

EUR 2985.e

EUROPEAN ATOMIC ENERGY COMMUNITY - EURATOM

**RDMM - A CODE FOR FAST
NEUTRON SPECTRA DETERMINATION
BY ACTIVATION ANALYSIS**

by

G. DI COLA and A. ROTA

1966



Joint Nuclear Research Center
Ispra Establishment - Italy

Scientific Information Processing Center - CETIS
and
Chemistry Department
Nuclear Chemistry

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Printed by Guyot, s.a.
Brussels, June 1966

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RDMM (Relative Deviation Minimization Method) is the name of the FORTRAN Code here described. It allows the automatic interpretation of the activation data for the in-pile fast neutron spectra determination.

1. The first part of the document discusses the importance of maintaining accurate records of all transactions. It emphasizes that this is crucial for ensuring the integrity of the financial statements and for providing a clear audit trail.

2. The second part of the document outlines the various methods used to collect and analyze data. It describes how different types of information are gathered and how they are processed to identify trends and anomalies.

3. The third part of the document focuses on the results of the analysis. It presents the findings in a clear and concise manner, highlighting the key areas of concern and the potential implications for the organization.

4. The final part of the document provides recommendations for improving the system. It offers practical advice on how to address the identified issues and how to prevent similar problems from occurring in the future.

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SUMMARY

RDMM (Relative Deviation Minimization Method) is the name of the FORTRAN Code here described. It allows the automatic interpretation of the activation data for the in-pile fast neutron spectra determination.

1. STATEMENT OF THE PROBLEM

Measurements of in-pile fast neutron spectra are often obtained by threshold detector irradiations. The threshold detector technique give data usually expressed as normalized activation rates, that is:

$$A_i = \int \sigma_i(E) \varphi(E) dE \quad (1)$$

where A_i indicates the reaction probability per second for a nucleus of the i 'th isotope, having $\sigma_i(E)$ as differential cross-section, when immersed in the unknown neutron flux $\varphi(E)$ $n/(cm^2 sec MeV)$.

The code, here described in the following, allows the $\varphi(E)$ determination, on the hypothesis that the neutron flux shape can be approximated by an expression of the type:

$$\varphi(E) \approx \varphi^t(E) = W(E) \sum_{i=1}^t a_k \Psi_k(E) \quad (2)$$

where:

n = number of the threshold detectors used in the experiment of activation.

$W(E)$ = weighting function, chosen in agreement with the kind of the treated problem.

$\Psi_k(E)$ = complete set of arbitrary functions.

a_k = coefficients of the expansion, that will be determined by the calculation.

t = cut-off number.

Experimental and theoretical support of the proposed code can be found in detail in a paper by Di Cola and Rota (1).

(1) G. DI COLA and A. ROTA - Nucl. Sci. and Eng. 23, 344, (1965)

2. OUTLINE OF THE CODE

The Relative Deviation Minimization Method (R D M M) suggests to choose, as solution of the problem, the flux that minimize the quadratic form

$$Q(t, a_1, \dots, a_t) = \sum_1^n \left[\frac{A_i - \int \sigma_i(E) \varphi^t(E) dE}{A_i} \right]^2$$

with the condition $t \leq n$.

A second feature of the code is the determination of the standard deviation on the resulting flux, as function of the standard deviation on the activation rates and on the data of cross-sections. A limitation is that the standard deviation on the cross-sections can be expressed only in the form:

$$\sigma_i(E) \pm \alpha_i \sigma_i(E)$$

with $\alpha_i = \text{const.}$

Care should be taken to read correctly the output data in order to choose, as solution of the problem under study, the best approximation.

3. SOME REMARKS ON THE USE OF THE CODE

A correct running of the code requires the knowledge of the limitations described in the following:

3.1 Sets of the Ψ functions

The $\Psi(E)$ functions must belong to a class that can be described by the general formula:

$$\Psi_k(E) = \sum_1^k b_{j-1} E^{j-1}$$

This class includes, for instance, the Laguerre polynomials, the Chebyshev polynomials, the Legendre polynomials,

and so on. The use of trigonometric functions is not allowed without a suitable change in a part of the code.

The functions ψ must belong to a set of complete functions and it is better if they are normalized. This last condition is not strictly necessary.

3.2 Activation rates and scale factors

According to their definition, the activation rates are usually very small numbers. (e.g. $A_{Np} = 4 \cdot 10^{-13}$ for a fast flux, above 0.5 MeV, of $5 \cdot 10^{10}$ n/(cm² sec))

Numerical reasons require that these numbers are in the range (order of magnitude) $10^{-3} + 10^3$. It is then necessary to work with the A_i reduced by a suitable scale factor, that must be recalled when the results are read.

The cross-sections are given in mb and it follows that the flux is actually expressed in n/(10⁻²⁷ cm² sec MeV). If a scale factor of 10^{+c} is used for the activation rates, the scale factor 10^{27-c} must be applied to the results in order to obtain differential flux values in the usual unity, n/(cm² sec MeV), and integral flux values in n/(cm² sec).

4. DESCRIPTION OF THE FORTRAN PROGRAM

4.1 General information

RDMM has been written in FORTRAN IV or MAP language. It consists of a MAIN program and 10 subroutines.

Data transfer to and from subroutines is obtained both through COMMON and explicite arguments, as the case requires.

4.2 Subroutines

1 INPUT reads the input data and transforms them in the

way required for the processing. It writes the data of the problem.

