

**EUR 4203 e**

**EUROPEAN ATOMIC ENERGY COMMUNITY - EURATOM**

**E R U P T**

**A two-dimensional, two-energy group  
fuel management programme  
for the IBM - 360 computer**

by

**W. BÖTTCHER, A. DEGRESSIN AND F. LAFONTAINE**

**1968**



**ORGEL Program**

**Joint Nuclear Research Center  
Ispra Establishment - Italy**

**ORGEL Project**



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MANAGEMENT PROGRAMME FOR THE IBM - 360 COMPUTER  
by W. BÖTTCHER, A. DECRESSIN and F. LAFONTAINE

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## SUMMARY

ERUPT is a programme for survey calculations of fuel management on a two-dimensional (r, Z) model of a reactor core. The number of energy groups is two.

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The codification is in Fortran IV for the IBM-360/65 computer.

## KEYWORDS

E-CODES  
FUEL CYCLE  
REACTOR CORE  
AUTOMATION

FORTRAN  
IBM 360  
NUMERICALS

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ERUPTA two-dimensional, two-energy group, fuel management programme  
for the IBM - 360 computer1. INTRODUCTION

ERUPT is a two-dimensional, two-energy group, fuel management code for a reactor core in r-Z geometry. This geometry has been chosen in order to be able to study also variations of axial power shapes during burn-up.

The diffusion theory calculation has been based on the code Equipoise-3 (1) being a rapid two-dimensional code suitable for survey calculations.

ERUPT is applicable to a wide range of fuel management problems. Axial shufflings are not included. Using the code, the user has to be aware that fuel managements are performed with concentric annuli. As diffusion theory is concerned, the problem has to be adapted to a homogenization in two energy groups.

ERUPT does not carry out a calculation of the variation of the isotopic composition of the fuel with irradiation. The group constants of the diffusion equations have to be provided by a separate burn-up code. The test calculations of the code have been performed with data supplied by the codes Plutharco (2) and RLT (3).

Core-life is divided in a number of time steps. During a time interval, the power distribution is regarded as unchanged. At the end of each time interval, the fuel composition has changed according to the constant flux during the time interval

multiplied by the length of the time interval. The corresponding two-group parameters are obtained by interpolation in the data tables provided by the separate point burn-up code.

This report contains two parts; in the first part, the main characteristics of the code are reported; the second part describes "How to use the code".

## 2. THE PHYSICAL BASIS OF ERUPT

ERUPT solves the two-group diffusion equations using the same finite difference techniques as in the code Equipoise 3. It is remembered that the calculational procedure is accelerated by the use of the "extrapolated Liebmann process" which is characterized by an "over-relaxation factor"  $\beta$  being a number between 1 and 2. If this factor is not given in the input data, it is automatically calculated by the code. The form of the diffusion equations at each point can be written in shorthand as:

$$D_k \nabla^2 \phi_k + \sum_{R,3-k} \phi_{R,3-k} - (\sum_{A_k} + \sum_{R_k}) \phi_k + \lambda \eta_k \sum_k v_k \sum_{f_k} \phi_k = 0$$

$$\begin{aligned} k &= 1, 2 \\ \sum_k \eta_k &= 1 \end{aligned}$$

The notation of the symbols is the common one. The factor  $\lambda$  can be regarded as the inverse of a static multiplication factor.

As  $\sum_{R_2}$  is equal to zero and normally also  $\eta_2 = 0$  (only fast neutrons are created), the above system of equations reduces to:

$$D_1 \nabla^2 \phi_1 - (\sum_{A_1} + \sum_{R_1}) \phi_1 + \lambda (v_1 \sum_{f_1} \phi_1 + v_2 \sum_{f_2} \phi_2) = 0$$

$$D_2 \nabla^2 \phi_2 - \sum_{A_2} \phi_2 + \sum_{R_1} \phi_1 = 0$$



The code contains three convergence criteria; these are eigen value, flux and residue condition. The calculation stops automatically when these conditions are not met after the specified total number of iterations.

### 3. THE CORE CONFIGURATION

Regions in ERUPT are concentric annuli (maximum number 275) in which the group constants of the diffusion equations are spatially constant. Except for the reflector regions, all group constants may be given as function of the integrated flux. Every region consists of one or more mesh points with identical group constants. The maximum number of mesh points is 3200. They may be distributed according to the requirements of the problem.

All vertical regions containing fissionable materials within the same column form a cylinder of core height, named "RING". Fuel management operations are performed by means of these rings; this point is to be remembered when the size of the regions is defined.

As ERUPT treats only refuelling schemes with radial shuffling, an axial symmetry condition exists which can be used when top and bottom reflectors have identical properties.

If the core contains elements of different design, pitch or enrichment, it has to be noted that only identical cell types can be arranged to rings. For the batch cycle, this limitation is not existent. The maximum number of different cell types is four. Appendix A gives a picture of the core arrangement.

#### 4. PROCEDURE OF THE CALCULATION

The time-integrated flux, TAU, is the independent variable of the problem. Its initial distribution in the core has to be given for all fuel regions on input cards. For all further calculations, it is found by accumulating the TAU-steps which are the product of the point-wise fluxes and the fixed time interval in which the distribution is regarded as invariant. (The time step in days has to be specified in the input data).

The TAU-distribution at every time-step of reactor life allows to find, by linear interpolation in the library table (5), the distribution of group constants with means of which the steady state flux distribution and the critical value are calculated.

The total power is summed up assuming that thermal power is equal to fission rate multiplied by an appropriate proportionality constant given as input data. Normalizing the total power to the value specified in input data, the point-wise fluxes are converted into absolute values.

The reactivity level is the only criterion for the management decision process up to now incorporated in the code. If with the new distribution of group constants, the reactivity worth falls below a prescribed value within a specified reactivity band-width, a fuel management operation is performed (see also Appendix B). In the case that the reactivity level is higher than the prescribed value, the calculation is pursued in the above-described manner.

The reactivity can get negative values during the calculation or fall below the specified reactivity band-width. In this case, the code reduces automatically the last time step in order to maintain critically the core within the reactivity band-width. The necessary reduction is found by interpolation multiplying the fixed time interval by the ratio of the reactivity step wanted to the reactivity step obtained between



the last and preceding calculation. The reactivity level wanted is defined as that lying below the prescribed value and in the middle of the reactivity band.

The reactivity-band-width prevents that at every time step an interpolation becomes necessary. In Appendix B, a picture underlines the fuel management decision process.

## 5. THE FUEL MANAGEMENT OPERATIONS IN ERUPT

Three modes exist to run ERUPT.

