CVP-2005-00007
Rev. 0

Cleanup Verification Package for the 300-8 Waste Site

Prepared for the U.S. Department of Energy by Washington Closure Hanford

October 2005

EXECUTIVE SUMMARY

This cleanup verification package documents completion of remedial action for the 300-8 waste site. The 300-8 site is located within the 300-FF-2 Operable Unit in the 300 Area of the Hanford Site in southeastern Washington State. The site was formerly used to stage scrap metal from the 300 Area in support of a program to recycle aluminum. Staging and loading activities at the site scattered scrap metal over an approximately 34,000-m^2 (366,000-ft^2) area, with residual metallic debris generally present within the top 0.4 m (1.5 ft) of soil.

Site excavation and waste disposal are complete, and post-excavation geophysical surveys confirm the removal of residual metallic debris. The exposed surfaces have been sampled and analyzed to verify attainment of the remedial action goals. Results of the sampling, laboratory analyses, and data evaluations for the 300-8 site indicate that all remedial action objectives and goals for direct exposure, protection of groundwater, and protection of the Columbia River have been met for industrial land use (Table ES-1).

Because residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 300-8 site, a supplemental evaluation was performed against unrestricted land-use cleanup objectives established in the *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision* (EPA 2004). Results of the evaluation (Table ES-2) demonstrate that residual contaminant concentrations do not preclude any future uses (as bounded by the rural-residential scenario) and allow for unrestricted use of shallow zone soils (i.e., surface to 4.6 m [15 ft] deep). This site does not have a deep zone; therefore, no deep zone institutional controls are required.

The site meets cleanup standards and has been reclassified as "interim closed out" in accordance with the *Hanford Federal Facility Agreement and Consent Order* (Ecology et al. 1989) and the Waste Site Reclassification Guideline TPA-MP-14 (RL-TPA-90-0001) (DOE-RL 1998). A copy of the waste site reclassification form is included as Attachment ES-1.

Table ES-1. Summary of Cleanup Verification Results for the 300-8 Waste Site – Industrial Land Use.

Regulatory Requirement	Remedial Action Goals	Results	Remedial Action Objectives Attained?
Direct Exposure – Radionuclides	Attain 15 mrem/yr dose rate above background over 1,000 years. Attain the CERCLA risk range of 10^{-4} to 10^{-6}.	No radionuclide COCs were detected above background levels.	Yes
Direct Exposure – Nonradionuclides	Attain individual COC RAGs.	All individual COC concentrations are below the RAGs.	Yes
Meet Nonradionuclide Risk Requirements	Hazard quotient of <1 for noncarcinogens.	Hazard quotients were not calculated because concentrations of the only nonradionuclide COC (beryllium) were below statistical background levels.	Yes
	Cumulative hazard quotient of <1 for noncarcinogens.		
	Excess cancer risk of <1 x 10^{-5} for individual carcinogens.	Excess cancer risks were not calculated because concentrations of the only nonradionuclide COC (beryllium) were below statistical background levels.	
	Attain a total excess cancer risk of <1 x 10^{-5} for carcinogens.		
Groundwater/River Protection – Radionuclides	Attain single-COC groundwater and river protection RAGs.	All single-COC groundwater and river RAGs have been attained.	Yes
	Attain National Primary Drinking Water Standards: 4 mrem/yr (beta/gamma) dose rate to target receptor/organs.[a]	No beta/gamma-emitting COCs were identified for this site.	
	Meet drinking water standards for nonuranium alpha emitters: the more stringent of the 15 pCi/L MCL or 1/25th of the derived concentration guide per DOE Order 5400.5.[b]	No beta/gamma-emitting COCs were identified for this site.	
	Meet total uranium standard of 21.2 pCi/L.[c]	Uranium statistical values are below background levels for this site.	
Groundwater/River Protection – Nonradionuclides	Attain individual nonradionuclide groundwater and river cleanup requirements.	All the groundwater and river RAGs have been attained.	Yes
Supporting Information	300-8 Cleanup verification sample location design (Appendix C).[d]		
	300-8 Cleanup verification 95% UCL Calculation (Appendix C).[e]		

[a] "National Primary Drinking Water Regulations" (40 *Code of Federal Regulations* 141).

[b] *Radiation Protection of the Public and the Environment* (DOE Order 5400.5).

[c] Based on the isotopic distribution of uranium in the Hanford Site background, the 30 µg/L MCL (65 *Federal Register* 76708) corresponds to 21.2 pCi/L. Concentration-to-activity calculations are documented in *Calculation of Total Uranium Activity Corresponding to a Maximum Contaminant Level for Total Uranium of 30 Micrograms per Liter in Groundwater*, 0100X-CA-V0038 (BHI 2001).

[d] *300-8 Sites Shallow Zone Sampling Plan*, 0300X-CA-V0057, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

[e] *300-8 Waste Site Cleanup Verification 95% UCL Calculation*, 0300X-CA-V0056, Rev. 0, Washington Closure Hanford, Richland, Washington.

CERCLA = *Comprehensive Environmental Response, Compensation, and Liability Act of 1980*
COC = contaminant of concern
MCL = maximum contaminant level (drinking water standard)
RAG = remedial action goal
UCL = upper confidence limit

Table ES-2. Summary of Cleanup Verification Results for the 300-8 Waste Site – Unrestricted Land Use.

Regulatory Requirement	Remedial Action Goals	Results	Remedial Action Objectives Attained?
Direct Exposure – Radionuclides	Attain 15 mrem/yr dose rate above background over 1,000 years. Attain the CERCLA risk range of 10^{-4} to 10^{-6}.	No radionuclide COCs were detected above background levels.	Yes
Direct Exposure – Nonradionuclides	Attain individual COC RAGs.	All individual COC concentrations are below the RAGs.	Yes
Meet Nonradionuclide Risk Requirements	Hazard quotient of <1 for noncarcinogens.	Hazard quotients were not calculated because concentrations of the only nonradionuclide COC (beryllium) were below statistical background levels.	Yes
	Cumulative hazard quotient of <1 for noncarcinogens.		
	Excess cancer risk of $<1 \times 10^{-6}$ for individual carcinogens.	Excess cancer risks were not calculated because concentrations of the only nonradionuclide COC (beryllium) were below statistical background levels.	
	Attain a total excess cancer risk of $<1 \times 10^{-5}$ for carcinogens.		
Groundwater/River Protection – Radionuclides	Attain single-COC groundwater and river protection RAGs.	All single-COC groundwater and river RAGs have been attained.	Yes
	Attain National Primary Drinking Water Standards: 4 mrem/yr (beta/gamma) dose rate to target receptor/organs.[a]	No beta/gamma-emitting COCs were identified for this site.	
	Meet drinking water standards for nonuranium alpha emitters: the more stringent of the 15 pCi/L MCL or $1/25^{th}$ of the derived concentration guide per DOE Order 5400.5.[b]	No nonuranium alpha-emitting COCs were identified for this site.	
	Meet total uranium standard of 21.2 pCi/L.[c]	Uranium statistical values are below background levels for this site.	
Groundwater/River Protection – Nonradionuclides	Attain individual nonradionuclide groundwater and river cleanup requirements.	All the groundwater and river RAGs have been attained.	Yes
Supporting Information	300-8 Cleanup verification sample location design (Appendix C).[d] 300-8 Cleanup verification 95% UCL Calculation (Appendix C).[e]		

[a] "National Primary Drinking Water Regulations" (40 *Code of Federal Regulations* 141).

[b] *Radiation Protection of the Public and the Environment* (DOE Order 5400.5).

[c] Based on the isotopic distribution of uranium in the Hanford Site background, the 30 µg/L MCL (65 *Federal Register* 76708) corresponds to 21.2 pCi/L. Concentration-to-activity calculations are documented in *Calculation of Total Uranium Activity Corresponding to a Maximum Contaminant Level for Total Uranium of 30 Micrograms per Liter in Groundwater*, 0100X-CA-V0038 (BHI 2001).

[d] *300-8 Sites Shallow Zone Sampling Plan*, 0300X-CA-V0057, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

[e] *300-8 Waste Site Cleanup Verification 95% UCL Calculation*, 0300X-CA-V0056, Rev. 0, Washington Closure Hanford, Richland, Washington.

CERCLA = *Comprehensive Environmental Response, Compensation, and Liability Act of 1980*
COC = contaminant of concern
MCL = maximum contaminant level (drinking water standard)
RAG = remedial action goal
UCL = upper confidence limit

Attachment ES-1
Waste Site Reclassification Form

Date Submitted: 10/06/05 **Originator:** R. A. Carlson **Phone:** 373-1440	**Operable Unit(s):** 300-FF-2 **Waste Site ID:** 300-8 **Type of Reclassification Action:** Rejected ☐ Closed Out ☐ Interim Closed Out ☒ No Action ☐	**Control Number:** 2005-039 **Lead Agency:** EPA

This form documents agreement among the parties listed below authorizing classification of the subject unit as rejected, closed out, or no action, and authorizing backfill of the site, if appropriate. Final removal from the National Priorities List of no action or closed-out sites will occur at a future date.

Description of current waste site condition:

Remedial action at this site has been performed in accordance with remedial action objectives and goals established by the U.S. Environmental Protection Agency and the U.S. Department of Energy, Richland Operations Office, in concurrence with the Washington State Department of Ecology. The selected remedial action involved (1) excavating the site to the extent required to remove scrap metal shavings and meet specified soil cleanup levels, (2) disposing of contaminated excavated materials at the Environmental Restoration Disposal Facility in the 200 Area of the Hanford Site, and (3) contouring the site to match the surrounding surface. The excavation and disposal activities have been completed.

Basis for reclassification:

The 300-8 waste site has been remediated to meet the cleanup standards specified in the *Record of Decision for the 300-FF-2 Operable Unit, Hanford Site*, U.S. Environmental Protection Agency, Region 10, Seattle, Washington. Remedial actions were performed to support future industrial land use and to protect groundwater and the Columbia River. Further, the residual contaminant concentrations achieved do not preclude any future uses (as bounded by the rural-residential scenario) and allow for unrestricted use of shallow zone soils (i.e., surface to 4.6 m [15 ft] deep). This site has no deep zone; therefore, no deep zone institutional controls are required. The basis for reclassification is described in detail in the *Cleanup Verification Package for the 300-8 Waste Site* (CVP-2005-00007), Washington Closure Hanford, Richland, Washington.

D. C. Smith DOE-RL Project Manager	Signature	10/14/05 Date
N/A Ecology Project Manager	Signature	Date
A. Boyd EPA Project Manager	A. L. Boyd Signature	10-28-05 Date

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APPENDICES

FIGURES

TABLES

ACRONYMS AND ABBREVIATIONS

COC	contaminant of concern
DQA	data quality assessment
ERDF	Environmental Restoration Disposal Facility
ESD	explanation of significant differences
RAG	remedial action goal
RAO	remedial action objective
RDR/RAWP	remedial design report/remedial action work plan
RESRAD	RESidual RADioactivity dose assessment model
ROD	record of decision
SAP	sampling and analysis plan
UCL	upper confidence limit
WAC	*Washington Administrative Code*
WCH	Washington Closure Hanford

1.0 INTRODUCTION

The purpose of this cleanup verification package is to document that the 300-8 waste site was remediated in accordance with the *Record of Decision for the 300-FF-2 Operable Unit, Hanford Site* (ROD) (EPA 2001). Remedial action objectives (RAOs) and remedial action goals (RAGs) for the 300-8 site are documented in the ROD (EPA 2001) and the *Remedial Design Report/Remedial Action Work Plan for the 300 Area* (RDR/RAWP) (DOE-RL 2004b). The ROD provides the U.S. Department of Energy, Richland Operations Office the authority, guidance, and objectives to conduct this remedial action.

The preferred remedy specified in the ROD (EPA 2001) and conducted for the 300-8 site included (1) excavating the site to the extent required to remove scrap metal shavings and meet specified soil cleanup levels, (2) disposing of contaminated excavated materials at the Environmental Restoration Disposal Facility (ERDF) at the 200 Areas of the Hanford Site, and (3) contouring the site to match surrounding grade elevation. Excavation was driven by RAOs for direct exposure, protection of groundwater, and protection of the Columbia River. For the respective points of compliance, RAGs, summarized in Table 1, were established for the radionuclide and nonradionuclide contaminants of concern (COCs) in the RDR/RAWP (DOE-RL 2004b). Preliminary waste site contaminants of potential concern were identified in the *300 Area Remedial Action Sampling and Analysis Plan* (SAP) (DOE-RL 2004a). Following excavation of the site, final COCs were identified in the *Closeout Plan for Waste Site 300-8* (WCH 2005b) and are listed in Table 1.