- 2 PESI defines the $W(E)$ in the form e^{-E} . If the use of a different weighting function is required, this subroutine must be suitably changed.
- 3 MTRIX calculates the elements of the matrices S and $R^T R$ defined in formulas (3) and (11) of reference (1).
- 4 ABC performs an approximate evaluation of an integral of the form
$$\int_a^b f(x) dx$$
by the n -point Newton-Cotes integration formulas, with n variable from 2 to 7.
- 5 SIMH calculates the inverse of a given matrix.
- 6 FLXCAL calculates the differential and integral neutron fluxes as in formula (5) or reference (1).
- 7 RNG performs a transformation of random numbers to obtain statistically independent parameters.
- 8 CAS computes a random number by using a congruential method.
- 9 AVERAG calculates the mean and the standard deviation of a given frequency distribution.
- 10 CØE reads, normalizes and prints the coefficients b_j (see 3.1). This routine now refers to Laguerre polynomials.

4.3 Input description

The following table contains the list of the input cards.

Data Cards.

Order of cards	Names	Description	Format
1	RCRD1	Titles	12A6
2	RCRD2	Titles	12A6
3	NS, NP, NPP	NS = number of detectors: n NP = number of points in which the $\sigma(E)$ are tabulated NPP= step of the print-out results.	1016
4	MIN, MAX	MIN = minimum t MAX = maximum t ($\leq n$)	1016
5	NHIS (I)	Number of Montecarlo hystories for each approximation order. If NHIS (I) = 0 no error evaluation is performed for t=I	1016
6	E(1), H	E(1)= the first energy value in which the $\sigma_i(E)$ are tabulated (MeV). H = step of the energy tabulation (MeV).	3E10.6
7	NS blocks of data cards, for I = 1, NS as follows		
	SIGNME(I)	Name of detector (6 characters) Cross sections of the detector I expressed in mbarn for J=1, NP.	12A6 5F10.3
8	NS data cards, for I = 1, NS, as follows:		
	A(I), SA(I), SV(I)	A= Activation rate SA= Relative standard error on the A SV= Relative standard error on the SIGMA.	3E10.6
9	MAX cards, for I = 1, MAX, as fdlows:		
	CF(J, I)	CF = coefficients of the polynomial forms (J=1, I)	

4.4 Output description

The first page contains:

- The title of the problem.
- The coefficients b_i of the functions ψ_k (see 3.1)
- The activation rates, its relative standard deviations and the cross-section relative standard deviations.
- The "cut-off" numbers of the series expansion and the number of the Monte-Carlo histories required.

In the pages that follow the results of the computation are listed. For each cut-off number one or two pages are printed. The first one contains:

- The list of the numbers:

$$\frac{\int \sigma_i \varphi^k dE}{A_i} \quad \begin{array}{l} i = 1, \dots, n \\ k = \text{cut-off number} \end{array}$$

These numbers tend to 1 as φ^k tends to φ .

- The value of the quadratic form Q (see form (1))
- The value of $\Phi = \int_{0.5} \varphi^k(E) dE$
- The table of the actual values of $\varphi^k(E)$

A second page is printed only if a statistical analysis on the results is required. This page contains all the information concerning this analysis on integral and differential neutron fluxes.

```

DIMENSION V(10)
DIMENSION AV(141),SD(141)
DIMENSION A(10),E(141),NHIS(10),SA(10),SIGNME(10),SV(10),TN(10),VV
1(10),W(141)
DIMENSION CA(10,141),CF(10,10),S(10,10),SIGMA(10,141),T(10,10),TI(
10,10)
COMMON MIN,MAX,NG,NS,NP,NPP,H
COMMON A,E,NHIS,SA,SIGME,SV,TN,VV,W
COMMON CA,CF,S,SIGMA,T,TI
DIMENSION FI(141),FXT(150),FLX(150,30)
100 FORMAT (1H1,30X,26HAPPROXIMATION NUMBER ***,12,4H ***,////,1H ,1
10X,5HCHECK,/)
101 FORMAT (14X,3HQ =,E10.3,33H - INTEGRAL FLUX ABOVE .5 MEV =,E10.3
1,/,1H ,4X,1HE,7X,2HFI,8X,1HE,7X,2HFI,8X,1HE,7X,2HFI,8X,1HE,7X,2H
21,8X,1HE,7X,2HFI,/)
102 FORMAT ((1H ,5(F7.2,E11.3)))
103 FORMAT (1H1 ,28X,22HSTATISTICAL ANALYSIS (,13,11H HISTORIES),/)
104 FORMAT (1H ,8H INT.FLX,E10.3,7H MEAN,E10.3,4H +- ,E10.3,6H %/0
1,F6.2,4H (,E10.3,2H -,E10.3,1H))
105 FORMAT (//,1H ,64H E / NEUTRON FLUX STANDARD DEVIAT.
1 68 PER CENT,/,67H (MEV) CALCUL. MONTECARLO ABSOLUTE R
2ELAT. CONFIDENCE INT.,/)
106 FORMAT (1H ,F6.2,3E11.3,F8.3,2E11.3)
107 FORMAT (1H0,10X,A6,F8.3)
108 FORMAT (///)

```

MAIN

```

CALL INPUT
KP=NP/NPP+1
CALL MTRIX
DO 1 K=1,KP
L=K*NPP-(NPP-1)
E(K)=E(L)
1 CONTINUE
DO 12 NG=MIN,MAX
WRITE (6,100) NG
CALL SIMH(T,TI,NG,1)
CALL FLXCAL(FI,FLXINT,V)
Q=0
DO 3 K=1,NS
C=0
DO 2 L=1,NG
2 C=C+S(K,L)*V(L)
WRITE (6,107) SIGNME(K),C
3 Q=Q+(1.-C)**2
WRITE (6,108)
WRITE (6,101) Q,FLXINT
WRITE (6,102) (E(I),FI(I),I=1,KP)
NHS=NHIS(NG)
IF(NHS)4,4,5
4 GO TO 12
5 CONTINUE
DO 8 NN=1,NHS
CALL RNG(A,SA,NS)
CALL RNG(VV,SV,NS)
DO 6 I=1,NS

