

**EUR 4673 e**

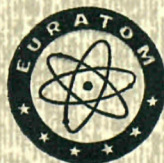
COMMISSION OF THE EUROPEAN COMMUNITIES

**COSTANZA (R, Z)**

by

**E. VINCENTI and A. CLUSAZ**

**1971**



**Joint Nuclear Research Centre  
Ispra Establishment - Italy**

**Nuclear Studies Division**

## LEGAL NOTICE

This document was prepared under the sponsorship of the Commission of the European Communities.

Neither the Commission of the European Communities, its contractors nor any person acting on their behalf :

make any warranty or representation, express or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this document, or that the use of any information, apparatus, method or process disclosed in this document may not infringe privately owned rights; or

assume any liability with respect to the use of, or for damages resulting from the use of any information, apparatus, method or process disclosed in this document.

This report is on sale at the addresses listed on cover page 4

at the price of F.Fr. 13.08	B.Fr. 125.—	DM 9.20	It.Lire 1.560	Fl. 9.—
-----------------------------	-------------	---------	---------------	---------

**When ordering, please quote the EUR number and the title which are indicated on the cover of each report.**

Printed by Guyot s.a., Brussels  
Luxembourg, July 1971

This document was reproduced on the basis of the best available copy.

## EUR 4673 e

COSTANZA (R, Z) by E. VINCENTI and A. CLUSAZ

Commission of the European Communities  
Joint Nuclear Research Centre - Ispra Establishment (Italy)  
Nuclear Studies Division  
Luxembourg, July 1971 - 78 Pages - 7 Figures - B.Fr. 125,—

The code Costanza (R, Z) written in Fortran for IBM 360/65 solves the kinetic diffusion equations in two groups and two dimensions (radial and axial for a cylindrical reactor). The neutronic calculation is coupled with the calculation of the heat transmission from the fuel to the cladding and to the coolant, and with the thermohydraulics of channels with forced circulation of a single phase coolant.

Fuel elements of prismatic shape are considered. Up to 6 groups of delayed neutrons are allowed.

Temperature feedback of fuel and coolant are considered independently and affect the nuclear constants. Control rod movement or diffused poison

---

## EUR 4673 e

COSTANZA (R, Z) by E. VINCENTI and A. CLUSAZ

Commission of the European Communities  
Joint Nuclear Research Centre - Ispra Establishment (Italy)  
Nuclear Studies Division  
Luxembourg, July 1971 - 78 Pages - 7 Figures - B.Fr. 125,—

The code Costanza (R, Z) written in Fortran for IBM 360/65 solves the kinetic diffusion equations in two groups and two dimensions (radial and axial for a cylindrical reactor). The neutronic calculation is coupled with the calculation of the heat transmission from the fuel to the cladding and to the coolant, and with the thermohydraulics of channels with forced circulation of a single phase coolant.

Fuel elements of prismatic shape are considered. Up to 6 groups of delayed neutrons are allowed.

Temperature feedback of fuel and coolant are considered independently and affect the nuclear constants. Control rod movement or diffused poison

---

## EUR 4673 e

COSTANZA (R, Z) by E. VINCENTI and A. CLUSAZ

Commission of the European Communities  
Joint Nuclear Research Centre - Ispra Establishment (Italy)  
Nuclear Studies Division  
Luxembourg, July 1971 - 78 Pages - 7 Figures - B.Fr. 125,—

The code Costanza (R, Z) written in Fortran for IBM 360/65 solves the kinetic diffusion equations in two groups and two dimensions (radial and axial for a cylindrical reactor). The neutronic calculation is coupled with the calculation of the heat transmission from the fuel to the cladding and to the coolant, and with the thermohydraulics of channels with forced circulation of a single phase coolant.

Fuel elements of prismatic shape are considered. Up to 6 groups of delayed neutrons are allowed.

Temperature feedback of fuel and coolant are considered independently and affect the nuclear constants. Control rod movement or diffused poison

concentrations are simulated by externally imposed variations of the thermal absorption cross section in the different regions of the reactors.

Inlet temperatures and mass flow in the coolant channels may be varied according to any externally given time table.

Before starting the transient initial calculations put the reactor in criticality conditions at a given power level in equilibrium with the temperature feedback.

---

concentrations are simulated by externally imposed variations of the thermal absorption cross section in the different regions of the reactors.

Inlet temperatures and mass flow in the coolant channels may be varied according to any externally given time table.

Before starting the transient initial calculations put the reactor in criticality conditions at a given power level in equilibrium with the temperature feedback.

---

concentrations are simulated by externally imposed variations of the thermal absorption cross section in the different regions of the reactors.

Inlet temperatures and mass flow in the coolant channels may be varied according to any externally given time table.

Before starting the transient initial calculations put the reactor in criticality conditions at a given power level in equilibrium with the temperature feedback.

**EUR 4673 e**

COMMISSION OF THE EUROPEAN COMMUNITIES

**COSTANZA (R, Z)**

by

**E. VINCENTI and A. CLUSAZ**

1971



**Joint Nuclear Research Centre  
Ispra Establishment - Italy**

**Nuclear Studies Division**

## ABSTRACT

The code Costanza (R, Z) written in Fortran for IBM 360/65 solves the kinetic diffusion equations in two groups and two dimensions (radial and axial for a cylindrical reactor). The neutronic calculation is coupled with the calculation of the heat transmission from the fuel to the cladding and to the coolant, and with the thermohydraulics of channels with forced circulation of a single phase coolant.

Fuel elements of prismatic shape are considered. Up to 6 groups of delayed neutrons are allowed.

Temperature feedback of fuel and coolant are considered independently and affect the nuclear constants. Control rod movement or diffused poison concentrations are simulated by externally imposed variations of the thermal absorption cross section in the different regions of the reactors.

Inlet temperatures and mass flow in the coolant channels may be varied according to any externally given time table.

Before starting the transient initial calculations put the reactor in criticality conditions at a given power level in equilibrium with the temperature feedback.

## KEYWORDS

FORTRAN  
2-DIMENSIONAL CALCULATIONS  
CYLINDERS  
PROGRAMMING  
NEUTRON DIFFUSION EQUATION  
HEAT TRANSFER  
CANNING

COOLANTS  
FUEL PINS  
FORCED CONVECTION  
TEMPERATURE COEFFICIENT  
DOPPLER REACTIVITY  
POWER COEFFICIENT  
CRITICALITY

Contents:

Purpose	page 5
Subdivision of the reactor	" 5
Control rods	" 9
Neutronic initialization	" 10
Criticality search	" 10
Dynamic Calculations	" 11
Method	" 13
Thermal calculation	" 17
Temperature reaction	" 19
Programme composition	" 21
First overlay	" 21
Second overlay	" 22
Input instructions	" 27
Restart and interrupted calculation	" 44
Sample problem input	" 48
Sample problem output	" 57
Appendix A	" 71
Appendix B	" 76





Purpose \*)

The code COSTANZA (R,Z) treats the dynamics of a cylindrical nuclear reactor. Two geometrical dimensions: radius R and height Z are considered, in the supposition that there is an axial symmetry.

The nuclear time dependent diffusion equations, in two energy groups are solved numerically with the finite-difference approximation.

The heat transfer equation from heating element to coolant in the channel are also solved numerically.

The geometry of the heating element can be of any type, because the element is characterized by thermal resistances and thermal capacities given in input.

Flow rates and inlet temperatures of coolant are externally imposed as input function of time.

The coolant is a single phase fluid which may be liquid or gas, and is not supposed to boil or condense during the transient.

This code has been developed to treat accidents of intermediate gravity in which detailed spatial aspects of the phenomena are essential, but which are not so grave as to produce alterations or damages of the core lattice, or changes of the state in the coolant.

Subdivision of the reactor

The reactor is supposed to be cylindrical with axial symmetry. Therefore only a vertical section of it

---

\*) Manuscript received on March 15, 1971

will be considered. (Fig.1). This section is a rectangular surface limited by the axis of the cylinder at one side and by the extrapolated boundary of the reactor at the other three sides. This area is subdivided with a cartesian point-lattice. In the radial direction the points may be arbitrarily spaced. In the axial direction the points are located at equal intervals within the core and they may be arbitrarily distributed in the upper and lower reflector. Within the core the axial point lattice is the same for neutronic and thermal calculations.

The outer points are located at the extrapolate boundaries of the reactor. The first two vertical arrays of points are located symmetrically to the axis of the cylinder to obtain the condition of zero space derivative at the axis. The lattice may have a maximum of 400 points, product of the radial and axial points. The radial and axial points cannot be more than 40 each. In Fig. 1 is a point-lattice of 12 axial x 10 radial = 120 points. The core may be subdivided in a maximum number of 30 regions having different core compositions; their boundaries are defined by points of the lattice. The initial criticality condition may be obtained by diffused poison or by the insertion of control rods.

Each region contains a uniformly distributed poison, whose concentration may be varied for criticality search to obtain the initial equilibrium condition, but will remain constant during the transient.

The core is also subdivided in vertical control zones. There can be a maximum number of 10 control zones. They have the same height of the reactor, including top and bottom reflectors if existing. Each zone contains control rods which move in vertical direction independently from the rods of the other zones. Each zone is also characterized by a certain type of cooling channel which may be different from those of the other zones. The coolant and the rods may enter into the core from apposite sides (rod from top and coolant from bottom or viceversa) or they may enter from the same side. Conventionally we assume that the coolant inlet is at the top and the points of the lattice are numerated starting from the top left along the radii down to the bottom right. The entrance side of the rods will be indicated in input.

The vertical boundaries of the control zones are defined by points of the lattice. They may not coincide with the vertical boundaries of the composition regions.

Fig. 1 indicates 5 regions, the first of which includes the top, bottom and radial reflector considered all of the same composition in this case; regions 2,3,4,5 belong to the core. Two vertical central zones (I,II) contain the rods.

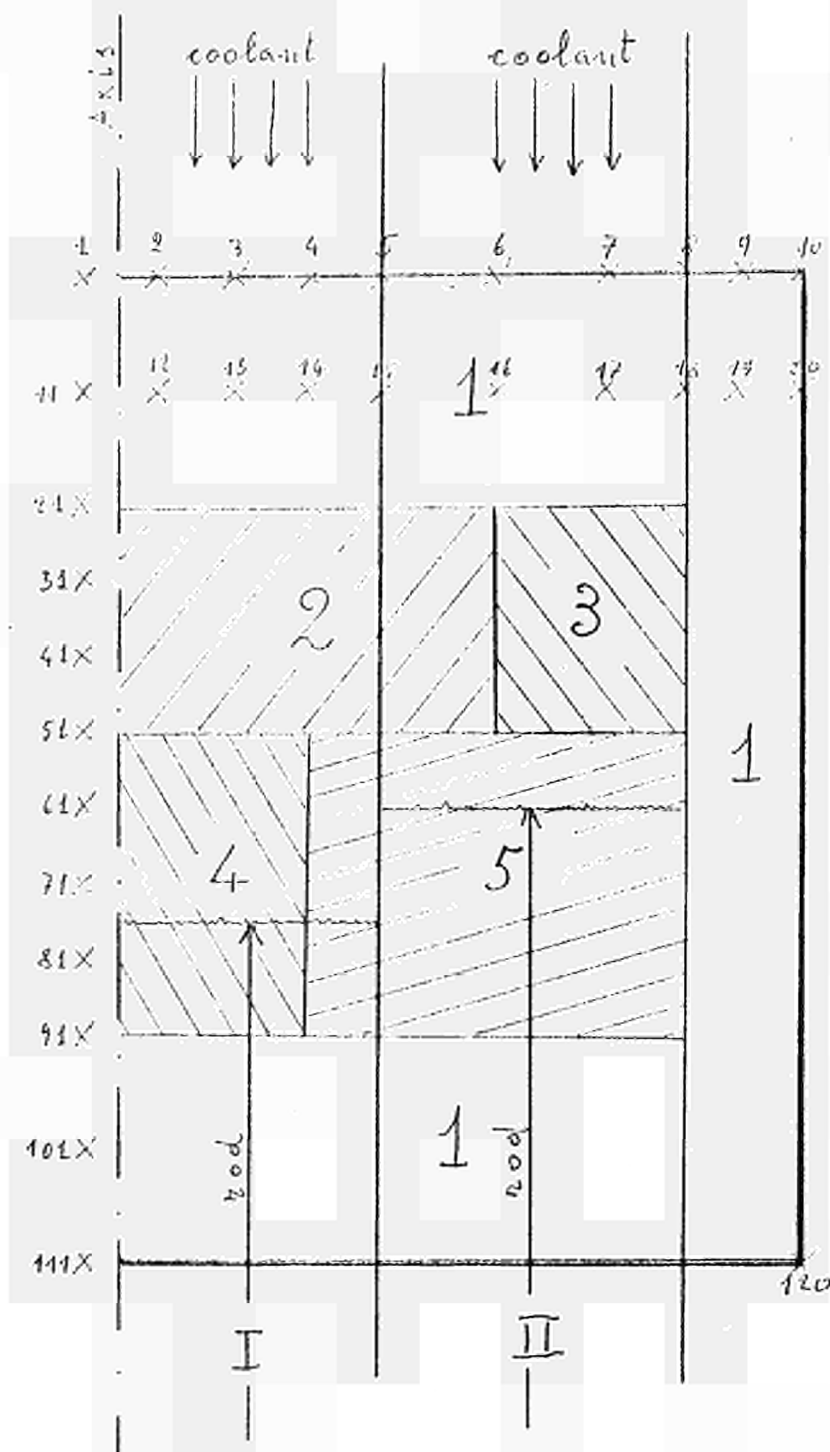


Fig. 1

Fac simile of a core subdivision with a  
10 x 12 = 120 point-lattice

5 composition regions (1:reflector; 2,3,4,5:core)

Two vertical control zones I and II

Control rods inserted from bottom

8 axial points in the active part of the core

Control rods

In each control zone the rods move as a bank all together. They are represented by a diffused equivalent poison. The fast and thermal macroscopic cross sections of this equivalent poison are given in input. The poisoned part of the zone is limited

by a movable boundary A-B of Fig. 2. When the movable boundary lay between two points of the vertical array, a linearly interpolated value of poison is assigned to the space between the two points.