2.0 SITE DESCRIPTION AND SUPPORTING INFORMATION

2.1 SITE HISTORY

The 300-8 waste site is located in the 300-FF-2 Operable Unit of the 300 Area, along the railroad line north of the 300 Area and adjacent to the 618-8 and 618-3 Burial Grounds (Figure 1). Beginning in 1962, the area adjacent to the railroad line was used to stage scrap metal from the 300 Area in support of a program to recycle aluminum. Some of the metal was contaminated with low levels of uranium and beryllium from 300 Area operations. Scrap metal was staged in the area until sufficient quantities were available to solicit bids from offsite salvage vendors. Sold scrap metal was loaded into open rail cars with clamshell buckets. This process of staging and loading the scrap metal scattered the material over an area greater than 30,000 m^2 (321,000 ft^2). Geophysical surveys of the area performed as part of the 300-FF-2 Operable Unit limited field investigation (DOE-RL 1997) suggested that the scrap metal was dispersed in the top 0.3 m (1 ft) of soil. During remediation, it was determined that scrap metal was predominantly in the top 0.6 m (2 ft) of soil. The posted soil contamination areas are separated by unposted dirt roads that were cleared of contamination and a railroad

line. The railroad line is excluded from the 300-8 site and has not been investigated because it may have future potential uses in support of ongoing or new industrial activities in the 300 Area.

Table 1. Summary of Remedial Action Goals – Industrial Land Use.

COCs	Direct Exposure RAG (pCi/g)	Soil RAG for Groundwater Protection (pCi/g)	Soil RAG for Columbia River Protection (pCi/g)
Radionuclides			
Uranium (total)	350[a]	267[b]	267[b]
COCs	**Direct Exposure RAG (mg/kg)**	**Soil RAG for Groundwater Protection (mg/kg)**	**Soil RAG for Columbia River Protection (mg/kg)**
Nonradionuclides			
Beryllium	104[c]	NA[d]	NA[d]

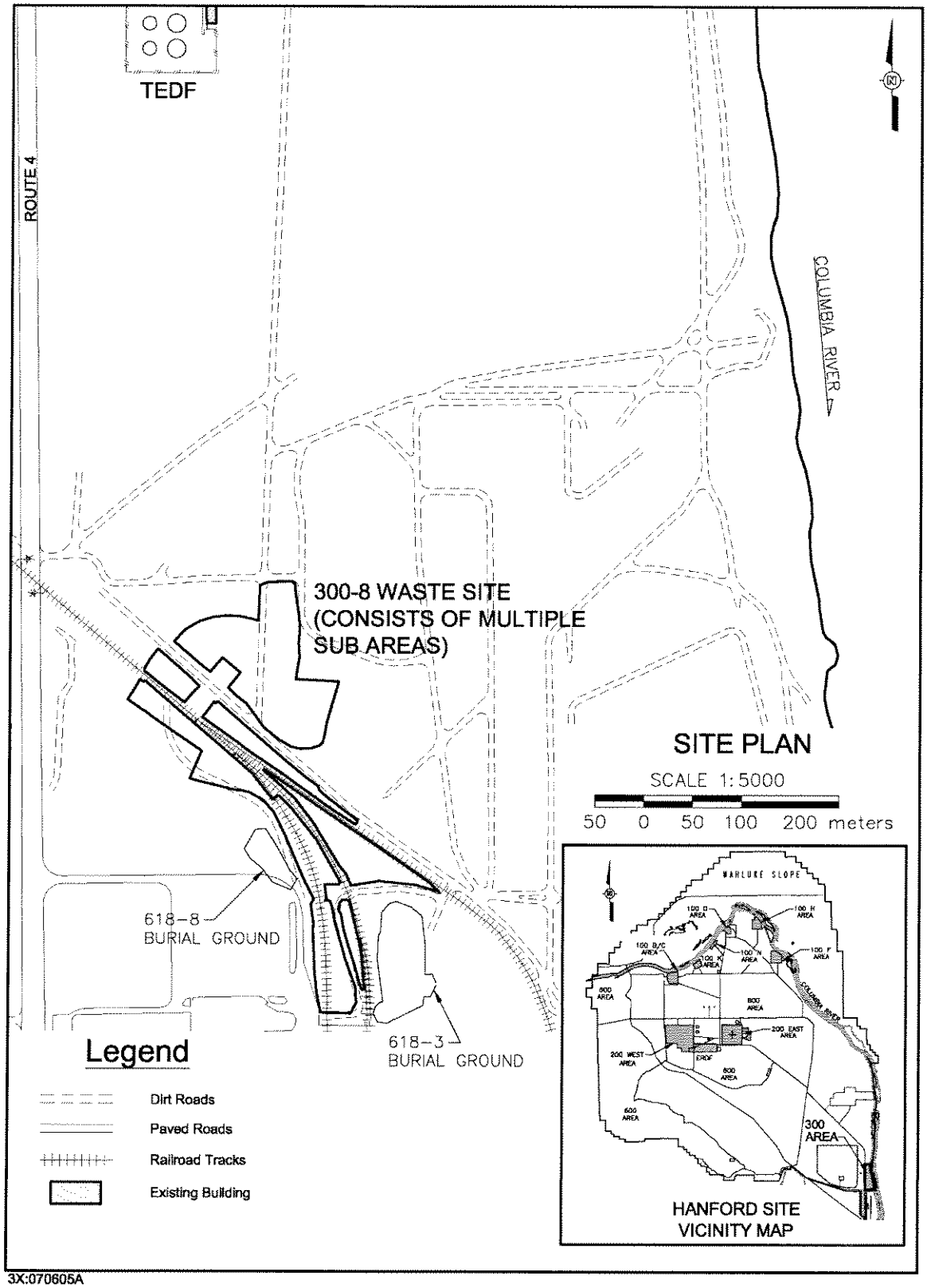
[a] Listed value is equal to a 15 mrem/yr dose for the industrial exposure scenario (DOE-RL 2004b)
[b] Value calculated using RESRAD, based on the generic site model (DOE-RL 2004b).
[c] Value calculated based on the inhalation exposure pathway per *Washington Administrative Code* 173-340-750(4)(b)(ii)(A) or (B).
[d] RESRAD modeling predicts the constituent will not reach groundwater within 1,000 years based on a generic site profile (DOE-RL 2004b).
COC = contaminant of concern
NA = not applicable
RAG = remedial action goal
RESRAD = RESidual RADioactivity (dose assessment model)
WAC = *Washington Administrative Code*

2.2 SUBSURFACE CONDITIONS

The soil column (vadose zone) underlying the waste site and extending to groundwater consists of the Hanford and Ringold Formations. The shallower Hanford Formation consists predominantly of medium-dense to dense sand and gravel, with varying amounts of silt and cobble. The underlying Ringold Formation consists of dense, well-cemented gravels with sand and silt interbedding. The Hanford/Ringold contact is approximately 9 to 21 m (30 to 69 ft) below the surface grade level.

The long-term groundwater level beneath the site is estimated at El. 104.6 m (North American Vertical Datum of 1988) based on information from local groundwater wells. Groundwater levels are influenced by the nearby Columbia River and other factors such as atmospheric pressure. The depth to groundwater is approximately 12.5 m (41 ft) beneath the maximum depth of soil removal at the 300-8 waste site.

Figure 1. Hanford Site Map and Location of the 300-8 Site.



2.3 INITIAL GEOPHYSICAL SURVEYS

Geophysical surveys were conducted at the 300-8 waste site prior to remediation activities to evaluate the effectiveness of survey equipment and to assess the changing density of metal fragments across the site. Both metal detectors evaluated (an EM-61 High Sensitivity Metal Detector and a Fisher 1270 Metal Detector) were determined to be effective for the detection of small aluminum shavings up to 0.13 m (5 in.) below ground surface (WCH 2005a). In general, significantly more aluminum shavings were identified north of the haul road than between the road and the railroad tracks. Additional detail regarding the distribution of metallic debris prior to remediation activities is provided in WCH (2005a).

3.0 REMEDIAL ACTION FIELD ACTIVITIES

3.1 EXCAVATION AND DISPOSAL

Remedial action at the 300-8 site was conducted from December 2004 to May 2005. Excavation of the site included the removal of small quantities of miscellaneous metal construction-type debris (e.g., nuts, bolts), aluminum metal shavings, and soil. No indications of liquid waste disposal or land disposal restricted materials were observed during excavation.

Initially, material within the site boundaries was removed to a depth of 0.3 m (1 ft). Following excavation, geophysical surveys and ground-truthing excavations indicated that significant quantities of metal shavings, on the order of one to a few per square meter, remained at the site (WCH 2005a). Consequently, an additional 0.3 m (1 ft) of material was removed from the entire area. Following this excavation, additional geophysical surveys were performed at thirty-five randomly located 3- by 3-m (10- by 10-ft) test areas within the 300-8 waste site boundaries (Figure 2). Within the 324 m^2 (3,488 ft^2) surveyed, fewer than 10 discrete pieces of metallic debris were detected (WCH 2005a). Based on these results and ground-truthing excavations, it was concluded that remediation was complete.

Approximately 39,750 metric tons (43,820 U.S. tons) of material was removed for disposal at the ERDF. Pre- and post-remediation topographic civil survey results for the 300-8 waste site are depicted in Figures 3 and 4.

3.2 FIELD SCREENING

Radiological surveys were performed in May 2005 after excavation operations were complete at the 300-8 waste site to provide an initial assessment of attainment of radiological cleanup levels. The survey methodology was based on an assumption of

Figure 2. Locations of Test Plots for Geophysical Verification Surveys.

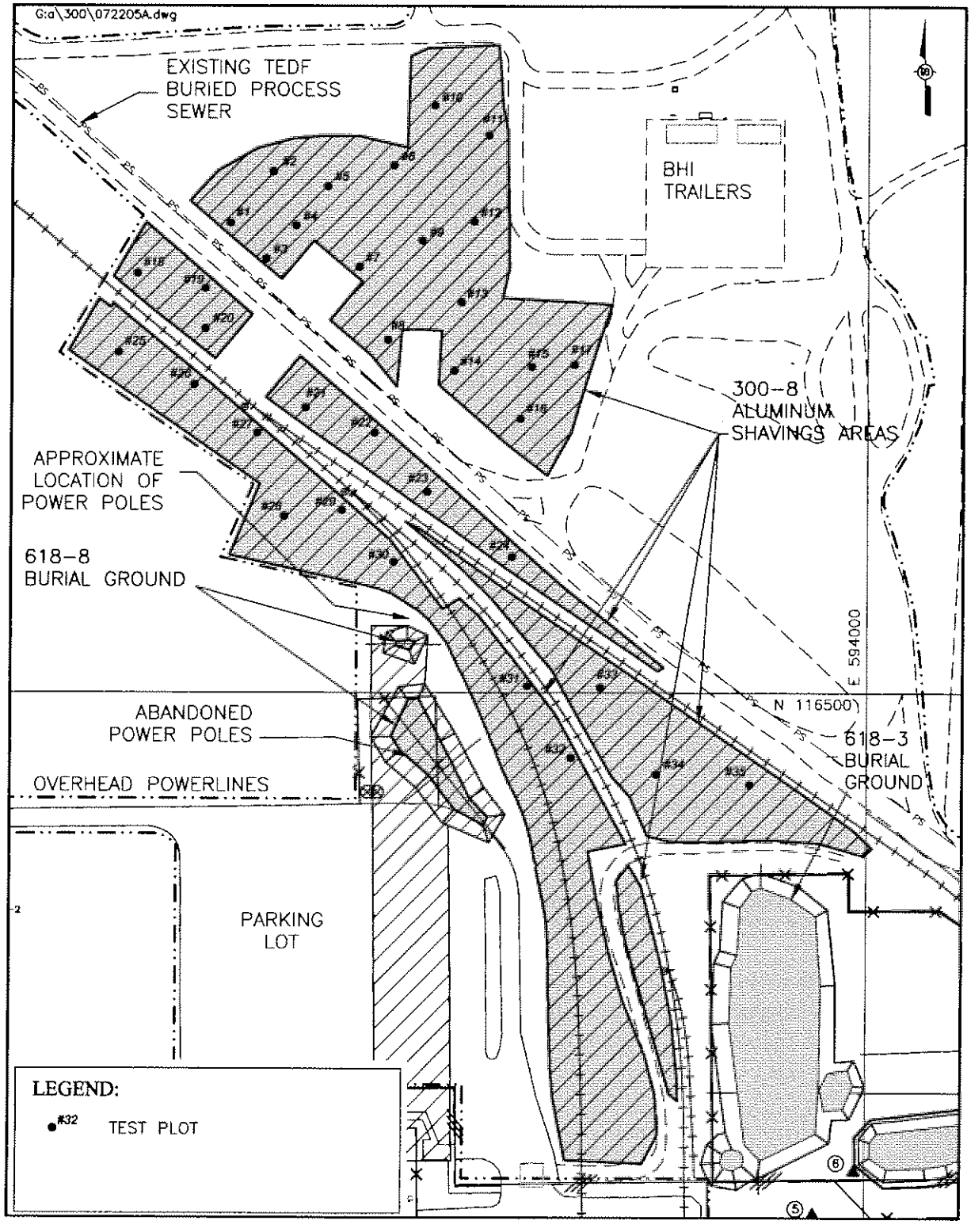


Figure 3. Pre-Remediation Topographic Plan for the 300-8 Site.

