SWELLING AND MECHANICAL PROPERTY CHANGES IN NEUTRON-IRRADIATED COLD-ROLLED TYPE 316 STAINLESS STEEL

Dr-1882

ERDA Research and Development Report

Prepared for the United States Energy Research and Development Administration, Division of Reactor Research and Development, under Contract Number AT(04-3)-824



12 19 the 200

RD

# **Rockwell International**

Atomics International Division 8900 DeSoto Avenue Canoga Park, California 91304

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

# DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency Thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

# DISCLAIMER

Portions of this document may be illegible in electronic image products. Images are produced from the best available original document.

#### NOTICE

This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Energy Research and Development Administration, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

> Printed in the United States of America Available from National Technical Information Service U.S. Department of Commerce 5285 Port Royal Road Springfield, Virginia 22151 Price: Printed Copy \$4.00 Microfiche \$2.25

#### AI-ERDA-13164 LMFBR FUELS AND MATERIALS ENGINEERING AND DEVELOPMENT UC-79b

SWELLING

AND

#### MECHANICAL PROPERTY CHANGES

IN

#### NEUTRON-IRRADIATED COLD-ROLLED

#### **TYPE 316 STAINLESS STEEL**

By K. R. GARR A. G. PARD

# **Rockwell International**

Atomics International Division 8900 DeSoto Avenue Canoga Park, California 91304 . ..

CONTRACT: AI(04-3)-824 ISSUED: OCTOBER 22, 1975

1 • 1

 $\mathbf{\hat{g}}$ 

....

## DISTRIBUTION

This report has been distributed according to the category "LMFBR Fuels and Materials Engineering and Development," as given in the Standard Distribution for Unclassified Scientific and Technical Reports, TID-4500.

#### ACKNOWLEDGMENT

We are pleased to acknowledge the assistance of E. M. Dzalak with sample preparation, J. A. Mellor with the tensile testing, and M. D. Ellis and H. Hori with part of the metallography. We thank Dr. E. E. Bloom of ORNL for his cooperation on the irradiation of the samples in EBR-II. We also thank Dr. D. W. Keefer and Mr. D. Kramer for a critical reading of the manuscript.

#### AI-ERDA-13164

# CONTENTS

		Page
Abs	tract	4
I.	Introduction	5
II.	Experimental	7
III.	Results and Discussion	9
	A. Swelling	9
	B. Metallography	10
	C. Tensile Tests	11
IV.	Conclusions	18
Refe	erences	19

## TABLES

· . · ·

1	Composition of Type 316 Stainless Steel Samples	6
±.	Composition of Type 510 Stanness Steel Samples.	
2.	Irradiation Initial and Final Temperature	6
3.	Immersion Density and TEM Swelling Results	8
4.	Summary of Tensile Data for 25% Cold-Rolled Type 316 Stainless Steel	1 <b>.</b> 7

# FIGURES

響ない。

1.	Swelling vs Irradiation Temperature for Type 316 Stainless Steel, 20 to 27% Cold-Rolled and Irradiated to a Fluence >7 x $10^{26}$ n/m <sup>2</sup> (E > 0.1 MeV)	9
2.	Pre-irradiation Microstructure of 20% Cold-Rolled Type 316 Stainless Steel	12
3.	Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 500 to $530^{\circ}$ C to a Fluence of $3.8 \times 10^{26} \text{ n/m}^2$ (E > 0.1 MeV)	13
4.	Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 600 to 620°C to a Fluence of $3.8 \times 10^{26} \text{ n/m}^2$ (E > 0.1 MeV)	14
5.	Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 500°C to a Fluence of $\sim 8 \times 10^{26} \text{ n/m}^2$ (E > 0.1 MeV)	15
6.	Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 600°C to a Fluence of $\sim 8 \ge 10^{26} \text{ n/m}^2$ (E>0.1 MeV).	16

đ

AI-ERDA-13164

#### ABSTRACT

Samples of cold-rolled Type 316 stainless steel were irradiated in EBR-II to a fluence of about  $8 \times 10^{26}$  n/m<sup>2</sup> (E>0.1 MoV) at 500 and 600°C. Three sample configurations were used - small sheet tensile samples, small right-circular cylinders for immersion density, and thin foils for transmission-electron microscopy (TEM). TEM revealed voids in the foils irradiated at both temperatures. Immersion density results, however, indicated swelling only in the sample irradiated at 600°C. Considerable recovery and precipitation were observed in foils irradiated at both temperatures. Results of tensile tests on irradiated samples showed a decrease in yield strength and an increase in ductility compared to unirradiated controls.