Fig. 2

The initial insertion of the rods in each zone is given in input. However, in all the zones chosen for criticality search, the depth of insertion will be determined by the programme and will be the same for all these zones.

In all other zones not chosen for criticality search, the rods maintain their initial insertion.

During the transient the rods can be moved in any zone arbitrarily chosen. The movement will be specified by a tabulation of insertions versus times given in input for each chosen zone separately. For intermediate times a linear interpolation will be made. To obtain a step-wise movement two tabulated values must be given having times equal to  $n \cdot \Delta t$  and  $(n+1) \cdot \Delta t$ .

If the time of the transient overtakes the last tabulated time, the rods will maintain the last tabulated position.

In all other zones the rods will maintain their initial positions or the positions determined by the criticality search.

### Neutronic initialization

The flux calculation starts with flat fluxes. A fixed number of calculations are made (Sub. INIZ) and at every step the fluxes are normalized with the factor

$$F = \frac{\text{Total Power}}{\int_{\text{core}} \epsilon \left[ \sum_{\text{fission}}^{\text{fast}} \psi + \sum_{\text{fission}}^{\text{ther.}} \varphi \right] dV}$$

in order to obtain a wanted value of total power. Although the reactor is not critical the distribution of fluxes, thus obtained, is very close to the static distribution.

### Criticality search

The criticality search can be made either by varying the concentration of poison in any wanted composition region or by varying the depth of insertion of the

rods in any wanted vertical control zone. The calculation is repeated until the reciprocal of the period of the average thermal flux differs from zero by less than a fixed amount. To accelerate the convergence, only prompt neutrons are considered during this process. The delayed neutron precursor concentrations will be calculated at their equilibrium values once obtained the critical condition.

After every iteration the fluxes are normalized at a wanted power and the temperatures and corresponding feed back on neutronic are calculated. Once obtained the equilibrium condition at the wanted power the physical constants of the regions may be different from their initial values given in input.

#### Dynamic Calculations

Starting from the initial equilibrium conditions the code calculates the dynamic behaviour of the reactor in time.

The calculations of the flux distributions and delayed neutrons precursor concentrations will be repeated at every time step of constant length  $\Delta t$ . The thermohydraulic calculations and the corresponding temperature reactions are calculated at every  $n$  time steps,  $n$  to be specified in input.

During the transient the power level and flux distributions evolve freely according to the reactivity introduced by any perturbation and to the temperature feed-back. The perturbations may be given by varying the depth of insertion of the rods, or the mass flow, or the inlet temperature in any wanted zone independently. The variation of these parameters as functions of time are given in input in tabular form. A convenient value of the time interval  $\Delta t$  is given in Appendix B as ratio  $\Delta t/T$ , where  $T$  is the shortest reactor period in the transient to be studied.

The maximum increase of power and the admissible error are considered. It is easy to have an a priori estimate of these values.



Method

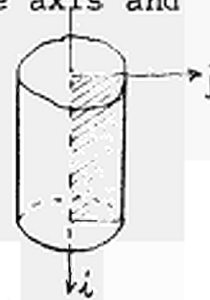
The two-groups time-dependent diffusion equations and the equation of the delayed neutron precursors are:

$$1) D_1 \nabla^2 \psi - \Sigma_R \psi + (1 - \beta) (\Sigma_{f1} \psi + \Sigma_{f2} \psi) + \sum_{i=1}^N \lambda_i C_i = \frac{1}{\omega} \frac{\delta \psi}{\delta t}$$

$$2) D_2 \nabla^2 \psi - \Sigma_a \psi + \Sigma_R \cdot \rho \cdot \psi = \frac{1}{\nu} \frac{\delta \psi}{\delta t}$$

$$3) \frac{\delta C_i}{\delta t} = \beta_i (\nu \Sigma_{f1} \psi + \nu \Sigma_{f2} \psi) - \lambda_i C_i$$

As the reactor is cylindrical, with axial symmetry, a calculation of the fluxes and other physical magnitudes made in a vertical section delimited by the axis and outer boundaries, as shown in fig.3, describes completely the behaviour of the whole reactor.

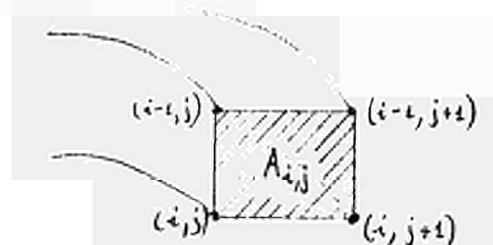


This section is covered with a point lattice. The reactor volume is subdivided into cylindrical annuli each of which is delimited by four points of the lattice.

$j$  = is the radial index increasing from the axis outwards

$i$  = is the axial index increasing from the upper boundary downwards

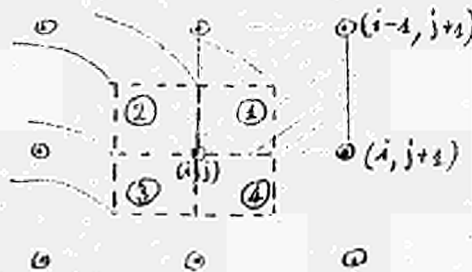
fig. 4



The physical constants of the equations 1) 2) 3), generally indicated with  $A_{ij}$ , may have different values for each annular part of the space, but are constant within the annulus. There is a correspondance of indexes between the part of the space and the lower left of the four points which delimit it.

The equations 1) 2) 3) are integrated over the volume corresponding to the point  $(i,j)$  which is subdivided into four sub-volumes

see fig. 5



Each sub-volume belongs to a different part of space, which may have different physical constants. Thus for every point of the lattice, the integrated equations are averaged over the surrounding parts of space. This is evidently significant for points on the interferences of different regions, but it is also significant for the internal points of a region, because the variations of the physical constants due to the temperature reaction are calculated pointwise. By this method the formal expression of the integrated equations results to be the same everywhere with a considerable simplification of the programme.

For the details of the integration see Appendix A.

The general form of the integrated equation for the generic point (i,j) is:

$$4) -\alpha_{ij} \varphi_{i-1,j} - \beta_{ij} \varphi_{i,j-1} - \gamma_{ij} \varphi_{i+1,j} - \delta_{ij} \varphi_{i,j+1} + \varepsilon_{ij} \varphi_{ij} = b_{ij} \varphi_{ij} + c_{ij}$$

This is the eq. corresponding to the thermal flux and contains, in the known term of the right side the source term  $b_{ij} \varphi_{ij}$  with the fast flux  $\varphi_{ij}$ . The equation corresponding to the fast flux  $\psi$  is formally the same and contains a source term with the thermal flux  $\varphi$ .

There are two systems of  $n$  linear equations, one for each energy group;  $n$  is the number of the points of the lattice.

#### Note

The boundary condition  $\frac{d\varphi}{dr} = 0$  at the axis of the cylindrical core, is obtained by putting all the points (i,1) and (i,2) of the first two vertical columns, symmetrically to the axis. i.e.  $r(1) = -r(2)$  ( $r(j)$  being the radial distance of the point (i,j) from the axis), and by imposing  $\varphi_{i,1} = \varphi_{i,2}$ . This is obtained by putting  $\beta_{i,1} = 0$ . In fact the equation corresponding to the first point of each radius is of the type:

$$8) -\alpha_{i,2} \varphi_{i,2} - \beta_{i,2} \varphi_{i,1} - \gamma_{i,2} \varphi_{i+1,2} - \delta_{i,2} \varphi_{i,3} + \varepsilon_{i,2} \varphi_{i,2} = b_{i,2} \varphi_{i,2} + c_{i,2}$$

if  $\varphi_{i,1} = \varphi_{i,2}$  then the equation can be written:

$$9) -\alpha_{i,2} \varphi_{i,2} - \gamma_{i,2} \varphi_{i+1,2} - \delta_{i,2} \varphi_{i,3} + [\varepsilon_{i,2} - \beta_{i,2}] \varphi_{i,2} = b_{i,2} \varphi_{i,2} + c_{i,2}$$

as  $\beta_{i,2}$  is also contained in  $\varepsilon_{i,2}$  with the + sign the equation 9) can be obtained from 8) by simply putting  $\beta_{i,2} = 0$ .

The outer boundary conditions are obtained by imposing  $\varphi = 0$  and  $\psi = 0$  in all the points of the outer boundary.

In a first version of the code we have used the Gauss-Seidel iterative method with overrelaxation factor for the solution of each groups separately, using than the obtained solution as source for the other groups and iterating between the groups.

This way of proceeding has been abandoned for a much faster method in part direct and in part iterative.

In eq. 4) the fluxes  $\psi_{i-1,j}$  and  $\psi_{i+1,j}$  correspond to points belonging to the radius  $(i-1)$  above and to the radius  $(i+1)$  below the point  $(i,j)$ .

Let us consider the values  $\psi_{i-1,j}$  and  $\psi_{i+1,j}$  as already known. All the equations of the  $i^{\text{th}}$  radius form a system of JMAX equations (JMAX = number of points of the radius), which has a tridiagonal matrix of coefficients. In fact each equation contains only three incognita. This could be solved with the well known method of forward elimination and backward substitution, for each group separately. However by intercalating the equation of the fast with those of the thermal group and rearranging the terms, it is possible to obtain a system of  $2 \times (\text{IMAX})$  equations for both groups, with a pentadiagonal matrix of coefficients. This can be partitioned into  $(2 \times 2)$  submatrices and considered as a tridiagonal matrix with entries consisting of  $(2 \times 2)$  submatrices. The method of forward elimination on backward substitution can be applied operating on these  $(2 \times 2)$  submatrices and the direct solution can be obtained for both groups at the same time. (See: Finite Difference Method for solving the Spatio-Temporal Diffusion Equation in the Two-group Approximation - EUR 596 by B.Monterosso and E.Vincenti).

All the point lattice has to be scanned by repeating this direct calculation at each radius. For  $\psi_{i-1,j}$  will be used the value obtained in the calculation of the  $(i-1)^{\text{th}}$  radius. In the actual iteration, and for  $\psi_{i+1,j}$  will be used the value obtained in the preceding iteration. The scanning of the lattice will be repeated alternatively downward and upward until convergence is reached. This method which is direct in radial direction and iterative in axial direction eliminates the iterations between the groups and in the normal cases reduces the computing time of a factor 3 compared to the usual overrelaxation method.

#### Thermal calculation

The reactor is subdivided in a number of vertical zones (max. 10). Each zone contains many vertical cooling channels of the same type. Channels belonging to different zones may however be of different types. For channel is meant here a fuel rod with the corresponding coolant. The channels go through the reactor from top to bottom including the reflectors, but only the part contained in the active core is considered for thermal calculation. The part is subdivided with a point-lattice of constant mesh  $\Delta z$ , which coincides with the axial lattice for neutronic calculation.

To determine the power developed in a fuel rod, in any zone, in a part  $\Delta Z$  between the  $(i-1)^{th}$  and the  $(i)^{th}$  level, the power is integrated in the volume A (Fig.6) of the zone between  $(i-1)$  and  $(i)$  and this is divided by the number of rods of the zone.

