



University of Pisa  
DESTEC-GRNSPG

Nuclear Research Group in San Piero a Grado (Pisa)- Italy

# Panel Session: **V&V and more in Nuclear Thermal-Hydraulics**

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***NUTHOS-12***

***12<sup>th</sup> International Topical Meeting on Nuclear Reactor Thermal-Hydraulics***

***Qingdao (PRC) - Oct. 14-18, 2018***

# PREFACE



## CONNECTION

NUCLEAR TECHNOLOGY & NRS  $\leftrightarrow$  SYS TH  $\leftrightarrow$  V & V

- 1950: NPP designed, no computers, no SYS TH
- 1960: Fundamental TH, computers appeared
- 1970: Complex TH, numerical codes appeared
- 1980: V & V needs, experimental Data Base available
- 1990: Uncertainty Methods (UM) appeared, V & V 'finalized' (SYS TH words)
- 2000: UM, V & V, Best Estimate Plus Uncertainty (BEPU) attempts
- 2010: Domain for BEPU

INDUSTRY & REGULATORS ACCEPTANCE NEEDED FOR BEPU

V & V ESSENTIAL

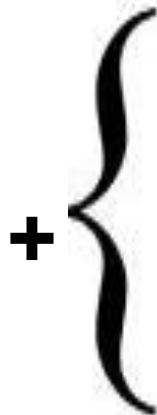
# LIST OF CONTENT



## 1) OUTLINE OF CURRENT/ADVANCED V&V

IDENTIFICATION

OF KEY TOPICS



**DOMAIN OF SIMULATION FOR SYSTEM CODES**

**Precision Objectives**

**Qualitative and Quantitative Accuracy Evaluation**

**CONSIDERATION OF 116 TH PHENOMENA**

**SCALING PART OF V&V – RECENT FINDINGS**

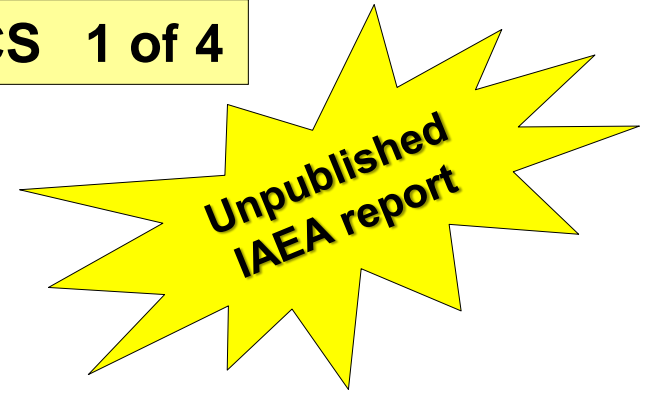
**INDEPENDENT ASSESSMENT & ASSESSMENT OF ASSESSMENT**

## 2) THE MOTIVATION FOR V&V&C

## 3) THE DEFINITION OF ‘C’ = CONSISTENCY

## 4) THE ROADMAP FOR V&V&C

## IDENTIFICATION OF KEY TOPICS 1 of 4



### 1. Design of system thermal-hydraulic codes:

- Domain of simulation
- Precision objective
- Attributes for safety analyses
- Scaling requirements

### 2. The development of codes implies the consideration of the following elements:

- Physical models
  - Fundamental models for thermal-hydraulics
  - Special thermal-hydraulic models
  - Non-thermal-hydraulic systems (noticeably nuclear fuel and neutron physics)
- Numerics
- Code implementation
  - Structure
  - Programming
  - Software Quality Engineering
- Code assessment strategy within the development process
- Code manual
- Life cycle
  - Quality Assurance

## IDENTIFICATION OF KEY TOPICS 2 of 4

### 3. The following areas form the Verification:

- Numerical algorithm and numerical solution

Numerical scheme

Verification matrix for numerical algorithm and solution

Types of verification solutions

Accuracy definition and numerical error estimation

Qualitative

Quantitative

Checklist for review and inspection

- Source code

Tools for verification

Portability of the code

Review and inspection

- Development of nodalization



Unpublished  
IAEA report

## IDENTIFICATION OF KEY TOPICS 3 of 4

### 4. The following areas form the Validation:

- **Physical laws and closure relations**
- **Validation matrix**
  - Basic tests**
  - Separate Effect Tests**
  - Integral Effect Tests**
  - Large scale experiments**
  - Containment experiments**
  - NPP data**
  - Scaling**
  - Code-to-code comparison**
- **Accuracy definition**
  - Qualitative**
  - Quantitative**
- **Validation report**
  - Assessment of validation**
- **Use of validation for uncertainty**
  - Sources of uncertainty**
- **Sensitivity tests**
- **Qualification of nodalization**
  - Thresholds of acceptability**
- **User effect**
- **User guidelines**

Unpublished  
IAEA report

# OUTLINE OF CURRENT V&V



## IDENTIFICATION OF KEY TOPICS 4 of 4

### 5. Independent assessment must be connected with code maintenance and improvement:

- Independent assessment (meaning of)
  - The matrix
  - Use of sensitivity analyses
  - Assessment of validation
- Maintenance (meaning of)

### 6. Code application in nuclear technology implies the consideration of the following topics:

- Key applications
  - Licensing and V & V
  - Technology consistent independent assessment matrix
  - Assessment of an individual calculation
  - Documents for user qualification
- Code application and uncertainty

Unpublished  
IAEA report

# OUTLINE OF CURRENT V&V



## DOMAIN OF SIMULATION FOR SYS TH CODES

### NPP FEATURES

### OTHER NEEDS

**NEEDS FOR THE  
OPERATION & DESIGN**

**THE DBA  
ENVELOPE**

- TH PHENOMENA
- OPERATOR TRAINING
- SCALING
- USER FRIENDLY
- ETC.