Mode 1 performs a batch calculation. The core is loaded with fuel according to a given scheme in order to obtain the surplus reactivity reserved for burn-up. For this core, after each fixed time step, ERUPT gives the variation of the eigen value, fluxes and form factors. As soon as the reactivity exceeds the prescribed reactivity level, the programme stops.

With mode 2, all imaginable radial shufflings can be performed. Before applying the code to on-line refuelling schemes in which equilibrium of a core is maintained by charging and discharging only few elements every few days, it has to be remembered that fuel operations are performed by displacing or changing complete rings.

The variety of radial management operations has been made possible by means of the load vector which is described in detail in Appendix B.

The third mode to use ERUPT is a cycle without any shuffling called once-through cycle. The ring with the highest burn-up is pushed out of the core and replaced by unirradiated fuel.

## 6. DATA PREPARATION FOR THE CODE

For the data preparation, at first the group constants of the diffusion equations have to be calculated for every cell type, by a separate burn-up code, and tabulated as function of integrated flux. If some group constants remain unchanged as burn-up proceeds, they are tabulated only in the first step of the table. Then the preparation of input data for ERUPT is easily carried out following the "How to use the code" with the attached data punch sheet and sample problem.

Note that two possibilities exist to start the problem. The initial TAU-distribution may be zero for all regions or it may be assessed or known from preceding calculations. Both cases are treated in the same way, choosing the option RUN=1. In this option, cards are read in, which have to contain for every region the fuel type and the initial TAU-value.

An important feature of the code is the Restart possibility. Thus, an investigation which is too long for a single run may be broken down into several computer runs. The necessary data are automatically transferred from one execution to the other by means of the library tape, and it is sufficient to use the option RUN=2.

## 7. GENERAL EXPERIENCE WITH ERUPT

ERUPT has been applied to a number of survey calculations carried out in the frame of the ORGEL prototype, a heavy water-moderated, organic-cooled reactor of the pressure tube type. The fuel management used was an on-line refuelling scheme with radial shuffling. A comparison with a three-dimensional fuel management code based on the source-sink method has shown satisfactory results, though ERUPT is not best adapted to



on-line refuelling schemes. The two group parameters for all calculations have been provided by the codes PLUTHARCO (2) and RLT-4 (3), which are appropriate for the treatment of a reactor lattice composed of individual cells.

Studies of time-dependent cores, for which large running times are necessary, can be easily carried out by ERUPT. The Restart facility allows the segmentation of the overall running time in several shorter pieces which is often advantageous from the point of view of availability of the computer. The main reason, however, for the Restart facility is to make it possible to intervene as burn-up proceeds in order to improve the management policy and to optimize the time-dependent core.

The total running time of a problem depends mainly, apart from the number of mesh points and the precision desired for the flux and reactivity value calculations, on the number of time steps which are necessary to solve the problem realistically. The length of a time step is given by the requirement that it must be short enough in order not to lead to unrealistic results because of the assumption of constant flux distribution during the time step. Calculations have shown that time steps of 10-25 days are acceptable.

For the sample problem, 1950 mesh points, convergence criteria of eigen value equal to  $4 \times 10^{-4}$ , of flux and residue equal to  $1 \times 10^{-3}$ , the execution time is about 0.7 minutes per time step.

In order not to obtain large amounts of output, it is recommended to use the option of a detailed printout in only a few selected cases.

---

REFERENCES

- (1) T.B. Fowler, M.O. Tobias  
A two-dimensional, two-group, neutron diffusion code  
for the IBM-7090 computer  
ORNL 3199
  
  - (2) W. de Haan, R. Meelhuysen  
Plutharco, A Plutonium, Uranium, Thorium Assembly  
Reactivity Code  
Physical Concepts, Comparisons with Experiments and  
Code Description  
EUR 3141 e (1964)
  
  - (3) G. Blaesser, G. Casini, J. Pillon  
Variation de la Réactivité à long terme pour différents  
types de circulation du combustible  
EUR-129 f (1962)
-



HOW TO USE THE CODE

1. Identification

Name	ERUPT - A two-dimensional, two-energy group fuel management programme for the IBM-360 computer
Job n°	68.8125
Codification	FORTRAN IV
System	IBM 360/65
Date	1968, September
Origin	ORGEL Project, Common Research Center, Ispra, Varese, Italy

2. Input data

The following list explains all input data items. Its arrangement on punched cards is shown in the table at the end of this chapter.

<u>Symbol</u>	<u>Description</u>
HØL	Any alphanumeric comment
ITMAX	Max. number of iterations (eg. 300)
IMAX	Total number of rows $\leq 100$ ) (see also APP. 1) )
JMAX	Total number of columns $\leq 100$ ) IMAXxJMAX $\leq 3200$ (see also APP. A) )
NREG	Total number of regions included reflectors $\leq 275$
NMAT	Total number of compositions included reflectors $\leq$ NREG
NGEM	Geometry indicator ( = 1 )
NBUC	Transverse buckling indicator ( = 0 )
NSØR )	
NADJ )	Not used
NFX )	
NVG	Convergence indicator = 1 problem converges only on $\lambda$ - critical = 2 problem converges on $\lambda$ -critical <u>and</u> flux = 3 problem converges on $\lambda$ -critical, flux <u>and</u> residue
NDIAG	Diagonal symmetry indicator ( = 0 )
NLB	Left boundary indicator ( = 1 )
NTB	Top boundary indicator ( = 0 )
NRB	Right boundary indicator ( = 0 )
NBB	bottom boundary indicator = 0 no symmetry = 1 axial symmetry
FACTOR	Total fission power in MW for normalization
EPI1	Convergence criteria for $\lambda$ -critical ( $4 \cdot 10^{-4}$ )
EPI2	Convergence criteria for flux ( $1 \cdot 10^{-3}$ )
EPI3	Convergence criteria for residue ( $1 \cdot 10^{-3}$ )
BETA	Over-relaxation factor (if this number is not given, it is automatically calculated)

XI1                    Fraction of fission neutrons in fast group ( $n^{\circ}1$ )  
XI2                    Fraction of fission neutrons in thermal group  
                          ( $n^{\circ}2$ )  
                          (XI1 + XI2 = 1)  
BSG1                    Buckling ( = 0 )  
BSG2                    Buckling ( = 0 )

(ELY, Y)<sub>I</sub>                ELY is the spacing between two succeeding rows  
                          and is applied from row Y<sub>I-1</sub> to Y<sub>I</sub>.  
                          (see also APP.A)

(ELX, X)<sub>J</sub>                ELX is the spacing between two succeeding columns  
                          and is applied from column X<sub>J-1</sub> to X<sub>J</sub>.  
                          (see also APP. A)

MNR                    Composition number  
I1                      Top row number  
I2                      Bottom row number  
J1                      Left column number  
J2                      Right column number

TAG                    Additional data (numerated from 1 to 15)  
                          - Blank if there are several cards with  
                          "Additional Data"  
                          - Set "\*" if it is the last card of this set

N1    )  
N2    )                The following data items are numerated from 1  
          )                to 15; they have to be sequentially punched  
                          on cards, in which N1 and N2 are the number of  
                          the first and the last data item on a card.