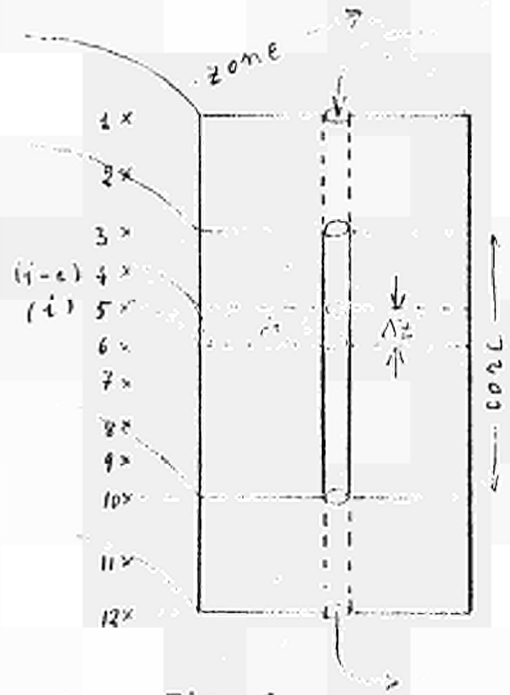


Fig. 6

$$\frac{\int_A \epsilon (\sum f_{fast} \psi + \sum f_{th.} \phi) dv}{\text{number of rods in the zone}}$$

number of rods in the zone

The heat is generated in the rod and propagates to the coolant into which it is injected at the point  $(i)$ . The fuel rod is subdivided in a number of concentric shells (5-10). The relative distribution of power among the shells is given in input and remains constant during the transient, although the total power varies with time. The time-dependent heat balance equation, transformed with the finite-difference method, is solved in implicit way at each level  $(i)$  and gives the temperature distribution at a given instant in the shells of the fuel and in the coolant. The results obtained at level  $(i)$  will be used as input for the calculation at level  $(i+1)$ . A complete description of this procedure is in the report EUR 3171 by A. Agazzi, G. Forti, E. Vincenti, pages 8-14.

Temperature reaction

Eight physical constants are allowed to vary with the temperature, i.e.:

$$D_{\text{fast}}; \Sigma_{\text{removal}}; \nu \Sigma_{\text{fission fast}}; P \text{ (reserve prob.)}$$

$$D_{\text{thermal}}; \Sigma_{\text{a th.}}; \nu \Sigma_{\text{fission th.}}; \Sigma_{\text{poison}}$$

this temperature dependence has been expressed in the form of a Taylor expansion:

$$C = C_0 + \alpha (T_f - T_{f_0}) + \beta (T_m - T_{m_0}) + \gamma (T_f - T_{f_0})^2 + \delta (T_m - T_{m_0})^2$$

where

$C_0$  = initial value of the physical constant  
 $\alpha, \beta, \gamma, \delta$  = temperature coefficients  
 $T_{f_0}$  and  $T_{m_0}$  = Reference temperatures of fuel and moderator corresponding to the initial value  $C_0$  (1)

All these magnitudes are given in input region wise.

$T_f$  and  $T_m$  = New values of the fuel and moderator temperatures. (1)

---

(1) Note This version of the code is made for pressurized water reactors where the moderator and coolant are at the same temperature. Another version of the code has been made for HTG pebble-bed reactors, where the moderator (graphite) is contained in the fuel element mixed with the fuel.

Although all other values, including the reference temperatures  $T_{fo}$  and  $T_{mo}$ , are given for each composition region, the temperatures  $T_f$  and  $T_m$  are calculated in each vertical control zone. The calculation is made at every thermal time step.  $T_f$  and  $T_m$  are values of temperatures averaged over that part of the radius within the control zone, they have however distinct values at every axial level. The temperature reaction is therefore made pointwise in axial direction, and zonewise in radial direction.



### Programme Composition

The code consists of a Main and 19 Subroutines of which 5 belong to the first Overlay and the remaining 14 belong to the second Overlay.

The 5 routines of the first group are called only once. They calculate all the coefficients which will not be changed anymore and will be transferred to the second overlay in order to make available the rest of the memory.

#### First overlay :

- 1) MAIN - Calls the first routines of the first group and PRINC 2, control routine of the second group.
- 2) INPUT - Reads the vector DATA from cards or tape.
- 3) PRINC - It takes the data which are read region-wise in Input and transforms them into point-wise data. Calls GEOM and DCAN.
- 4) DCAN - Calculates all the thermohydraulic coefficients which will remain unchanged during the transient.
- 5) GEOM - Calculates all the coefficient depending from the geometry of the core and the point lattice.
- 6) AZER 1 - Called from Input, before reading the data. Puts equal zero all the commons of the first Overlay.

Second Overlay :

- 7) PRINC-2 - In its first part, before label 1000, this routine determines the initial conditions. It calls AZER 2 to put equal zero all the commons belonging only to the second Overlay. Puts the control rod in the initial position. Calls INIZ to give a first approximate distribution of the fluxes. Calls CRITIC to calculate the initial equilibrium distributions of fluxes and temperatures.
- In the second part after the label 1000 it determines the transient behaviour.
- At every time step it calls BARRE to determine the rod position; it calls Flussi to calculate the flux distribution; optionally it calculates the accumulated energy per  $\text{cm}^3$  at every point; it calculates the delayed neutrons precursors concentrations at every point.
- At every KPC (see input key) time steps it calls the thermal routines CANPALL and CØVAR to determine the temperature reactions.
- Calls STAMPA to print according to the orders given in input.
- Controls the restart operations to store on tape the results at the end of a calculation and to start from tape an interrupted calculation (see Restart Instructions).
- 8) AZER 2 - Puts equal zero all the commons belonging only to the second Overlay.

9) INIZ - It calls FLUSSI a number of times specified in input. Starting from an initial flat distribution determines a first approximate distribution of the fluxes. In these calculations the delayed neutrons are not considered. It normalizes the fluxes at a given initial power level. Uses a neutronic time step DELTI which may be different from DELTC used in CRITIC, and from DELT used for the transient calculation.

10) FLUSSI - It solves directly along each radius the two-group system of linear equations. (see Method). It scans axially the point lattice, in alternate directions, until the residuals of each group

$$\frac{\sum_{i=1}^N |\Delta \psi_i|}{\sum_{i=1}^N \psi_i} < EP1 \quad \text{and} \quad \frac{\sum_{i=1}^N |\Delta \phi_i|}{\sum_{i=1}^N \phi_i} < EP2$$

are smaller of the quantities EP1 and EP2 given in input.

It calculates the fast and thermal flux average values and the reciprocal of the period.

11) MAT1 - It calculates the coefficients of the system of linear eq. to be solved by Flussi. This calculation is made using the invariant coefficients transferred from the first overlay and variable physical magnitudes depending from the temperature reaction and the rod position.

This routine will be called at every thermal calculation, from Princ 2 and from Critic.

When only the poison concentration is varied it is enough to call MAT2, because, as the routine contains the Entry Mat 2, the first part of it will be bypassed.

- 12) STAMPA - It prints the output. Any wanted result may be printed in several optional types of print (see input instructions).
- 13) CRITC - searches the criticality by varying iteratively either the concentration of poison in any wanted region, or the insertion of the rod in any wanted zone. In the first part of this iterative method, from two values of poison and the corresponding values of reciprocal period, it determines, by extrapolation, the next value of poison and so on. When the reciprocal period is below a given value it varies the poison at each iteration of one thousand of the last concentration, until the convergency criterium is satisfied or the reciprocal of period changes of sign.

The same proceeding is followed when the criticality is searched with rod movement. After a first part with the extrapolation method, the rod are displaced of 1 millimeter at each iteration.

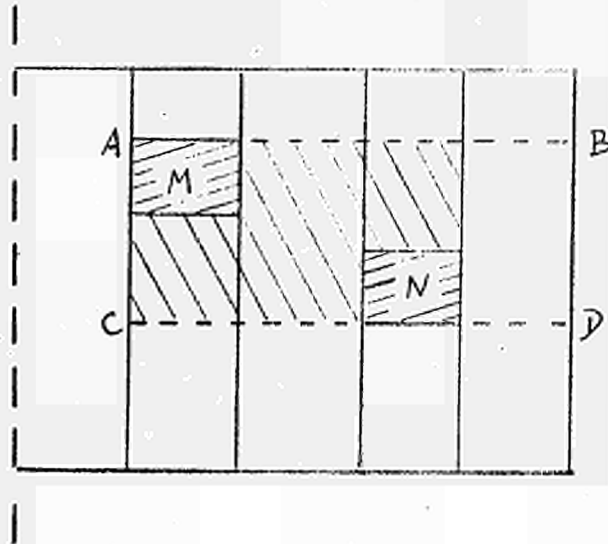
In both cases at each iteration the thermohydraulic routines are called to determine the temperature reaction.

The calculation is made without considering the delayed neutrons.

- 14) BARRE - This routine calculates the new coefficients of the system of linear equations only where a new condition of poison was determined by the displacement of the rods e.g. see M and N of Fig. 7....  
The recursive coefficients of the forward elimination

and backward substitution method however are calculated by BARRE in all the points of the region A B C D of Fig. 7 determined by the rods displacement and the outer boundary.

Fig. 7



It is called from Critic and Princ 2. When called from Princ 2 it determines the rod position by interpolation of the tabulation given in input.

- 15) TEST - This is a dummy routine which contains all the commons and may be used to introduce any new statement in the programme. It is called at every time step from Princ 2.
- 16) CANPAL - This routine calculates from each channel the temperature distribution in the fuel rod and in the coolant at all axial levels (see thermal calculation). It is called during the transient from Princ 2 at every KPC time step, and from Critic to determine the equilibrium initial condition.

- 17) VINIZ - Determines for each channel the coolant inlet temperatures and mass flow at every time step, interpolating the values given in input as function of time in tabulated form.
- 18) INTEGR - Calculates for each channel independently at every axial level the fuel and coolant average temperatures.
- 19) COVAR - calculates the new values of the physical constants given by Integr. (See temperature reaction).
- 20) RESTAR - stores on tape the results at a given time. It restarts an interrupted calculation, reading the results stored on tape (See restant instructions).

### Input Instructions

A title card is the first input card of each problem.

A vector of 10000 memory positions : DATA (1).....

DATA (10000) contains all the data.

Since entire groups of memory positions are zero, it is possible to read separate sets of significant data; each set of data must be preceded by a card containing the integers  $Ki_1$ ,  $Ki_2$ ,  $Ki_3$ , written in Format (2I6, I12) where  $Ki_1$  is normally zero,  $Ki_2$  and  $Ki_3$  are the indexes of the first and last datum of the set.

Only in the last set of a problem  $Ki_1$  must be any positive integer number. The data are in floating form, six per card (Format 6 E 12.8).

Any number of problems may be run in sequence; for all the subsequent problem only the data different from the corresponding data of the first problem need to be given. The vector DATA of the first problem must be memorized on an auxiliary tape. (See Restart Instructions).

### Input Key

Title card : Columns 1-2-3 if left blank the data are read from cards and the results will not be stored on tape. Numbers in these columns correspond to store and restart (see Restart Instructions). Columns 4-5-6. If only one problem is treated they must contain any positive integer. If a series of problems are to be treated in the same run, only the last title card of the last problem must have any positive integer, in the preceding title cards these columns must be blank. Columns 7-70 may contain any alphanumerical information which will be printed in output.

DATA Number	Variable Name	DESCRIPTION	Allowable Values	NOTES		
First card	1	TEMAC	Computer time (min.)		Only if the first three columns of the title card contain a number see Restart Instructions.	
	2	DELT	Neutronic time step $\Delta t$ for transient calculation (sec.)			
	3	DELTC	Neutronic time step $\Delta t$ for criticality search (sec.)			
	4	IMAX	Number of axial mesh points	$\leq 40$		$(IMAX * JMAX) \leq 400$
	5	JMAX	" " radial " "	$\leq 40$		
	6	NREG	Number of regions with different composition	$\leq 30$		
— . —						
Second card	7	NRIT	Number of delayed neutron groups	$\leq 6$		
	8	ITMAX	Maximum allowable number of axial iterations	suggested value = 30		
	9	IDST	Number of initial iterations before criticality search	Suggested value = 20		
	10		blank			
	11		blank			
	12	DELTI	Neutronic time step $\Delta t$ for the initial iterations before criticality			



DATA Number	Variable Name	DESCRIPTION	Allowable Values	NOTES
third card	13	Blank		
	14	EP1	Convergence criterium for the fast group	s.v.=0,01
	15	EP2	Same for the thermal group	s.v.=0,01
	16	SI	Initial reactor power (Watts)	
	17	KPC	The thermal calculation is done every KPC neutronic steps	
	18	NCAN	Number of radial zones containing channels	
— • —				
19		Blank		
20		Blank		
21	KTE	If $KTE > 0$ Subroutine TEST will be called.		
22	ICA	Axial index of the points laying on the upper core boundary. (Reflector excluded)		
23	ICB	Same for the lower boundary		
24	JCI	Radial index of the points laying on the inner core boundary		Usually the central part of the reactor belongs to the core, then <u>JCI=2</u>
25	JCE	Radial index of the points laying on the outer boundary of the core. (Radial reflector excluded)		

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
26	W	Fast group neutron velocity ( $\frac{\text{cm}}{\text{sec}}$ )		
27	V	Thermal " " " "		
28	SPRG	First guess of control poison concentration		Blank if critically search is made with rod movement
29	DAPF	Convergence criterion for reciprocal of period in criticality search	s.v.=0.1	
30	LF	Maximum allowable number of criticality iterations	s.v.=30	
31	IDIR	<ul style="list-style-type: none"> <li>= 1 if the control rods enter into the core at the same side as the coolant.</li> <li>= 2 if they enter at the opposite side</li> </ul>		
32	IBCR	<ul style="list-style-type: none"> <li>= 0 criticality search made with diffused poison</li> <li>= 1 criticality search made with rod movement</li> </ul>		
33	NZB	Number of the vertical zones	≤10	The vertical zones have the same height of the reactor. Each zone is characterized by the same rod depth of insertion and by the same typical channel.