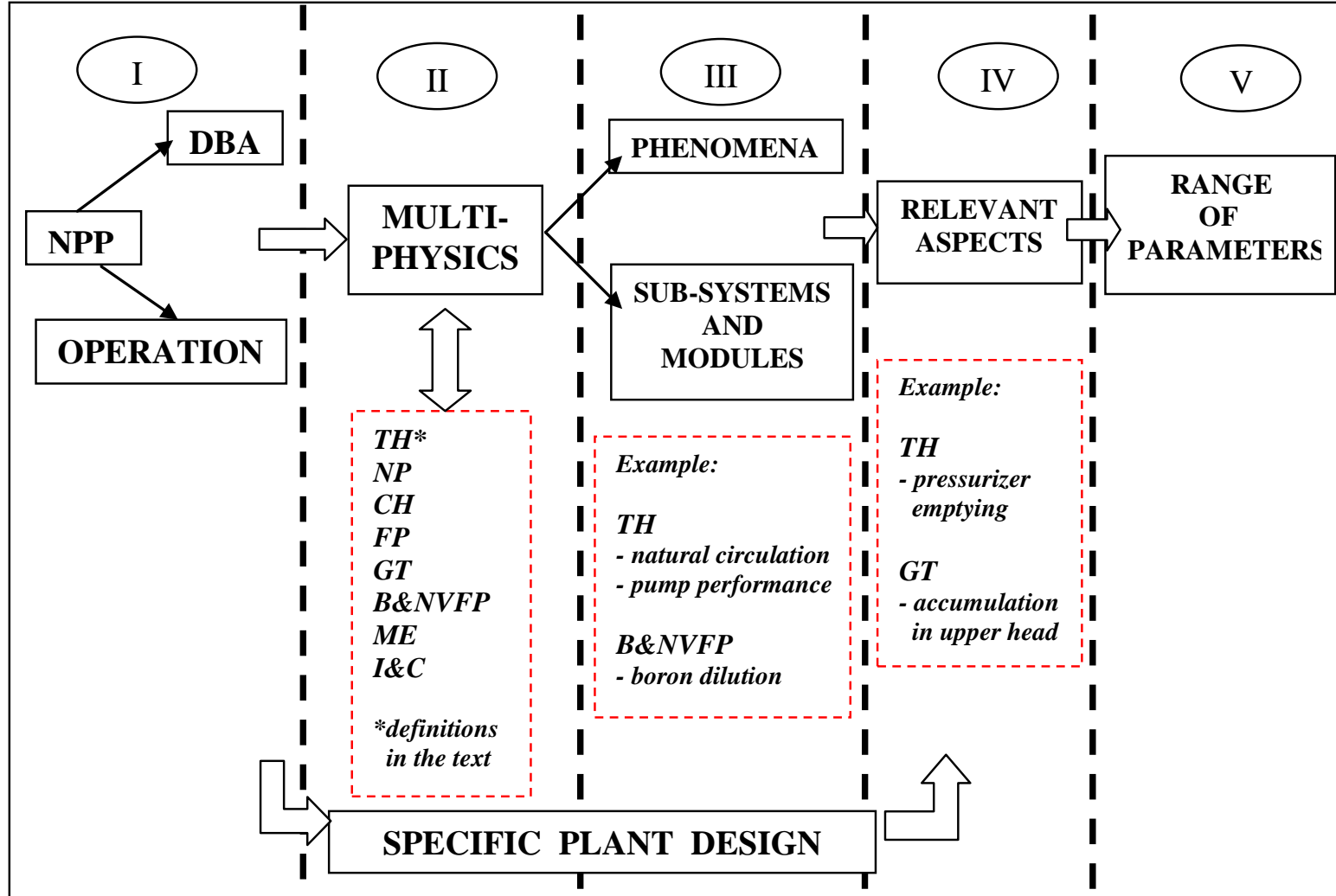


**ENTERING THE DIAGRAM IN  
THE NEXT SLIDE**



# OUTLINE OF CURRENT V&V

## DOMAIN OF SIMULATION FOR SYS TH CODES



# OUTLINE OF CURRENT V&V



## NOMENCLATURE & PRECISION OBJECTIVES

### *Nomenclature / Disciplines of Multi-physics*

B	= Boron including mixing and separation.
CH	= Chemistry restricted to the zirconium-water reaction.
FP	= Fuel performance.
GT	= Gas Transport (N <sub>2</sub> , H <sub>2</sub> and fission products) including mixing and separation.
I&C	= Instrumentation and Control including the logics.
ME	= Mechanics, e.g. dealing with the inertial behavior of rotors.
NP	= Neutron Physics.
NVFP	= Non-volatile fission products transport including mixing and separation.
TH	= Thermal-Hydraulics including conduction and radiation heat transfer.

### *Precision objectives*

The precision objectives shall be fixed in relation to each of the listed disciplines, making reference to:

- **Steady state.**
- **Transient at qualitative level.**
- **Transient at quantitative level.**

# OUTLINE OF CURRENT V&V



## PRECISION OBJECTIVES

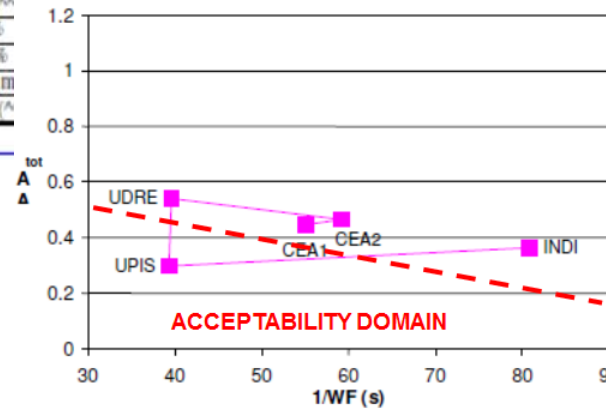
### Acceptance criteria for Nodalization

QUANTITY	ACCEPTABLE ERROR (%)
1 Primary circuit volume	1 %
2 Secondary circuit volume	2 %
3 Non-active structures heat transfer area (overall)	10 %
4 Active structures heat transfer area (overall)	0.1 %
5 Non-active structures heat transfer volume (overall)	14 %
6 Active structures heat transfer volume (overall)	0.2 %
7 Volume vs. height curve (i.e. "local" primary and secondary circuit volume)	10 %
8 Component relative elevation	0.01 m
9 Axial and radial power distribution (**)	1 %
10 Flow area of components like valves, pumps orifices	1 %
11 Generic flow area	10 %
(*)	
12 Primary circuit power balance	2 %
13 Secondary circuit power balance	2 %
14 Absolute pressure (PRZ, SG, ACC)	0.1 %
15 Fluid temperature	0.5 % (**)
16 Rod surface temperature	10 K
17 Pump velocity	1 %
18 Heat losses	10 %
19 Local pressure drops	10 % (*)
20 Mass inventory in primary circuit	2 % (**)
21 Mass inventory in secondary circuit	5 % (**)
22 Flow rates (primary and secondary circuit)	2 %
23 Bypass mass flow rates	10 %
24 Pressurizer level (collapsed)	0.05 m
25 Secondary side or downcomer level	0.1 m (*)

### Qualitative Accuracy

	BL-34	BL-44	SB-03	SB-04	6.2-TC	CL-21
Phase (a)	Pressurizer emptying	E	E	R	R	R
	Maximum break flowrate/initial loop flowrate	R	R	R	R	E
	Average specific break flowrate during phase (a)	R	R	R	R	E
Subcooled blowdown and first core dry out rewetting	First dry out duration	M	M	M	M	M
	Period for dry out starting at bottom level	E	E	E	E	E
	Period for dry out starting at middle level	M	E	M	E	M
	Period for dry out starting at high level	M	M	M	E	M
Pump stop or pump velocity	E	E	E	E	E	-