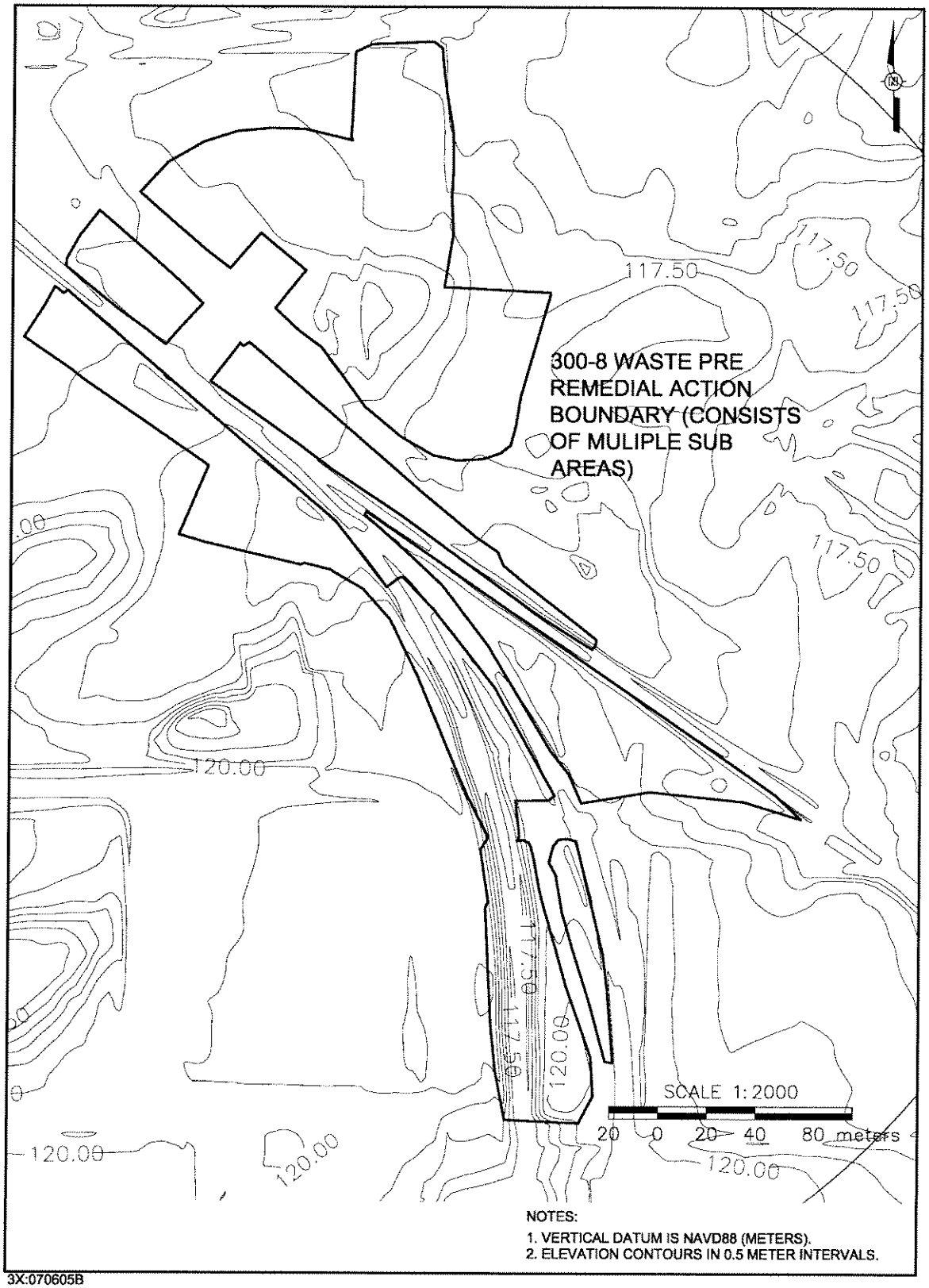


Figure 4. Post-Remediation Topographic Plan for the 300-8 Site.

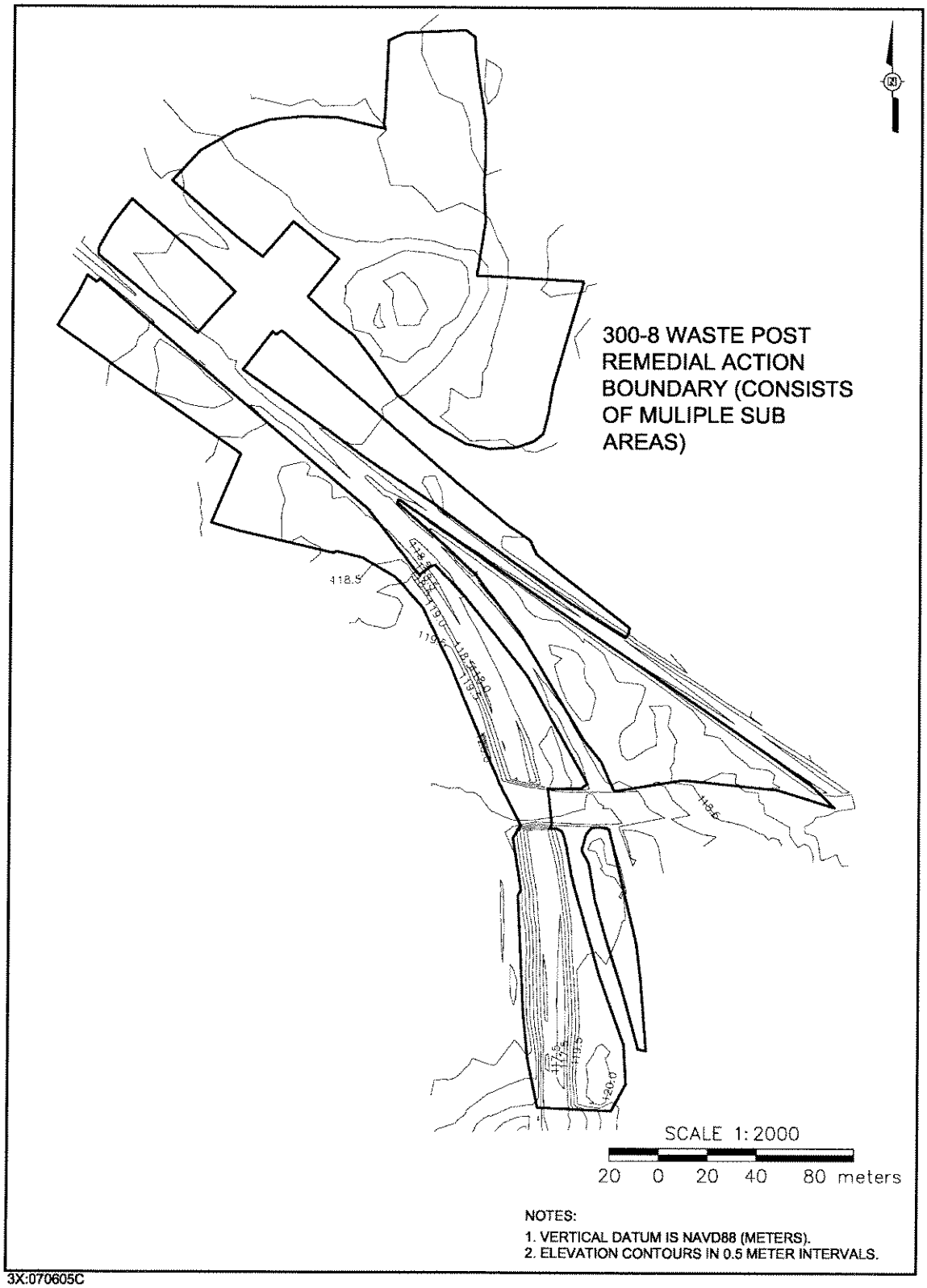
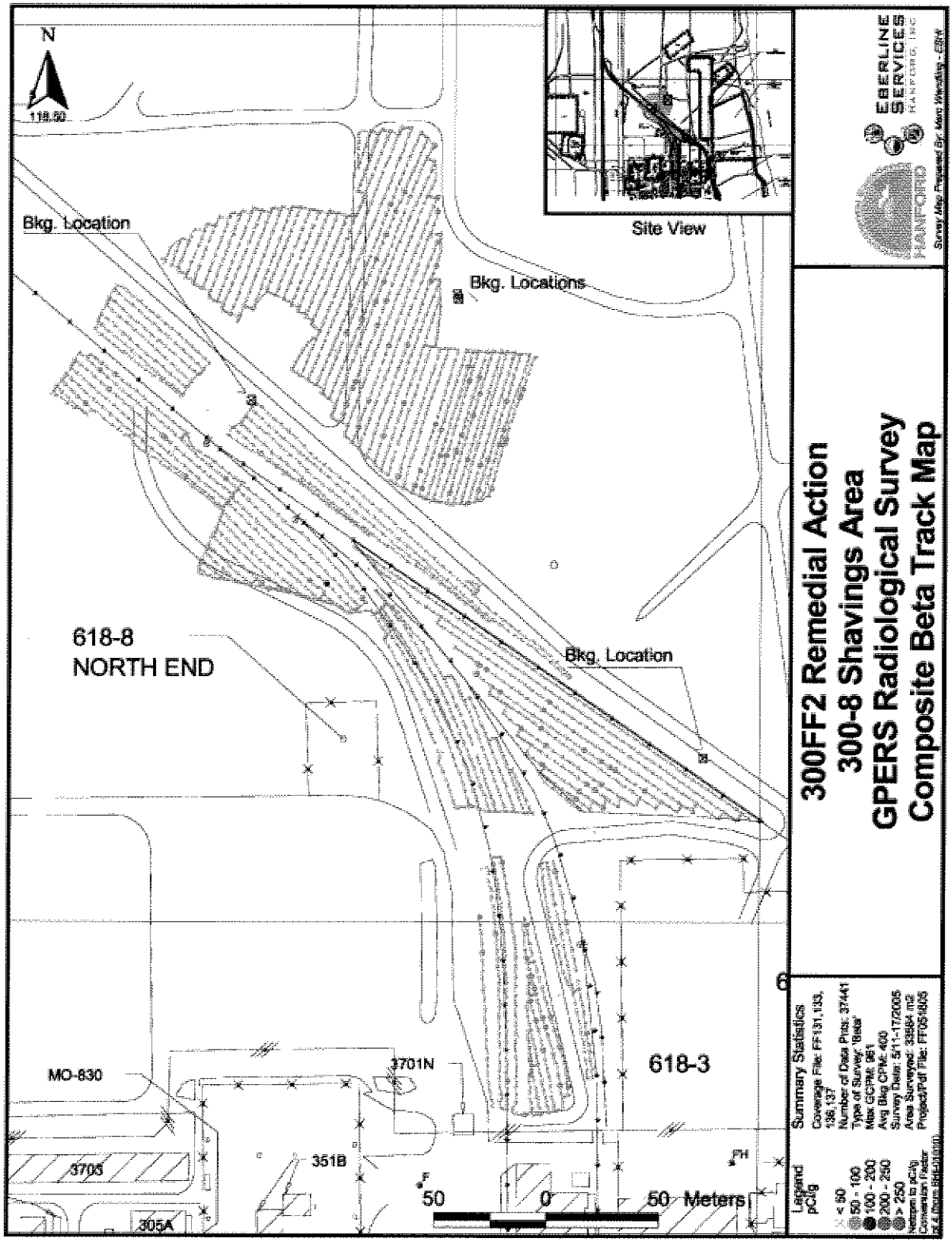


Figure 5. Radiological Mapping Survey Results for the 300-8 Site.



CALCULATION COVER SHEET

Project Title:	300 Area Remedial Action	**Job No.**	14655
Area	300		
Discipline	Environmental	***Calc. No.**	0300X-CA-V0056
Subject	300-8 Waste Site Cleanup Verification 95% UCL Calculation		
Computer Program	Excel	**Program No.**	Excel 2003

The attached calculations have been generated to document compliance with established cleanup levels. These documents should be used in conjunction with other relevant documents in the administrative record.

Committed Calculation ☒ **Preliminary** ☐ **Superseded** ☐ **Voided** ☐

Rev.	Sheet Numbers	Originator	Checker	Reviewer	Approval	Date
0	Cover = 1 Sheets = 5 Total = 6	9/6/05 J. M. Capron	T. M. Blakley 9/8/05 T. B. Miley 9-13-05	9-15-05 L. M. Dittmer	J. A. Lerch	9/15/05

SUMMARY OF REVISIONS

* Obtain calc no. from DIS

DE01437.03 (12/09/2004)

Washington Closure Hanford

CALCULATION SHEET

Originator J. M. Capron **Date** 09/06/05 **Calc. No.** 0300X-CA-V0056 **Rev. No.** 0
Project 300 Area Remedial Action **Job No.** 14655 **Checked** T. M. Blakley **Date** 9/8/05
Subject 300-8 Waste Site Cleanup Verification 95% UCL Calculation **Checked** T. B. Miley **Date** 9-15-05
Sheet No. 1 of 5

Purpose:
Calculate the 95% upper confidence limit (UCL) to evaluate compliance with cleanup standards for the subject site. Also, calculate the hazard quotient and carcinogenic risk for applicable nonradionuclide analytes, perform the *Washington Administrative Code* (WAC) 173-340 (Model Toxics Control Act [MTCA]) 3-part test (all nonradionuclide analytes), and calculate the relative percent difference (RPD) for each contaminant of concern (COC).