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
34	SPB1	Microscopic control poison (barn) cross section for the fast groups.		they are multiplied by SPR in each region to obtain the correspondin macroscopic cross section
35	SPB2	Same for the thermal group		
201-206	BETA	$\beta_i$ delayed neutrons yield per fission		
211-216	DL	$\lambda_i$ delayed neutrons precursors decay constants		
231	INTG	<p>{ = 1 The power will be integrated in the time at every mesh point</p> <p>{ = 0 No integration will be effectuated</p>		
232	NUSO	Number of thresholds of integrated energies	$\leq 10$	only if INTG = 1
233-242	SØLEN	Thresholds of the integrated energies in increasing order from the lower to the higher	[ Joule ] [ cm <sup>3</sup> ]	(See printing order N° 8 and 9)
301	I1	Index of the upper boundary of the first region		This group of data for energy regions is included in one card. For the region bounded by the axis of cylinder J1=1
302	I2	Index of the lower boundary of the first region		
303	J1	Index of the left (inner) boundary of the first region		
304	J2	Index of the right (outer) boundary of the first region		
305	-	blank		
306	-	blank		
307-312		for the second region		
375-480		for the 30th region		

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
1001	D1	Diffusion coefficient fast group		
1002	SR	Removal cross section		
1003	SF1			
1004	P	Resonance escape probability		
1005	-	blank		
1006	-	blank		
1007-1012		Second region		
1175-1180		30th region		
— • —				
1601	D2	Diffusion coefficient thermal group		
1602	SA	thermal absorption cross-section		
1603	SF2			
1604	SPR	Poison concentration; it is multiplied by SPB1=Data (34) and SPR2=Data (35)		
1605	-	blank		
1606	-	blank		
1607-1612		Second region		

One card for each region

One card for each region

If IBCR = 0 SPR will be varied to obtain criticality in the control regions

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
2201	EQUI1	Thermal equivalent of fission (fast) [Watt-sec]		
2202	EQUI2	Thermal equivalent of fission (thermal) [Watt-sec]		
2203	FNU	$\nu$ Neutron yield per fission		
2204	KV	$\left\{ \begin{array}{l} KV = \text{order number of the region, then} \\ \text{in that region SPR will be varied} \\ \text{for criticality search} \\ KV = \text{Zero, SPR will not change} \end{array} \right.$		Only if IBCR=0
2205	TRU	The initial values of the physical constants given in input were calculated according to a fuel temperature = TRU and a moderator temperature = TMU for each composition region.		
2206	TMU			
2207-2212		for the second region		
2375-2380		for the 30 <sup>th</sup> region		
— . —				
2801	KB	$\left\{ \begin{array}{l} KB = \text{order number of the zone, this} \\ \text{zone contains control rods} \\ KB = 0, \text{ no rods} \end{array} \right.$		See note of Data (33)
2802	KBC	$\left\{ \begin{array}{l} KBC = \text{order number of the zone, the} \\ \text{rods of the zone are used for} \\ \text{criticality search} \\ KBC = \text{zero, no movement of rod for} \\ \text{criticality search} \end{array} \right.$	Only if IBCR=1	The order number of the zone, must increase from left to right

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
2803	J1B	Index of the left (inner) boundary of the zone		For the region bounded by the axis of cylinder J1B = 1
2804	J2B	Index of the right (outer) boundary of the zone		
2805	KCA	{ KCA= order number of zone, the zone contains channels KCA= zero, no channel		
2806	-		blank	
2807-2812		Second zone		
2855-2860		10 <sup>th</sup> zone		
— . —				
2861	VEBA1	Macroscopic cross section of the rod equivalent poison (fast)		
2862	VEBA2	Macroscopic cross section of the rod equivalent poison (thermal)		
2863	-	blank		
2864	-	blank		
2865	-	blank		
2866	-	blank		
2867-2872		Second zone		
2915-2920		10 <sup>th</sup> zone		

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
3001-3030	TOBA	Times of the rod movement tabulation for the first zone sec		DATA (3001) must always be zero
3031-3060	PRIN	Corresponding depth of rod insertion in the tabulation for the first zone cm		
3061-3090		Same for the second zone	DATA (3061) = 0	
3091-3120		etc.	DATA (3091) = 0	
4001-4040	X(J)	Distance of mesh points from the axis		
4101-4140	Y(I)	Distance of mesh points from the upper boundary.		The upper boundary is for definition the side from which the coolant enters. The first distance is $Y(1) = 0.0$

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
<u>CHANNEL DATA</u>				
5001	DZ	Constant axial mesh distance	cm	
5002	SEE	Section of channel (Coolant corresponding to a fuel rod)	cm <sup>2</sup>	
5003	SC	Coolant specific heat	$\frac{\text{Joule}}{\text{g } ^\circ\text{C}}$	
5004	RO	Coolant density	g/cm <sup>3</sup>	
5005		blank		
5006	-	blank		



DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
5007	N <sub>1</sub> (KC)	Number of shells in the fuel rod	$N_1 \leq 9$	
5008	PALN <sub>1</sub> (KC)	Number of fuel rods in the zone KC		
5009		blank		
5010	ØSE1	Outer surface of a part of rod of height DZ		
5011	-	blank		
5012	-	blank		
— • —				
5013-5018		These six DATA contained in one card correspond to the first shell of the rod.		
5013	RES1	Thermal resistance between the shell and the next one [°C/Watt]		
5014	CPT1	Thermal capacity of the shell [Joules/°C]		
5015	VOS1	Shell volume [cm <sup>3</sup> ]		
5016	PZ01	Power fraction corresponding to the shell		Relative values are significant. Normalization is performed by the code.

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
5019-5024		Same data for the second shell etc.....		
5061-5066		Same data for the ninth shell		
— . —				
5190	VITE	Inlet temperature of coolant at equilibrium initial conditions (Put zero if inlet temperature is tabulated as function of time) [°C]		
5191	STEPT	Step of coolant inlet temperature		Only if VITE ≠ 0
5192	RAMPT	Value of $dT/dt$ for ramps in coolant inlet temperature [°C/sec]		"
5193	VIPO	Initial value of coolant mass-flow (put zero if tabulated) [g/sec]		
5194	STEPPO	Step of coolant mass flow		Only if VIPO ≠ 0
5195	RAMFO	Value of $\frac{dW}{dt}$ for ramp in mass flow [g/sec <sup>2</sup> ]		"
— . —				

DATA Number	Variable Name	DESCRIPTION	Allowabl Values	NOTES
5201-5329		SECOND CHANNEL DATA		
6801-6999		TENTH CHANNEL DATA		
<p>NOTE: All the cards must be repeated for every channel. The order number of the existent channel must be the same of the order number of the radial zone in which the channel is contained.  (Ex. Suppose that zone one and three contain channel and zone two do not contain any channel, then the channel of zone three must be considered as the third and its data are in the group Data (5401) ....(5599).)</p>				
7001-7060		Tabulation of inlet temperatures and mass flow for the <u>first channel</u> . Each point of the tabulation for both magnitudes is contained in one card (4 data), as follows:		
7001	TOTE	Time of the temperature tabulation (Zero in the first card)		
7002	TENT	Value of the corresponding temperature		
7003	TOPO	Time of the mass-flow tabulation (zero in the first card)		
7004	PENT	Value of the corresponding mass flow		
7005	PENT	blank		
7006	PENT	blank		

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
7007-7012		Second card of the tabulation etc.		
7061-7120		Same for the second channel etc.		
7541-7600		Same for the 10 <sup>th</sup> channel		

— . —  
TEMPERATURE COEFFICIENTS (8001 → 8960)

The temperature dependence of the physical constants is expressed in the form of a Taylor expansion,

$$C = C_0 + \alpha_1 (T_f - T_{f_0}) + \alpha_2 (T_m - T_{m_0}) + \alpha_3 (T_f - T_{f_0})^2 + \alpha_4 (T_m - T_{m_0})^2$$

40 -

The variable physical constants are 8 in the following order :

- |   |                              |
|---|------------------------------|
| 1 | D1 = $\mathcal{D}_1$         |
| 2 | SR = $\Sigma_R$              |
| 3 | SF1 = $\nu \bar{\Sigma}_f^1$ |
| 4 | P = $\rho$                   |
| 5 | D2 = $\mathcal{D}_2$         |
| 6 | SA = $\Sigma_{a2}$           |
| 7 | SF2 = $\nu \bar{\Sigma}_f^2$ |
| 8 | SPR = $\Sigma$ poison        |

The algorithm giving the position of the temperature coefficient in the reactor DATA is as follows:

$$CTN(K, L, N) = \text{DATA} [8000 + 32 * (N - 1) + 8 * (L - 1) + K]$$

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
----------------	------------------	-------------	---------------------	-------

where K (1,...8) = order number of the physical constant  
L (1,...4) = order number of  $\alpha_i$  in the Taylor series  
N (1,...30) = order number of the composition region

— • —

PRINTING ORDERS

9901-9960                    It is possible to give in input up to 5  
different printing orders.  
                              Each order is contained  
                                  in two cards (12 data)

9901-9912                    First order

9913-9924                    Second "

"

9949-9960                    Fifth "

In each group of 12 data (two cards)  
corresponding to an order the first  
datum = n1 indicates that the prin-  
ting order is valid up to the  $n_1^{\text{th}}$   
time-step.

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
		<p>The other data of the group <math>n_i \leq n_1</math> (<math>i= 2,3,\dots,12</math>) indicate that the corresponding type of printing will be made at every <math>n_i</math> time-step. If some of the data = <math>n_i</math> are left blank the corresponding type of printing will not be made.</p>		
— . —				
<u>Ex. For the first order :</u>				
9901	n1	The first printing order will be performed up to the $n_1^{\text{th}}$ time step.	} First card	
9902	n2	Flux core average, reciprocal period and total power		
9903	n3	Flux map		
9904	n4	Map of rod insertions		
9905	n5			
9906	n6	Map of the concentrations of the delayed neutron precursors		
— . —				

DATA Number	Variable Name	DESCRIPTION	Allowable values	NOTES
9907	n7	Maximal fuel temperature, max. cladding temperature, max. heat flux, inlet and outlet temperature, mass flow.		
9908	n8	Map of temperatures		
9909	n9	Percent of core (in volume) below a given integrated energy threshold		
9910	n10	Map of integrated energies		
9911		nothing		
9912				

Second card

It is possible to give as many order as wanted up to five.  
The calculation stops at the time-step indicated by the first datum =n1 of the last printing order.

### Restart an interrupted calculation

Two dimensional calculations require long computer time. It may be interesting to have the possibility of restarting an interrupted calculation, without repeating it from the beginning. In particular it may be interesting to have the possibility to initiate again a calculation from the equilibrium condition, without repeating the criticality search. For this purpose it is necessary to store on tape the results already obtained.

It is possible after a restart to change the printing orders. The first card of the new printing orders must contain in the first twelve columns the number IT of the time steps at which the calculation was interrupted; this card is blank in all other columns. A second blank card must follow, then groups of two cards must follow containing the new printing orders. It is also possible to change the tabulations given in input for the rod movements, inlet temperatures, and mass flow. Of course only changements after the instant  $t$  of restart have a meaning.

The new tabulations must contain a first point at time zero, with any value of the variable, the second point will be at the instant  $t$  and the value of the variable (insertion of rod, inlet temperature, mass flow) must be that indicated in the output of the results which were stored on tape. From  $t$  on any new point may be introduced in the tabulation.

At any subsequent restart after having changed the original tabulation, it is necessary to use the cards of the new tabulation, otherwise the programme will calculate according



to the original one which remains stored on tape.

It is also possible in a restarted calculation to introduce a new value of the time step Data (2). In this case a new printing order must be given, as described in the preceding page. The original vector Data is stored on tape, only the new changed values must be given by cards. The orders of restart are given in the first three columns of the title card:

Titel Card:

Blank = Input data read from card, no storing of results on tape

001 = Input data read from cards, three files are stored on tape

file 1 input data  
file 2 results of criticality search  
file 3 results of instant t.

002 = Only if more than one problem is treated. Input data corresponding to the first problem are read from tape, file 1. Only those input data of the new problem which differ from those of the first problem are read from cards. At every new restart of the problem the cards corresponding to changement of the input data must be introduced, because on tape will always remain stored the input of the first problem.

003 = Restart from equilibrium conditions after criticality search, and store results at the end of calculation at time t.

004 }  
005 } = restart from results stored on file  
006 } (n-1) and store results on file n.  
00n }

-03 } Restart from the results stored in any file  
-04 } without destroying the subsequent files.  
-05 } This in case of tentative changements of  
-0n } rods, inlet temperature, mass flow tabulation  
and time step in any part of the transient.

First data card:

In addition to the titel card the first data card, containing DATA (1) to (6) must always be present.

In DATA (1) = TEMAC is indicated the computer time (minutes). At every thermal calculation the programme will compare the remaining computer time with the computer time necessary to make a thermal time step. If it will not be possible to make the next thermal calculation, then the results will be stored on tape.

If no thermal calculations are made then the programme makes this check at every 100 steps.