### Quantitative Accuracy – Application of FFTBM



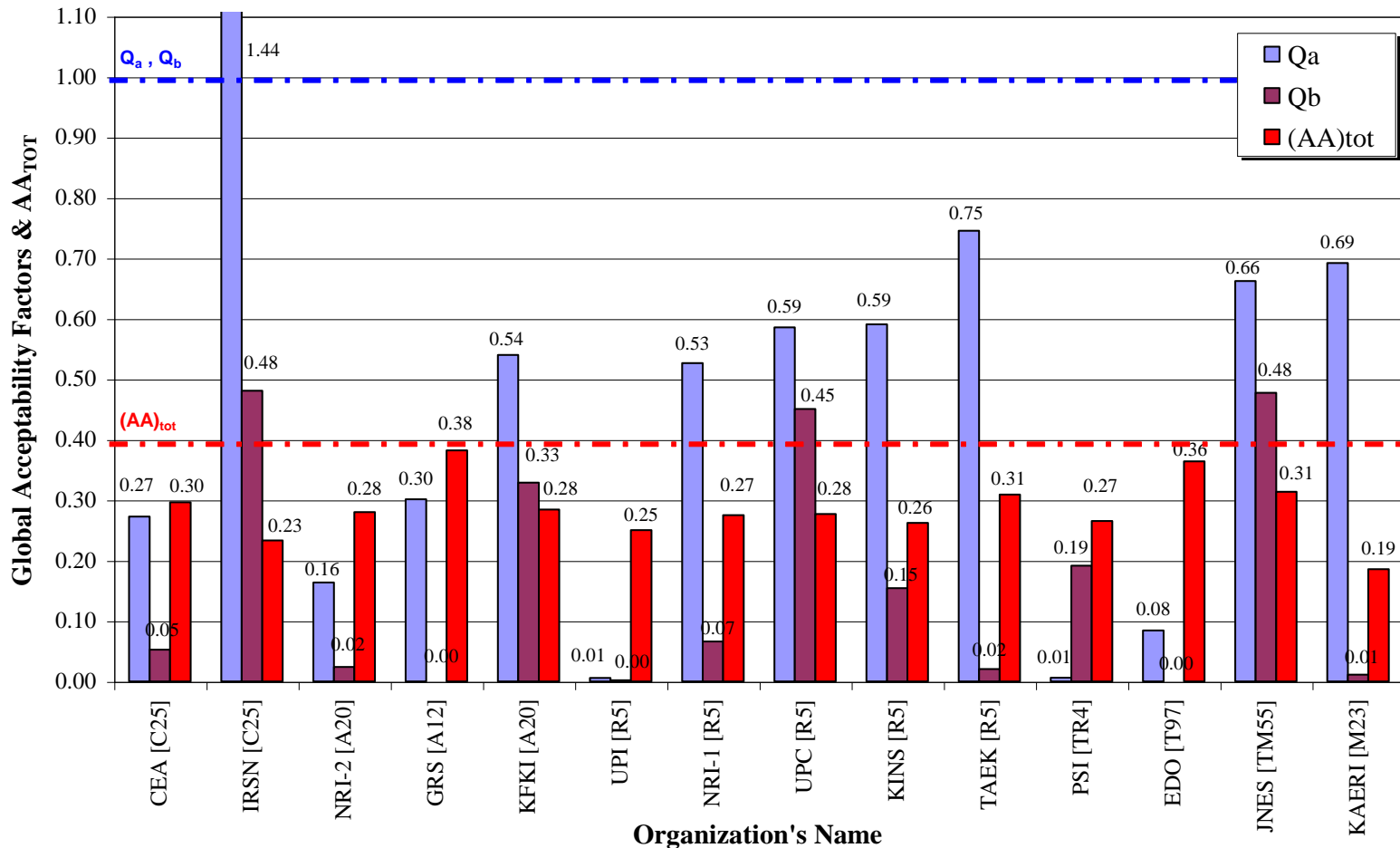
ID	Participant/Country/Code	NV	Time interval of analysis					
			0-3000 s		0-6000 s		0-8000 s	
			L/WF <sub>tot</sub>	AA <sub>tot</sub>	L/WF <sub>tot</sub>	AA <sub>tot</sub>	L/WF <sub>tot</sub>	AA <sub>tot</sub>
1	ZfK Dresden, Germany, R5/M2	25	9.2	0.30	*	*	*	*
2	KINS, Korea, R5/M3	25	10.0	0.68	*	*	*	*
3	LITK, Finland, CATH. 2 V1.2(3L)	25	9.4	0.62	*	*	*	*
4	Kurchatov Institute, Russia, R5/M3	25	9.7	0.40	*	*	*	*
5	CAE Beijing, China, R5/M2	25	8.8	0.31	*	*	*	*
6	JINS, Japan, R5/M2	24	8.6	0.40	*	*	*	*
7	JAERI, Japan, CATH. 2 V1.2(3L)	22	8.5	0.47	*	*	*	*
8	Studsvik Nuclear, Sweden, R5/M3	25	8.4	0.25	10.5	0.26	*	*
9	NNC, England, NOTRUMP	24	7.3	0.51	12.3	0.51	*	*
10	Kurchatov Institute, Russia, R5/M2	25	9.4	0.46	14.9	0.60	*	*
11	JAERI, Japan, CATH. 2 V1.2(2L)	22	8.5	0.47	11.1	0.33	*	*
12	Paul Scherrer, Switzerland, R5/M2	25	10.0	0.50	14.3	0.39	*	*
13	GRS, Germany, ATHLET MLE	25	8.4	0.39	10.4	0.36	*	*
14	Tractebel, Belgium, R5/M2	25	8.8	0.41	14.0	0.44	*	*
15	UKAEA, England, R5/M2	25	8.6	0.27	11.5	0.29	*	*
16	VTT, Finland, R5/M3	25	9.7	0.38	11.5	0.33	10.9	0.39
17	Texas AML, USA, R5/M3	25	9.6	0.39	12.0	0.49	14.2	0.44
18	UKAEA, England, R5/M3	25	8.6	0.35	10.6	0.30	10.8	0.30
19	KAERI, Korea, CATH. 2 V1.2(2L)	23	8.1	0.49	11.5	0.41	11.6	0.42
20	IPENCNEN, Brazil, TRAGOPP1	22	8.2	0.35	12.0	0.35	10.8	0.40
21	DCMENEVA, Italy-Croatia, R5/M2	22	8.0	0.36	12.6	0.36	12.5	0.38
22	AELI, Hungary, R5/M2	25	10.0	0.52	15.8	0.55	14.1	0.55
23	Jozef Stefan Institute, Slovenia, R5/M3	25	8.4	0.38	14.8	0.40	13.8	0.39
24	Jozef Stefan Institute, Slovenia, R5/M2	25	8.4	0.39	11.9	0.55	14.9	0.44
25	NPPRI, Slovakia, R5/M2	22	9.1	0.40	10.6	0.56	9.9	0.64

# OUTLINE OF CURRENT V&V



## PRECISION OBJECTIVES – Nodalization & Calculation Acceptance

Results related to the “acceptability of input deck” are reported in the same diagram as results of “acceptability of calculation results”.