Table of Contents:

Given/References:

1) Sample Results
2) Lookup values, background values, and remedial action goals (RAGs) are taken from DOE-RL (1996), DOE-RL (2001), the remedial design report/remedial action work plan (RDR/RAWP) (DOE-RL 2004b), and Ecology (1996).
3) DOE-RL, 1996, *Hanford Site Background: Part 2, Soil Background for Radionuclides*, DOE/RL-96-12, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
4) DOE-RL, 2001, *Hanford Site Background: Part 1, Soil Background for Nonradioactive Analytes*, DOE/RL-92-24, Rev. 4, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
5) DOE-RL, 2004a, *300 Area Remedial Action Sampling and Analysis Plan*, DOE/RL-2001-48, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
6) DOE-RL, 2004b, *Remedial Design Report/Remedial Action Work Plan for the 300 Area*, DOE/RL-2001-47, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.
7) Ecology, 1992, *Statistical Guidance for Ecology Site Managers*, Publication #92-54, Washington State Department of Ecology, Olympia, Washington.
8) Ecology, 1993, *Statistical Guidance for Ecology Site Managers, Supplement S-6, Analyzing Site or Background Data with Below-Detection Limit or Below-PQL Values (Censored Data Sets)*, Publication #92-54, Washington State Department of Ecology, Olympia, Washington.
9) Ecology, 1996, *Model Toxics Control Act Cleanup Levels and Risk Calculations (CLARC II)*, Publication #94-145, Washington State Department of Ecology, Olympia, Washington.
10) EPA, 1994, *USEPA Contract Laboratory Program National Functional Guidelines for Inorganic Data Review*, EPA 540/R-94/013, U.S. Environmental Protection Agency, Washington, D.C.
11) WAC 173-340, 1996, "Model Toxics Control Act--Cleanup," *Washington Administrative Code*.

Solution:
Calculation methodology is described in Ecology Pub. #92-54 (Ecology 1992, 1993), below, and in the RDR/RAWP (DOE-RL 2004b). Use data from attached worksheets to calculate the 95% UCL, hazard quotient, carcinogenic risk, and the RPD for each analyte and to perform the WAC 173-340 3-part test for nonradionuclides.

Calculation Description:
The subject calculations were performed on data from soil verification samples from waste site 300-8. The data were entered into an EXCEL 2003 spreadsheet and calculations performed by utilizing the built-in spreadsheet functions and/or creating formulae within the cells. The statistical evaluation of data for use in accordance with the RDR/RAWP (DOE-RL 2004b) is documented by this calculation. Split and duplicate RPD results are used in evaluation of data quality and are presented in the cleanup verification package (CVP) for this site.

Methodology:
The statistical value calculated to evaluate the effectiveness of cleanup was the 95% UCL. For nonradioactive analytes with > 50% of the data below detection limits, the maximum value for the sample data is used instead of the 95% UCL. All nonradionuclide data reported as being below detection limits are set to ½ the detection limit value for calculation of the statistics (Ecology 1993). For radionuclide data, calculation of the statistics was done on the reported value. In cases where the laboratory does not report a value below the minimal detectable activity (MDA), half of the MDA is used in the calculation.

For the statistical evaluation of duplicate sample pairs, the samples are averaged before being included in the data set, after adjustments for censored data as described above.

For nonradionuclides, the WAC 173-340 statistical guidance suggests that a test for distributional form be performed on the data, and the 95% UCL calculated on the appropriate distribution using Ecology software. For nonradionuclide small data sets ($n < 10$) and all radionuclide data sets, the calculations are performed assuming nonparametric distribution, so no test for distribution is performed. For nonradionuclide data sets of ten or greater, distributional testing is done using Ecology's MTCAStat software (Ecology 1993).

The estimated hazard quotient (for applicable nonradionuclide COCs) is determined by dividing the statistical value (derived in this calculation) by the WAC 173-340 noncarcinogenic cleanup limit. The nonradionuclide carcinogenic risk, above background, is determined by dividing the statistical value by the WAC 173-340 carcinogenic cleanup limit and then multiplying by 10^{-6}. For data sets where all values are below detection or background levels, neither of these calculations are required.

The WAC 173-340 3-part test is performed for nonradionuclide analytes only and determines if:
1) the 95% UCL value exceeds the most stringent cleanup limit for each non-radionuclide COC,
2) greater than 10% of the raw data exceed the most stringent cleanup limit for each non-radionuclide COC,
3) the maximum value of the raw data set exceeds two times the most stringent cleanup limit for each non-radionuclide COC.

The RPD is calculated when both the main value and either the duplicate or split values are above detection limits and are greater than 5 times the target detection limit (TDL). The TDL is a laboratory detection limit pre-determined for each analytical method. These detection limit requirements are listed in Table 2-1 of the sampling and analysis plan (DOE-RL 2004a). The RPD calculations use the following formula:

$$RPD = [\,|M-S|/((M+S)/2)]*100$$

where, M = Main Sample Value S = Split (or duplicate) Sample Value

For quality assurance/quality control (QA/QC) split and duplicate RPD calculations, a value less than +/- 30% indicates the data compare favorably. For regulatory splits, a threshold of +/- 35% is used (EPA 1994). If the RPD is greater than +/- 30% (or +/- 35% for regulatory split data), further investigation regarding the usability of the data is performed. Additional discussion as necessary is provided in the data quality assessment section of the applicable CVP.

If regulator split comparison is required, an additional parameter is evaluated. A control limit of +/- 2 times the TDL shall be used if either the main or regulator split value is less than 5 times the TDL and above detection. In the case where only one result is greater than 5 times the TDL and the other is below, the +/- 2 times the TDL criteria applies. Therefore, the following calculation is performed as part of the evaluation for these two cases involving regulator split data: difference = main - regulator split.

If the difference is greater than +/- 2 times the TDL, then further investigation regarding the usability of the data is performed and presented in the applicable CVP data quality assessment section.

No regulatory split samples were collected for the 300-8 site.

Washington Closure Hanford

CALCULATION SHEET

Originator J. M. Capron
Project 300 Area Remedial Action
Subject 300-8 Waste Site Cleanup Verification 95% UCL Calculation

Date 09/06/05
Job No. 14655

Calc. No. 0300X-CA-V0056
Checked T. M. Blakley 9/8/05
Checked T. B. Miley 9/8/05

Rev. No. 0
Date 9/8/05
Date 9/8/05
Sheet No. 2 of 5

Summary (continued)

Results:

The results presented in the summary tables that follow are for use in RESRAD dose/risk analysis and the CVP for this site.

Results Summary - Shallow Zone			
Analyte	**Result**	**Qualifier**	**Units**
Beryllium	5.7E-01		mg/kg
Uranium (Total)	0 (< BG)		pCi/g

WAC 173-340 Evaluation (Shallow Zone)

3-Part Test:

95% UCL > Cleanup Limit?	NA
> 10% above Cleanup Limit?	NA
Any sample > 2x Cleanup Limit?	NA

Risk Estimate:

Nonrad noncarcinogenic index sum:	NA
Nonrad carcinogenic risk:	NA

Relative Percent Difference (RPD) Results (Shallow Zone)* QA/QC Analysis		
Analyte	**Duplicate Analysis**	**Split Analysis**
Beryllium		
Uranium (Total)		

*A blank cell indicates that RPD evaluation was not required.

BG = background
QA/QC = quality assurance/quality control
RESRAD = RESidual RADioactivity (dose model)
UCL = upper confidence limit
WAC = *Washington Administrative Code*

CALCULATION SHEET

Washington Closure Hanford

Originator J. M. Capron Date 09/06/05 Calc. No. 0300X-CA-V0058 Rev. No. 0
Project 300 Area Remedial Action Job No. 14655 Checked T. M. Blakley Date 9/8/05
Checked T. B. Miley Date 9-13-05
Subject 300-8 Waste Site Cleanup Verification 95% UCL Calculation Sheet No. 3 of 5

300-8 Shallow Zone Sample Data

Sampling Area	HEIS Number	Sample Date	Beryllium mg/kg	Q	PQL	Uranium (Total) pCi/g	Q	MDA
A1	J03VD4	7/27/2005	4.7E-01		9.E-03	1.341E+00		1.9E-01
Duplicate of J03VD4	J03VH0	7/27/2005	6.0E-01		9.E-03	1.095E+00		2.4E-01
A2	J03VD5	7/27/2005	4.9E-01		9.E-03	1.141E+00		1.8E-01
A3	J03VD6	7/27/2005	4.9E-01		9.E-03	1.145E+00		2.0E-01
A4	J03VD7	7/27/2005	5.1E-01		9.E-03	1.171E+00		1.9E-01
B5	J03VD8	7/27/2005	6.1E-01		1.E-02	1.449E+00		2.1E-01
B6	J03VD9	7/27/2005	5.4E-01		9.E-03	1.204E+00		1.8E-01
B7	J03VF0	7/27/2005	5.3E-01		9.E-03	1.133E+00		2.0E-01
B8	J03VF1	7/27/2005	6.0E-01		9.E-03	1.190E+00		1.9E-01
C9	J03VF2	7/28/2005	5.5E-01		9.E-03	1.089E+00		1.9E-01
C10	J03VF3	7/28/2005	6.5E-01		9.E-03	1.124E+00		1.7E-01
C1	J03VF4	7/28/2005	5.7E-01		9.E-03	6.92E-01		1.9E-01
C2	J03VF5	7/28/2005	5.7E-01		9.E-03	2.081E+00		1.8E-01
D3	J03VF6	7/28/2005	5.5E-01		9.E-03	1.763E+00		2.0E-01
D4	J03VF7	7/28/2005	5.2E-01		9.E-03	3.175E+00		1.9E-01
D5	J03VF8	7/28/2005	5.1E-01		9.E-03	1.646E+00		2.1E-01
D6	J03VF9	7/28/2005	5.8E-01		9.E-03	8.950E-01		1.9E-01

Statistical Computation Input Data

Sampling Area	HEIS Number	Sample Date	Beryllium mg/kg			Uranium (Total) pCi/g		
A1	J03VD4/ J03VH0	7/27/2005	5.4E-01			1.218E+00		
A2	J03VD5	7/27/2005	4.9E-01			1.141E+00		
A3	J03VD6	7/27/2005	4.9E-01			1.145E+00		
A4	J03VD7	7/27/2005	5.1E-01			1.171E+00		
B5	J03VD8	7/27/2005	6.1E-01			1.449E+00		
B6	J03VD9	7/27/2005	5.4E-01			1.204E+00		
B7	J03VF0	7/27/2005	5.3E-01			1.133E+00		
B8	J03VF1	7/27/2005	6.0E-01			1.190E+00		
C9	J03VF2	7/28/2005	5.5E-01			1.089E+00		
C10	J03VF3	7/28/2005	6.5E-01			1.124E+00		
C1	J03VF4	7/28/2005	5.7E-01			6.92E-01		
C2	J03VF5	7/28/2005	5.7E-01			2.081E+00		
D3	J03VF6	7/28/2005	5.5E-01			1.763E+00		
D4	J03VF7	7/28/2005	5.2E-01			3.175E+00		
D5	J03VF8	7/28/2005	5.1E-01			1.646E+00		
D6	J03VF9	7/28/2005	5.8E-01			8.950E-01		

Statistical Computations

	Beryllium			Uranium (Total)		
Statistical value based on	Large data set (n >10), use MTCAStat lognormal distribution.			Radionuclide data set. Use nonparametric z-stat.		
N	16			16		
% < Detection limit	0%			0%		
mean	5.5E-01			1.382E+00		
st. dev.	4.5E-02			5.83E-01		
Z-statistic	1.645			1.645		
95% UCL on mean	5.7E-01			1.622E+00		
max value	6.5E-01			3.175E+00		
Statistical value	5.7E-01			1.622E+00		
Background	NA			2.27		
Statistical value above background	5.7E-01			0 (< BG)		
Most Stringent Industrial Use Cleanup Limit for nonradionuclide and RAG type	104 [a]		Direct Exposure			
3-PART TEST						
95% UCL > Cleanup Limit?	NA					
> 10% above Cleanup Limit?	NA					
Any sample > 2X Cleanup Limit?	NA					
RISK EVALUATION						
WAC 173-340 Non-Carcinogenic Cleanup:	7,000					
Hazard quotient for each nonradionuclide:	NA					
WAC 173-340 Carcinogenic Cleanup:	104					
Risk for each carcinogenic nonradionuclide:	NA					
WAC 173-340 Compliance? NA	Because all beryllium values are below background (1.51 mg/kg), performance of the 3-part test and calculation of excess risk are not required.					
Nonrad noncarcinogenic index sum: NA						
Nonrad carcinogenic risk: NA						

a = Based on the generic site RESRAD assessment included in the RDR/RAWP (DOE/RL-2001-47), as well as numerous site-specific assessments, these contaminants will not migrate to groundwater or the river and are therefore not a threat to groundwater or the river. For the shallow zone, the direct exposure criteria is the most stringent cleanup criteria for this contaminant.