In the control cards are DD statements. At every file stored on tape corresponds a DD statement. When in the third column of the title card there 1 or 2 or 3 the DD statements are three, otherwise the number of the DD statements is the same as the number contained in column three of the Title card.

Every DD statement corresponding to a file stored on tape must contain the parameter DISP = (NEW, DELETE), and every DD st. corresponding to a file read from tape must contain the parameter DISP = (OLD, DELETE).

DATA (Zones de 10 colonnes)

PROBLEM										DATE										PAGE										OF																																																	
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
Sample Problem Input and Control Cards																																																																															
Print calculation. Input data read from cards																																																																															
TIME																																																																															
LINES																																																																															
\$ TAPES																																																																															
11 Identification card																																																																															
11 EXEC FTLG, $\phi V = \phi VLY$																																																																															
11 LKED SY SIN DD *																																																																															
Main Program:																																																																															
OVERLAY A																																																																															
Binary 1 <sup>st</sup> Overlay																																																																															
OVERLAY A																																																																															
Binary 2 <sup>nd</sup> Overlay																																																																															
/*																																																																															
11 G $\phi$ FT09 F00(1) DD UNIT=L91, VOLUME=(PRIVATE, SER=EU2747), C																																																																															
11 LABEL=(1, SL), DSNAME=EU2747(1), C																																																																															
11 DISP=(NEW, DELETE)																																																																															
11 G $\phi$ FT09 F00(2) DD UNIT=L91, VOLUME=(PRIVATE, SER=EU2747), C																																																																															
11 LABEL=(2, SL), DSNAME=EU2747(2), C																																																																															
11 DISP=(NEW, DELETE)																																																																															
11 G $\phi$ FT09 F00(3) DD UNIT=L91, VOLUME=(PRIVATE, SER=EU2747), C																																																																															
11 LABEL=(3, SL), DSNAME=EU2747(3), C																																																																															
11 DISP=(NEW, DELETE)																																																																															
11 G $\phi$ SY SIN DD *																																																																															

CETIS/CADI (EURATOM)

PROBLEM																								DATE																								PAGE 1		OF																																																																																																																																																																																																																																																																					
1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80																																																																																																																																																																																																																																								
1																								SAMPLE																								PROBLEM																																																																																																																																																																																																																																																																							
2																								36																																																																																																																																																																																																																																																																																															
3																								5.0																								0.001																								0.001																								12.0																								10.0																								5.0																																																																																																																																																																							
4																								6.0																								30.0																								20.0																																																0.001																																																																																																																																																																																															
5																																																0.01																								0.01																								500.0																								+06																								100.0																								2.0																																																																																																																																															
6																																																																																																3.0																																																10.0																								2.0																																																																																																																																															
7																								8.0																								0.1																								+10																								0.26																								+06																								0.2																								+21																								0.1																								30.0																																																																																															
8																								1.0																								0.0																																																2.0																								0.2																								-23																								0.8																								-23																																																																																																																							
9																								201																																																206																																																																																																																																																																																																																																															
10																								0.218																								-03																								0.1116																								-02																								0.977																								-03																								0.1687																								-02																								0.455																								-03																								0.185																								-03																							
11																																																911																																																216																																																																																																																																																																																																																							
12																								0.125																								-01																								0.319																								-01																								0.123																								+00																								0.312																								+00																								0.115																								+01																								0.274																								+02																							
13																								231																																																233																																																																																																																																																																																																																																															
14																								1.0																								1.0																																																10.0																																																																																																																																																																																																																							
15																								301																																																330																																																																																																																																																																																																																																															
16																								1.0																								12.0																																																1.0																								10.0																																																																																																																																																																																															
17																								3.0																								6.0																																																1.0																								6.0																																																																																																																																																																																															
18																								3.0																								6.0																																																6.0																								8.0																																																																																																																																																																																															
19																								6.0																								10.0																																																1.0																								4.0																																																																																																																																																																																															
20																								6.0																								10.0																																																4.0																								8.0																																																																																																																																																																																															
21																								1001																																																1030																																																																																																																																																																																																																																															
22																								1.37																								0.91																								-02																								1.0																																																																																																																																																																																																																							
23																								1.35																								0.838																								-02																								1.0																																																																																																																																																																																																																							
24																								1.35																								0.838																								-02																								1.0																																																																																																																																																																																																																							
25																								1.35																								0.838																								-02																								1.0																																																																																																																																																																																																																							

(S)

(Pb)









PROBLEM		DATE	PAGE	OF
1	2	3	4	5
1	1	9901	9924	
2	400.0	10.0	200.0	50.0
3	100.0	100.0	50.0	200.0
4	3000.0	10.0	200.0	50.0
5	200.0	500.0	200.0	500.0
6	/*			
7	F/P			
8	Restart from equilibrium conditions			
9	after criticality search.			
10	TIME			
11	LINES			
12	TAPES			
13	Identification Card			
14	EXEC FTLG, $\Phi V = \Phi VLY$			
15	LINKED SYSTEM DD*			
16	Main Binary			
17	OVERLAY A			
18	Binary 1 <sup>st</sup> Overlay			
19	OVERLAY A			
20	Binary 2 <sup>nd</sup> Overlay			
21	/*			
22	//G $\phi$ .FTO9FOO $\textcircled{1}$ DD UNIT=L91, VOLUME=(PRIVATE, SER=EU2747),			
23	// LABEL=( $\textcircled{1}$ , SL), DSNNAME=EU2747 $\textcircled{1}$ ,			
24	// DISP=(OLD, DELETE)			
25	//G $\phi$ .FTO9FOO $\textcircled{2}$ DD UNIT=L91, VOLUME=(PRIVATE, SER=EU2747),			

VS

PO



# CETIS/CADI (EURATOM)

PROBLEM	DATE	PAGE	7	OF
1 1Gφ·FT09F00①	DD UNIT=L91, VOLUME=(PRIVATE, SER=EUZF4Z),			
2 11	LABEL=(①, SL), DSNNAME=EUZF4Z①,			
3 11	DISP=(OLD, DELETE)			
4 11Gφ·FT09F00②	DD UNIT=L91, VOLUME=(PRIVATE, SER=EUZF4Z),			
5 11	LABEL=(②, SL), DSNNAME=EUZF4Z②,			
6 11	DISP=(OLD, DELETE)			
7 11Gφ·FT09F00③	DD UNIT=L91, VOLUME=(PRIVATE, SER=EUZF4Z),			
8 11	LABEL=(③, SL), DSNNAME=EUZF4Z③,			
9 11	DISP=(OLD, DELETE)			
10 11G0·FT09F00④	DD UNIT=L91, VOLUME=(PRIVATE, SER=EUZF4Z),			
11 11	LABEL=(④, SL), DSNNAME=EUZF4Z④,			
12 11	DISP=(NEW, DELETE)			
13 11Gφ SXSIN	DD *			
14 4 1	SAMPLE PRφBLEM			
15 1	6			
16 5.0	0.002	0.001	12.0	10.0
17 3061	3063			5.0
18 0.0	0.2	0.5		
19 3091	3093			
20 0.0	245.0	612.0		
21 1 9901	9936			
22 200.0				
23 Blank Card				
24 300.0	5.0	100.0	25.0	
25 50.0	50.0	25.0	100.0	



COSTANZA R.Z.

SAMPLE PROBLEM

1	0.500000E-01	2	0.100000E-02	3	0.100000E-02	4	0.120000E-02	5	0.100000E-02	6	0.500000E-01
7	0.500000E-01	8	0.300000E-02	9	0.200000E-02	10	0.0	11	0.0	12	0.100000E-02
13	0.0	14	0.100000E-02	15	0.100000E-02	16	0.500000E-09	17	0.100000E-03	18	0.200000E-01
19	0.0	20	0.0	21	0.0	22	0.300000E-01	23	0.100000E-02	24	0.200000E-01
25	0.800000E-01	26	0.100000E-01	27	0.200000E-06	28	0.200000E-21	29	0.100000E-00	30	0.300000E-02
31	0.100000E-01	32	0.0	33	0.200000E-01	34	0.200000E-23	35	0.800000E-23	36	0.0
201	0.215000E-03	202	0.111600E-02	203	0.477000E-03	204	0.168700E-02	205	0.455000E-03	206	0.185000E-03
211	0.125000E-01	212	0.319000E-01	213	0.123000E-00	214	0.312000E-00	215	0.115000E-01	216	0.274000E-01
231	0.100000E-01	232	0.100000E-01	233	0.100000E-02						
301	0.100000E-01	302	0.120000E-02	303	0.100000E-01	304	0.100000E-02	305	0.0	306	0.0
307	0.300000E-01	308	0.300000E-01	309	0.100000E-01	310	0.600000E-01	311	0.0	312	0.0
313	0.300000E-01	314	0.300000E-01	315	0.600000E-01	316	0.800000E-01	317	0.0	318	0.0
319	0.600000E-01	320	0.100000E-02	321	0.100000E-01	322	0.400000E-01	323	0.0	324	0.0
325	0.500000E-01	326	0.100000E-02	327	0.400000E-01	328	0.800000E-01	329	0.0	330	0.0
1001	0.137000E-01	1002	0.338000E-02	1003	0.0	1004	0.100000E-01	1005	0.0	1006	0.0
1007	0.135000E-01	1008	0.338000E-02	1009	0.0	1010	0.100000E-01	1011	0.0	1012	0.0
1013	0.135000E-01	1014	0.338000E-02	1015	0.0	1016	0.100000E-01	1017	0.0	1018	0.0
1019	0.135000E-01	1020	0.338000E-02	1021	0.0	1022	0.100000E-01	1023	0.0	1024	0.0
1025	0.135000E-01	1026	0.338000E-02	1027	0.0	1028	0.100000E-01	1029	0.0	1030	0.0
1601	0.366000E-00	1602	0.477000E-02	1603	0.0	1604	0.0	1605	0.0	1606	0.0
1607	0.377000E-00	1608	0.477000E-02	1609	0.520000E-02	1610	0.0	1611	0.0	1612	0.0
1613	0.377000E-00	1614	0.477000E-02	1615	0.513000E-02	1616	0.0	1617	0.0	1618	0.0
1619	0.377000E-00	1620	0.477000E-02	1621	0.505000E-02	1622	0.0	1623	0.0	1624	0.0
1625	0.377000E-00	1626	0.477000E-02	1627	0.515000E-02	1628	0.0	1629	0.0	1630	0.0
2201	0.0	2202	0.0	2203	0.0	2204	0.100000E-01	2205	0.0	2206	0.0
2207	0.317200E-10	2208	0.317200E-10	2209	0.246000E-01	2210	0.200000E-01	2211	0.800000E-03	2212	0.290000E-03
2213	0.317200E-10	2214	0.317200E-10	2215	0.246000E-01	2216	0.300000E-01	2217	0.800000E-03	2218	0.290000E-03
2219	0.317200E-10	2220	0.317200E-10	2221	0.246000E-01	2222	0.400000E-01	2223	0.800000E-03	2224	0.290000E-03
2225	0.317200E-10	2226	0.317200E-10	2227	0.246000E-01	2228	0.500000E-01	2229	0.800000E-03	2230	0.290000E-03
2801	0.100000E-01	2802	0.0	2803	0.100000E-01	2804	0.500000E-01	2805	0.100000E-01	2806	0.0
2807	0.200000E-01	2808	0.0	2809	0.500000E-01	2810	0.800000E-01	2811	0.200000E-01	2812	0.0
2861	0.400000E-04	2862	0.400000E-03	2863	0.0	2864	0.0	2865	0.0	2866	0.0
2867	0.400000E-04	2868	0.400000E-03	2869	0.0	2870	0.0	2871	0.0	2872	0.0
3001	0.0	3002	0.200000E-00								
3031	0.612000E-03	3032	0.337000E-03								
3061	0.0	3062	0.200000E-00	3063	0.100000E-01						