Nr

(1) NRT                Number of introduced tables (fuel compos.)  
(2) RUN                = 1 for the initial run ( $\tau_i$  specified by input  
                          card(s)  
                          = 2 if  $\tau$  -distribution of a preceding calculation  
                          is available on library tape

(3) LD1                Number of fuel regions  
(4) LD2                Number of vertical fuel regions    )  
(5) LD3                Number of horizontal fuel regions )    LD1 = LD2 \* LD3

- (6) CIRC Calculation mode (see also APP. B)  
= 1 no fuel management (batch)  
= 2 radial fuel management with fixed loading scheme according to the load vector  
= 3 only the ring with the highest burn-up is discharged and replaced by fresh fuel the type of which is TYPE
- (7) TYPE Type of fuel for recharge (only used if CIRC = 3)
- (8) KEF1 ) higher  
(9) KEF2 ) lower Limit of reactivity band for management process
- (10) DT Time interval in which the fluxes are assumed to remain constant days
- (11) P/F Power per fission MeV/fission
- (12) XEQ Execution time available for the calculation min.
- (13) PRCC Print index  
= 1 if the Core Composition is to be printed  
= 0 if this information is not desired
- (14) PRTF Print index  
= 1 if the Thermal cell Fluxes per region are to be printed  
= 0 if the results are not desired
- (15) PRSP Print index  
= 1 if Specific Power distribution is to be printed  
= 0 if this result is not desired
- LDCØ Number of positions occupied in the LOAD vector
- LØAD Load vector which contains the radial fuel movement scheme (see APP.B)
- NM Region number of reflectors ) For the  
NG Group number ) two groups  
D Diffusion coefficient ) of axial  
SIGR Macroscopic removal cross-section ) and  
SIGA Macroscopic absorption cross-section ) radial  
NUSF ~~U~~\*macroscopic fission cross-section (=0) ) reflectors
- K Table number
- J Number of burn-up steps in table k



IND1

Interpolation index vector

Contains 11 positions corresponding to the 11 program variables (BU, D1, SR1, SA1, NSF1, SF1, D2, SR2, SA2, NSF2, SF2)

A number 1 means that the variable corresponding to the position is given as a function of irradiations TAU1 ; the number 0 means that the parameter is kept constant during irradiation and equal to the initial value (specified in the 1st TAU-step in the table)

TAU	Cell irradiation (time-integrated flux)	$\frac{n/kb}{}$
BU	Burn-up	$\frac{MwD/T}{}$
D1	Fast diffusion length	$\frac{cm}{}$
SR1	Macroscopic removal cross-section (fast)	$\frac{cm^{-1}}{}$
SA1	Macroscopic absorption cross-section (fast)	$\frac{cm^{-1}}{}$
NSF1	$\nu$ * times macroscopic fission cross-section (fast)	$\frac{cm^{-1}}{}$
D2	Thermal diffusion length	$\frac{cm}{}$
SR2	Macroscopic removal cross-section (therm)	$\frac{cm^{-1}}{}$
SA2	Macroscopic absorption cross-section (therm)	$\frac{cm^{-1}}{}$
NSF2	$\nu$ * times macroscopic fission cross-section (therm)	$\frac{cm^{-1}}{}$
SF2	Macroscopic fission cross-section (therm)	$\frac{cm^{-1}}{}$
LIFE	Reactor life at initial definition of the core	$\frac{days}{}$
NM } NG }	Lower and upper delimiting point for succeeding regions in the reactor with the same reactor state	
CØMP	Composition number )	Applied to all regions NM to NG
CHAR	Charge number )	
TAU	Irradiation )	

,DATA CARDS		
Card(s)	Format	Symbols
1	20A6	HØL
1	16I3	ITMAX, IMAX, JMAX, NREG, NMAT, NGEM, NBUC, NSØR, NADJ, NFX, NVG, NDIAG, NLB, NTB, NRB, NBB
1	5E10.5	FACTØR, EPI1, EPI2, EPI3, BETA
1	4E10.5	XI1, XI2, BSG1, BSG2
variable	8(E8.5,I2)	(ELY,Y) <sub>I</sub> I sets, the last one containing Y = IMAX
variable	8(E8.5,I2)	(ELY,X) <sub>J</sub> J sets, the last one containing X = JMAX
NREG/6	6(I3,4I2,1x)	(MNR,I1,I2,J1,J2) <sub>K</sub> K = 1, NREG
variable	A1,2I2,7E10.5	TAG, N1, N2, DATA (N1), ....., ....., ....., DATA (N2)
1	I3	LDCØ
LDCØ/20	20I3	LØAD <sub>L</sub> L=1,LDCØ } only if CIRC = 2
4 or 6	2I3,5E10.5	NM, NG, D, SIGR, SIGA, NUSF, BS
1	2I3	K, J
1	11I3	IND1 <sub>M</sub> M = 1,11
	2E10.5	TAU, BU
J	5E10.5	D1, SR1, SA1, NSF1, SF1
	5E10.5	D2, SR2, SA2, NSF2, SF2
1	E10.5	LIFE
variable	4I3, E10.5	NM, NG, CØMP, CHAR, TAU } Only of RUN = 1; a value NG = LD1 stops input sequence

### 3. Output data

The output printed is largely self-explaining; at first, all input data are reprinted, then the program prints the following results (some of them only on request):

- a table with the cell volumes (cm<sup>3</sup>);
- $f_t$ ,  $\Lambda_t$ ,  $\rho_t$ ,  $R_{max}$ ,  $\nu$ -critical printed every ten iterations;
- the final value of reactivity  $keff = 1/\nu$ -critical;
- the form factors for fast and thermal fluxes and for the power distribution;
- total fission power (MW);
- a table with thermal cell fluxes (on request);
- a table with power distribution (on request);
- execution time per each cycle;
- core composition for each region (on request);  
this table contains the region number (REG), the fuel type (CØMP), the charge number (CHAR), and the variables TAU, BU, D1, SR1, NSF1, SF1, D2, SR2, NSF2, SF2 for each fuel region;  
in any case, this information is printed before and after each fuel movement;
- for the discharged fuel, the mean irradiation in n/kb (TAUM), the mean burn-up (BUM), the max. burn-up (BUMAX) and the form factor of burn-up is printed.