```

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RDMM	MAIN	EXTERNAL FORMULA NUMBER	SOURCE STATEMENT	INTERNAL FORMULA NUMBER(S)
			A(I)=A(I)/VV(I)	,44
6			CONTINUE	,45 ,46
			CALL FLXCAL(W,WW,V)	,47
			DO 7 K=1,KP	,48
			FLX(NN,K)=W(K)	,49
7			CONTINUE	,50 ,51
			FXT(NN)=WW	,52
8			CONTINUE	,53 ,54
			DO 9 K=1,KP	,55
			CALL AVERAG(FLX(I,K),NHS,AV(K),SD(K))	,56
9			CONTINUE	,57 ,58
			CALL AVERAG(FXT,NHS,TFM,SDIF)	,59
			WRITE (6,103) NHS	,60 ,61 ,62
			S1=SDIF/TFM	,63
			S2=TFM-SDIF	,64
			S3=TFM+SDIF	,65
			WRITE (6,104) FLXINT,TFM,SDIF,S1,S2,S3	,66 ,67 ,68
			WRITE (6,105)	,69 ,70
			DO 10 K=1,KP	,71
			S1=SD(K)/AV(K)	,72
			S2=AV(K)-SD(K)	,73
			S3=AV(K)+SD(K)	,74
			WRITE (6,106) E(K),FI(K),AV(K),SD(K),S1,S2,S3	,75 ,76 ,77
10			CONTINUE	,78 ,79
			DO 11 I=1,NS	,80
			A(I)=1.	,81
11			CONTINUE	,82 ,83
12			CONTINUE	,84 ,85
			STOP	,86
			END	,87

SUBROUTINE INPUT

```

DIMENSION A(10),E(141),NHIS(10),SA(10),SIGNME(10),SV(10),TN(10),VV
1(10),W(141)
DIMENSION CA(10,141),CF(10,10),S(10,10),SIGMA(10,141),T(10,10),TI(
110,10)
COMMON MIN,MAX,NG,NS,NP,NPP,H
COMMON A,E,NHIS,SA,SIGNME,SV,TN,VV,W
COMMON CA,CF,S,SIGMA,T,TI
DIMENSION RCRD1(12),RCRD2(12)
1 FORMAT (12A6)
2 FORMAT (1H1,20X,12A6,////,1H ,20X,12A6,////)
3 FORMAT (10I6)
4 FORMAT (3E10.6)
5 FORMAT (5F10.3)
7 FORMAT (1H ///
1 1H ,6X,4HDATA//1H0,13X,41HDETECTOR ACTIV.RATE 0/0 A
10/0 SIGMA)
8 FORMAT (1H0,13X,A6,E13.4,F8.3,F11.3)
9 FORMAT (////,1H )
10 FORMAT (10X,10HPOL. ORDER,13,15H NUMB. OF HIS.,14)

READ (5,1) RCRD1,RCRD2
WRITE (6,2) RCRD1,RCRD2
READ (5,3) NS,NP,NPP
READ (5,3) MIN,MAX
READ (5,3) (NHIS(I),I=MIN,MAX)
READ (5,4) E(1),H
DO 11 K=2, NP
11 E(K)=E(K-1)+H
DO 12 I=1, NS
READ (5,1) SIGNME(I)
12 READ (5,5) (SIGMA(I,J),J=1, NP)

READ (5,4) ((A(I),SA(I),SV(I)),I=1,NS)
DO 13 I=1, NS
DO 13 J=1, NP
13 SIGMA(I,J)=SIGMA(I,J)/A(I)
CALL COE(MAX,CF)
CALL PESI(W,E, NP)
WRITE (6,7)
WRITE (6,8) ((SIGNME(I),A(I),SA(I),SV(I)),I=1,NS)
WRITE (6,9)
WRITE (6,10) (K,NHIS(K),K=MIN,MAX)
DO 116 I=1, NS
116 A(I)=1.
RETURN
END
    
```

INPUT

,1	,2	,3			
,4	,5	,6			
,7	,8	,9	,10	,11	
,12	,13	,14	,15	,16	
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,63					
,64	,65				
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,67					

RDMM PESI 08/15/65
EXTERNAL FORMULA NUMBER - SOURCE STATEMENT - INTERNAL FORMULA NUMBER(S)

C SUBROUTINE PESI(W,E,NP)
C OPTIONAL SUBROUTINE
C TO BE MODIFIED ACCORDING TO THE NECESSITIES
DIMENSION W(1),E(1)
DO 1 K=1,NP
W(K)=EXP(-E(K))
1 CONTINUE
RETURN
END

,1
,2
,3 ,4
,5
,6


```

SUBROUTINE MTRIX
DIMENSION A(10),E(141),NHIS(10),SA(10),SIGNME(10),SV(10),TN(10),VV
1(10),W(141)
DIMENSION CA(10,141),CF(10,10),S(10,10),SIGMA(10,141),T(10,10),TI(
110,10)
COMMON MIN,MAX,NG,NS,NP,NPP,H
COMMON A,E,NHIS,SA,SIGNME,SV,TN,VV,W
COMMON CA,CF,S,SIGMA,T,TI
DIMENSION V(141)
    
```

MTRIX

```

NG=MAX
DO 2 I=1,NG
DO 2 J=1,NP
POL=0.
DO 1 K=1,I
K1=I-K+1
1 POL=POL*E(J)+CF(K1,I)
2 CA(I,J)=POL*W(J)

DO 5 K=1,NG
DO 5 I=1,NS
DO 3 J=1,NP
3 V(J)=SIGMA(I,J)*CA(K,J)
5 S(I,K)=ABC(NP,5,H,V)
DO 6 J=1,NG
DO 6 K=1,NG
T(J,K)=0
DO 6 L=1,NS
6 T(J,K)=T(J,K)+S(L,J)*S(L,K)
RETURN
END
    
```

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C

RDMM ABC 08/15/65
 EXTERNAL FORMULA NUMBER - SOURCE STATEMENT - INTERNAL FORMULA NUMBER(S)