BG = background
HEIS = Hanford Environmental Information System
MDA = minimum detectable activity
NA = not applicable
PQL = practical quantitation limit
Q = qualifier
RAG = remedial action goal
WAC = *Washington Administrative Code*

CALCULATION SHEET

Washington Closure Hanford

Originator J. M. Capron JMC **Date** 09/06/05 **Calc. No.** 0300X-CA-V0056 **Rev. No.** 0
Project 300 Area Remedial Action **Job No.** 14655 **Checked** T. M. Blakley TMB **Date** 9/8/05
Checked T. B. Miley TBM **Date** 9/13-05
Subject 300-8 Waste Site Cleanup Verification 95% UCL Calculation **Sheet No.** 4 of 5

Split/Duplicate Analysis:

Shallow Zone

Composite Area	HEIS Number	Beryllium mg/kg	Q	Beryllium PQL	Uranium (Total) pCi/g	Q	Uranium (Total) MDA
A1	J03VD4	4.7E-01		9.0E-03	1.341E+00		1.9E-01
Duplicate of J03VD4	J03VH0	6.0E-01		9.0E-03	1.095E+00		2.4E-01
Split of J03VD4	J03VH1	5.3E-01		5.0E-01	1.487E+00		1.17E-01

Shallow Zone Analysis:

TDL		0.5	1
Duplicate Analysis	Both >PQL/MDA?	**Yes (continue)**	**Yes (continue)**
	Both >5xTDL?	No-Stop (acceptable)	No-Stop (acceptable)
	RPD		
Split Analysis	Both >PQL/MDA?	**Yes (continue)**	**Yes (continue)**
	Both >5xTDL?	No-Stop (acceptable)	No-Stop (acceptable)
	RPD		

HEIS = Hanford Environmental Information System
MDA = minimum detectable activity
PQL = practical quantitation limit
Q = qualifier
RPD = relative percent difference
TDL = target detection limit

CALCULATION SHEET

Washington Closure Hanford

Originator J. M. Capron **Date** 09/06/05 **Calc. No.** 0300X-CA-V0056 **Rev. No.** 0
Project 300 Area Remedial Action **Job No.** 14655 **Checked** T. M. Blakley **Date** 9/8/05
Subject 300-8 Waste Site Cleanup Verification 95% UCL Calculation **Checked** T. B. Miley **Date** 9-13-05
Sheet No. 5 of 5

Ecology Software (MTCAStat) Results

Beryllium 95% UCL Calculation

DATA	ID					
5.4E-01	J03VD4/J03VH0					
4.9E-01	J03VD5					
4.9E-01	J03VD6	Number of samples			Uncensored values	
5.1E-01	J03VD7	Uncensored	16		Mean	0.55
6.1E-01	J03VD8	Censored			Lognormal mean	0.55
5.4E-01	J03VD9	Detection limit or PQL			Std. devn.	0.04
5.3E-01	J03VF0	Method detection limit			Median	0.55
6.0E-01	J03VF1	TOTAL	16		Min.	0.49
5.5E-01	J03VF2				Max.	0.65
6.5E-01	J03VF3					
5.7E-01	J03VF4	Lognormal distribution?		Normal distribution?		
5.7E-01	J03VF5	r-squared is:	0.973	r-squared is:	0.961	
5.5E-01	J03VF6	Recommendations:				
5.2E-01	J03VF7	Use lognormal distribution.				
5.1E-01	J03VF8					
5.8E-01	J03VF9	UCL (Land's method) is	0.57			

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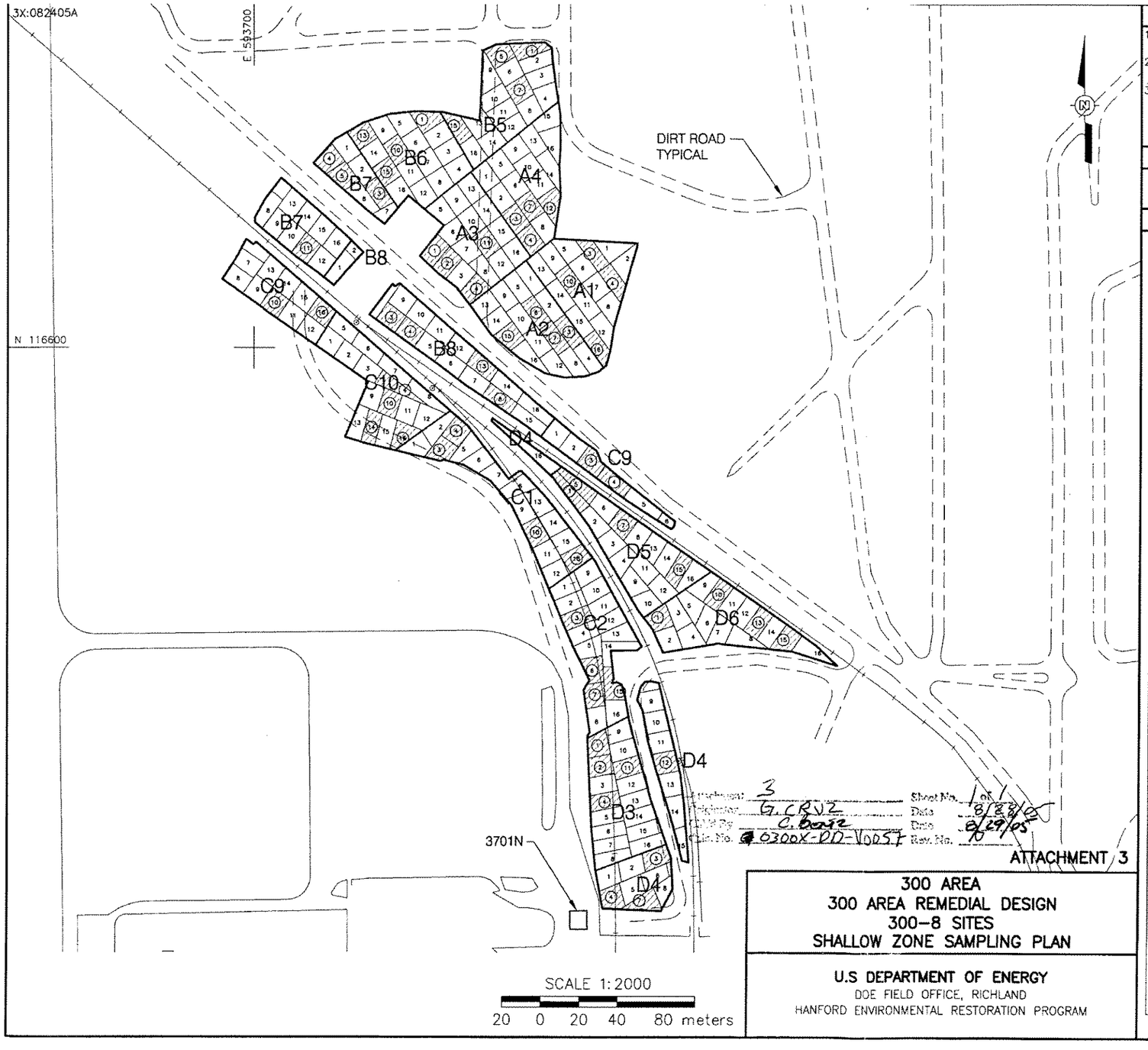

NOTES

1. SHALLOW ZONE NODE AREAS ARE APPROXIMATELY 127.65 SQUARE METERS.
2. SAMPLES ARE TAKEN FROM THE APPROXIMATE CENTER OF EACH NODE.
3. THE SHALLOW ZONE CONSISTS OF SAMPLING AREAS A1, A2, A3, AND A4 WITHIN DECISION SUBUNIT 1. AREAS B5, B6, B7, AND B8 WITHIN DECISION SUBUNIT 2. AREAS C9, C10, C1, AND C2 WITHIN DECISION SUBUNIT 3. AREAS D3, D4, D5, AND D6 WITHIN DECISION SUBUNIT 4.

LEGEND

(8) CLEAN UP VERIFICATION SAMPLING NODE

SAMPLE LOCATION TABLE

DECISION SUBUNIT	SAMPLING AREA	SAMPLE NODE	NORTHING	EASTING
1	A1	S-A1-3	116648.21	593873.60
		S-A1-4	116633.00	593885.52
		S-A1-10	116634.07	593863.21
		S-A1-16	116599.23	593877.59
	A2	S-A2-3	116608.21	593862.81
		S-A2-6	116618.08	593845.93
		S-A2-7	116605.07	593855.52
		S-A2-15	116606.02	593831.33
	A3	S-A3-1	116649.58	593793.59
		S-A3-2	116643.22	593800.10
		S-A3-4	116630.19	593815.09
		S-A3-11	116653.67	593820.09
	A4	S-A4-3	116665.80	593835.18
		S-A4-4	116655.05	593842.92
		S-A4-7	116672.17	593842.53
		S-A4-12	116671.34	593852.70
2	B5	S-B5-1	116751.11	593843.47
		S-B5-5	116748.37	593827.65
		S-B5-7	116731.21	593837.07
		S-B5-15	116713.12	593803.33
	B6	S-B6-1	116716.24	593786.87
		S-B6-10	116700.65	593773.87
		S-B6-13	116708.03	593756.17
		S-B6-15	116689.65	593768.40
	B7	S-B7-3	116678.71	593764.63
		S-B7-4	116696.38	593738.69
		S-B7-5	116687.58	593745.28
		S-B7-11	116650.89	593726.79
	B8	S-B8-3	116615.49	593771.08
		S-B8-4	116608.23	593781.03
		S-B8-8	116573.77	593827.36
		S-B8-13	116590.55	593818.30
3	C9	S-C9-3	116542.64	593874.53
		S-C9-4	116531.68	593886.69
		S-C9-10	116623.01	593710.62
		S-C9-16	116618.11	593734.94
	C10	S-C10-4	116578.24	593778.31
		S-C10-10	116571.48	593770.38
		S-C10-14	116559.37	593761.00
		S-C10-16	116553.73	593777.29
	C1	S-C1-3	116547.83	593796.12
		S-C1-4	116557.51	593804.89
		S-C1-10	116506.15	593846.03
		S-C1-16	116492.90	593867.21
	C2	S-C2-3	116462.77	593867.49
		S-C2-6	116436.00	593875.52
		S-C2-7	116423.70	593877.05
		S-C2-15	116425.35	593890.05
4	D3	S-D3-1	116397.76	593878.44
		S-D3-2	116386.78	593879.95
		S-D3-4	116369.15	593882.31
		S-D3-11	116386.39	593894.16
	D4	S-D4-3	116340.59	593909.14
		S-D4-4	116321.26	593886.08
		S-D4-7	116319.61	593900.55
		S-D4-12	116389.13	593913.84
	D5	S-D5-1	116527.04	593863.32
		S-D5-5	116530.42	593866.75
		S-D5-7	116509.75	593891.47
		S-D5-15	116487.36	593920.37
	D6	S-D6-1	116463.08	593909.33
		S-D6-10	116474.74	593940.49
		S-D6-13	116460.51	593961.12
		S-D6-15	116451.79	593974.18

Attachment 3
Sheet No. 1 of 1
Drawn By G.CRUZ Date 8/28/05
Checked By C.Bonz Date 8/29/05
Eng. No. 0300X-DD-V0057 Rev. No. 0

ATTACHMENT 3

300 AREA
300 AREA REMEDIAL DESIGN
300-8 SITES
SHALLOW ZONE SAMPLING PLAN

U.S DEPARTMENT OF ENERGY
DOE FIELD OFFICE, RICHLAND
HANFORD ENVIRONMENTAL RESTORATION PROGRAM

uranium as the primary radiological contaminant. Results of the surveys are depicted on a map based on various ranges of detected uranium activity (Figure 5), with <50 pCi/g being the lowest reported range based on instrument sensitivity. Locations where survey results indicated uranium activities >50 pCi/g were investigated further in the field by radiological control technicians assigned to the project. Contaminated items identified by the technicians during the field investigation were hand-removed for disposal at the ERDF. Results from the radiological surveys provided an initial indication that residual soil concentrations of uranium were statistically below the applicable cleanup criteria.

3.3 BIASED SAMPLING AND ANALYSIS

Biased samples are typically collected at locations where significant quantities of specific waste streams were unearthed from a common area to help verify the absence of hot spots in the residual soil. At the 300-8 waste site, aluminum shavings were spread throughout the excavation rather than being concentrated in any discrete area. No containerized liquid was found, and no evidence of historical liquid disposal was identified during the excavation. Consequently, it was determined that radiological surveys and statistical verification sampling would be adequate for site closeout, and biased samples were not collected as per the approved closeout plan (WCH 2005b).