Input  
Data

3091	0.0		3092	0.245000E	(3	3093	0.612000E	03										
4001	-0.300000E	02	4002	0.300000E	02	4003	0.600000E	02	4004	0.300000E	02	4005	0.120000E	03	4006	0.160000E	03	
4007	0.200000E	03	4008	0.200000E	03	4009	0.240000E	03	4010	0.250000E	03							
4101	0.0		4102	0.200000E	02	4103	0.400000E	02	4104	0.116000E	03	4105	0.192000E	03	4106	0.266000E	03	
4107	0.344000E	03	4108	0.420000E	03	4109	0.496000E	03	4110	0.572000E	03	4111	0.592000E	03	4112	0.612000E	03	
5001	0.760000E	02	5002	0.100000E	01	5003	0.540000E	01	5004	0.800000E	00	5005	0.0		5006	0.0		
5007	0.500000E	01	5008	0.297000E	04	5009	0.0		5010	0.285000E	03	5011	0.0		5012	0.0		
5013	0.775000E	01	5014	0.447000E	01	5015	0.113383E	02	5016	0.200000E	00	5017	0.0		5018	0.0		
5019	0.233000E	01	5020	0.447000E	01	5021	0.119383E	02	5022	0.200000E	00	5023	0.0		5024	0.0		
5025	0.134700E	01	5026	0.447000E	01	5027	0.119383E	02	5028	0.200000E	00	5029	0.0		5030	0.0		
5031	0.139300E	01	5032	0.447000E	01	5033	0.119383E	02	5034	0.200000E	00	5035	0.0		5036	0.0		
5037	0.636000E	02	5038	0.447000E	01	5039	0.119383E	02	5040	0.200000E	00	5041	0.0		5042	0.0		
5043	0.377000E	02	5044	0.531000E	01	5045	0.125324E	02	5046	0.0		5047	0.0		5048	0.0		
5100	0.270000E	03	5101	0.0		5102	0.0		5103	0.585000E	03							
5201	0.750000E	02	5202	0.100000E	01	5203	0.540000E	01	5204	0.800000E	00	5205	0.0		5206	0.0		
5207	0.500000E	01	5208	0.297000E	04	5209	0.0		5210	0.285000E	03	5211	0.0		5212	0.0		
5213	0.775000E	01	5214	0.447000E	01	5215	0.113383E	02	5216	0.200000E	00	5217	0.0		5218	0.0		
5219	0.233000E	01	5220	0.447000E	01	5221	0.119383E	02	5222	0.200000E	00	5223	0.0		5224	0.0		
5225	0.134700E	01	5226	0.447000E	01	5227	0.119383E	02	5228	0.200000E	00	5229	0.0		5230	0.0		
5231	0.139300E	01	5232	0.447000E	01	5233	0.119383E	02	5234	0.200000E	00	5235	0.0		5236	0.0		
5237	0.636000E	02	5238	0.447000E	01	5239	0.119383E	02	5240	0.200000E	00	5241	0.0		5242	0.0		
5243	0.377000E	02	5244	0.531000E	01	5245	0.125324E	02	5246	0.0		5247	0.0		5248	0.0		
5300	0.270000E	03	5301	0.0		5302	0.0		5303	0.585000E	03							
8030	-0.125500E	-05																
8063	-0.125500E	-05																
8100	-0.125500E	-05																
8132	-0.125500E	-05																
9901	0.400000E	03	9902	0.100000E	02	9903	0.200000E	03	9904	0.500000E	02	9905	0.0		9906	0.0		
9907	0.100000E	03	9908	0.100000E	03	9909	0.500000E	02	9910	0.200000E	03	9911	0.0		9912	0.0		
9913	0.300000E	04	9914	0.100000E	02	9915	0.200000E	03	9916	0.500000E	02	9917	0.0		9918	0.0		
9919	0.200000E	03	9920	0.300000E	03	9921	0.200000E	03	9922	0.500000E	03	9923	0.0		9924	0.0		

Input  
Data

NREG= 5 NRIT= 6 NCAN= 2 NZB= 2 KPC= 100  
 IMAX= 12 JMAX= 10 ICA= 3 ICB= 10 JCI= 2  
 JCE= 8 DELT= 0.00100 SI= 0.500000E 09 W= 0.100000E 10 V= 0.260000E 06  
 IDST= 20 INGR= 30 LF= 30 IBCR= 0 IDIR= 1  
 EP1= 0.0010 EP2= 0.0010 DAPF= 0.1000 DELTC= 0.001000 DELTI= 0.001000  
 SPRG= 0.200000E 21 SPB1= 0.200000E-23 SPB2= 0.800000E-23 INTG= 1 NUSO= 1

REG	I1	I2	J1	J2
1	1	12	1	10
2	3	6	1	6
3	3	6	6	8
4	6	10	1	4
5	6	10	4	8

REG	D1	SR	P	SF1	SOR	EQUI1	KV
1	0.13700000E C1	0.00333398E-02	0.10000000E 01	0.0	0.0	0.0	1
2	0.13499999E C1	0.03300000E-02	0.10000000E 01	0.0	0.0	0.31719999E-10	2
3	0.13499999E C1	0.03330000E-02	0.10000000E 01	0.0	0.0	0.31719999E-10	3
4	0.13499999E C1	0.03330000E-02	0.10000000E 01	0.0	0.0	0.31719999E-10	4
5	0.13499999E C1	0.03330000E-02	0.10000000E 01	0.0	0.0	0.31719999E-10	5

REG	D2	SA	SPR	SF2	FNU	EQUI2	TRU	TRM
1	0.00000000E CC	0.61339999E-04	0.0	0.0	0.0	0.0	0.0	0.0
2	0.07700000E CC	0.45199999E-02	0.0	0.51999999E-02	0.24599999E 01	0.31719999E-10	800.00	290.00
3	0.07700000E CC	0.45339999E-02	0.0	0.51799998E-02	0.24599999E 01	0.31719999E-10	800.00	290.00
4	0.07700000E CC	0.42399999E-02	0.0	0.50500000E-02	0.24599999E 01	0.31719999E-10	800.00	290.00
5	0.07700000E CC	0.47700000E-02	0.0	0.51499998E-02	0.24599999E 01	0.31719999E-10	800.00	290.00

RIT	DL	BETA
1	0.12500000E-01	0.21300000E-03
2	0.01300000E-01	0.11150000E-02
3	0.12300000E CC	0.07700000E-03
4	0.01200000E CC	0.16370000E-02
5	0.11500000E C1	0.03300000E-03
6	0.27400000E C1	0.13500000E-03

X

-30.000	30.000	60.000	90.000	120.000	160.000	200.000	230.000	240.000	250.000
---------	--------	--------	--------	---------	---------	---------	---------	---------	---------

Y

0.0	20.000	40.000	116.000	192.000	268.000	344.000	420.000	496.000	572.000
0.0	612.000								

ZONE	J13	J23	K3C	K8	KCA	VEBA1	VEBA2
1	1	5	0	1	1	0.4000000E-04	0.44999999E-03
2	5	3	0	2	2	0.4000000E-04	0.44999999E-03

	1	2	3	4	5	6	7	8	9	10	11	12
1	1	1	1	1	1	1	1	1	1	1	1	1
2	1	1	1	1	1	1	1	1	1	1	1	1
3	1	1	1	1	1	1	1	1	1	1	1	1
4	2	2	2	2	2	3	3	1	1	1	1	1
5	2	2	2	2	2	3	3	1	1	1	1	1
6	2	2	2	2	2	3	3	1	1	1	1	1
7	4	4	4	5	5	5	5	1	1	1	1	1
8	4	4	4	5	5	5	5	1	1	1	1	1
9	4	4	4	5	5	5	5	1	1	1	1	1
10	4	4	4	5	5	5	5	1	1	1	1	1
11	1	1	1	1	1	1	1	1	1	1	1	1
12	1	1	1	1	1	1	1	1	1	1	1	1

Core  
map



CANALE 1  
 SEZ.CAN.= 0.130000E C1 DZ= C.760000E 02 SC= C.540000E 01 RO= C.900000E 00  
 PALN1= 2473. N1= 3 CSE1= C.286000E C3

RES1	CPT1	PZQ1	VGL
0.775000E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.283000E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.184700E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.139300E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.686000E-02	0.447000E-01	0.200000E 00	0.119383E 02
0.377000E-02	0.591000E-01	0.0	0.125324E 02

CANALE 2  
 SEZ.CAN.= 0.130000E C1 DZ= C.760000E 02 SC= C.540000E 01 RO= C.900000E 00  
 PALN1= 6528. N1= 3 CSE1= C.286000E C3

RES1	CPT1	PZQ1	VGL
0.775000E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.283000E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.184700E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.139300E-01	0.447000E-01	0.200000E 00	0.119383E 02
0.686000E-02	0.447000E-01	0.200000E 00	0.119383E 02
0.377000E-02	0.591000E-01	0.0	0.125324E 02

Channel  
data

TEMPERATURE COEFFICIENTS

REG.	ALFA 1	ALFA 2	ALFA 3	ALFA 4
1	0.0	0.0	0.0	0.0
P	0.0	0.0	0.0	0.0
2	ALFA 1	ALFA 2	ALFA 3	ALFA 4
P	-0.1255399E-05	0.0	0.0	0.0
3	ALFA 1	ALFA 2	ALFA 3	ALFA 4
P	-0.1255399E-05	0.0	0.0	0.0
4	ALFA 1	ALFA 2	ALFA 3	ALFA 4
P	-0.1255399E-05	0.0	0.0	0.0
5	ALFA 1	ALFA 2	ALFA 3	ALFA 4
P	-0.1255399E-05	0.0	0.0	0.0

RICERCA CRITICITA

REGIONI AVVELENATE

1	2	3	4	5
ITERAZIONI	DLT	REP	VELFNO	IDER
1	0.10000E-02	0.28735D 02	0.0	2
2	0.10000E-02	-0.36366D 03	0.19999999D 21	1
3	0.10000E-02	-0.48248D 01	0.14645926D 20	15
4	0.10000E-02	0.15345D 01	0.12153696D 20	18
5	0.10000E-02	0.31339E 00	0.12755075D 20	31
6	0.10000E-02	0.19298D-01	0.12912832D 20	31
7	0.10000E-02	-0.34035E-02	0.12925745D 20	
8	0.10000E-02	-0.27554D-01	0.12938658D 20	
9	0.10000E-02	-0.52522E-01	0.12951571D 20	
10	0.10000E-02	-0.76358D-01	0.12964483D 20	
TEMPO TOTALE DI CALCOLO SEC.		19.090	TEMPO DI SOLO CRITIC SEC.	11.340

Criticality  
Search

FLUX 1				RISULTATI DEL CRITIC					
C.O	O.O	J.O	C.C	O.O	O.O	O.C	O.O	O.C	O.O
0.51905E 13	0.51905E 13	0.52421E 13	0.53269E 13	0.53728E 13	0.45240E 13	0.25849E 13	0.86094E 12	0.37967E 12	0.0
0.24241E 14	0.24241E 14	0.24433E 14	0.24917E 14	0.25378E 14	0.21462E 14	0.12267E 14	0.35230E 13	0.14363E 13	0.0
0.13432E 14	0.13432E 14	0.13553E 14	0.13938E 14	0.14259E 14	0.10504E 14	0.75323E 14	0.42674E 14	0.13700E 14	0.0
0.11935E 15	0.11935E 15	0.11948E 15	0.11925E 15	0.11839E 15	0.38239E 14	0.55553E 14	0.17531E 14	0.65332E 13	0.0
0.11990E 15	0.11990E 15	0.11921E 15	0.10310E 15	0.10240E 15	0.33603E 14	0.47727E 14	0.15391E 14	0.56441E 13	0.0
0.10007E 15	0.10007E 15	0.92613E 14	0.81433E 14	0.72810E 14	0.58357E 14	0.33557E 14	0.10954E 14	0.39884E 13	0.0
0.71489E 14	0.71489E 14	0.65441E 14	0.55502E 14	0.45624E 14	0.39241E 14	0.22476E 14	0.72641E 13	0.26690E 13	0.0
0.41771E 14	0.41771E 14	0.33133E 14	0.32783E 14	0.22672E 14	0.22575E 14	0.12904E 14	0.41680E 13	0.15309E 13	0.0
0.10513E 14	0.10513E 14	0.96512E 13	0.84506E 13	0.75400E 13	0.60234E 13	0.34581E 13	0.11111E 13	0.40787E 12	0.0
0.22263E 13	0.22263E 13	0.20505E 13	0.13046E 13	0.16059E 13	0.12742E 13	0.72881E 12	0.24359E 12	0.10754E 12	0.0
C.O	O.O	J.O	C.C	C.C	C.O	O.C	O.O	O.C	O.C

Print type m3

Flux map

These flux maps (pages 60 and 61) correspond to the equilibrium conditions after criticality search. Results are stored on tape. The calculation was restarted from this point. See page 48.