FUNCTION ABC(NP,MP,H,Y)
 DIMENSION S(7,7),D(7),Y(300)

C	DATA((S(I,J),J=1,7),I=2,7)/1.,1.,0.,0.,0.,0.,0.,		
	11.,4.,1.,0.,0.,0.,0.,		
	21.,3.,3.,1.,0.,0.,0.,		
	37.,32.,12.,32.,7.,0.,0.,		
	419.,75.,50.,50.,75.,19.,0.,		
	541.,216.,27.,272.,27.,216.,41./		
	DATA(D(I),I=2,7)/.5,.33333333,.375,.04444444,.01736111,.00714286/		
	NI=NP-1	,1	
	MI=MP-1	,2	
	M=NI/MI	,3	
	K=NI-MI*M	,4	
C	Z=0	,5	
	MM=0	,6	
	DO 1 I=1,M	,7	
	W=0	,8	
	DO 2 J=1,MP	,9	
	JM=MM+J	,10	
2	W=W+S(MP,J)*Y(JM)	,11	,12
	Z=Z+W	,13	
1	MM=JM-1	,14	,15
	Z=Z*D(MP)		
C	IF(K)3,3,4	,16	
4	KP=K+1	,17	
	W=0	,18	
	DO 5 J=1,KP	,19	
	JM=MM+J	,20	
5	W=W+S(KP,J)*Y(JM)	,21	
	Z=Z+W*D(KP)	,22	,23
C	3 ABC=Z*H	,24	
	RETURN	,25	
	END	,26	
		,27	

RDMM FLXCAL 08/15/65
 EXTERNAL FORMULA NUMBER - SOURCE STATEMENT - INTERNAL FORMULA NUMBER(S)

```

SUBROUTINE FLXCAL(FI,FLXINT,V)
DIMENSION A(10),E(141),NHIS(10),SA(10),SIGNME(10),SV(10),TN(10),VV
1(10),W(141)
DIMENSION CA(10,141),CF(10,10),S(10,10),SIGMA(10,141),T(10,10),TI(
110,10)
COMMON MIN,MAX,NG,NS,NP,NPP,H
COMMON A,E,NHIS,SA,SIGNME,SV,TN,VV,W
COMMON CA,CF,S,SIGMA,T,TI
DIMENSION FI(141),V(10)
DO 1 J=1,NG
TN(J)=0.
DO 1 K=1,NS
1 TN(J)=TN(J)+S(K,J)*A(K)
DO 2 K=1,NG
V(K)=0.
DO 2 I=1,NG
2 V(K)=V(K)+TI(K,I)*TN(I)
DO 3 I=1,NP
FI(I)=0.
DO 3 L=1,NG
3 FI(I)=FI(I)+CA(L,I)*V(L)
FLXINT=ABC(NP-5,5,H,FI(6))
K=0
DO 4 I=1,NP,NPP
K=K+1
4 FI(K)=FI(I)
RETURN
END

```

FLXCAL

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```

C SUBROUTINE RNG(PAR,SPAR,N)
  RANDOM NUMBER GENERATION
  DIMENSION PAR(10),TAB(2,30),SPAR(10)
  DATA ((TAB(J,K),J=1,2),K=1,29)/-4.,0.,-2.33,.01,-2.06,.02,-1.89,.0
13,-1.76,.04,-1.65,.05,-1.29,.1,-1.04,.15,-.85,.2,-.68,.25,-.53,.3,
2-.39,.35,-.26,.4,-.13,.45,.0,.5,.13,.55,.26,.60,.39,.65,.53,.70,.6
38,.75,.85,.80,1.04,.85,1.29,.9,1.65,.95,1.76,.96,1.89,.97,2.06,.98
4,2.33,.99,.4,.1.7
  DO 3 K=1,N
    CALL CAS(RN)
    DO 1 I=1,29
      IF(TAB(2,I)-RN)1,1,2
1 CONTINUE
2 CONTINUE
  PAR(K)=TAB(1,I)-((TAB(2,I)-RN)/(TAB(2,I)-TAB(2,I-1))*(TAB(1,I)-TAB
1(1,I-1)))
  PAR(K)=1.+SPAR(K)*PAR(K)
3 CONTINUE
  RETURN
  END
    ,1
    ,2
    ,3
    ,4
    ,5 ,6
    ,7
    ,8
    ,9
    ,10 ,11
    ,12
    ,13
  
```

RDMM CAS
ASSEMBLED TEXT.

08/15/65

\$TEXT CAS

CAS 0001

				ENTRY	CAS
BINARY	CARD ID.	CAS	0002		
	00000	1 00000	0 00003	10001	CAS SAVE
	00001	0774 00	4 00000	10000	
	00002	0020 00	4 00001	10000	
	00003	0634 00	4 04000	10011	
	00004	0634 00	4 00020	10001	
	00005	0634 00	4 00000	10011	
		1 00000	7 00001	10001	
	00006	0560 00	0 00016	10001	LDQ U
	00007	0200 00	0 00017	10001	MPY X
	00010	4600 00	0 00016	10001	STQ U
	00011	0500 00	0 00022	10001	CLA =0200
	00012	0763 00	0 00033	10000	LLS 27
	00013	0300 00	0 00023	10001	FAD =0200000000000
	00014	0601 60	4 00003	10000	STO* 3,4
			00015		RETURN CAS
	00016	000000000	335	10000	OCT 00000000335
	00017	000000037	035	10000	OCT 00000037035
	00020	00000000000		10000	*LDIR
	00021	232162606	060	10000	
BINARY	CARD ID.	CAS	0003		
	00022	000000002	00	10000	*LORG
	00023	200000000	000	10000	
			00000	01111	END

RDMM AVERAG 08/15/65
EXTERNAL FORMULA NUMBER - SOURCE STATEMENT - INTERNAL FORMULA NUMBER(S)