# OUTLINE OF CURRENT V&V



## CONSIDERATION OF 116 PHENOMENA

Nuclear Engineering and Design 330 (2018) 166–186



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Nuclear Engineering and Design

journal homepage: [www.elsevier.com/locate/nucengdes](http://www.elsevier.com/locate/nucengdes)

Thermal-hydraulic phenomena for water cooled nuclear reactors

N. Aksan<sup>a</sup>, F. D'Auria<sup>b,\*</sup>, H. Glaeser<sup>c</sup>

Excerpt from list of 116

CROSS-LINK WITH  
ACCIDENT SCENARIOS  
PARAMETERS



			PWR-V	
I-13-NC6	NC with horizontal SG			
A-11-NC	NC RPV and containment & various system configurations		New Reactors	Also containment
S-42-NC0-c	Natural convection and H2 distribution	SETF		Inside containment
S-43-NCG	Non condensable gas effect including condensation HT in RCS	SETF	PWR	Also ITF
	Nuclear fuel behavior		SETF/ITF	See I-29-NTF2
I-16-NTF1	Nuclear thermal-hydraulics feedback and spatial effect (see also I-29-NTF2)	ITF	BWR	Also RBMK, ABWR, etc.
	Nuclear thermal-hydraulics instabilities	ITF	BWR	See I-16 and S-44
S-44-PCEI	Parallel channel effects and instabilities PCEI	SETF	BWR	
S-45-PSB	Phase separation at branches (including effect on TPCF)	SETF	All	Also ITF (T-branches)
S-46-PS1	Phase separation/vertical flow with and w/o mixture level-Core	SETF		Also ITF
S-47-PS2	Phase separation/vertical flow with and w/o mixture level-Downcomer			
S-48-PS3	Phase separation/vertical flow with and w/o mixture level-Pipes & Plena			
I-17-PFU	Pool formation in UP	ITF	PWR	See also S-8-CCF6
B-7-PD-c	Pressure drops at geometric discontinuities, including containment	Basic	N/A	Also New Reactors
B-8-PW	Pressure wave propagation including CIWH	Basic		
I-18-PRB	Pressure-temperature increase & boiling due to energy and mass input	ITF	PW/BWR	Containment & Shutdown
I-19-PRZ	PRZ thermal-hydraulics	ITF	PWR	
S-49-QF1	QF propagation/rewet-Fuel rods	SETF	PWR/BWR	
S-50-QF2	QF propagation/rewet-Channel walls, Water rods		BWR	
I-20-RF	Refill including loop refill in PWR-O	ITF	PWR/BWR	PHW rather than PH
I-21-RE	Reflow			
I-22-RCM	Reflux condenser mode and CCFL	ITF	PWR	
	Return to Nucleate Boiling (RNB)			See Reflood & QF
S-51-SEP	Separator behavior (&* flooding, steam penetration, liquid carry-over)	SETF	PWR/BWR	*Mainly for BWR
I-23-SIP	SG siphon draining (SG interaction with ESF, including gravity driven)	ITF	PWR	Shutdown conditions
S-63-SPR-c	Spray effects-Containment (added T-HP)	SETF	All	
S-52-SPR1	Spray effects-Core (including cooling and distribution)		BWR	

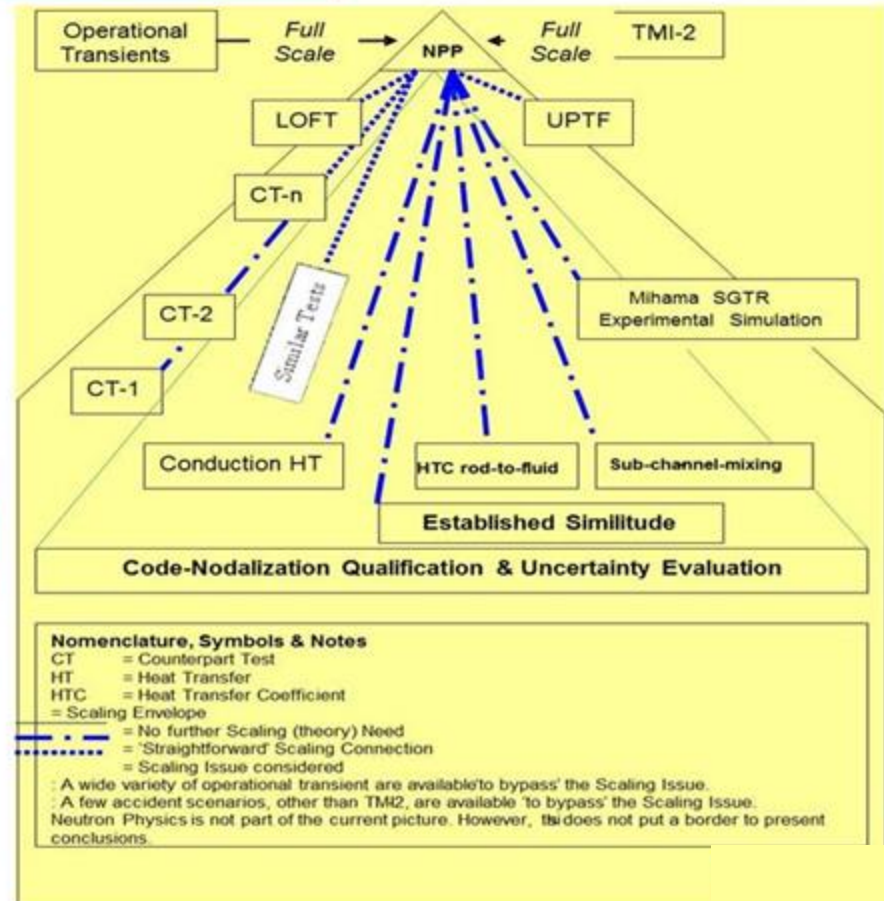
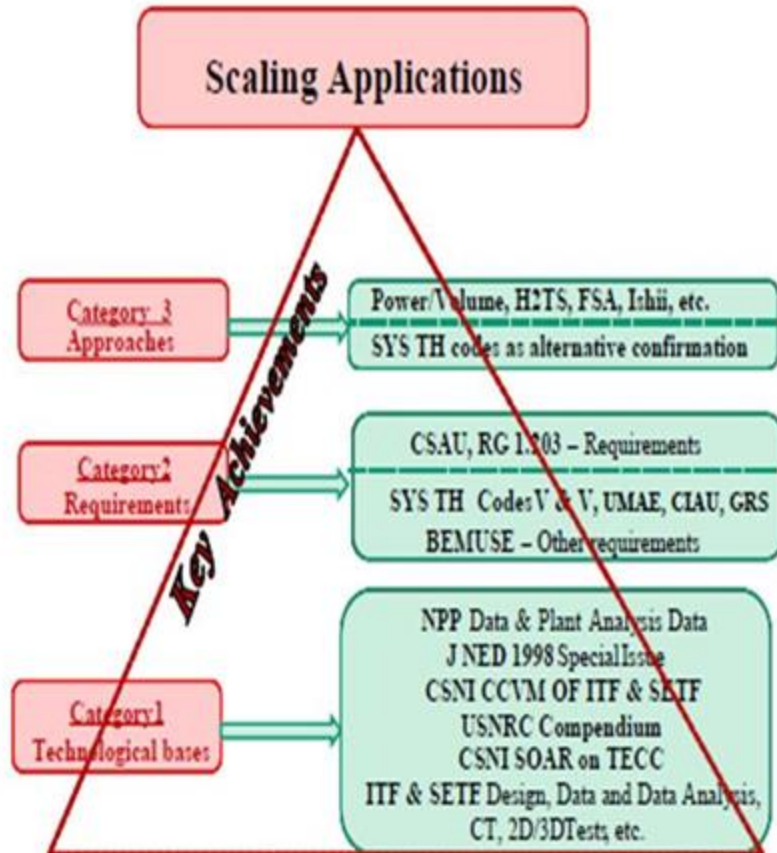
# OUTLINE OF CURRENT V&V