FLUX 2

0.0	0.0	0.0	0.0	C.0	C.0	0.0	0.0	0.0	0.0	0.0
0.31530E 14	0.31530E 14	0.31887E 14	0.32322E 14	C.34250E 14	0.29805E 14	0.17290E 14	0.68421E 13	0.34206E 13	0.0	0.0
0.51329E 14	C.51329E 14	C.51360E 14	0.52316E 14	C.55098E 14	0.47155E 14	0.27095E 14	0.11401E 14	0.62117E 13	0.0	0.0
0.14500E 15	C.14500E 15	0.14536E 15	0.14761E 15	0.15158E 15	C.12722E 15	C.72331E 14	C.30938E 14	C.17311E 14	0.0	0.0
0.19623E 15	C.19623E 15	0.19565E 15	0.19558E 15	0.19878E 15	C.16570E 15	0.94035E 14	0.40210E 14	0.22499E 14	0.0	0.0
C.20034E 15	0.20034E 15	C.19223E 15	C.17314E 15	C.17001E 15	0.14033E 15	0.80621E 14	0.34659E 14	0.19437E 14	0.0	0.0
0.17072E 15	C.17072E 15	0.15767E 15	C.13488E 15	C.12002E 15	0.97472E 14	0.56571E 14	0.24437E 14	C.13735E 14	0.0	0.0
0.12201E 15	C.12201E 15	0.11142E 15	C.93607E 14	C.81797E 14	0.65540E 14	0.37889E 14	0.16354E 14	0.91909E 13	0.0	0.0
0.71372E 14	C.71372E 14	C.65003E 14	C.54386E 14	C.47318E 14	0.37751E 14	0.21781E 14	0.93954E 13	0.52792E 13	0.0	0.0
0.23101E 14	0.23101E 14	0.21192E 14	0.18113E 14	0.16175E 14	0.13124E 14	0.76081E 13	0.32215E 13	C.17611E 13	0.0	0.0
0.13687E 14	0.13687E 14	0.12613E 14	0.11100E 14	0.10230E 14	C.83839E 13	0.48623E 13	C.19306E 13	C.96621E 12	0.0	0.0
0.0	C.0	0.0	0.0	0.0	C.0	0.0	C.0	C.0	0.0	0.0

Print  
type n3

1  
65  
1

ITS = 1    RESIDUAL 0.41350E 12    RE2 0.52446E 16    TO = 0.0    IT = 0

CORE AVERAGE	FLUX 1	FLUX 2	POWER	DP2	REP	TO
	0.49333E 14	0.34948E 14	C.50000E 09	-0.66795E 13	-0.78558E-01	C.0

Print  
type n3

CHANNEL N 1      TIME = 0.100      MASS FLCW = 0.5850000E 03

3	0.0	0.0	0.0	0.0	0.0	0.0	270.000
4	671.836	567.035	491.000	416.971	342.916	297.564	272.617
5	1037.563	838.768	694.205	553.187	411.859	325.152	277.471
6	1206.237	965.557	750.196	618.856	446.878	341.205	283.111
7	1119.143	902.837	745.132	590.818	435.768	340.401	287.982
8	969.476	733.597	665.059	539.177	412.537	334.548	291.690
9	838.803	638.194	595.126	493.920	391.858	328.855	294.248
10	643.919	554.410	488.593	423.798	358.298	317.769	295.517

AVERAGE TEMPERATURES

3	0.0	0.0
4	497.951	272.617
5	707.116	277.471
6	805.545	233.111
7	758.752	287.982
8	675.968	291.690
9	603.580	294.248
10	493.804	295.517

} Print  
type n8

CHANNEL N 2 TIME = C.100 MASS FLOW = C.585000CE 03

3	0.0	C.C	C.0	0.0	0.C	0.0	270.000
4	486.575	430.035	338.902	349.001	309.226	284.952	271.591
5	716.701	600.734	516.597	434.609	352.552	302.276	274.621
6	816.607	676.034	573.747	473.795	373.508	311.909	278.042
7	733.384	615.548	529.930	445.812	361.311	309.347	280.793
8	620.879	533.055	468.910	406.109	342.953	304.073	282.706
9	527.331	434.327	418.219	373.002	327.460	299.380	283.953
10	422.485	336.893	360.793	335.158	309.299	293.332	284.561

Print  
type n8

AVERAGE TEMPERATURES

3	C.C	C.C
4	392.823	271.591
5	524.245	274.621
6	532.743	278.042
7	537.377	280.793
8	471.381	282.706
9	422.063	283.953
10	352.920	284.561

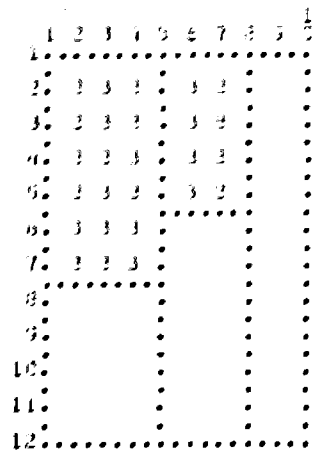
TIME 0.100 TOTAL ENERGY 0.48166E 03 AVERAGE ENERGY 0.54478E CC

PERCENT OF CORE WITH ENERGY LESS THAN  
100.000 C.1000CE 02  
0.C

Print  
type n9

CORE AVERAGE FLUX 1 FLUX 2 POWER OP2 RFP TO

MAX. INVERSION AT TIME 0.200  
 CONTROL ZONE            1            2  
 INVERSION CH.           337.00    243.00



Print  
type n4

Print  
type n7

MAX. TEMP. AT TIME 0.200

TYPE 1

CHANNEL	LEVEL	CENTER	LEVEL	SURFACE	LEVEL	H.FLOW	INLET	OUTLET	M.FLOW
1	7	3567.123	9	540.023	9	219.701	270.000	303.881	0.58500E 03
2	9	1399.875	9	353.221	9	66.643	270.000	296.847	0.58500E 03



MAP OF INTEGRATED ENERGIES AT TIME C.400

0.2338E 01	0.2338E 01	0.21913E 01	0.19651E 01	0.16672E 01	0.11370E 01	0.59145E 00	0.24701E 00
0.76115E 01	0.75116E 01	0.71243E 01	0.63627E 01	0.53940E 01	0.36607E 01	0.18925E 01	0.90195E 00
0.15631E 02	0.15871E 02	0.14710E 02	0.12329E 02	0.10833E 02	0.73170E 01	0.37821E 01	0.16029E 01
0.32320E 02	0.32020E 02	0.28745E 02	0.23775E 02	0.19140E 02	0.13012E 02	0.68300E 01	0.29029E 01
0.57259E 02	0.57259E 02	0.56800E 02	0.45852E 02	0.35153E 02	0.23677E 02	0.12454E 02	0.52928E 01
0.11917E 03	0.11917E 03	0.99555E 02	0.75343E 02	0.55640E 02	0.35611E 02	0.18340E 02	0.76727E 01
0.11993E 03	0.11993E 03	0.93570E 02	0.74971E 02	0.53467E 02	0.33016E 02	0.16701E 02	0.69468E 01
0.48914E 02	0.48914E 02	0.40507E 02	0.30607E 02	0.22208E 02	0.13649E 02	0.68803E 01	0.27996E 01

Print  
Type n10

TIME C.400 TOTAL ENERGY C.16203E 10 AVERAGE ENERGY 0.20588E 02

PERCENT OF CORE WITH ENERGY LESS THAN  
44.133 C.10000E 02  
25.867

Print  
type n9

CORE AVERAGE	FLUX 1	FLUX 2	POWER	DP2	RFP	TO
	0.89275E 15	0.19344E 16	0.86618E 10	0.50035D 16	0.32562E 01	0.41000E 00
CORE AVERAGE	FLUX 1	FLUX 2	POWER	DP2	RFP	TO
	0.92345E 15	0.15306D 16	0.92392E 10	0.58416D 16	0.36793E 01	0.42000E 00
CORE AVERAGE	FLUX 1	FLUX 2	POWER	DP2	RFP	TO
	0.93902E 15	0.15196D 16	0.96269E 10	0.57043D 16	0.34661D 01	0.43000E 00
CORE AVERAGE	FLUX 1	FLUX 2	POWER	DP2	RFP	TO

Print  
Type n2

④ DATI SU NASTRO

RISULTATI DEL CALCOLO AL TEMPO 0.200 IT = 200

This print indicates that at time 0.2 sec.  
 the results were stored on tape. The number ④  
 indicates that, to restart from this point, the  
 title card must contain 4 in the third column  
 and 4 DD statements must be given.  
 (See pages 42, 50, 52)

Some data were changed, they are printed  
 in the new calculation:

1	0.500000E 01	2	0.200000E-02	3	0.100000E-02	4	0.120000E 02	5	0.100000E 02	6	0.500000E 01
3061	0.0	3062	0.200000E 00	3063	0.500000E 00						
3091	0.0	3092	0.245000E 03	3093	0.612000E 03						
9901	0.200000E 03	9902	0.0	9903	0.0	9904	0.0	9905	0.0	9906	0.0
9907	0.0	9908	0.0	9909	0.0	9910	0.0	9911	0.0	9912	0.0
9913	0.300000E 03	9914	0.500000E 01	9915	0.100000E 03	9916	0.250000E 02	9917	0.0	9918	0.0
9919	0.500000E 02	9920	0.500000E 02	9921	0.250000E 02	9922	0.100000E 03	9923	0.0	9924	0.0
9925	0.600000E 03	9926	0.500000E 01	9927	0.100000E 03	9928	0.250000E 02	9929	0.0	9930	0.0
9931	0.100000E 03	9932	0.250000E 03	9933	0.100000E 03	9934	0.250000E 03	9935	0.0	9936	0.0

Appendix A

The two diffusion equations are:

$$1) \quad D_2 \nabla^2 \Psi - \Sigma_R \Psi + (1-\beta) \cdot (\Sigma_{f_1} \Psi + \Sigma_{f_2} \Psi) + \sum_{i=1}^N \lambda_i C_i = \frac{1}{W} \frac{d\Psi}{dt}$$

$$2) \quad D_1 \nabla^2 \varphi - \Sigma_a \varphi + \Sigma_R \cdot \rho \cdot \Psi = \frac{1}{V} \frac{d\varphi}{dt}$$

by reordering and substituting the time derivative with its finite-different expression for a time interval  $\Delta t$ , equation 2) takes the new expression

$$3) \quad D_2 \nabla^2 \varphi - A' \varphi + B \Psi + C' = \frac{1}{V} \frac{\varphi - \varphi^*}{\Delta t}$$

the new expression for eq. 1) is formally the same, therefore all the following discussions will refer only to eq. 3) as representative for both fast and thermal group.

By putting:

$$A = A' + \frac{1}{V \cdot \Delta t} \quad \text{and} \quad C = C' + \frac{\varphi^*}{V \cdot \Delta t}$$

and reordering, then eq. 3) becomes:

$$4) \quad D \nabla^2 \varphi - A \cdot \varphi + B \cdot \Psi + C = 0$$

This equation is now integrated over the sector

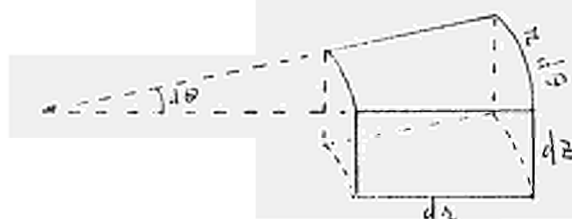


Fig. 5

$$5) \quad D \iiint_V [\nabla^2 \varphi] \cdot dz \cdot dz \cdot n d\theta - \iiint_V A \varphi + \iiint_V B \varphi + \iiint_V C = 0$$

For the theorem of Green the volume integral of the Laplace operator of  $\varphi$ , is equal to the integral over the surface of the volume, of the gradient of  $\varphi$  normal to the surface:

$$D \iiint_V \nabla^2 \varphi \cdot dz \cdot dz \cdot n d\theta = D \iint_S \frac{\delta \varphi}{\delta n} ds$$

Equation 5) then becomes:

$$6) \quad -D \iint_S \frac{\delta \varphi}{\delta n} ds + \iiint_V A \varphi - \iiint_V B \varphi - \iiint_V C = 0$$

all the signs have been changed in order to have a coefficient matrix with positive entries in the main diagonal and negative all the other entries.

Each integral term of eq. 6) will now be transformed into a finite-different expression where the flux  $\varphi_{i,j}$  is considered constant in the volume corresponding to the point  $(i,j)$  of the lattice. This volume has been divided into four sub-volumes Vol. 1,2,3,4; within each sub-volume the physical characteristic of the medium are constant.

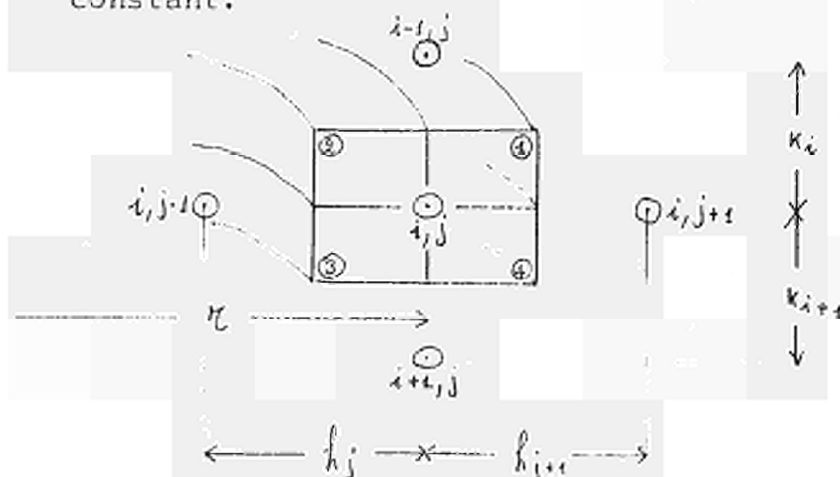


Fig. 6

Here below are the expressions for each term; with the help of fig. 6 they are self explanatory:

$$\begin{aligned}
 7) \quad \mathcal{D} \iint_S \frac{d\varphi}{h_n} dS &= - \frac{\varphi_{ij} - \varphi_{i,j+1}}{h_j} \left[ \mathcal{D}_2 \frac{k_i}{2} + \mathcal{D}_3 \frac{k_{i+1}}{2} \right] \cdot \left( r - \frac{h_j}{2} \right) 2\pi \\
 &+ \frac{\varphi_{i,j+1} - \varphi_{i,j}}{h_{j+1}} \left[ \mathcal{D}_1 \frac{k_i}{2} + \mathcal{D}_4 \frac{k_{i+1}}{2} \right] \cdot \left( r + \frac{h_{j+1}}{2} \right) 2\pi \\
 &- \frac{\varphi_{i,j} - \varphi_{i-1,j}}{k_i} \left[ \mathcal{D}_1 \frac{k_{i+1}}{2} \left( r + \frac{h_{j+1}}{4} \right) + \mathcal{D}_2 \frac{h_j}{2} \left( r - \frac{h_j}{4} \right) \right] 2\pi \\
 &- \frac{\varphi_{i+1,j} - \varphi_{i,j}}{k_{i+1}} \left[ \mathcal{D}_3 \frac{h_j}{2} \left( r - \frac{h_j}{4} \right) + \mathcal{D}_4 \frac{h_{j+1}}{2} \left( r + \frac{h_{j+1}}{4} \right) \right] 2\pi
 \end{aligned}$$

$$8) \quad \iiint A \cdot \varphi \cdot dz \cdot dr \cdot r d\theta =$$

$$\begin{aligned}
 &\varphi_{is} \cdot A \otimes \left[ \frac{k_i}{2} \frac{k_{i+1}}{2} \left( r + \frac{h_{j+1}}{4} \right) \right] \cdot 2\pi \\
 &+ \varphi_{ij} \cdot A \otimes \left[ \frac{k_i}{2} \frac{h_j}{2} \left( r - \frac{h_j}{4} \right) \right] \cdot 2\pi \\
 &+ \varphi_{ij} \cdot A \otimes \left[ \frac{k_{i+1}}{2} \frac{h_j}{2} \left( r - \frac{h_j}{4} \right) \right] \cdot 2\pi \\
 &+ \varphi_{i+1,j} \cdot A \otimes \left[ \frac{k_{i+1}}{2} \frac{h_{j+1}}{2} \left( r + \frac{h_{j+1}}{4} \right) \right] \cdot 2\pi
 \end{aligned}$$

$$\begin{aligned}
 9) \quad \iiint B \psi &= \psi_{ij} \cdot B_{\Theta} \cdot V\phi L_1 \cdot 2\pi \\
 &+ \psi_{ij} \cdot B_{\Theta} \cdot V\phi L_2 \cdot 2\pi \\
 &+ \psi_{ij} \cdot B_{\Theta} \cdot V\phi L_3 \cdot 2\pi \\
 &+ \psi_{ij} \cdot B_{\Theta} \cdot V\phi L_4 \cdot 2\pi
 \end{aligned}$$

$$\begin{aligned}
 10) \quad \iiint C &= C_{ij} \quad V\phi L_1 \cdot 2\pi \\
 &+ C_{ij} \quad V\phi L_2 \cdot 2\pi \\
 &+ C_{ij} \quad V\phi L_3 \cdot 2\pi \\
 &+ C_{ij} \quad V\phi L_4 \cdot 2\pi
 \end{aligned}$$

where  $V\phi L_1, 2, 3, 4 \cdot 2\pi$  are the subvolumes of fig. 6, or the expressions in parenthesis  $[ ]$  of eq. 8),

the integration in  $\Theta$  has been made over the whole circumference and due to the axial symmetry, it is only a multiplication with  $2\pi$ . This factor being common to all terms can be eliminated.

The final expression of the equation after reordering is:

$$11) \quad -\alpha_{ij} \cdot \psi_{i-1,j} - \beta_{ij} \cdot \psi_{i,j-1} - \gamma_{ij} \cdot \psi_{i+1,j} - \delta_{ij} \cdot \psi_{i,j+1} + \varepsilon_{ij} \cdot \psi_{ij} = b_{ij} \cdot \psi_{ij} + c_{ij}$$

where 
$$\alpha_{ij} = \frac{D_1 \frac{h_{j+1}}{2} \left( \kappa + \frac{h_{j+1}}{4} \right) + D_2 \frac{h_j}{2} \left( \kappa - \frac{h_j}{4} \right)}{K_i}$$

$$\beta_{ij} = \frac{\left( \kappa - \frac{h_j}{2} \right) \left[ D_2 \frac{K_i}{2} + D_3 \frac{K_{i+1}}{2} \right]}{h_j}$$

$$\gamma_{ij} = \frac{D_4 \frac{h_{j+1}}{2} \left( \kappa + \frac{h_{j+1}}{4} \right) + D_3 \frac{h_j}{2} \left( \kappa - \frac{h_j}{4} \right)}{K_{i+1}}$$

$$\delta_{ij} = \frac{\left( \kappa + \frac{h_{j+1}}{2} \right) \left[ D_1 \frac{K_i}{2} + D_4 \frac{K_{i+1}}{2} \right]}{h_{j+1}}$$

$$\epsilon_{ij} = \alpha_{ij} + \beta_{ij} + \gamma_{ij} + \delta_{ij} +$$

$$+ \frac{h_{j+1}}{2} \left( \kappa + \frac{h_{j+1}}{4} \right) \left[ A_{\textcircled{1}} \frac{K_i}{2} + A_{\textcircled{2}} \frac{K_{i+1}}{2} \right] + \frac{h_j}{2} \left( \kappa - \frac{h_j}{4} \right) \left[ A_{\textcircled{2}} \frac{K_i}{2} + A_{\textcircled{3}} \frac{K_{i+1}}{2} \right]$$

$$b_{ij} \psi_{ij} + C_{ij} = \psi_{ij} \left[ B_1 \text{Vol}_1 + B_2 \text{Vol}_2 + B_3 \text{Vol}_3 + B_4 \text{Vol}_4 \right] + \left[ C_1 \text{Vol}_1 + C_2 \text{Vol}_2 + C_3 \text{Vol}_3 + C_4 \text{Vol}_4 \right]$$

It can be verified that:  $\alpha_{ij} = \beta_{i,j+1}$

and  $\gamma_{ij} = \alpha_{i+1,j}$

the pentadiagonal matrix of the coefficients is symmetric.

The equation 11) can be written:

$$\alpha) -\alpha_{ij} \cdot \psi_{i-1,j} - \beta_{ij} \cdot \psi_{i,j-1} - \alpha_{i+1,j} \cdot \psi_{i+1,j} - \beta_{i,j+1} \cdot \psi_{i,j+1} + \epsilon_{ij} \cdot \psi_{ij} = b_{ij} \psi_{ij} + C_{ij}$$

it is possible therefore to avoid the storage of the two reactor  $\gamma$  and  $\delta$ .

4673c.

Appendix B

Error due to the time discretization in steps of length  $\Delta t$ .

Let us consider the one-group time dependent diffusion equation:

$$\cancel{D \nabla^2 \varphi} - \Sigma_a \varphi + k \Sigma_a \varphi = \frac{1}{v} \frac{d\varphi}{dt}$$

neglecting the spatial flux distribution and the diffusion term the eq. reduces to:

$$\frac{\delta k}{l} \varphi = \frac{\delta \varphi}{\delta t}$$

where  $l = \frac{1}{v \Sigma_a}$

let  $\frac{\delta k}{l} = \alpha$

substituting the time derivative with its finite-difference expression we have:

$$\alpha \varphi = \frac{\varphi - \varphi^*}{\Delta t}$$

at the time  $t = n \Delta t$  the value of the flux as determined by this approximation will be :

$$\varphi' = \varphi_0 \left( \frac{1}{1 - \alpha \Delta t} \right)^n \quad \text{a)}$$



whereas the exact analytical solution will give :

$$\varphi'' = \varphi_0 e^{m \alpha \Delta t} \quad \text{b)}$$

The appropriate solution  $\varphi'$  exceeds the exact one  $\varphi''$  by an error :

$$\xi = \frac{\varphi' - \varphi''}{\varphi''} = \frac{\left(1 - \frac{\Delta t}{T}\right)^{-n} - e^{n \frac{\Delta t}{T}}}{e^{n \frac{\Delta t}{T}}}$$

In the range of accidents which can be treated by this code, the power level does not become greater than  $10^3$  the nominal value. A scram or a temperature reaction will interrupt the increase of power, and in the great majority of cases the increase of power is less than 10 times the nominal value.

Here below are calculated the values of  $\xi$  for various values of the ratio  $\Delta t/T$  between the time step and the reactor period, for an increase of power of  $10^3$  and 10.

Three decades increase

$$e^{n \frac{\Delta t}{T}} = 10^3$$

$$n \frac{\Delta t}{T} = 3 \cdot \lg_e 10 = 6.90776$$

$$n = 6.9 \dots / \frac{\Delta t}{T}$$

$$\varepsilon = \frac{(1 - \frac{\Delta t}{T})^{\frac{-6.9 \dots}{\Delta t/T}} - 1000}{1000}$$

$\Delta t/T$	$\varepsilon$
0.05	19.7 %
0.01	3.18 %
0.005	—

One decade increase

$$e^{n \frac{\Delta t}{T}} = 10$$

$$n \frac{\Delta t}{T} = \lg_e 10 = 2.3026$$

$$n = 2.3 / \frac{\Delta t}{T}$$

$$\varepsilon = \frac{(1 - \frac{\Delta t}{T})^{\frac{-2.3}{\Delta t/T}} - 10}{10}$$

$\Delta t/T$	$\varepsilon$
0.1	13.1 %
0.05	6.2 %
0.01	1 %

Convergence criterium for the fast and thermal flux

The iterations stop when the residual is smaller than a given value of  $\varepsilon$ , criterium of convergence :

$$\frac{\sum_{i=1}^N |\Delta \varphi_i|}{\sum_{i=1}^N \varphi_i} < \varepsilon$$

$\varepsilon$  should be smaller than the possible variation of  $\varphi$  in  $\Delta t$

$$\varphi = \varphi_0 e^{\frac{\Delta t}{T}}$$

$$\varepsilon < \frac{|\Delta \varphi|}{\varphi} = \left| 1 - e^{\frac{\Delta t}{T}} \right|$$

$$e^{\frac{\Delta t}{T}} = 1 + \frac{\Delta t}{T} + \left\{ \left( \frac{\Delta t}{T} \right)^2 \frac{1}{2!} + \dots \right.$$

truncating at the second term and substituting

$$\varepsilon < \frac{\Delta t}{T}$$



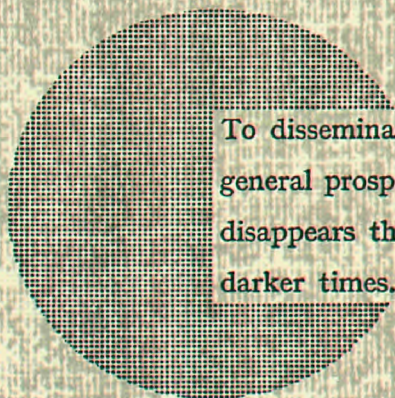
## NOTICE TO THE READER

All scientific and technical reports published by the Commission of the European Communities are announced in the monthly periodical "euro-abstracts". For subscription (1 year : US\$ 16.40, £ 6.17, Bfr 820,-) or free specimen copies please write to :

**Handelsblatt GmbH**  
**"euro-abstracts"**  
**D-4 Düsseldorf 1**  
**Postfach 1102**  
**Germany**

or

**Office for Official Publications**  
**of the European Communities**  
**P.O. Box 1003 - Luxembourg 1**



To disseminate knowledge is to disseminate prosperity — I mean general prosperity and not individual riches — and with prosperity disappears the greater part of the evil which is our heritage from darker times.

Alfred Nobel

## SALES OFFICES

All reports published by the Commission of the European Communities are on sale at the offices listed below, at the prices given on the back of the front cover. When ordering, specify clearly the EUR number and the title of the report which are shown on the front cover.

### OFFICE FOR OFFICIAL PUBLICATIONS OF THE EUROPEAN COMMUNITIES

P.O. Box 1003 - Luxembourg 1  
(Compte chèque postal N° 191-90)

#### BELGIQUE — BELGIË

MONITEUR BELGE  
Rue de Louvain, 40-42 - B-1000 Bruxelles  
BELGISCH STAATSBLAD  
Leuvenseweg 40-42 - B-1000 Brussel

#### LUXEMBOURG

OFFICE DES  
PUBLICATIONS OFFICIELLES DES  
COMMUNAUTÉS EUROPÉENNES  
Case Postale 1003 - Luxembourg 1

#### DEUTSCHLAND

VERLAG BUNDESANZEIGER  
Postfach 108 006 - D 5 Köln 1

#### NEDERLAND

STAATSDRUKKERIJ-  
en UITGEVERIJBEDRIJF  
Christoffel Plantijnstraat - Den Haag

#### FRANCE

SERVICE DE VENTE EN FRANCE  
DES PUBLICATIONS DES  
COMMUNAUTÉS EUROPÉENNES  
rue Desaix, 26 - F-75 Paris 15<sup>e</sup>

#### ITALIA

LIBRERIA DELLO STATO  
Piazza G. Verdi, 10 - I-00198 Roma

#### UNITED KINGDOM

H. M. STATIONERY OFFICE  
P.O. Box 569 - London S.E.1

Commission of the  
European Communities  
D.G. XIII - C.I.D.  
29, rue Aldringen  
L u x e m b o u r g

CDNA04673ENC