```
SUBROUTINE AVERAG(X,NX,AV,SD)
DIMENSION X(2)
CNX=NX
VQM=0
AV=0
DO 1 I=1,NX
1 AV=AV+X(I)
AV=AV/CNX
DO 2 I=1,NX
2 VQM=VQM+(X(I)-AV)**2
SD=SQRT(VQM/(CNX-1.))
RETURN
END
```

```
,1
,2
,3
,4
,5 ,6
,7
,8
,9 ,10
,11
,12
,13
```

RDMM COE 08/15/65
 EXTERNAL FORMULA NUMBER - SOURCE STATEMENT - INTERNAL FORMULA NUMBER(S)

SUBROUTINE COE(MAX,CF)					
DIMENSION CF(10,10)					
6 FORMAT (10E7.4)					
16 FORMAT (1H0,1P8E15.4)					
CN=1					,1
DO 14 I=1,MAX					,2
READ (5,6) (CF(J,I),J=1,I)					,3
DO 15 J=1,I					,4
15 CF(J,I)=CF(J,I)/CN					,5
CN=CN*FLOAT(I)**2					,6
WRITE (6,16) (CF(J,I),J=1,I)					,7
14 CONTINUE					,8
RETURN					,9
END					,10
					,11
					,12
					,13
					,14
					,15
					,16
					,17
					,18
					,19
					,20

SAMPLE PROBLEM *** EXP. DATA ISPRA 1 REACTOR 3.7 MW

LAGUERRE POLYNOMIALS

8 DETECTORS

1.0000E 00							
1.0000E 00	-1.0000E 00						
5.0000E-01	-1.0000E 00	2.5000E-01					
1.6667E-01	-5.0000E-01	2.5000E-01	-2.7778E-02				
4.1667E-02	-1.6667E-01	1.2500E-01	-2.7778E-02	1.7361E-03			
8.3333E-03	-4.1667E-02	4.1667E-02	-1.3889E-02	1.7361E-03	-6.9444E-05		
1.3889E-03	-8.3333E-03	1.0417E-02	-4.6296E-03	8.6806E-04	-6.9444E-05	1.9290E-06	
1.9841E-04	-1.3889E-03	2.0833E-03	-1.1574E-03	2.8935E-04	-3.4722E-05	1.9290E-06	-3.9368E-08

DATA

DETECTOR	ACTIV.RATE	O/O A	O/O SIGMA
NI-NEW	0.6150E 02	0.050	0.050
NEPT.	0.4170E 04	0.050	0.050
AL-N.P	0.2540E 01	0.050	0.050
URANIO	0.3390E 03	0.050	0.050
FERRO	0.7630E 00	0.050	0.050
TCRIO	0.9210E 02	0.050	0.050
AL-ALF	0.5280E 00	0.050	0.050
ZCLFO	0.4070E 02	0.050	0.050

POL.	CRDER	4	NUMB.	OF	HIS.	150
POL.	CRDER	5	NUMB.	OF	HIS.	150
POL.	CRDER	6	NUMB.	OF	HIS.	150
POL.	CRDER	7	NUMB.	OF	HIS.	150
POL.	CRDER	8	NUMB.	OF	HIS.	150

APPROXIMATION NUMBER *** 4 ***

CHECK

NI-NEW 1.082
 NEPT. 0.996
 AL-N.P 0.970
 URANIC 1.079
 FERRO 1.042
 TORIC 0.888
 AL-ALF 0.975
 ZCLFC 0.937

Q = 0.328E-01 - INTEGRAL FLUX ABOVE .5 MEV = 0.355E 01

E	FI	E	FI	E	FI	E	FI	E	FI
0.	0.140E-02	0.50	0.619E-01	1.00	0.262E-01	1.50	0.105E-01	2.00	0.401E-00
2.50	0.146E-00	3.00	0.557E-01	3.50	0.284E-01	4.00	0.219E-01	4.50	0.203E-01
5.00	0.187E-01	5.50	0.164E-01	6.00	0.136E-01	6.50	0.107E-01	7.00	0.809E-02
7.50	0.592E-02	8.00	0.422E-02	8.50	0.293E-02	9.00	0.199E-02	9.50	0.133E-02
10.00	0.873E-03	10.50	0.563E-03	11.00	0.357E-03	11.50	0.223E-03	12.00	0.137E-03
12.50	0.821E-04	13.00	0.483E-04	13.50	0.276E-04	14.00	0.153E-04		

APPROXIMATION NUMBER *** 5 ***

CHECK

NI-NEW 1.088
 NEPT. 1.035
 AL-N.P 0.964
 URANIC 1.059
 FERRC 1.026
 TORIC 0.873
 AL-ALF 0.991
 ZCLFC 0.951

Q = 0.330E-01 - INTEGRAL FLUX ABOVE .5 MEV = 0.371E 01

E	FI	E	FI	E	FI	E	FI	E	FI
0.	0.162E-02	0.50	0.680E-01	1.00	0.273E-01	1.50	0.104E-01	2.00	0.380E-00
2.50	0.138E-02	3.00	0.568E-01	3.50	0.329E-01	4.00	0.261E-01	4.50	0.229E-01
5.00	0.198E-01	5.50	0.163E-01	6.00	0.129E-01	6.50	0.983E-02	7.00	0.729E-02
7.50	0.530E-02	8.00	0.380E-02	8.50	0.271E-02	9.00	0.191E-02	9.50	0.135E-02
10.00	0.956E-03	10.50	0.677E-03	11.00	0.480E-03	11.50	0.342E-03	12.00	0.243E-03
12.50	0.174E-03	13.00	0.124E-03	13.50	0.889E-04	14.00	0.635E-04		

STATISTICAL ANALISYS (150 HISTORIES)

INT.FLX 0.371E 01 MEAN 0.372E 01 +- 0.407E-00 0/0 0.11 (0.332E 01 - 0.413E 01)

E (MEV)	NELTRON FLUX		STANDARD DEVIAT.		68 PER CENT	