## SCALING

### HIERARCHY & KNOWLEDGE MANAGEMENT

### 'BRIDGES' & ACHIEVEMENTS

OECD/NEA/CSNI S-SOAR – 2017



# OUTLINE OF CURRENT V&V



## INDEPENDENT ASSESSMENT & ASSESSMENT OF ASSESSMENT

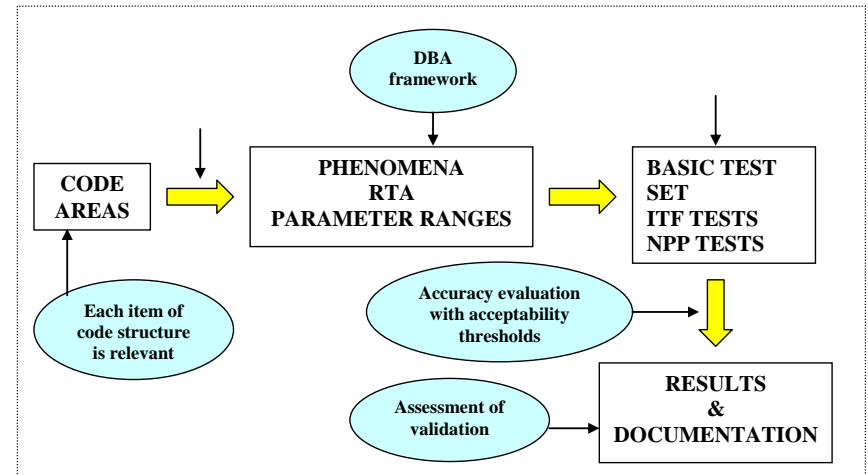
**QUALIFIED CODE USER / USER GROUP**

**(demonstration of)  
INDEPENDENT ASSESSMENT**

**SENIOR EXPERT**  
*(not involved with calculation preparation-outcomes)*

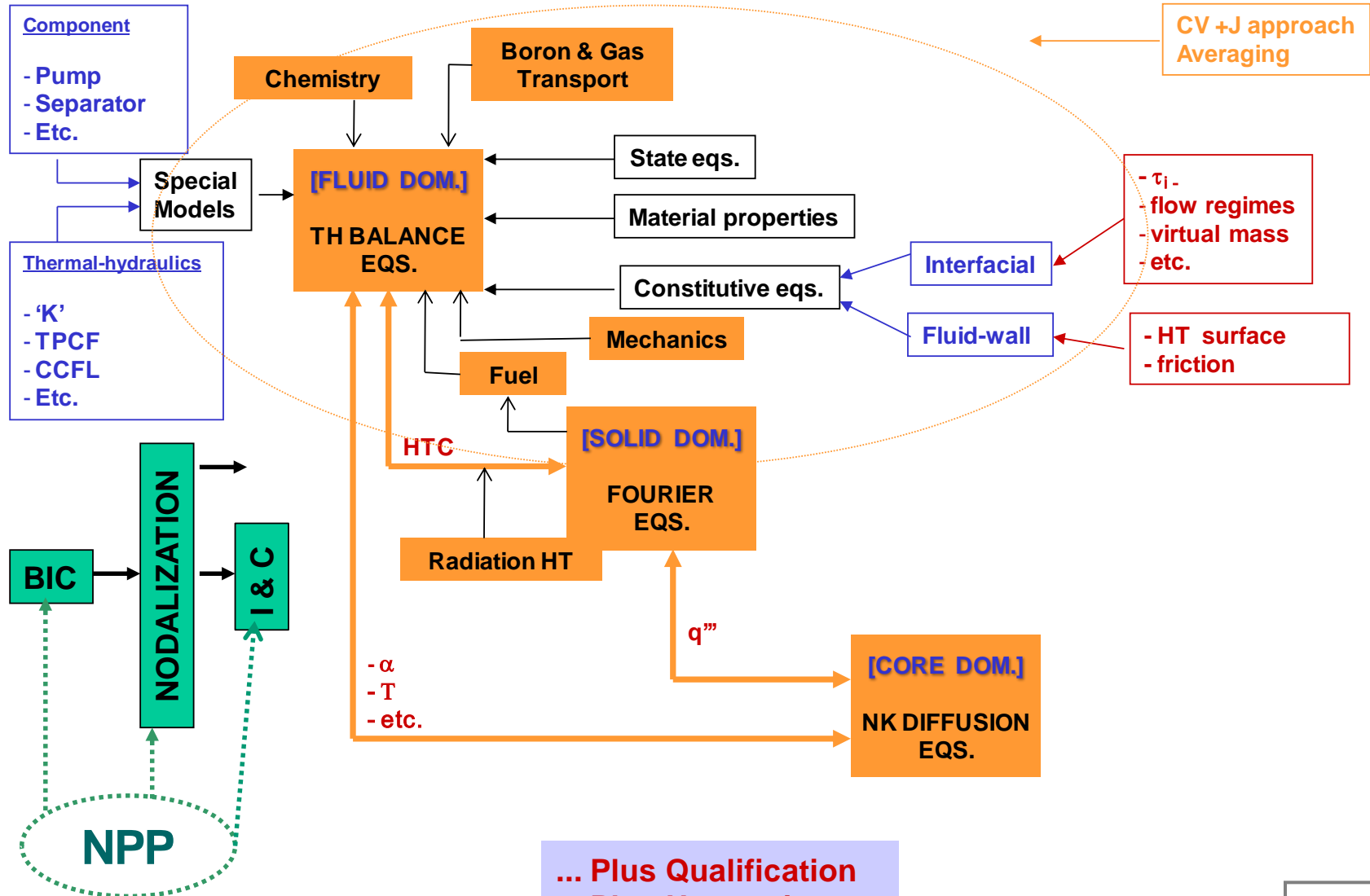
**(demonstration of)  
ASSESSMENT OF ASSESSMENT**

No	Type	Concerned NPP	Concerned phenomenon or DBA	Notes
1	Basic	-	Bottle emptying	To test code features
2			U-tube manometer	To test code features and dependency of results upon boundary conditions
3			Pressure drops in two phase flow	
4	SET	PWR	TPCF	Key phenomenon for DBA Analysis
5	ITF		Transient CHF	
6		SBLOCA		
7	ITF	PWR		Counterpart Test, to address the scaling issue
8	NPP			To perform Kv-scaled calculation