### 4. Remarks

#### 4.1. Program limitations :

The actual program dimensions allow the treatment of problems with:

IMAX	< 100
JMAX	< 100
IMAX*JMAX	≤ 3200
NREG	≤ 275
NREG > NMAT	≤ 275
LDCO	< 1000
NRT	≤ 4
J ≤ 50	(number of burn-up steps in the tables)

#### 4.2. Memory occupation

The program, including library functions, occupies about 230k bytes.

#### 4.3. Magnetic tapes

The program uses 1 library tape (symbolic tape unit 9) for writing the  $\tau$ -distribution at each time step: thus a calculation which cannot be finished in the given execution time can be restarted with the last registered  $\tau$ -distribution. If this option will not be used, define a scratch tape as dummy library.

#### 4.4. Execution time

The execution time is dependent on the number of mesh points, the iteration number according to the precision required and the output desired at each time-step. For example, a problem with 1950 mesh points will take about 0.7 minutes per time step.

#### 4.5. Printed lines

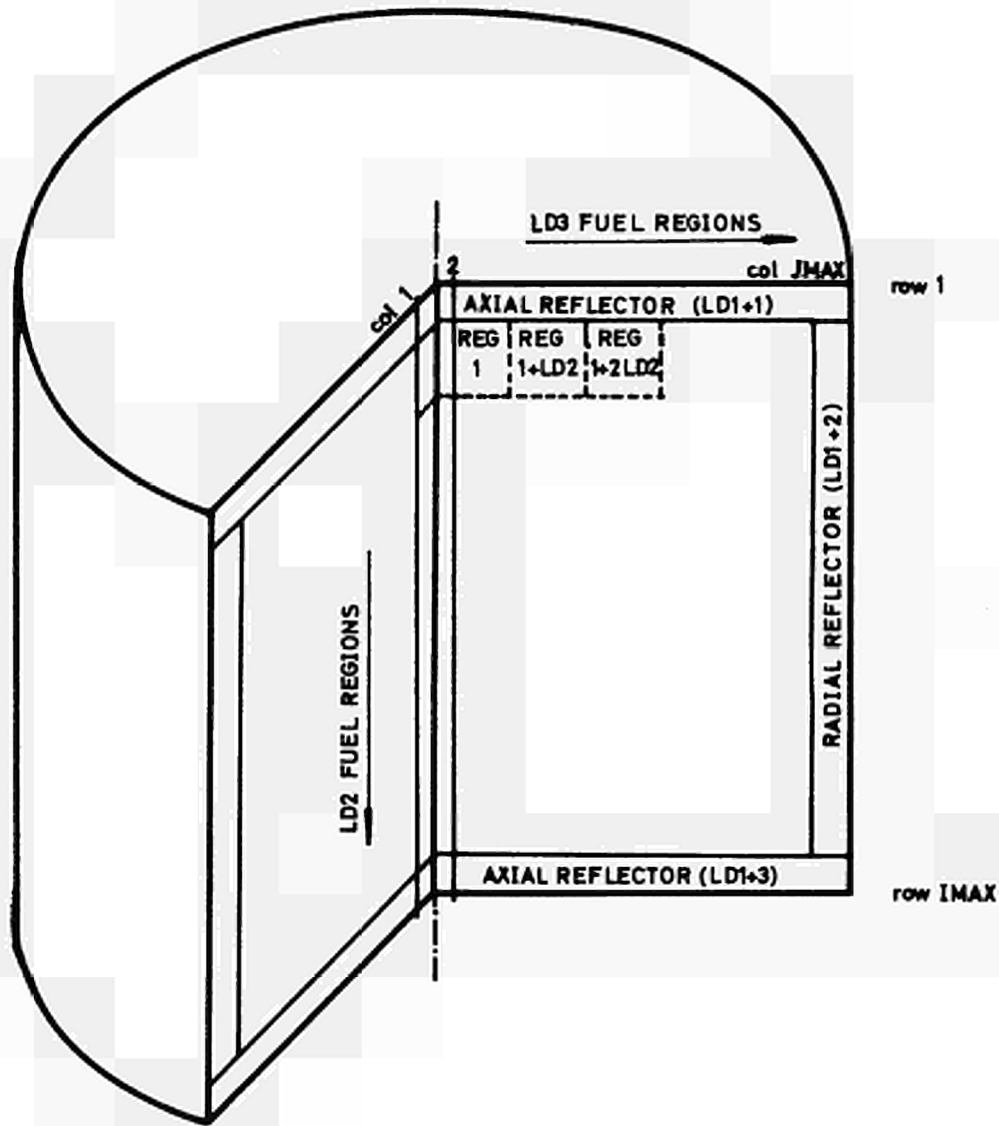
The number of printed lines depends on the output options. Printing of available information takes about 1250 lines max/400/lines min. output per 1 minute of execution time.

#### 4.6. Clock overflow

A clock overflow may happen if the available execution time is too short for a complete investigation. The program checks however that such a clock overflow cannot happen during the "write-tape" operation of the TAU-distribution.



## APPENDIX A

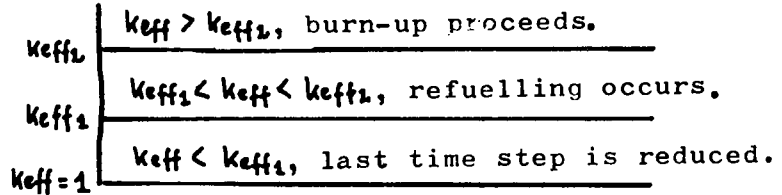
Reactor geometryExplanation

- 1) The reactor axis is always situated between columns 1 and 2. For instance, if an equidistant mesh is used, take attention that the distance between Col. 1 and 2 is the double of one 2-3 a.s.o.
- 2) If axial symmetry is used, the symmetry axis is situated between row IMAX-1 and IMAX.
- 3) Regions are defined by top/bottom row and left/right column numbers, and are numbered in vertical sense. We name a RING LD2 vertical regions.
- 4) The reflector regions are numbered LD1+1, LD1+2, LD1+3.