	CALCUL.	MCNTECARLO	ABSOLUTE	RELAT.	CONFIDENCE	INT.
0.	0.162E 02	0.161E 02	0.495E 01	0.307	0.112E 02	0.211E 02
0.5	0.680E 01	0.680E 01	0.142E 01	0.208	0.538E 01	0.821E 01
1.	0.273E 01	0.274E 01	0.299E-00	0.109	0.244E 01	0.304E 01
1.5	0.104E 01	0.105E 01	0.699E-01	0.066	0.985E 00	0.112E 01
2.	0.380E 00	0.386E-00	0.530E-01	0.137	0.333E-00	0.439E-00
2.5	0.138E 00	0.139E-00	0.244E-01	0.175	0.115E-00	0.164E-00
3.	0.568E 00	0.568E-01	0.101E-01	0.178	0.467E-01	0.669E-01
3.5	0.329E 01	0.323E-01	0.114E-01	0.353	0.209E-01	0.437E-01
4.	0.261E 01	0.225E-01	0.121E-01	0.398	0.153E-01	0.356E-01
4.5	0.229E 01	0.224E-01	0.671E-02	0.299	0.157E-01	0.291E-01
5.	0.198E 01	0.195E-01	0.358E-02	0.184	0.159E-01	0.231E-01
5.5	0.163E 01	0.162E-01	0.214E-02	0.132	0.141E-01	0.183E-01
6.	0.129E 01	0.129E-02	0.215E-02	0.167	0.107E-01	0.150E-01
6.5	0.983E 00	0.985E-02	0.217E-02	0.221	0.767E-02	0.120E-01
7.	0.729E 00	0.733E-02	0.188E-02	0.257	0.545E-02	0.921E-02
7.5	0.530E 00	0.535E-02	0.142E-02	0.266	0.392E-02	0.677E-02
8.	0.338E 00	0.338E-02	0.934E-03	0.243	0.291E-02	0.477E-02
8.5	0.271E 00	0.273E-02	0.518E-03	0.190	0.221E-02	0.325E-02
9.	0.191E 00	0.193E-02	0.224E-03	0.116	0.171E-02	0.216E-02
10.	0.135E 00	0.136E-02	0.146E-03	0.107	0.122E-02	0.151E-02
10.5	0.956E 00	0.959E-03	0.223E-03	0.233	0.736E-03	0.118E-02
11.	0.677E 00	0.676E-03	0.275E-03	0.406	0.401E-03	0.950E-03
11.5	0.480E 00	0.477E-03	0.287E-03	0.602	0.190E-03	0.764E-03
12.	0.342E 00	0.337E-03	0.273E-03	0.809	0.646E-04	0.610E-03
12.5	0.243E 00	0.233E-03	0.243E-03	1.018	0.436E-05	0.482E-03
13.	0.174E 00	0.170E-03	0.208E-03	1.224	0.381E-04	0.377E-03
13.5	0.124E 00	0.121E-03	0.171E-03	1.421	0.508E-04	0.292E-03
14.	0.868E 00	0.855E-04	0.138E-03	1.606	0.519E-04	0.223E-03
14.5	0.635E 00	0.609E-04	0.108E-03	1.775	0.472E-04	0.169E-03

APPROXIMATION NUMBER *** 6 ***

CHECK

NI-NEW 1.087
 NEPT. 1.013
 AL-N.P 0.963
 URANIC 1.065
 FERRC 1.031
 TCRC 0.877
 AL-ALF 0.988
 ZCLFC 0.948

Q = 0.321E-01 - INTEGRAL FLUX ABOVE .5 MEV = 0.362E 01

E	FI	E	FI	E	FI	E	FI	E	FI
0.	0.152E-02	0.50	0.651E-01	1.00	0.267E-01	1.50	0.104E-01	2.00	0.388E-00
2.50	0.142E-02	3.00	0.573E-01	3.50	0.318E-01	4.00	0.247E-01	4.50	0.219E-01
5.00	0.193E-01	5.50	0.162E-01	6.00	0.130E-01	6.50	0.101E-01	7.00	0.753E-02
7.50	0.550E-02	8.00	0.394E-02	8.50	0.279E-02	9.00	0.195E-02	9.50	0.135E-02
10.00	0.935E-03	10.50	0.645E-03	11.00	0.444E-03	11.50	0.305E-03	12.00	0.210E-03
12.50	0.145E-03	13.00	1.000E-04	13.50	0.691E-04	14.00	0.479E-04		

STATISTICAL ANALYSIS (150 HISTORIES)

INT.FLX 0.362E 01 MEAN 0.362E 01 +- 0.345E-00 0/0 0.10 (0.328E 01 - 0.397E 01)

E (MEV)	NEUTRON FLUX		STANDARD DEVIAT.		68 PER CENT	
	CALCUL.	MCNTECARLO	ABSOLUTE	RELAT.	CONFIDENCE INT.	
0.000000	0.152E 02	0.152E 02	0.374E 01	0.246	0.114E 02	0.189E 02
0.000000	0.651E 01	0.649E 01	0.111E 01	0.170	0.539E 01	0.760E 01
0.000000	0.267E 01	0.267E 01	0.264E -00	0.099	0.240E 01	0.293E 01
0.000000	0.104E 01	0.104E 01	0.693E -01	0.066	0.975E 00	0.111E 01
0.000000	0.389E 00	0.389E 00	0.421E -01	0.108	0.347E -00	0.431E -00
0.000000	0.143E 00	0.143E 00	0.222E -01	0.156	0.123E -00	0.165E -00
0.000000	0.573E 00	0.577E 00	0.121E -01	0.210	0.455E -01	0.698E -01
