PREPRINT UCRL- 77735

CONF-751238--1

Lawrence Livermore Laboratory

SEPARATION OF TRIVALENT LANTHANIDES AND ACTINIDES BY SOLVENT EXTRACTION WITHOUT AQUEOUS COMPLEXING AGENTS

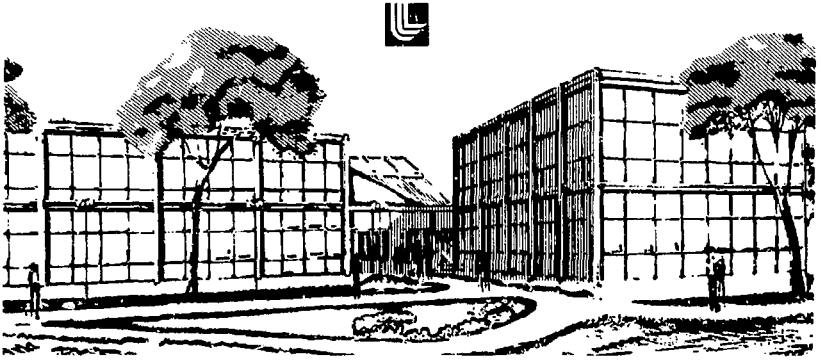
G. B. Kasting
J. A. Heppert
E. K. Hulet
J. F. Wild

January 30, 1976

This paper was prepared for submission to the Minutes of the JOWOG 20 Meeting, Los Alamos Scientific Laboratory, Los Alamos, New Mexico, December 8-10, 1975.

MASTER

This is a preprint of a paper intended for publication in a journal or proceedings. Since changes may be made before publication, this preprint is made available with the understanding that it will not be cited or reproduced without the permission of the author.



DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

SEPARATION OF TRIVALENT LANTHANIDES AND ACTINIDES BY SOLVENT EXTRACTION WITHOUT AQUEOUS COMPLEXING AGENTS*

by

G. B. Kasting**, J. A. Heppert†, E. K. Hulet, and J. F. Wild

Lawrence Livermore Laboratory, University of California
Livermore, California 94550

ABSTRACT

A method of separating the trivalent actinides, mainly Am and Cm, from trivalent lanthanides is presented. This method embodies the sequential use of two different solvent extractants; the first extractant would remove the heavy lanthanides from the lighter lanthanides and Am-Cm, while the second would extract Am-Cm in preference to the lighter lanthanides. In this scheme, no additional complexing agents are required. Thus, waste disposal and corrosion problems are minimized. Overall separation factors for Am-Cm from lanthanide fission products in reactor wastes may be as high as several thousand.

INTRODUCTION

The projected use of fission nuclear reactors to meet the power needs of the United States in the next several decades poses the serious problem of disposal of the highly radioactive waste products from fission and neutron capture in the reactors. One of the most critical aspects of this problem is long-term storage of the waste, necessitated by the need to protect the environment from the long-lived transplutonium elements generated as neutron-capture products of the nuclear fuel. Although the fission products from freshly-processed spent fuel elements present a more immediate radiation hazard, this danger decreases at a considerably faster rate than that for the α-emitting actinides. Thus, while there is a need to store the fission products safely for about a thousand years, the presence of the long-lived actinide elements in the nuclear waste doubles or triples the required storage period.

Several solutions to this problem have been suggested; among them is to separate the α-emitting species from the fission products, storing them separately or returning them to the reactors to be consumed by the (n, fission) reaction. This reaction has a considerable cross-section for the actinide elements, and would thus

*Work performed under the auspices of the U.S. Energy Research & Development Administration under contract No. W-7405-Eng-48.

**Present address: Department of Chemistry
Massachusetts Institute of Technology
77 Massachusetts Avenue
Cambridge, Mass. 02139

†Present address: Department of Chemistry
San Jose State University
San Jose, Calif.

NOTICE
This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Energy Research and Development Administration, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

transform the long-lived α-emitting nuclides into less dangerous fission products.

The chemical separation of transplutonium elements from fission products generally fails to eliminate the lanthanide fission products (primarily La to Eu) because of similarities in ionic radii and electronic structures between these two *families of elements. Thus, the separation of the* transplutonium actinides from the lanthanide elements is crucial to partitioning the short-lived from the long-lived waste products.