APPENDIX B

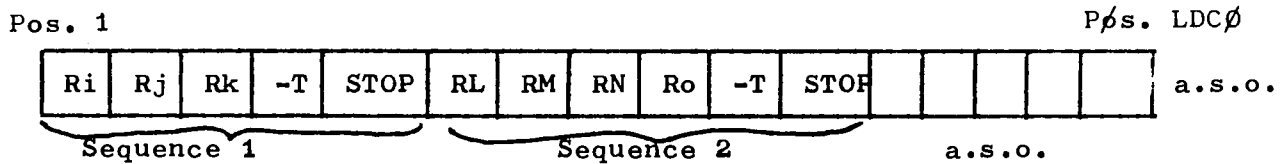
(Management)

1. Decision process



2. Load vector

The load vector contains references to RING-numbers (1 to LD3), TABLE-numbers (1 to 4) and STOP-sequence-numbers (0 or 1), as shown:



Explanation

The ring, the number of which is  $R_i$ , is discharged and replaced by the ring  $R_j$ , which is at the same time replaced by  $R_k$ . Then ring  $R_k$  is replaced by fresh fuel of type T. If STOP is 0, the program treats also immediately the following sequence in the same way as sequence 1. If stop is 1, the shuffling stops and, at the next loading process, the program enters in the load vector at this point.

Note that the number of rings to be moved (in the first sequence  $R_j, R_k$ ) is variable (from 0 to LD3-1) and may also vary from one sequence to the other.

Logically, at least in the position LDC0, the STOP-sequence-number must be 1. After having executed all management information within the LOAD vector, the program returns automatically at the beginning of the load vector.

***	SAMPLE	PROBLEM	ERUPT	***																
300	34	30	92	92	1	0	0	0	0	3	0	1	0	0	1					
350		+02	4		-04	1		-03	1		-03									
1		+00																		
18.5		3	6.0	33	0.00	0134														
44.444		2	22.222	4	9.205	7	7.063	10	5.954313	5.245	16	4.742719	4.361322							
4.059325		3	3.813	28	26.0	30														
103060104		4	206090104	104	3	3120104	41	2150104	51	5130104	104	618210104								
721240104		5	824270104	104	927300104	104	1030340104	104	11	3	60407	1206090407								
13	9120407	6	1412150407	407	1515130407	407	1618210407	407	17	21240407	407	1824270407								
1927300407		7	2030340407	407	21	3	60710	22	6	90710	23	9120710	2412150710							
2515180710		8	2618210710	10	2721240710	10	2824270710	10	29	27300710	10	3030340710								
3103061013		9	3206091013	13	3303121013	13	3412151013	13	35	151013	13	3618211013								
3721241013		10	3824271013	13	3927301013	13	4030341013	13	41	03061316	16	4206091316								
4300121316		11	4412151316	16	4515131316	16	4618211316	16	47	21241316	16	4824271316								
4927301316		12	5030341316	16	5103061619	19	5206091619	19	53	09121619	19	5412151619								
5515181619		13	5618211619	19	5721241619	19	5824271619	19	59	27301619	19	6030341619								
6103061922		14	6206091922	22	6303121922	22	6412151922	22	65	15181922	22	6618211922								
6721241922		15	6824271922	22	6927301922	22	7030341922	22	71	03062225	25	7206092225								
7309122225		16	7412152225	25	7515182225	25	7618212225	25	77	21242225	25	7824272225								
7927302225		17	8030342225	25	8103062528	28	8206092528	28	83	09122528	28	8412152528								
8515182528		18	8618212528	28	8721242528	28	8824272528	28	89	27302528	28	9030342528								
91	1	3	92	1342	830															
1	7	1	1	1	1	90	10	9	2	1										
814		1001	1000	30	185															
1515		1	1	1	1															
*1212		10																		
15																				
1	4	7	-1	0	2	5	8	-1	0	3	6	9	-1	-1						
91	1	1.1540	0	0.011524	0.0022325															
91	2	0.6355		0.012823	0.0000625															
92	1	1.2841																		
92	2	0.8413																		
1	21																			
1	0	0	0	0	0	0	1	1	1											
0		+00	0	+00																
1282		+00	1128	-02	3	-03	0	+00	0	+00	0	+00								
7268		-01	0	+00	528	-03	819	-03	333	-03										
6		-02	1091	+02																
15		-01	2004	+02	5405	-03	8211	-03	3328	-03										
28		-01	3511	+02	545	-03	822	-03	3325	-03										
46		-01	5302	+02	55	-03	8235	-03	332	-03										
					5545	-03	8252	-03	3317	-03										