# OUTLINE OF CURRENT V&V

**(RESULTING) ATTRIBUTE FOR A SYS TH CODE** *(unchanged in 40+ years)*





# OUTLINE OF CURRENT V&V



## SEVERAL INTERCONNECTED TOPICS

NUMERICS

COMPUTER-COMPILER ISSUES

USER CONVENIENCES

QUALITY ASSURANCE

LIFE CYCLE

UNCERTAINTY

SCALING

USER QUALIFICATION

INPUT - OUTPUT

USER EFFECT

**MULTI-DISCIPLINARY  
MULTI-PHYSICS  
MULTI-SCALE  
MULTI-ACTORS**

**CONTEXT**

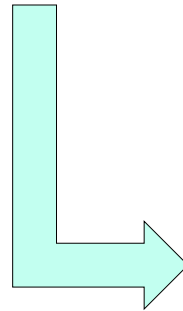
# THE MOTIVATION FOR V & V & C



## 1) Accuracy of NTH models lags behind technological needs

Thermal-hydraulic model capabilities are found to lag behind industry needs, requiring innovative ideas and investigations; the following log-frame to be considered:

**V&V**  
Capability  $\leftrightarrow$  Development and Qualification.



**DESIGN LEVEL**  
**INDUSTRY PROPOSAL FOR NEW RESEARCHES**  
*(e.g. spacer grids, CHF margins)*

**SAFETY LEVEL**  
**NC FLOWRATE**  
*(e.g. Max NC flow,  
Conditions for flow reversal in SG U-tubes)*



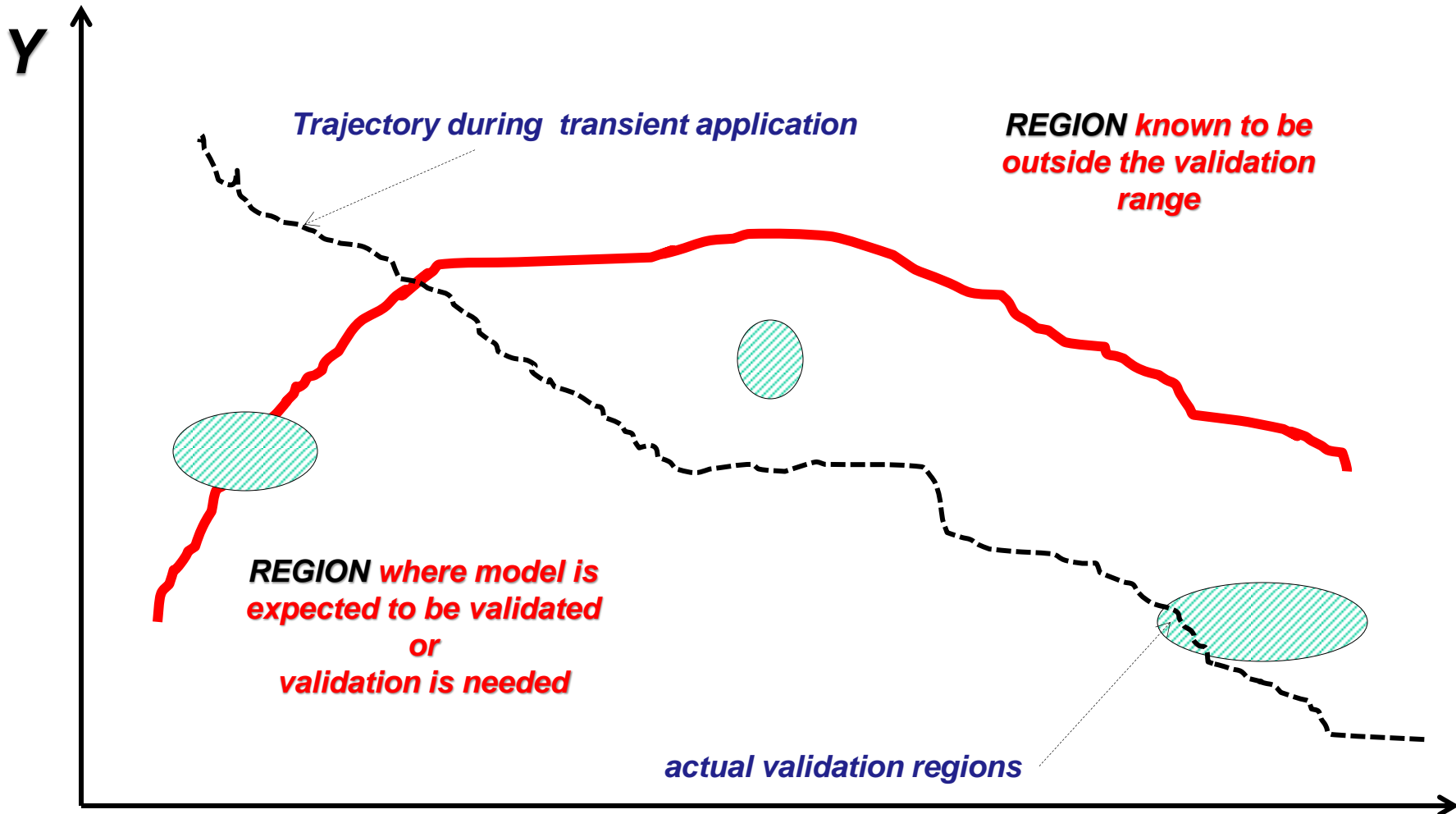
**PRIORITIZATION OF RESEARCH  
NEEDED**  
*(to confirm adequacy of  
research strategy and needed V&V)*

**NOT DONE! (method available)**

# THE MOTIVATION FOR V & V & C

## 2) Inadequate experimental database (EXP-DB) - 1 of 2

(showing) NARROW PARAMETER RANGES FOR VALIDATION



# THE MOTIVATION FOR V & V & C

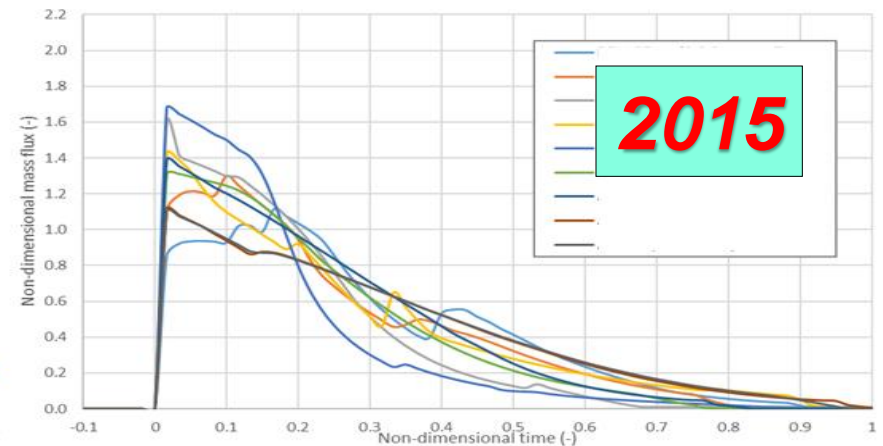
## 2) Inadequate experimental database (EXP-DB) - 2 of 2

CSNI SETF & ITF EXP-DB COVERS (<) < 1% parameter ranges / combinations expected in DBA of WCNR → Situation may be worse looking at 116 TH phenomena.