0.000000	0.318E 00	0.319E 00	0.982E -02	0.308	0.220E -01	0.417E -01
0.000000	0.247E 00	0.247E 00	0.770E -02	0.312	0.170E -01	0.324E -01
0.000000	0.219E 00	0.219E 00	0.515E -02	0.235	0.167E -01	0.270E -01
0.000000	0.193E 00	0.193E 00	0.327E -02	0.170	0.160E -01	0.226E -01
0.000000	0.162E 00	0.163E 00	0.250E -02	0.154	0.137E -01	0.188E -01
0.000000	0.130E 00	0.131E 00	0.226E -02	0.173	0.108E -01	0.153E -01
0.000000	0.101E 00	0.101E 00	0.198E -02	0.196	0.812E -02	0.121E -01
0.000000	0.753E 00	0.757E 00	0.158E -02	0.208	0.600E -02	0.915E -02
0.000000	0.550E 00	0.554E 00	0.112E -02	0.202	0.442E -02	0.666E -02
0.000000	0.399E 00	0.398E 00	0.706E -03	0.177	0.327E -02	0.468E -02
0.000000	0.274E 00	0.281E 00	0.380E -03	0.135	0.243E -02	0.319E -02
0.000000	0.195E 00	0.197E 00	0.187E -03	0.095	0.178E -02	0.216E -02
0.000000	0.135E 00	0.137E 00	0.171E -03	0.125	0.120E -02	0.154E -02
0.000000	0.935E 00	0.945E 00	0.220E -03	0.233	0.726E -03	0.117E -02
0.000000	0.645E 00	0.651E 00	0.246E -03	0.377	0.406E -03	0.897E -03
0.000000	0.444E 00	0.448E 00	0.245E -03	0.546	0.203E -03	0.693E -03
0.000000	0.305E 00	0.308E 00	0.226E -03	0.733	0.822E -04	0.534E -03
0.000000	0.210E 00	0.212E 00	0.198E -03	0.933	0.142E -04	0.409E -03
0.000000	0.145E 00	0.146E 00	0.166E -03	1.141	0.205E -04	0.312E -03
0.000000	0.090E 00	0.100E 00	0.136E -03	1.350	0.352E -04	0.236E -03
0.000000	0.691E 00	0.693E 00	0.108E -03	1.558	0.387E -04	0.177E -03
0.000000	0.479E 00	0.479E 00	0.842E -04	1.759	0.363E -04	0.132E -03

APPROXIMATION NUMBER *** 7 ***

CHECK

NI-NEW 1.088
 NEPT. 1.022
 AL-N.P 0.963
 URANIC 1.064
 FERRC 1.030
 TORIC 0.876
 AL-ALF 0.989
 ZCLFC 0.951

Q = 0.324E-01 - INTEGRAL FLUX ABOVE .5 MEV = 0.366E 01

E	FI	E	FI	E	FI	E	FI	E	FI
0.	0.157E-02	0.50	0.663E-01	1.00	0.269E-01	1.50	0.104E-01	2.00	0.385E-00
2.50	0.141E-02	3.00	0.575E-01	3.50	0.324E-01	4.00	0.252E-01	4.50	0.222E-01
5.00	0.194E-01	5.50	0.162E-01	6.00	0.129E-01	6.50	0.998E-02	7.00	0.746E-02
7.50	0.546E-02	8.00	0.392E-02	8.50	0.278E-02	9.00	0.196E-02	9.50	0.136E-02
10.00	0.947E-03	10.50	0.655E-03	11.00	0.452E-03	11.50	0.311E-03	12.00	0.214E-03
12.50	0.147E-03	13.00	0.101E-03	13.50	0.697E-04	14.00	0.479E-04		

STATISTICAL ANALYSIS (150 HISTORIES)

INT.FLX 0.356E 01 MEAN 0.359E 01 +- 0.383E-00 0/0 0.11 (0.320E 01 - 0.397E 01)

E (MEV)	NEUTRON FLUX		STANDARD DEVIAT.		68 PER CENT	
	CALCUL.	MCNTECARLO	ABSOLUTE	RELAT.	CONFIDENCE	INT.
0.0	0.167E 02	0.165E 02	0.713E 01	0.433	0.933E 01	0.236E 02
0.1	0.667E 01	0.662E 01	0.116E 01	0.175	0.546E 01	0.778E 01
0.2	0.257E 01	0.260E 01	0.433E 00	0.166	0.217E 01	0.304E 01
0.3	0.991E 00	0.100E 00	0.191E 00	0.190	0.813E 00	0.119E 01
0.4	0.344E 00	0.383E 00	0.427E 00	0.111	0.341E 00	0.426E 00
0.5	0.753E 00	0.649E 00	0.590E 00	0.393	0.910E 00	0.209E 00
0.6	0.288E 00	0.347E 00	0.418E 00	0.645	0.230E 00	0.107E 00
0.7	0.288E 00	0.247E 00	0.142E 00	0.410	0.205E 00	0.490E 00
0.8	0.188E 00	0.249E 00	0.128E 00	0.532	0.112E 00	0.368E 00
0.9	0.157E 00	0.196E 00	0.184E 00	0.933	0.119E 00	0.380E 00
1.0	0.139E 00	0.169E 00	0.162E 00	0.955	0.757E 00	0.331E 00
1.1	0.121E 00	0.146E 00	0.103E 00	0.701	0.438E 00	0.249E 00
1.2	0.830E 00	0.124E 00	0.429E 00	0.347	0.809E 00	0.167E 00
1.3	0.652E 00	0.807E 00	0.190E 00	0.188	0.825E 00	0.121E 00