3.4 CLEANUP VERIFICATION SAMPLING AND ANALYSIS

Final cleanup verification samples were collected on July 27 and 28, 2005, to confirm acceptability of residual contaminant concentrations in soil at the 300-8 waste site. Based on the overall footprint of the area and depth of excavation, the 300-8 waste site was classified as four shallow zone decision units. The final verification samples were submitted to offsite laboratories for analysis using approved U.S. Environmental Protection Agency analytical methods as described in the SAP (DOE-RL 2004a).

In accordance with the SAP (DOE-RL 2004a), each verification sample was collected as a composite sample formed by combining soil collected at four random locations within the sampling area (excluding the quality assurance/quality control samples). The sample design methodology and sample location figures are presented in the calculation brief for sample design in Appendix C.

4.0 CLEANUP VERIFICATION DATA EVALUATION

This section presents the evaluation and modeling of the 300-8 waste site cleanup verification data for comparison with the data quality criteria and RAGs.

4.1 DATA QUALITY ASSESSMENT PROCESS

A data quality assessment (DQA) is performed to compare the verification sampling approach and resulting analytical data with the sampling and data quality requirements specified by the project objectives and performance specifications.

The DQA for the 300-8 waste site determined that the data are of the right type, quality, and quantity to support site verification decisions within specified error tolerances. All analytical data were found to be acceptable for decision-making purposes.
The evaluation also found that the sample design was sufficient to support clean site verification. The cleanup verification sample analytical data are stored in the Hanford Environmental Information System and are summarized in Appendix A. The detailed DQA is presented in Appendix B.

4.2 CONTAMINANTS OF CONCERN 95% UPPER CONFIDENCE LIMIT

The primary statistical calculation to support cleanup verification is the 95% upper confidence limit (UCL) on the arithmetic mean of the data. Prior to calculating the 95% UCL, the individual sample results are reviewed and, as appropriate, adjusted per the SAP (DOE-RL 2004a). This process is summarized below.

- **Radionuclides:** The laboratory-reported value is used in the calculation of the 95% UCL. In cases where the laboratory does not report a value for data qualified with a "U" (i.e., less than the detection limit), half of the minimum detectable activity is used in the calculation of the 95% UCL.

- **Nonradionuclides:** For data flagged with a "U" (i.e., less than detection), a value equal to one-half the practical quantitation limit is used in the calculation of the 95% UCL, consistent with Washington State Department of Ecology regulations (*Washington Administrative Code* [WAC] 173-340-740[7][g]). If greater than half of the sample results for a given nonradionuclide COC are below detection, the statistical value is set equal to the maximum concentration detected (i.e., versus computing a 95% UCL).

Statistical calculations are presented in the 300-8 waste site cleanup verification 95% UCL calculation brief (Appendix C). Verification sampling summary statistics (95% UCL values) are listed in Table 2. The columns on the left side of Table 2 are the COCs and the 95% UCL values before subtraction of background. The third column of Table 2 presents the background, where values exist, and the last column presents the statistical values adjusted for background, if appropriate, which become the cleanup verification data set used for evaluation against RAGs. Typically, Hanford Site background concentration values are subtracted only for uranium.

4.3 SITE-SPECIFIC CLEANUP VERIFICATION MODEL

A site-specific vadose zone model was not developed for the 300-8 site, as the cleanup verification data set statistical values were all determined to be below statistical background levels, as shown in Table 2.

4.4 RESRAD MODELING

A site-specific RESidual RADioactivity (RESRAD) model was not developed for the 300-8 waste site, as the statistical value for total uranium presented in Table 2 was determined to be below the statistical background level as reported in *Hanford Site Background: Part 2, Soil Background for Radionuclides* (DOE-RL 1996).

Table 2. Cleanup Verification Data Set.

COCs	Shallow Zone 95% UCL Statistical Values	Hanford Site Background	Shallow Zone Cleanup Verification Data Set[a]
Radionuclide Concentration (pCi/g)[b]			
Uranium (total)	1.622	2.27[c]	0 (<BG)
Nonradionuclide Concentration (mg/kg)[b]			
Beryllium	0.57	1.51[d]	0.57 (<BG)

[a] For overburden, anthropogenic background (DOE-RL 1996) and naturally occurring background is subtracted from all radionuclides. For other decision units (e.g., shallow zone and deep zone), naturally occurring background (uranium) is subtracted. Refer to the 95% UCL calculation brief in Appendix C for additional details on determination of statistical values.
[b] Laboratory data, including the minimum detectable activity or practical quantitation limit for the individual cleanup verification samples, are included in Appendix A and the 95% UCL calculation brief in Appendix C.
[c] Value published in *Hanford Site Background: Part 2, Soil Background for Radionuclides* (DOE-RL 1996).
[d] Value published in *Hanford Site Background: Part 1, Soil Background for Nonradioactive Analytes* (DOE-RL 2001).
BG = background
COC = contaminant of concern
UCL = upper confidence limit

5.0 EVALUATION OF REMEDIAL ACTION GOAL ATTAINMENT FOR INDUSTRIAL LAND USE

This section demonstrates that remedial actions at the 300-8 waste site have achieved the RAGs developed to support industrial land use as documented in the RDR/RAWP (DOE-RL 2004b).

5.1 DIRECT EXPOSURE SOIL REMEDIAL ACTION GOALS ATTAINED

5.1.1 Radionuclides

5.1.1.1 Direct Comparison to RAGs. The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the direct exposure RAG of 350 pCi/g, the concentration corresponding to a 15 mrem/yr excess dose (DOE-RL 2004b). No other radionuclide COCs were identified for the 300-8 waste site.

5.1.1.2 Radionuclide Risk. Residual concentrations of total uranium at the 300-8 waste site were detected below the statistical background value and therefore do not contribute to residual excess carcinogenic risk for the site.

5.1.2 Nonradionuclides

5.1.2.1 Direct Comparison to RAGs. Table 3 compares the cleanup verification data set statistical value for beryllium presented in Table 2 to the direct exposure RAG presented in Table 1. The statistical value is less than the corresponding statistical background level and the RAG.

Table 3. Attainment of Nonradionuclide Direct Exposure Standards – Industrial Land Use.

Nonradionuclides	RAG (mg/kg)[a]	Shallow Zone Verification Data Set Values (mg/kg)	Direct Exposure RAG Attained?[b]
Beryllium	104	0.57	Yes

[a]Listed value for industrial land use as presented in *Remedial Design Report/Remedial Action Work Plan for the 300 Area* (DOE-RL 2004b).
[b]Criterion is comparison to direct exposure RAG.
RAG = remedial action goal

5.1.2.2 Noncarcinogenic Hazard Quotient RAG Attained. For noncarcinogenic COCs, WAC 173-340-740(5)(a) and (b) specify the evaluation of the hazard quotient, which is given as daily intake divided by a reference dose (DOE-RL 2001). The hazard quotient for beryllium (the only nonradionuclide COC) was not calculated because the associated statistical verification value was less than the statistical background value within the shallow zone.

5.1.2.3 Carcinogenic Risk RAG Attained. For individual nonradionuclide carcinogenic COCs, the WAC 173-340-745(4)(a)(iii) Method C cleanup limits are based on an industrial land-use incremental cancer risk of 1×10^{-5}. The cumulative excess cancer risk for all nonradionuclide carcinogenic COCs must also be less than 1×10^{-5} (WAC 173-340). The only nonradionuclide carcinogenic COC at the 300-8 waste site

was beryllium, which was detected at less than the applicable background value. Consequently, an excess cancer risk value was not calculated.

5.2 GROUNDWATER REMEDIAL ACTION GOALS ATTAINED

5.2.1 Radionuclides

The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the RAG for the protection of groundwater (267 pCi/g), as calculated by RESRAD based on the exposure scenario (DOE-RL 2004b). No other radionuclide COCs were identified for the 300-8 waste site.

5.2.2 Nonradionuclides

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, is not predicted to reach groundwater within 1,000 years based on a generic site profile for the 300 Area (DOE-RL 2004b). Further, beryllium was not detected above its statistical background level in the cleanup verification data set, as shown in Table 2.

5.3 COLUMBIA RIVER REMEDIAL ACTION GOALS ATTAINED

5.3.1 Radionuclides

The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the RAG for the protection of the Columbia River (267 pCi/g), as calculated by RESRAD based on the exposure scenario and the maximum contaminant level (DOE-RL 2004b). No other radionuclide COCs were identified for the 300-8 waste site.

5.3.2 Nonradionuclides

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, is not predicted to reach groundwater, and thus the Columbia River, within 1,000 years based on a generic site profile for the 300 Area (DOE-RL 2004b). Further, beryllium was not detected above its statistical background level in the cleanup verification data set, as shown in Table 2.

5.4 WAC 173-340 THREE-PART TEST FOR NONRADIONUCLIDES

The WAC 173-340-740(7)(e) three-part test is applicable to nonradionuclide COCs and consists of the following criteria: (1) the cleanup verification statistical value must be less than the cleanup level, (2) no single detection can exceed two times the cleanup criteria, and (3) the percentage of samples exceeding the cleanup criteria must be less than 10%. The most restrictive RAG (defined as the lowest of the direct exposure, groundwater protection, and river protection RAGs) is used for the test.

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, was detected at levels less than its statistical background value. Consequently, the WAC 173-340-740(e) three-part test was not performed.

6.0 EVALUATION OF REMEDIAL ACTION GOAL ATTAINMENT FOR UNRESTRICTED LAND USE

The information presented in the previous section demonstrates that the cleanup objectives established in the ROD (EPA 2001) for industrial land use have been achieved. In addition, residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 300-8 waste site. The information presented in this section evaluates the remedial action results against cleanup criteria established for unrestricted land use to be implemented at selected sites in the 300-FF-2 Operable Unit through the *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision* (ESD) (EPA 2004).

The 300 Area unrestricted land-use scenario is represented by an individual in a rural-residential setting. The exposure pathways considered in estimating dose from radionuclides in soil are inhalation; soil ingestion; ingestion of crops, meat, fish, drinking water, and milk; and external gamma exposure. This individual is conservatively assumed to spend 80% of his/her lifetime onsite. It is assumed that drinking water and irrigation water are obtained from groundwater, as impacted by the waste site.

Unrestricted land-use cleanup levels for chemicals or nonradionuclides are based on WAC 173-340-740(3), which assumes that the exposure pathway for residual contamination will be from ingestion of contaminated soil. Soil cleanup levels are calculated using the equations provided by WAC 173-340-740(3) for carcinogens and for noncarcinogens. For both carcinogens and noncarcinogens, the calculations assume that a resident with an average body weight 16 kg (35 lb) over the period of exposure ingests soil at a rate of 200 mg/day (73 g/yr [2.6 oz/yr]), with a frequency of contact of 100% and a gastrointestinal absorption rate of 100%. For carcinogens, the calculation is based on achieving a lifetime cancer risk goal of 1 in 1,000,000 (1×10^{-6}) for an exposure duration of 6 years and a lifetime of 75 years. For noncarcinogens, the calculation is based on achieving a hazard quotient of 1.

The key assumptions in the 300 Area unrestricted land-use scenario that affect groundwater protection are irrigation at agronomic rates (76 cm/yr [30 in./yr]), surface vegetation resulting in an evapotranspiration coefficient of 91%, and inclusion of drinking water ingestion as an exposure pathway. Details of this land-use scenario and associated RAGs are documented in the ESD (EPA 2004).

A comparison of the 300-8 waste site cleanup verification data set to the cleanup objectives for unrestricted land use as established in the ESD (EPA 2004) is presented in the following section.

6.1 DIRECT EXPOSURE SOIL REMEDIAL ACTION GOALS ATTAINED

6.1.1 Radionuclides

6.1.1.1 Direct Comparison to RAGs. The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the direct exposure RAG of 56 pCi/g, the concentration corresponding to a 15 mrem/yr excess dose (EPA 2004). No other radionuclide COCs were identified for the 300-8 waste site.

6.1.1.2 Radionuclide Risk. Residual concentrations of total uranium at the 300-8 waste site were detected below the statistical background value and therefore do not contribute to residual excess carcinogenic risk for the site.

6.1.2 Nonradionuclides

6.1.2.1 Direct Comparison to RAGs. Table 4 compares the cleanup verification data set statistical value for beryllium presented in Table 2 to the direct exposure RAG for unrestricted land use. The statistical value is less than the corresponding statistical background level and the RAG.

Table 4. Attainment of Nonradionuclide Direct Exposure Standards – Unrestricted Land Use.

Nonradionuclides	RAG (mg/kg)[a]	Shallow Zone Verification Data Set Values (mg/kg)	Direct Exposure RAG Attained?[b]
Beryllium	10.4	0.57	Yes

[a]Listed value for unrestricted land use as presented in *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision* (EPA 2004).
[b]Criterion is comparison to direct exposure RAG.
RAG = remedial action goal

6.1.2.2 Noncarcinogenic Hazard Quotient. For noncarcinogenic COCs, WAC 173-340-740(5)(a) and (b) specify the evaluation of the hazard quotient, which is

given as daily intake divided by a reference dose (DOE-RL 2001). The hazard quotient for beryllium (the only nonradionuclide COC) was not calculated because the associated statistical verification value was less than the statistical background value within the shallow zone.