**EXP DB**  
**ESSENTIAL FOR VALIDATION, BUT NOT ENOUGH**

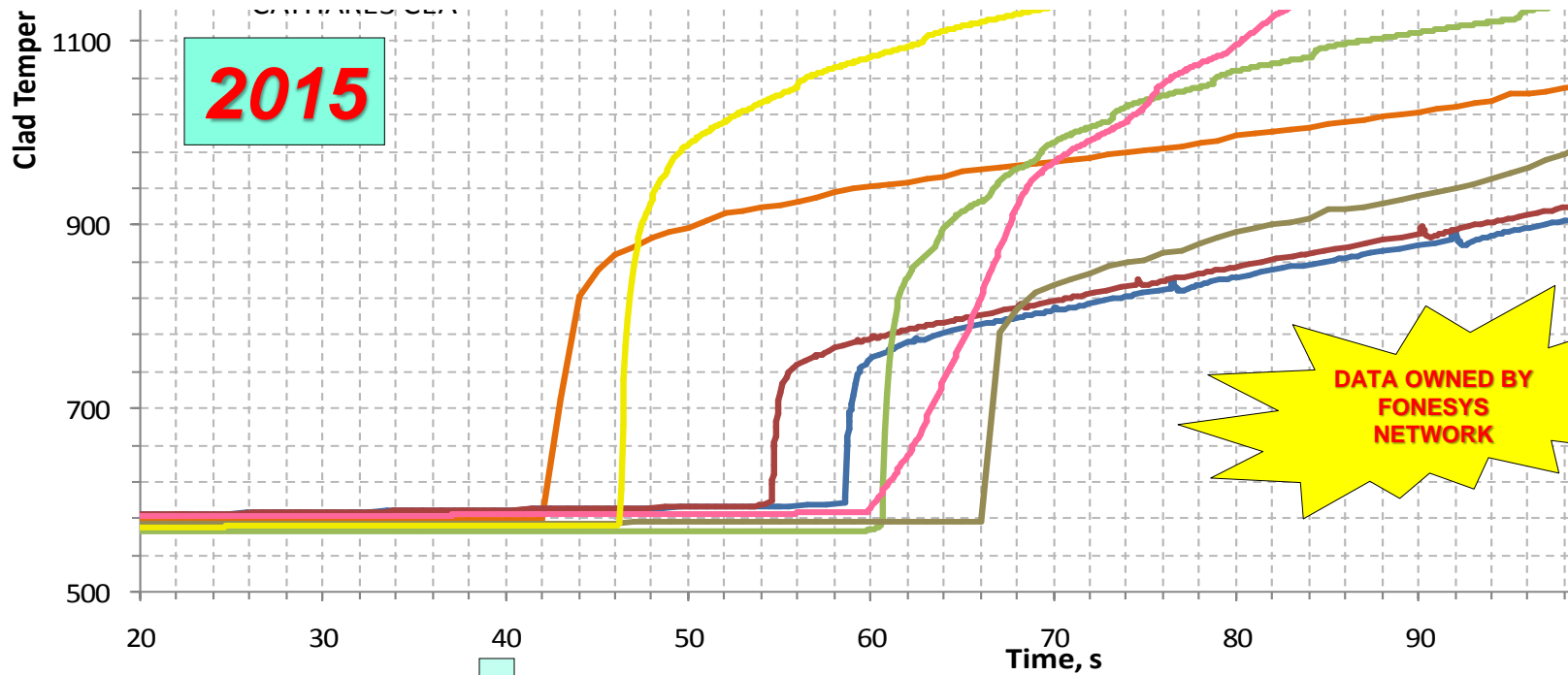
**SURROGATE OF EXPERIMENTS**  
**NEEDED**

## 3) Inadequate V & V role: (a) - TPCF



# THE MOTIVATION FOR V & V & C

## 3) Inadequate V & V role: (b) – CHF & FILM BOILING



**CURRENT V & V**  
**NOT ENOUGH TO IMPROVE MODELING CAPABILITIES**

**NEED TO ENLARGE**  
**THE SCOPE FOR V&V**

→ 'C'

# THE DEFINITION OF 'C' = CONSISTENCY



The rudimentary and broad definition for consistency is:

***'Consistency is an activity connected with the development and the qualification of numerical codes which covers topics not considered by current V&V'.***

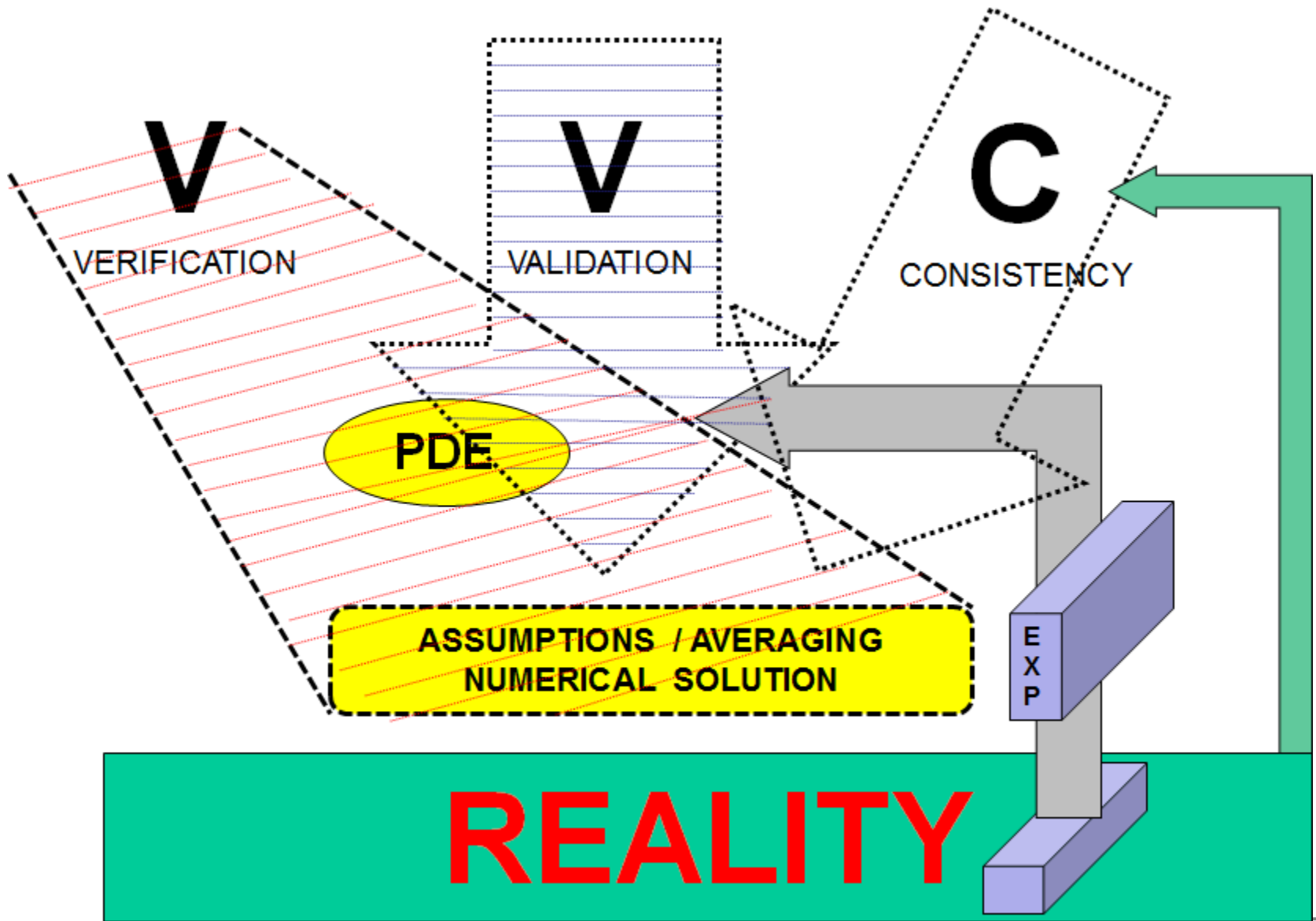
A detailed definition is expected to cover the above issues, to take into account of (fundamental) physical laws even in the absence of experimental evidence, to provide a hint for the development of computational tools and to not contradict the current V&V (rather being complementary to the existing procedures):