Sample problem

APPENDIX C

55	-01	6321	+02						
82	-01	9062	+02	5561	-03	8261	-03	3314	-03
145	+00	1531	+03	5605	-03	8276	-03	3304	-03
202	+00	2039	+03	5603	-03	8299	-03	3283	-03
249	+00	2552	+03	5749	-03	8287	-03	3265	-03
303	+00	3074	+03	5705	-03	8272	-03	3237	-03
348	+00	3521	+03	5846	-03	8246	-03	3206	-03
396	+00	4002	+03	5882	-03	8219	-03	3184	-03
455	+00	4565	+03	5920	-03	8190	-03	3160	-03
501	+00	5015	+03	5954	-03	8145	-03	3126	-03
556	+00	5569	+03	5981	-03	8109	-03	3099	-03
600	+00	6019	+03	6014	-03	8059	-03	3066	-03
652	+00	6552	+03	6041	-03	8018	-03	3039	-03
696	+00	7009	+03	6068	-03	7970	-03	3004	-03
750	+00	7548	+03	6090	-03	7929	-03	2974	-03
786	+00	7913	+03	6117	-03	7881	-03	2939	-03
0	+00			6136	-03	7848	-03	2916	-03
1 30	1	1 0.0							



```

//ERUPT   JOB ('68.0123.37171960130',12,007,005,090,0000,5,0,0,1), CJOB 55
//        '023-BJETTICHER',
//        MSGLEVEL=1
//JOBLIB DD DSNNAME=SYS1.CJPLIB,DISP=SHR
//EXEC   EXEC FTLG
//FTLG   PRCC XR=MAP,XC=LET,OV=BC TR,PR=,VL3=2,CLU=TRK,VU=0,DSL=NEW, C00032000
//        STEL=,SL=4,SD=00 C00032010
//LKED   EXEC PGM=IEHL,PARM='*XR,LIST,+XC,+OV' C00032020
//SYSPRINT DD SYSOUT=A C00032030
//SYSLIB DD DSNNAME=SYS1.FORTLIB,DISP=SHR C00032040
//        DD DSNNAME=SYS1.LIB+PRN,UNIT=SYSSQ,DISP=SHR, C00032050
//        VOLUME=SER=EURSY+VL3 C00032060
//        DD DSNNAME=SYS1.SSPLIB,DISP=SHR C00032070
//SYJUT1 DD JUT=SYSSQ,SPACE=(+CLU,(16,4)),VOLUME=SER=EURSY+VU C00032080
//SYSLMOD DD DSNNAME=++SSSET(+SD),DISP=(+DGL,PASS,DELETE),UNIT=SYSSQ, C00032090
//        SPACE=(TRK,(4),30,1),RLSE),VOLUME=SER=EURSY3 C00032100
//SYSLIN DD DSNNAME=SYSIN C00032110
//LKED.SYSIN DD *
IEF236I ALLOC. FJR ERUPT LKED
IEF237I JOBLIB ON 100
IEF237I SYSLIB ON 100
IEF237I ON 201
IEF237I ON 231
IEF237I SYSJUT1 ON 100
IEF237I SYSLMOD ON 232
IEF237I SYSLIN ON 300

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IEF235I SYS1.CMPLIB KEPT
IEF235I VOL SER NOS= EURGYO.
IEF235I SYSJUT SYJUT
IEF235I VOL SER NOS= .
IEF235I SYS1.FORTLIB KEPT
IEF235I VOL SER NOS= EURGYO.
IEF235I SYS1.LIB KEPT
IEF235I VOL SER NOS= EURGY2.
IEF235I SYS1.SSPLIB KEPT
IEF235I VOL SER NOS= EURGY2.
IEF235I SYS03275.T034007.RP001.ERUPT.R0000440 DELETED
IEF235I VOL SER NOS= EURGYO.
IEF235I SYS03275.T034007.RP001.ERUPT.G0SET PASSED
IEF235I VOL SER NOS= EURGYO.
//GO LKED DATE=33.10.01 BEG.T.=10.333 DURATION=0.003 N.OPER=463
//GO EXEC PGM=*.+STEL.LKED.SYSLMOD,CO.ID=(+GL,LT,+STEL.LKED) C0032120
//FT05F001 DD DSN=SYSIN C0032130
//FT06F001 DD SYSJUT=A C0032140
//FT07F001 DD UNIT=SYSOP,LABEL=(,NL),DISP=(MOD,PASS),DSNAME=PUNCH1, C00032150
// VOLUME=SER=PUNCH1 C0032160
//SYSUDUMP DD SYSOUT=A C00032170
//GO.FT09F001 DD UNIT=L91,VOLUME=(PRIVATE,SER=TP0458), C
// LABEL=(1,SL),DSNAME=ERUPT, C
// DISP=(OLD,DELETE)
//GO.SYSIN DD *
IEF236I ALLOC. FOR ERUPT GO
IEF237I JOBLIB ON 100
IEF237I PGM=* DD CN 232
IEF237I FT05F001 ON 000
IEF237I FT07F001 ON 000
IEF237I FT09F001 ON 332

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\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REACTOR SPECIFICATIONS

34 ROWS 30 COLUMNS 92 REGIONS 92 COMPS 2 GROUPS RZ GEOM B.C.(L,T,R,B)=1,0,0,1 N.F.= 3.5000E 02  
CHI1= 1.0000 CHI2= 0.0

MESH SPECIFICATIONS

I DELTA  
1 18.500 3 6.000 33 0.000 34  
J DELTA  
1 44.444 2 22.222 4 9.205 7 7.063 10 5.954 13 5.246 16 4.743 19 4.361 22 4.059 25 3.813 28  
26.000 30

DIMENSION SPECIFICATIONS

I	DIST.	3	4	5	6	7	8	9	10
1	18.500	37.000	43.000	49.000	55.000	61.000	67.000	73.000	79.000
11	85.000	91.000	97.000	103.000	109.000	115.000	121.000	127.000	133.000
20	139.000	145.000	151.000	157.000	163.000	169.000	175.000	181.000	187.000
29	193.000	199.000	205.000	211.000	217.000	217.000			

J	DIST.	3	4	5	6	7	8	9	10
2	22.222	44.444	66.666	75.371	85.076	94.281	101.344	108.407	115.470
11	121.424	127.373	133.333	138.579	143.825	149.071	153.913	158.556	163.299
20	167.666	172.021	176.383	180.442	184.501	188.561	192.374	196.187	200.000
29	226.000	252.000							

ADDITIONAL DATA

NRT	RUN	LD1	LD2	LD3	CIRC	TYPE	KEF1	KEF2
1.0000E 00	1.0000E 00	9.0000E 01	1.0000E 01	9.0000E 00	2.0000E 00	1.0000E 00	1.0010E 00	1.0000E 00
DT	P/F	KEJ	PRCC	PRTF	PRSP			
3.0000E 01	1.8500E 02	1.0000E 01	1.0000E 00	1.0000E 00	1.0000E 00			

MOVE-SEQUENCES FOR RADIAL FUEL MANAGEMENT

1 4 7 -1 0 2 5 3 -1 0 3 6 9 -1 -1



TAU	BU	D	**TABLE SIGR 1**	SIGA	NUSF	SIGF
0.0	0.0	1.2820E 00 7.2630E-01	1.1280E-02 0.0	3.0000E-03 5.2800E-03	0.0 8.1900E-03	0.0 3.3300E-03
6.0000E-02	1.0910E 02	0.0 0.0	0.0 0.0	0.0 5.4050E-03	0.0 8.2110E-03	0.0 3.3280E-03
1.5000E-01	2.0040E 02	0.0 0.0	0.0 0.0	0.0 5.4500E-03	0.0 8.2200E-03	0.0 3.3250E-03
2.8000E-01	3.5110E 02	0.0 0.0	0.0 0.0	0.0 5.5000E-03	0.0 8.2350E-03	0.0 3.3200E-03
4.6000E-01	5.3020E 02	0.0 0.0	0.0 0.0	0.0 5.5450E-03	0.0 8.2520E-03	0.0 3.3170E-03
5.5000E-01	6.3210E 02	0.0 0.0	0.0 0.0	0.0 5.5610E-03	0.0 8.2610E-03	0.0 3.3140E-03
8.2000E-01	9.0620E 02	0.0 0.0	0.0 0.0	0.0 5.6050E-03	0.0 8.2760E-03	0.0 3.3040E-03
1.4500E 00	1.5310E 03	0.0 0.0	0.0 0.0	0.0 5.6930E-03	0.0 8.2990E-03	0.0 3.2830E-03
2.0200E 00	2.0890E 03	0.0 0.0	0.0 0.0	0.0 5.7490E-03	0.0 8.2870E-03	0.0 3.2650E-03
2.4900E 00	2.5520E 03	0.0 0.0	0.0 0.0	0.0 5.7950E-03	0.0 8.2720E-03	0.0 3.2370E-03
3.0300E 00	3.0740E 03	0.0 0.0	0.0 0.0	0.0 5.8460E-03	0.0 8.2460E-03	0.0 3.2060E-03
3.4800E 00	3.5210E 03	0.0 0.0	0.0 0.0	0.0 5.8320E-03	0.0 8.2190E-03	0.0 3.1840E-03
3.9600E 00	4.0020E 03	0.0 0.0	0.0 0.0	0.0 5.9200E-03	0.0 8.1900E-03	0.0 3.1600E-03
4.5500E 00	4.5650E 03	0.0 0.0	0.0 0.0	0.0 5.9540E-03	0.0 8.1450E-03	0.0 3.1260E-03
5.0100E 00	5.0150E 03	0.0 0.0	0.0 0.0	0.0 5.9310E-03	0.0 8.1090E-03	0.0 3.0990E-03