6.1.2.3 Carcinogenic Risk. For individual nonradionuclide carcinogenic COCs, the WAC 173-340-750(3) Method B cleanup limits are based on an unrestricted land-use incremental cancer risk of 1×10^{-6}. The cumulative excess cancer risk for all nonradionuclide carcinogenic COCs must also be less than 1×10^{-5} (WAC 173-340). The only nonradionuclide carcinogenic COC at the 300-8 waste site was beryllium, which was detected at less than the applicable background value. Consequently, an excess cancer risk value was not calculated.

6.2 GROUNDWATER REMEDIAL ACTION GOALS ATTAINED

6.2.1 Radionuclides

The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the RAG for the protection of groundwater (37 pCi/g), as calculated by RESRAD based on the exposure scenario (EPA 2004). No other radionuclide COCs were identified for the 300-8 waste site.

6.2.2 Nonradionuclides

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, is not predicted to reach groundwater within 1,000 years based on a generic site profile for the 300 Area (DOE-RL 2004b). Further, beryllium was not detected above its statistical background level in the cleanup verification data set, as shown in Table 2.

6.3 COLUMBIA RIVER REMEDIAL ACTION GOALS ATTAINED

6.3.1 Radionuclides

The cleanup verification statistical value for total uranium (1.622 pCi/g) is below the statistical background level (2.27 pCi/g) and meets the RAG for the protection of the Columbia River (74 pCi/g), as calculated by RESRAD based on the exposure scenario (DOE-RL 2004b). No other radionuclide COCs were identified for the 300-8 waste site.

6.3.2 Nonradionuclides

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, is not predicted to reach groundwater, and thus the Columbia River, within 1,000 years based on a generic site profile for the 300 Area (DOE-RL 2004b). Further, beryllium was not detected above its statistical background level in the cleanup verification data set, as shown in Table 2.

6.4 WAC 173-340 THREE-PART TEST FOR NONRADIONUCLIDES

Beryllium, the sole nonradionuclide COC for the 300-8 waste site, was detected at levels less than its statistical background value. Consequently, the WAC 173-340-740(e) three-part test was not performed.

7.0 STATEMENT OF PROTECTIVENESS

This cleanup verification package demonstrates that remedial action at the 300-8 waste site has achieved the RAOs and corresponding RAGs established in the ROD (EPA 2001) and RDR/RAWP (DOE-RL 2004b). The contaminated materials from the site have been excavated and disposed at ERDF. Results of post-remediation geophysical surveys demonstrate that only trace levels of metallic debris remain at the site. The remaining soil at the 300-8 site has been sampled, analyzed, and evaluated. Results indicate that the site supports future land uses that can be represented (or bounded) by the industrial land-use scenario and poses no threat to groundwater or the Columbia River. Consequently, the 300-8 waste site is verified to be remediated in accordance with the ROD.

Because residual soil concentrations indicated that cleanup levels for more stringent land uses may have been achieved for the 300-8 waste site, a supplemental evaluation was performed against the unrestricted land-use RAGs established for the 300 Area in the ESD (EPA 2004). This evaluation demonstrated that the results of verification sampling do not preclude any future uses (as bounded by the rural-residential scenario) and allow unrestricted use of shallow zone soils. In consideration of this and because the site has no deep zone, no institutional controls are required at the 300-8 waste site.

8.0 REFERENCES

40 CFR 141, "National Primary Drinking Water Regulations," *Code of Federal Regulations,* as amended.

65 FR 76708, "National Primary Drinking Water Regulations; Radionuclides; Final Rule," *Federal Register*, Vol. 65, No. 236, p. 76708, December 7, 2000.

BHI, 2001, *Calculation of Total Uranium Activity Corresponding to a Maximum Contaminant Level for Total Uranium of 30 Micrograms per Liter in Groundwater*, 0100X-CA-V0038, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

Comprehensive Environmental Response, Compensation, and Liability Act of 1980, 42 U.S.C. 601, et seq.

DOE Order 5400.5, *Radiation Protection of the Public and the Environment*, U.S. Department of Energy, Washington, D.C.

DOE-RL, 1996, *Hanford Site Background: Part 2, Soil Background for Radionuclides*, DOE/RL-96-12, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 1997, *Limited Field Investigation Report for the 300-FF-2 Operable Unit*, DOE/RL-96-42, Rev. 0, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 1998, *Tri-Party Agreement Handbook Management Procedures*, RL-TPA-90-0001, Guideline Number TPA-MP-14, "Maintenance of the Waste Information Data System (WIDS)," U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 2001, *Hanford Site Background: Part 1, Soil Background for Nonradioactive Analytes*, DOE/RL-92-24, Rev. 4, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 2004a, *300 Area Remedial Action Sampling and Analysis Plan*, DOE/RL-2001-48, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 2004b, *Remedial Design Report/Remedial Action Work Plan for the 300 Area*, DOE/RL-2001-47, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

Ecology, EPA, and DOE, 1989, *Hanford Federal Facility Agreement and Consent Order*, 2 vols., as amended, Washington State Department of Ecology, U.S. Environmental Protection Agency, and U.S. Department of Energy, Olympia, Washington.

EPA, 2001, *Record of Decision for the 300-FF-2 Operable Unit, Hanford Site*, U.S. Environmental Protection Agency, Region 10, Seattle, Washington.

EPA, 2004, *Explanation of Significant Differences for the 300-FF-2 Operable Unit Record of Decision*, U.S. Environmental Protection Agency, Region 10, Seattle, Washington.

WAC 173-340, 1996, "Model Toxics Control Act – Cleanup" *Washington Administrative Code*.

WCH, 2005a, *300-8 Aluminum Shavings Area Geophysical Survey*, CCN 122368, Washington Closure Hanford, Richland, Washington.

WCH, 2005b, *Closeout Plan for Waste Site 300-8*, CCN 123843, Washington Closure Hanford, Richland, Washington.

APPENDIX A

SUMMARY OF VERIFICATION SOIL SAMPLING AND ANALYTICAL RESULTS

Table A-1. 300-8 Shallow Zone Cleanup Verification Data.

Sampling Area	HEIS Number	Sample Date	Beryllium			Uranium (Total)[a]		
			mg/kg	Q	PQL	pCi/g	Q	MDA
A1	J03VD4	7/27/05	4.7E-01		9E-03	1.341E+00		1.9E-01
Duplicate of J03VD4	J03VH0	7/27/05	6.0E-01		9E-03	1.095E+00		2.4E-01
Split of J03VD4	J03VH1	7/27/05	5.3E-01		5.0E-01	1.487E+00		1.17E-01
A2	J03VD5	7/27/2005	4.9E-01		9.E-03	1.141E+00		1.8E-01
A3	J03VD6	7/27/2005	4.9E-01		9.E-03	1.145E+00		2.0E-01
A4	J03VD7	7/27/2005	5.1E-01		9.E-03	1.171E+00		1.9E-01
B5	J03VD8	7/27/2005	6.1E-01		1.E-02	1.449E+00		2.1E-01
B6	J03VD9	7/27/2005	5.4E-01		9.E-03	1.204E+00		1.8E-01
B7	J03VF0	7/27/2005	5.3E-01		9.E-03	1.133E+00		2.0E-01
B8	J03VF1	7/27/2005	6.0E-01		9.E-03	1.190E+00		1.9E-01
C9	J03VF2	7/28/2005	5.5E-01		9.E-03	1.089E+00		1.9E-01
C10	J03VF3	7/28/2005	6.5E-01		9.E-03	1.124E+00		1.7E-01
C1	J03VF4	7/28/2005	5.7E-01		9.E-03	6.92E-01		1.9E-01
C2	J03VF5	7/28/2005	5.7E-01		9.E-03	2.081E+00		1.8E-01
D3	J03VF6	7/28/2005	5.5E-01		9.E-03	1.763E+00		2.0E-01
D4	J03VF7	7/28/2005	5.2E-01		9.E-03	3.175E+00		1.9E-01
D5	J03VF8	7/28/2005	5.1E-01		9.E-03	1.646E+00		2.1E-01
D6	J03VF9	7/28/2005	5.8E-01		9.E-03	8.950E-01		1.9E-01

[a] Reported total uranium value is based on summation of laboratory-reported isotopic uranium concentrations. Reported MDA is calculated as the average of laboratory-reported isotopic MDA values.

HEIS = Hanford Environmental Information System
MDA = minimum detectable activity
PQL = practical quantitation limit
Q = qualifier

APPENDIX B

DATA QUALITY ASSESSMENT

APPENDIX B

DATA QUALITY ASSESSMENT FOR THE 300-8 WASTE SITE

B1.1 OVERVIEW

This DQA was performed in accordance with BHI-EE-01, *Environmental Investigations Procedures*. Specific data quality objectives for the site are found in the *300 Area Remedial Action Sampling and Analysis Plan* (SAP) (DOE-RL 2004a). The DQA is based on the guidelines presented in *Guidance for Data Quality Assessment* (EPA 2000). Statistical tests used in this DQA were performed as specified in the SAP and the *Remedial Design Report/Remedial Action Work Plan for the 300 Area* (RDR/RAWP) (DOE-RL 2004b). This DQA involves the scientific and statistical evaluations to determine if the data are of the right type, quality, and quantity to support the intended use (i.e., closeout decisions [EPA 2000]). This DQA completes the data life cycle (i.e., planning, implementation, and assessment) that was initiated by the data quality objectives process.

Prior to performing statistical tests, the field logbook (BHI 2005a), sample design, and sample analytical data are evaluated. A portion of the cleanup verification sample analytical data are validated for compliance requirements (DOE-RL 2004b). Data evaluation is performed to determine if the laboratory carried out all steps required by the SAP (DOE-RL 2004a) and the laboratory contract governing the conduct of the analysis and reporting of the data. This assessment also examines the available laboratory data to determine what analytes are present or absent in a sample and the degree of overall uncertainty associated with that determination. Data validation is done in accordance with validation procedures (BHI 2000a, 2000b) as part of data evaluation. After data evaluation and validation, the appropriate statistical test is performed on the adjusted raw analytical data (see calculation briefs in Appendix C) to determine statistical values for each contaminant. The cleanup verification sample analytical data are stored in the Hanford Environmental Information System and are summarized in Appendix A.

B1.2 LABORATORY QUALITY ASSURANCE/QUALITY CONTROL MEASURES

All verification samples are subject to laboratory-specific quality assurance (QA) requirements, including instrument procurement, maintenance, calibration, and operation. Additional laboratory quality control (QC) checks are performed as specified by the analytical method, at a rate of once per sample delivery group (SDG), or once for every 20 samples, whichever is more frequent. Laboratory internal QC checks include the following:

- **Laboratory Contamination:** Each analytical batch contains a laboratory (method) blank (material of similar composition as the samples with known/minimal

contamination of the analytes of interest) carried through the complete analytical process. The method blank is used to evaluate false-positive results in samples due to contamination during handling at the laboratory.

- **Analytical Accuracy:** For most analyses, known quantities of representative analytes of interest (matrix spike [MS]) are added to a separate aliquot of a sample from the analytical batch. The recovery percentage of the added MS is used to evaluate analytical accuracy. For analyses not amenable to MS techniques (e.g., gamma energy analysis) or where analytical recovery is corrected via internal standards (e.g., alpha spectral analyses), accuracy is evaluated from recovery of the QC reference sample (e.g., laboratory control spike or blank spike sample).

- **Analytical Precision:** Separate aliquots removed from one or more of the same sample containers (replicate samples) are analyzed for each analytical batch. The replicate sample results (evaluated as relative percent differences [RPDs]) are used to assess analytical precision.

- **QC Reference Samples:** A QC reference sample is prepared from an independent standard at a concentration other than that used for calibration, but within the calibration range. Reference samples provide an independent check on analytical technique, methodology, and quantitation.

Laboratories are also subject to periodic and random assessments of overall performance. These assessments are performed by the Washington Closure Hanford QA group to ensure that the laboratories are performing within laboratory contract requirements.

B1.3 DATA VALIDATION

The final laboratory data package for SDG H3292 (containing all verification samples and analyses) was validated to Level C per BHI-EE-01, Procedure 2.5, "Data Package Validation Process," by a third-party validator. Level C validation procedures are specified in *Data Validation Procedure for Chemical Analysis* (BHI 2000a) and *Data Validation Procedure for Radiochemical Analysis* (BHI 2000b).
Use of level C validation procedures included the review of the following items, as appropriate, for each analytical method:

- Sample holding times
- Method blanks
- MS recovery
- Surrogate recovery
- MS/matrix spike duplicate results
- Sample replicates
- Associated batch laboratory control sample results

- Achievement of required (or contractual) detection limits (RDLs)
- Data package completeness.

The laboratory QA/QC was evaluated for precision, accuracy, completeness, and RDLs pursuant to the SAP (DOE-RL 2004a). The organization performing the data validation reported that, of the data validated, the laboratory met the standards of performance for precision (±30%), accuracy (±30%), and completeness (>90%). Comparison of the RDL with the respective MDA or PQL is discussed in Section B1.4.