∴

***'Consistency aims at filling gaps of V&V in nuclear thermal-hydraulics, with focus:***

- a) to connect the modeling features and the technological needs,***
- b) to take into account of the limitations of the experimental database mainly in terms of the space covered by parameter ranges,***
- c) to streamline the conditions for developing improved capability models'.***

# THE DEFINITION OF 'C' = CONSISTENCY



# THE ROADMAP FOR V&V&C



Step I – **THE ELEMENTS** (consistent geometry and material properties parameter ranges)

Step II – **THE LOGFRAME** (cross linking phenomena and models)

Step III – **THE REQUIREMENTS** (consistency requirements for models)

**I) THE ELEMENTS**  
**CLASSIFYING SSC FOR CORE, RPV, PS, SS, BOP, AND CONTAINMENT,**  
**(including materials)**

**IDENTIFYING/CHARACTERIZING N (> 100)**

**II) THE LOGFRAME**

**N (geometry-material) ELEMENTS**  
**116 TH PHENOMENA**  
**M (a few dozen) MODELS**  
**L (a couple dozen) PARAMETERS**

**CROSSLINKING N-116-M-L**

**III) REQUIREMENTS**

**.... NEXT SLIDE ....**



## THE REQUIREMENTS FOR [V&V&]C

### IIIa – General requirements (selected)

Any model should be applicable inside the resulting parameters space, **i.e. inside and outside the range of validation** in relation to which experimental data exist. Outside the range of validation, model predictions should be physically sound (according to the best available information).

**Pressure drops at GD for 1 and 2-phase:** fluid velocities (from zero to values corresponding to TPCF or to sound speed in both directions) and void fraction (from zero to one) at the minimum section should be independent parameters; the generation of cavitation conditions should be detected (e.g. as a function of subcooling) where necessary.

**Each phenomenon shall have at least one associated model.** Special models and models forming the set of constitutive equations can be distinguished.

A study of each mathematical function (M-Models) must be performed checking for peaks, valleys and singular points (including cliff-edge effects and/or conditions for bifurcations); the study shall include **the derivative(s) of the function, e.g.  $\partial(\text{output quantity}) / \partial(\text{input quantity})$** . Continuity shall be demonstrated or discontinuities justified.

Each model shall be checked for zero and negative fluid velocities, different liquid and steam velocities, the full range of pressure and temperature (etc.).

**Acceptability criteria**, shall be established in relation to each requirement.

# THE ROADMAP FOR V&V&C



## THE REQUIREMENTS FOR [V&V&]C

### IIIb – Requirements for individual TH phenomena (one example)

	Topic / Parameters	Activity	Requirements
		Systematic calculations, rows 1-11, varying upstream pressure ( $p_0$ ) and void ( $\alpha$ ), rows 12-13, varying local pressure drop at pipe inlet, to derive:	
1	Liquid and steam velocities ( $w_g$ , $w_f$ ) and Slip ratio (S)	curves of $w_g$ , $w_f$ and S at CS	A) Demonstrate rate continuity of functions, derivatives, etc. (Step IIIa, requirement 5.4)
2			B) Justify differences related to the condition $S=1$
3			C) At critical pressure, S must be unity
4			D) Derive and explain differences between $w_g$ , $w_f$ and sound speed.
5	Supersonic flow downstream the CS	values of $w_f$ and $w_g$ in supersonic conditions	E) This must be computed by the model, together with return to subsonic flow.
6	Energy transfer across the CS	kinetic and thermal energy values	F) Thermal plus kinetic energy at pipe inlet must be consistent at pipe outlet
7	End of “critical conditions” at CS	occurrence of criticality conditions at CS	G) Demonstrate that non-critical flowrate $\geq$ critical flowrate (TPCF)
8		pressure at CS ( $p_{cr}$ )	H) Demonstrate that if containment pressure ( $p_{co}$ ) $>$ $p_{cr}$ , non-critical flowrate occurs
9	Derivative $\partial(p_0-p_{co}) / \partial(\text{TPCF})$	derivative in first column	I) Demonstrate continuity (or explain discontinuities)
10	Model a reservoir and a containment-blowdown analysis (isolated-ideal system, no structures)	entropy of the ‘isolated system’	J) Demonstrate the total entropy increase in the isolated system
11			K) Demonstrate peak pressure in containment not depending upon TPCF
12	Cavitation at pipe inlet	pressure along the pipe	L) Demonstrate dependency of TPCF upon subcooling in reservoir
13			M) Demonstrate the possibility of occurrence of TPCF at pipe inlet

# CONCLUSIONS (advanced V&V)



- **V & V TARGETS DERIVED FROM CODE DEVELOPMENT, NPP FEATURES & DBA ENVELOPE**
- **THE NODALIZATION IS A TARGET FOR V & V AND A SOURCE OF ERRORS**
- **ADDRESS THE SCALING ISSUE IS PART OF V & V**
- **CONNECT INPUT & OUTPUT ERRORS AND ERROR ACCEPTABILITY**
- **DISTINGUISH, WITHIN THE V & V PROCESS, BETWEEN SENSITIVITY AND UNCERTAINTY AND CONSIDER BOTH**
- **RECOGNIZE THE NEED OF QUALIFIED USERS FOR V & V, THEN 'INDEPENDENT ASSESSMENT' AND 'ASSESSMENT OF ASSESSMENT'**

# CONCLUSIONS (V&V&C)



The **current V&V proved inadequate** for model improvement & the **supporting experimental DB for validation covers only a small portion of the parameters space characterizing NPP nominal and transient conditions..**

**V&V&C is a (large) human-and-financial resource-consuming approach.**

*Checking the physical validity of the results in a parameter space where experimental data are not available, creating crosslink matrices involving parameters, phenomena, NPP components and models and imposing conditions to the mathematical formulation of models, can be used for a snapshot characterization of the 'C' field.*

'C' covers both Verification and Validation. V&V&C will have a role in model development while V&V only proves model capabilities.

**The change from V&V to V&V&C appears an epochal change in the area of code development and validation,**

**V&V&C IS NOT A TECHNOLOGY BY ITSELF**

**V&V&C MAY REVEAL AS A MULTIDISCIPLINARY TASK**

**V&V&C MUST BE ADAPTED TO ANY TECHNOLOGY**