1.4	0.492E 00	0.621E 00	0.381E 00	0.473	0.425E 00	0.119E 00
1.5	0.357E 00	0.463E 00	0.462E 00	0.745	0.158E 00	0.108E 00
1.6	0.357E 00	0.433E 00	0.433E 00	0.935	0.302E 00	0.895E 00
1.7	0.248E 00	0.334E 00	0.340E 00	0.020	0.673E 00	0.674E 00
1.8	0.164E 00	0.233E 00	0.227E 00	0.977	0.527E 00	0.459E 00
1.9	0.101E 00	0.155E 00	0.119E 00	0.768	0.360E 00	0.275E 00
2.0	0.569E 00	0.990E 00	0.349E 00	0.352	0.641E 00	0.134E 00
2.1	0.270E 00	0.320E 00	0.417E 00	0.526	0.174E 00	0.101E 00
2.2	0.774E 00	0.143E 00	0.808E 00	0.266	0.488E 00	0.113E 00
2.3	0.372E 00	0.143E 00	0.102E 00	0.160	0.881E 00	0.117E 00
2.4	0.983E 00	0.338E 00	0.110E 00	0.413	0.106E 00	0.113E 00
2.5	0.124E 00	0.286E 00	0.107E 00	0.345	0.110E 00	0.104E 00
2.6	0.128E 00	0.602E 00	0.981E 00	0.311	0.104E 00	0.921E 00
2.7	0.120E 00	0.721E 00	0.861E 00	0.945	0.934E 00	0.789E 00
2.8	0.120E 00	0.724E 00	0.731E 00	0.089	0.803E 00	0.658E 00

383 LINES OUTPUT THIS JOB.

APPROXIMATION NUMBER *** 8 ***

CHECK

NI-NEW 1.084
 NEPT. 0.996
 AL-N.P 0.954
 URANIC 1.057
 FERRC 1.041
 TORIC 0.871
 AL-ALF 0.983
 ZCLFC 0.957

Q = 0.330E-01 - INTEGRAL FLUX ABOVE .5 MEV = 0.356E 01

E	FI	E	FI	E	FI	E	FI	E	FI
0.	0.167E-02	0.50	0.661E-01	1.00	0.257E-01	1.50	0.991E-00	2.00	0.384E-00
2.50	0.153E-00	3.00	0.670E-01	3.50	0.348E-01	4.00	0.228E-01	4.50	0.181E-01
5.00	0.157E-01	5.50	0.139E-01	6.00	0.121E-01	6.50	0.102E-01	7.00	0.830E-02
7.50	0.652E-02	8.00	0.492E-02	8.50	0.357E-02	9.00	0.248E-02	9.50	0.164E-02
10.00	0.101E-02	10.50	0.569E-03	11.00	0.270E-03	11.50	0.774E-04	12.00	-0.372E-04
12.50	-0.983E-04	13.00	-0.124E-03	13.50	-0.128E-03	14.00	-0.120E-03		

STATISTICAL ANALYSIS (150 HISTORIES)

INT. FLX 1.366E 01 MEAN 0.364E 01 +- 0.287E 00 0/0 0.08 (0.335E 01 - 0.393E 01)

E (MEV)	NEUTRON FLUX		STANDARD DEVIAT.		68 PER CENT CONFIDENCE INT.	
	CALCUL.	MCNTECARLO	ABSOLUTE	RELAT.		
0.0	157	154	3.9	0.197	0.124E 02	0.185E 02
0.1	663	655	9.0	0.137	0.566E 01	0.746E 01
0.15	269	268	2.2	0.083	0.245E 01	0.290E 01
0.2	104	104	0.7	0.068	0.974E 00	0.112E 01
0.25	385	389	4.5	0.133	0.349E 01	0.429E 01
0.3	141	143	1.9	0.137	0.123E 01	0.163E 01
0.35	575	583	9.6	0.186	0.475E 01	0.692E 01
0.4	224	224	2.2	0.288	0.231E 01	0.417E 01
0.45	552	550	7.4	0.297	0.176E 01	0.324E 01
0.5	194	192	4.9	0.225	0.173E 01	0.269E 01
0.55	224	222	4.9	0.256	0.163E 01	0.222E 01
0.6	162	162	3.3	0.130	0.141E 01	0.183E 01
0.65	129	132	1.8	0.143	0.111E 01	0.149E 01
0.7	99	101	1.6	0.166	0.839E 00	0.117E 01
0.75	74	75	1.3	0.181	0.619E 00	0.892E 00
0.8	54	55	1.0	0.181	0.454E 00	0.654E 00
0.85	39	39	0.6	0.165	0.333E 00	0.464E 00
0.9	27	28	0.3	0.135	0.245E 00	0.321E 00
0.95	13	13	0.2	0.102	0.178E 00	0.218E 00
1.0	9	9	0.1	0.110	0.122E 00	0.153E 00
1.05	4	5	0.1	0.188	0.772E 00	0.113E 00
1.1	6	6	0.2	0.306	0.453E 00	0.852E 00
1.15	3	4	0.2	0.451	0.245E 00	0.648E 00
1.2	2	2	0.1	0.617	0.117E 00	0.492E 00
1.25	1	1	0.0	0.802	0.415E 00	0.373E 00
1.3	1	1	0.0	0.002	0.321E 00	0.282E 00
1.35	0	0	0.0	0.216	0.066E 00	0.211E 00
1.4	0	0	0.0	0.439	0.206E 00	0.158E 00
1.45	0	0	0.0	0.671	0.293E 00	0.117E 00

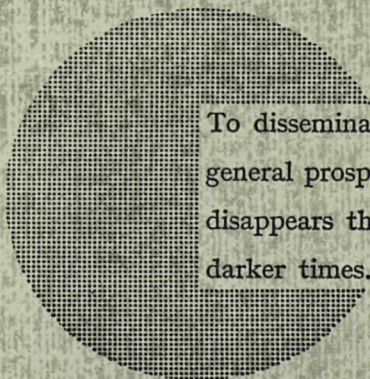
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Alfred Nobel

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