5.5600E 00	5.5690E 03	0.0 0.0	0.0 0.0	0.0 6.0140E-03	0.0 8.0590E-03	0.0 3.0660E-03
6.0000E 00	6.0190E 03	0.0 0.0	0.0 0.0	0.0 6.0410E-03	0.0 8.0180E-03	0.0 3.0390E-03
6.5200E 00	6.5520E 03	0.0 0.0	0.0 0.0	0.0 6.0680E-03	0.0 7.9700E-03	0.0 3.0040E-03
6.9600E 00	7.0090E 03	0.0 0.0	0.0 0.0	0.0 6.0900E-03	0.0 7.9290E-03	0.0 2.9740E-03

TAU	3U	D	**TABLE SIGR	1**	SIGA	NUSF	SIGF
7.5000E 00	7.5480E 03	0.0 0.0	0.0 0.0		0.0 6.1170E-03	0.0 7.8810E-03	0.0 2.9390E-03
7.8600E 00	7.9130E 03	0.0 0.0	0.0 0.0		0.0 6.1360E-03	0.0 7.8480E-03	0.0 2.9160E-03

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GROUP	D	SIGR	SIGA	NUSF	S SQ
1	3	6	1	4	1	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
2	6	9	1	4	2	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
3	9	12	1	4	3	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
4	12	15	1	4	4	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
5	15	18	1	4	5	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
6	18	21	1	4	6	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
7	21	24	1	4	7	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
8	24	27	1	4	8	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
9	27	30	1	4	9	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
10	30	33	1	4	10	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
11	3	6	4	7	11	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
12	6	9	4	7	12	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
13	9	12	4	7	13	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
14	12	15	4	7	14	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
15	15	18	4	7	15	1 2	1.232E-00 7.263E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GROUP	D	SIGR	SIGA	NUSF	B	SQ
15	13	21	4	7	15	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
17	21	24	4	7	17	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
18	24	27	4	7	18	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
19	27	30	4	7	19	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
20	30	34	4	7	20	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
21	3	5	7	10	21	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
22	5	9	7	10	22	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
23	9	12	7	10	23	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
24	12	15	7	10	24	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
25	15	13	7	10	25	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
26	13	21	7	10	26	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
27	21	24	7	10	27	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
28	24	27	7	10	28	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
29	27	30	7	10	29	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	
30	30	34	7	10	30	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.253E-01	0.0	5.280E-03	8.190E-03	0.0	

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GRJUP	D	SIGR	SIGA	NUSF	B	SQ
31	3	5	10	13	31	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
32	6	9	10	13	32	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
33	9	12	10	13	33	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
34	12	15	10	13	34	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
35	15	18	10	13	35	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
36	13	21	10	13	36	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
37	21	24	10	13	37	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
38	24	27	10	13	38	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
39	27	30	10	13	39	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
40	30	34	10	13	40	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
41	3	6	13	16	41	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
42	6	9	13	16	42	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
43	9	12	13	16	43	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
44	12	15	13	16	44	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	
45	15	18	13	16	45	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0	

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GROUP	D	SIGR	SIGA	NUSF	SQ
46	13	21	13	13	46	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
47	21	24	13	13	47	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
48	24	27	13	13	48	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
49	27	30	13	16	49	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
50	30	34	13	13	50	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
51	3	6	16	19	51	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
52	6	9	16	19	52	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
53	9	12	16	19	53	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
54	12	15	16	19	54	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
55	15	18	16	19	55	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
56	18	21	16	19	56	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
57	21	24	16	19	57	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
58	24	27	16	19	58	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
59	27	30	16	19	59	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0
60	30	34	16	19	60	1	1.232E-00	1.128E-02	3.000E-03	0.0	0.0
						2	7.268E-01	0.0	5.280E-03	8.190E-03	0.0



\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	CJMP	GROUP	D	SIGR	SIGA	NUSF	B	SQ
61	3	6	19	22	61	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
62	6	9	19	22	62	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
63	9	12	19	22	63	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
64	12	15	19	22	64	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