The validation process did not identify any major or minor deficiencies in the sample results. Consequently, no data qualifiers were assigned to the reported results through the validation process. Additional information is provided in the associated validation reports (BHI 2005b, 2005c).

B1.4 DATA EVALUATION

The context for assessing the data includes evaluating the sample data using the statistical methodology of the SAP (DOE-RL 2004a) (included in the calculation brief excerpts in Appendix C) and a comparison of analytical results to the parameters specified in the SAP. This section summarizes the results of the comparison and presents an evaluation of the affected data.

B1.4.1 RDL Comparison

Reported analytical detection levels for nondetected analytes were compared to the RDLs specified in the SAP (DOE-RL 2005a). When detected results are obtained, evaluation of detection limits is not performed. The data validation and supplemental data evaluation noted no analyses for which the detection limits (MDA or PQL) were above SAP RDLs for nondetected analytes.

B1.4.2 Precision and Accuracy Evaluation

Analytical accuracy and precision were evaluated by examination of the percent recovery and RPD of analytical spikes (MS and/or laboratory control samples) between the main and duplicate samples. Only the contaminants of concern (COCs) detected at more than five times the detection limit are used for data analysis with respect to accuracy and precision. The RPDs for all laboratory duplicates and the recoveries for all laboratory spikes were within acceptable limits.

B1.5 FIELD QUALITY ASSURANCE/QUALITY CONTROL

Field QA/QC measures were used to assess potential sources of error and cross-contamination of soil samples that could bias results. Field QA/QC samples included the following:

- Duplicate J03VH0, associated with sample J03VD4, and
- Split J03VH1, associated with sample J03VD4.

All main and QA/QC sample results are presented in Appendix A.

B1.5.1 Field Duplicate Samples

A duplicate sample was collected to provide a relative measure of the degree of local heterogeneity in the sampling medium, unlike laboratory duplicates that are used to evaluate precision in the analytical process. The field duplicates are evaluated by computing the RPD of the duplicate samples for each COC. Only analytes with values more than five times the contractual RDLs for both the main and duplicate samples are compared. Based on these criteria, RPD analysis was not required for the 300-8 waste site verification sample duplicate pair. The 95% upper confidence limit calculation brief in Appendix C provides details on duplicate pair evaluation and RPD calculation.

B1.5.2 Field Split Samples

A split sample was collected to provide a relative measure of the degree of variability in the sampling, sample handling, and analytical techniques used by commercial laboratories. The field main and split samples are evaluated by computing the RPD of the split samples for each COC to determine the usability of the verification data. The U.S. Environmental Protection Agency Contract Laboratory Program duplicate sample comparison methodology, *USEPA Contract Laboratory Program National Functional Guidelines for Inorganic Data Review* (EPA 1994), is used as an initial test of the data from the splits. Only analytes that had values more than five times the contractual RDL for both the main and split sample were compared. Based on these criteria, RPD analysis was not required for the split pair. The 95% upper confidence limit calculation brief in Appendix C provides details on split pair evaluation and RPD calculation.

B1.6 SUITABILITY OF DATA

The DQA for the 300-8 site determined that the data are of the right type, quality, and quantity to support site cleanup verification decisions within specified error tolerances. The evaluation verified that the sample design was sufficient for the purpose of clean site verification. All analytical data were found to be acceptable for decision-making purposes and acceptable for calculating the required statistical values.

B2.0 REFERENCES

BHI, 2000a, *Data Validation Procedure for Chemical Analysis*, BHI-01435, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

BHI, 2000b, *Data Validation Procedure for Radiochemical Analysis*, BHI-01433, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

BHI, 2005a, *300-FF-1/2 Analytical Field Services Logbook*, EFL-1395-11, Bechtel Hanford, Inc., Richland, Washington.

BHI, 2005b, *Data Validation Package - Inorganics (SDG No. H3292)*, Bechtel Hanford, Inc., Richland, Washington.

BHI, 2005c, *Data Validation Package - Radiochemistry (SDG No. H3292)*, Bechtel Hanford, Inc., Richland, Washington.

BHI-EE-01, *Environmental Investigations Procedures*, Bechtel Hanford, Inc., Richland, Washington.

DOE-RL, 2004a, *300 Area Remedial Action Sampling and Analysis Plan*, DOE/RL-2001-48, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

DOE-RL, 2004b, *Remedial Design Report/Remedial Action Work Plan for the 300 Area*, DOE/RL-2001-47, Rev. 1, U.S. Department of Energy, Richland Operations Office, Richland, Washington.

EPA, 1994, *USEPA Contract Laboratory Program National Functional Guidelines for Inorganic Data Review*, EPA 540/R-94/013, U.S. Environmental Protection Agency, Washington, D.C.

EPA, 2000, *Guidance for Data Quality Assessment*, EPA QA/G-9, QA00 Update, U.S. Environmental Protection Agency, Office of Environmental Information, Washington, D.C.

APPENDIX C

CALCULATION BRIEF EXCERPTS

DISCLAIMER FOR CALCULATIONS

The attached calculations have been generated for a specific purpose and task. Use of these calculations by persons who do not have access to all pertinent facts may lead to incorrect conclusions and/or results. Before applying these calculations to your work, the underlying basis, rationale, and other pertinent information relevant to these calculations must be thoroughly reviewed with appropriate WCH officials or other authorized personnel. The WCH is not responsible for the use of a calculation not under its direct control.

CALCULATION BRIEFS

The following calculation briefs have been prepared in accordance with BHI-DE-01, *Design Engineering Procedures Manual*, EDPI-4.37-01, "Project Calculations," Bechtel Hanford, Inc., Richland, Washington.

300-8 Sites Shallow Zone Sampling Plan, 0300X-CA-V0057, Rev. 0, Bechtel Hanford, Inc., Richland, Washington.

300-8 Waste Site Cleanup Verification 95% UCL Calculation, 0300X-CA-V0056, Rev. 0, Washington Closure Hanford, Richland, Washington.

NOTE: The calculation briefs referenced in this appendix are kept in the active Washington Closure Hanford project files and are available upon request. When the project is completed, the files will be stored in a U.S. Department of Energy, Richland Operations Office, repository. Only excerpts of the calculation briefs are included in this appendix.

CALCULATION COVER SHEET

Project Title:	**300-8 Sites Sample Design**	**Job No.**	**22192**
Area	**300 Area**		
Discipline	**Environmental Engineering**	**Calc. No.**	**0300X-CA-V0057**
Subject	**300-8 Sites Shallow Zone Sampling Plan**		
Computer Program	**Excel**	**Program No.**	**Excel 2003**

The attached calculations have been generated to document compliance with established cleanup levels. These documents should be used in conjuction with other relevent documents in the administrative record.

Committed Calculation ☒ **Preliminary** ☐ **Superseded** ☐

Rev.	Sheet Numbers	Originator	Checker	Reviewer	Approval	Date
0	Cover = 1 Sht Calc = 1 Shts Attach1 = 1 Sht Attach2 = 1 Sht Attach3 = 1 Shts Total = 5 Shts	G. Cruz 8/28/05	C.A. Bentz 8/29/05	J.A. Lerch 9/6/05	M.J. Haass	9/14/05

SUMMARY OF REVISIONS

*Obtain Calc. No. from DIS

January 2003

DE01-437.03

Bechtel Hanford, Inc. **CALCULATION SHEET**

Originator G. Cruz Date 8/24/2005 Calc. No. 0300X-CA-V0057 Rev. No. 0

Project 300-8 Sites Sample Design Job No. 22192 Checked CAB Date 8/29/05

Subject 300-8 Sites Shallow Zone Sampling Plan Sheet No. 1of1

Problem:	Calculate and display required sampling nodes in concurrence with 300 Area				
	SAP DOE/RL-2001-48 Rev. 0 for verification and closure.				
Given:	-SAP (DOE/RL-2001-48 Rev. 0) requirements				
	-Shallow Sampling Area (Surface area of each zone determined from CAD program,				
	Attachment 3, Sht 1of1, CAD file 3X:082405A, 300-8 Sites Shallow Zone Sampling Plan)				
SAP Requirements:					
	-Develop a 16 node sampling grid for the sampling area				
Shallow Zone	-Use table 3-2 of the SAP to determine which four of the sixteen nodes will be sampled				
	to collect clean up verification samples				
	-Develop a 16 node sampling grid for the sampling area				
Overburden:	-Use table 3-2 of the SAP to determine which four of the sixteen nodes will be sampled				
	to collect clean up verification samples				
	-Develop a 16 node sampling grid for the sampling area				
Deep Zone:	-Use table 3-2 of the SAP to determine which four of the sixteen nodes will be sampled				
	to collect clean up verification samples				
Determination of Shallow Zone Sampling Grid:					
Shallow Zone Sampling Grid Area determined from Table 3-2, SAP					
Attachment 2, Number of Decision Subunits Based on Area (Converted to Sq Meters)					
Total Area:			32680.61	m^2	
Area of Decision Subunits (total area 4 subunits)			8170.15	m^2	
Decision Subunit divided into 4 Sampling Areas:			2042.53	m^2	
Sampling Areas divided into a 16 node grid (node numbers 1-16):			127.65	m^2	
Nodes to be Sampled (as determined from Attachment 1, Table A-1, Sample Grid Point Lookup Table)					
	See Attachment 3, Sht 1of1, 300-8 Sites Shallow Zone Sampling Plan,				
	for Sample Location Table				

Bechtel Hanford, Inc.

Originator G. Cruz Date 8/24/2005 Calc. No. 0300X-CA-V0057 Rev. No. 0

Project 300-8 Sites Sample Design Job No. 22192 Checked CJB Date 8/29/05

Subject 300-8 Sites Shallow Zone Sampling Plan Sheet No 1of1

ATTACHMENT 1

Sample Grid Point Lookup Table.

Default Plan	Sampling Area 1	Sampling Area 2	Sampling Area 3	Sampling Area 4	Sampling Area 5	Sampling Area 6	Sampling Area 7	Sampling Area 8	Sampling Area 9	Sampling Area 10
Closeout	3	6	1	4	5	1	3	3	4	16
Closeout	4	7	11	3	15	15	5	13	10	10
Closeout	16	3	2	7	7	10	11	4	3	14
Closeout	10	15	4	12	1	13	4	8	16	4
Not Sampling	2	14	5	9	13	12	8	2	14	8
Not Sampling	13	10	9	13	2	16	1	12	5	3
Not Sampling	6	1	10	8	14	4	16	5	8	6
Not Sampling	1	9	13	1	10	5	12	1	1	15
Not Sampling	9	12	7	5	6	2	6	7	15	9
Not Sampling	15	16	15	14	16	6	2	15	11	1
Not Sampling	8	13	8	10	12	11	13	14	2	12
Not Sampling	5	2	3	11	4	3	9	10	7	11
Not Sampling	7	11	14	15	11	14	14	6	13	2
Not Sampling	11	4	6	2	9	7	7	11	9	7
Not Sampling	12	8	16	16	3	8	15	9	6	13
Not Sampling	14	5	12	6	8	9	10	16	12	5

** **Note:** Grid nodes for each sampling area in each waste site should be numbered consistently, e.g., begin numbering the nodes in the northwesternmost node. Then number consecutively left to right.

Bechtel Hanford, Inc.

Originator G. Cruz Date 8/24/2005 Calc. No. 0300X-CA-V0057 Rev. No. 0

Project 300-8 Sites Sample Design Job No. 22192 Checked CAB Date 8/29/05

Subject 300-8 Sites Shallow Zone Sampling Plan Sheet No. 1of1

ATTACHMENT 2

Number of Decision Subunits Based on Area.

Site Verification Sampling Frequencies Based on Area.					
Decision Unit[a]	Waste Site Size[b]	Decision Subunits	Blocks[c]	Discrete Samples	Composite Samples
Shallow zone – 0 10 15 ft	Small: < 100,000 ft^2	1	4	16	4
	Medium: >100,000 ft^2 < 400,000 ft^2	4	16	64	16
	Large: >400,000 ft^2	8	32	128	32
Deep Zone - >15 ft	Small: < 100,000 ft^2	1	4	16	4
	Medium: >100,000 ft^2 < 400,000 ft^2	4	16	64	16
	Large: >400,000 ft^2	8	32	128	32
Overburen/layback stockpiles	Small: < 100,000 ft^2	1	4	16	4
	Medium: >100,000 ft^2 < 400,000 ft^2	4	16	64	16
	Large: >400,000 ft^2	8	32	128	32
Staging pile areas (residual soil)	Small: < 100,000 ft^2	1	4	16	4
	Medium: >100,000 ft^2 < 400,000 ft^2	4	16	64	16
	Large: >400,000 ft^2	8	32	128	32

[a] The shallow zone, deep zone, overburden stockpile, and staging pile areas each represent single decision units. The total number of decision units will vary because individual waste sites may not have a deep zone, overburden stockpile, and/or staging pile areas.
[b] Area of exposed surface after excavation or area of stockpile base (as applicable)
[c] Decision subunits are divided into four blocks to ensure that random sampling locations are not bunched together in one area