Current laboratory separation schemes rely *upon the slightly larger complexation strengths of* the actinides over the lanthanides, followed by partitioning of the lanthanides into an organic solution by ion-exchange or solvent extraction.[1-3] The most effective complexants for this separation *are Cl^-, SCN^-, and certain aminocarboxylic acids* such as EDTA. However, each has drawbacks when applied to large-scale systems. High concentrations of Cl^- must be used, producing very corrosive solutions and adding significantly to the bulk of the wastes.[4] Thiocyanate shows a tendency to polymerize in acid solutions, especially in the presence of α-radiation.[5,6] The separation processes employing aminocarboxylic acids (i.e., TALSPEAK and related processes)[1,3] leave large quantities of these compounds in their various waste streams, and also require a low-acidity feed solution in which hydrolysis or precipitation of some of the fission products and actinides becomes a problem.[1]

We are investigating a solvent-extraction process for the separation of Am(III) and Cm(III) from the trivalent lanthanides in which no aqueous complexants are used. Instead, two successive extractions are performed. For the first, an extractant in which Am and Cm behave like the early lanthanides is used, and for the second, a different extractant in which Am and Cm behave like middle or heavy lanthanides is used.

Numerous studies have shown that Am and Cm behave like Pr or Nd in extractions with di-(2-ethylhexyl)orthophosphoric acid (HDEHP) from dilute mineral acids.[7,8,9] The heavier lanthanides are progressively more extractable, with an average separation factor $SF_{Z/Z-1}$ of 2.4,[10] where the separation factor is defined as the ratio of

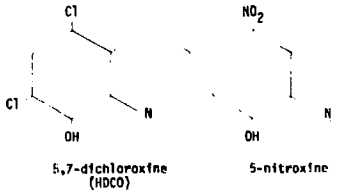


Fig. 1. Structural formulas of the extractants HDCO and 5-nitroxine.

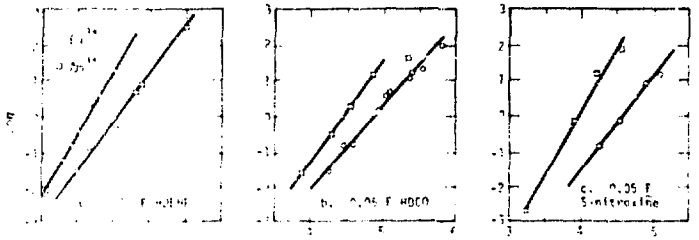
the distribution coefficients for each species between an organic and an aqueous phase. Hence, Am and Cm can be separated from the heavier lanthanides by extraction with HDEHP.

*To separate Am and Cm from the lighter lan*thanides, which contain the major lanthanide fission products, we are investigating several derivatives of 8-hydroxyquinoline (oxine) including 5,7-dichloroxine (HDCO) and 5-nitroxine (Fig. 1). Sekine and Dyrssen[11] have reported Am/Eu separation factors of 10 for extraction from dilute $HClO_4$ solutions with 0.05 F HDCO in chloroform. If the lanthanide extractabilities increase with Z as expected, Am would behave roughly like Dy, and a clean separation of Am and Cm from the lanthanide fission products should be feasible. The present work was concerned with testing the validity of the separation scheme and determining the optimum conditions for separations.

EXPERIMENTAL

The radioactive tracers used in this work for determining lanthanide-actinide distribution ratios were ^{241}Am and $^{152,154}Eu$. The Eu tracer was chosen because of its immediate availability and its identity as a medium-heavy lanthanide. The HDEHP and HDCO extractants were obtained commercially, and 5-nitroxine was graciously donated by Dr. H. Gershon, Boyce Thompson Institute for Plant Research, Yonkers, New York, who first synthesized this compound in 1972.

Stock solutions of the radioactive tracers were added to the extraction mixture immediately prior to extraction; the aqueous phases for the oxine-derivative extractions had previously been



[illegible] and Eu from 0.1 [illegible] Organic phases: [illegible] 5-nitroxoline; (b) 0.05 F HDCO in [illegible]; [illegible] 5-nitroxoline in chloroform.

[illegible] extraction [illegible] with [illegible] [illegible] [illegible]

TABLE I

...

...

...