65	15	18	19	22	65	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
66	18	21	19	22	66	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
67	21	24	19	22	67	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
68	24	27	19	22	68	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
69	27	30	19	22	69	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
70	30	34	19	22	70	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
71	3	6	22	25	71	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
72	6	9	22	25	72	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
73	9	12	22	25	73	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
74	12	15	22	25	74	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	
75	15	18	22	25	75	1	1.232E 00	1.128E-02	3.000E-03	0.0	0.0	
						2	7.263E-01	0.0	5.280E-03	8.190E-03	0.0	

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GRJUP	D	SIGR	SIGA	NUSF	B SQ
76	13	21	22	25	76	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
77	21	24	22	25	77	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
78	24	27	22	25	78	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
79	27	30	22	25	79	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
80	30	34	22	25	80	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
81	3	6	25	23	31	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
82	6	9	25	23	32	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
83	9	12	25	23	33	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
84	12	15	25	23	34	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
85	15	18	25	23	35	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
86	13	21	25	23	36	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
87	21	24	25	23	37	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
88	24	27	25	23	38	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
89	27	30	25	23	39	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0
90	30	34	25	23	90	1 2	1.232E-00 7.268E-01	1.128E-02 0.0	3.000E-03 5.280E-03	0.0 8.190E-03	0.0 0.0

\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

REGION SPECIFICATIONS

REG	I1	I2	J1	J2	COMP	GROUP	D	SIGR	SIGA	NUSF	B SQ
91	1	3	1	23	91	1	1.134E 00	1.152E-02	0.0	0.0	0.0
						2	6.355E-01	0.0	2.232E-03	0.0	0.0
92	1	34	28	30	92	1	1.284E 00	1.282E-02	0.0	0.0	0.0
						2	8.413E-01	0.0	6.250E-05	0.0	0.0



\*\*\* SAMPLE PROBLEM ERUPT \*\*\*

FLUX CALCULATION BEGINS FOR REACTOR STATE AT LIFE= 0.0 \* BETA= 1.8648E 00 \*

IT NO	FLUX CONVR	NU-CRIT CONVR	TOT RESIDUE	MAX RESIDUE	NU-CRITICAL
10	4.6555E-01	1.0000E-00	2.2271E-02	1.0903E-02	8.68663E-01
20	4.3368E-01	2.2432E-02	9.5933E-03	5.3382E-03	8.49579E-01
30	3.7153E-01	5.4474E-03	3.3762E-03	2.1232E-03	8.44976E-01
40	1.5369E-01	1.5068E-03	1.1805E-03	2.6473E-04	8.43704E-01
50	1.1089E-01	2.0027E-05	4.5992E-04	8.3443E-05	8.43687E-01
60	7.8807E-02	4.0656E-04	1.3812E-04	5.6340E-05	8.44030E-01
70	2.0490E-02	5.3055E-05	3.8651E-05	1.0472E-05	8.44079E-01
80	2.6665E-03	1.3351E-05	1.2634E-05	1.8732E-06	8.44068E-01
90	5.8895E-04	9.5367E-06	4.5956E-06	4.8721E-07	8.44059E-01

REACTIVITY KEFF = 1.1848E 00  
FORMFACTOR FAST FLUX = 4.3030E-01  
FORMFACTOR THERMAL FLUX= 4.4516E-01  
FORMFACTOR SPEC. POWER = 4.4516E-01  
TOTAL POWER (MW) = 3.5000E 02

\*THERMAL CELL FLUXES\*

RING 1		RING 2		RING 3		RING 4		RING 5		RING 6		RING 7		RING 8		RING 9	
8.4325E	13	7.6233E	13	6.8371E	13	6.0901E	13	5.3826E	13	4.7168E	13	4.1030E	13	3.5895E	13	3.3765E	13
1.2554E	14	1.1349E	14	1.0177E	14	9.0637E	13	8.0086E	13	7.0134E	13	6.0920E	13	5.3167E	13	5.0057E	13
1.6782E	14	1.5171E	14	1.3503E	14	1.2113E	14	1.0700E	14	9.3659E	13	8.1278E	13	7.0814E	13	6.6506E	13
2.0713E	14	1.8723E	14	1.6787E	14	1.4947E	14	1.3201E	14	1.1550E	14	1.0019E	14	8.7221E	13	8.1831E	13
2.4240E	14	2.1909E	14	1.9643E	14	1.7489E	14	1.5443E	14	1.3510E	14	1.1715E	14	1.0196E	14	9.5612E	13
2.7290E	14	2.4366E	14	2.2114E	14	1.9686E	14	1.7383E	14	1.5205E	14	1.3183E	14	1.1471E	14	1.0755E	14
2.9805E	14	2.6939E	14	2.4151E	14	2.1499E	14	1.8982E	14	1.6603E	14	1.4394E	14	1.2524E	14	1.1741E	14
3.1736E	14	2.8683E	14	2.5714E	14	2.2891E	14	2.0210E	14	1.7677E	14	1.5324E	14	1.3332E	14	1.2498E	14
3.3044E	14	2.9366E	14	2.6774E	14	2.3834E	14	2.1042E	14	1.8404E	14	1.5954E	14	1.3880E	14	1.3012E	14
3.3704E	14	3.0462E	14	2.7309E	14	2.4310E	14	2.1462E	14	1.8771E	14	1.6272E	14	1.4156E	14	1.3271E	14



\*SPECIFIC POWERS PER REGION\*

RING 1	RING 2	RING 3	RING 4	RING 5	RING 6	RING 7	RING 8	RING 9
1.3479E 00	1.2136E 00	1.0023E 00	0.7344E-01	9.6036E-01	7.5393E-01	6.5582E-01	5.7374E-01	5.3970E-01
2.0065E 00	1.8140E 00	1.6267E 00	1.4487E 00	1.2801E 00	1.1210E 00	9.7375E-01	8.4981E-01	8.0012E-01
2.6325E 00	2.4249E 00	2.1743E 00	1.9362E 00	1.7103E 00	1.4970E 00	1.2991E 00	1.1319E 00	1.0630E 00
3.3108E 00	2.9926E 00	2.6332E 00	2.3890E 00	2.1100E 00	1.9462E 00	1.6014E 00	1.3941E 00	1.3080E 00
3.8744E 00	3.5020E 00	3.1397E 00	2.7953E 00	2.4684E 00	2.1594E 00	1.8725E 00	1.6296E 00	1.5283E 00
4.3621E 00	3.9426E 00	3.5343E 00	3.1467E 00	2.7785E 00	2.4304E 00	2.1072E 00	1.8335E 00	1.7192E 00
4.7641E 00	4.3359E 00	3.8602E 00	3.4364E 00	3.0341E 00	2.6539E 00	2.3007E 00	2.0018E 00	1.8767E 00
5.0726E 00	4.5347E 00	4.1102E 00	3.6588E 00	3.2304E 00	2.8254E 00	2.4494E 00	2.1310E 00	1.9977E 00
5.2317E 00	4.7737E 00	4.2736E 00	3.8096E 00	3.3634E 00	2.9417E 00	2.5501E 00	2.2185E 00	2.0798E 00
5.3372E 00	4.8691E 00	4.3551E 00	3.8856E 00	3.4306E 00	3.0004E 00	2.6009E 00	2.2628E 00	2.1212E 00

\*XEQ TIME= 0.60 MINUTES\*





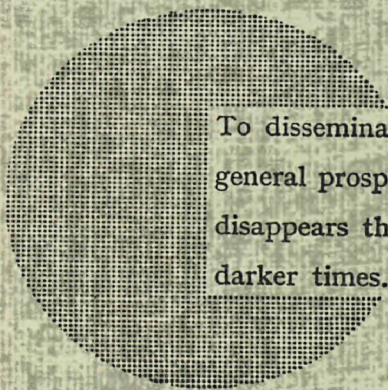
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**Alfred Nobel**



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