AI-ERDA-13164

ŝ

 $\bigcirc$ 

#### I. INTRODUCTION

High fluence swelling and mechanical properties data for cold-rolled Type 316 stainless steel is needed to provide data for design equations so that extrapolations to LMFBR goal fluence levels can be made with more confidence. The work reported here is a continuation of the EBR-II X100 irradiation experiment and extends these observations to a fluence of about  $8 \times 10^{26} \text{ n/m}^2$  (E>0.1 MeV).

|--|

COMPOSITION OF TYPE 316 STAINLESS STEEL SAMPLES

Sample Configuration	Fre-Irradiation	Composition (wt %)										
Configuration	Treatment	С	Mn	Р	S	Si	Ni	Cr	Cu	Mo	Co	Fe
Tensile	1120°C/30 min + 25% CR*	0.06	1.68	0_022	0.017	0,51	13.63	17.33	0.075	2.32	0.01	Balance
Cylinder	1120°C/30 min + 25% CR	0.037	1.51	0.021	0.015	0.42	11.56	16.65	0.13	2.39	0.13	Balance
Foil	980°C/l hr + 20% CR	0.050	1.72	0.018	0.023	0.51	13.70	17.86		2.32		Balance

\*CR = Cold rolled

## TABLE 2

#### IRRADIATION INITIAL AND FINAL TEMPERATURE

Pin	Initial Temperature (°C)	Final Temperature (°C)
Α	500	530
В	600	620
С	50C	500
D	600	535
E	500	545
F	600	600

.

•36

#### **II. EXPERIMENTAL**

Small sheet tensile samples (0.23 mm by 1.02 mm, with a gage length of 12.7 mm), small right-circular cylinders (3.8 mm diameter by 2.8 mm), and foils (3.7 mm square by 0.013 mm) of Type 316 stainless steel were irradiated in EBR-II Subassemblies X100 and X100A. The composition and pre-irradiation treatment of each type of sample are given in Table 1. These are the same alloy heats and pre-irradiation treatments as those reported earlier for Pins A and B of Subassembly X100.<sup>(1)</sup>

Data reported here are for samples irradiated in Pins C, D, E, or F to a fluence of about  $8.8 \times 10^{26} \text{ n/m}^2$  total with about  $8 \times 10^{26} \text{ n/m}^2$  (E>0.1 MeV). These fluences are tentative pending calculations by the Fast Reactor Materials Dosimetry Center (FRMDC) at HEDL. The initial irradiation temperatures and the final temperatures<sup>(2)</sup> for each pin are given in Table 2. Also included in Table 2 are the temperatures for Pins A and B.

Percent swelling was determined from immersion density measurements<sup>\*</sup> on the right-circular cylinders, and calculated from void size and concentration data for foil samples. Immersion densities for the unirradiated cylindrical samples were obtained from archive samples. Void size and concentrations in the foil samples were obtained from electron photomicrographs using a Zeiss Particle-Size Analyzer.

Tensile testing was done in a vacuum ( $\sim 7 \times 10^{-3}$  Pa) at a strain rate of  $3.3 \times 10^{-4}$  sec<sup>-1</sup>. Samples were tested at either 525 or 575°C, depending on their irradiation temperature. There was a 15-min hold prior to testing to allow for temperature stabilization.

Helium analysis was determined by mass spectrometry<sup>(3)</sup> at Atomics International (AI) on pieces cut from the grip ends of tensile samples. The analysis showed 8.6 appm for samples irradiated to  $3.8 \times 10^{26} \text{ n/m}^2$  and 17.8 appm for samples irradiated to about  $8 \times 10^{26} \text{ n/m}^2$ . The fluence of  $3.8 \times 10^{26} \text{ n/m}^2$  was calculated at the FRMDC.<sup>(4)</sup> Fluences quoted in this report are for E >0.1 MeV, except where noted.

Ť

<sup>&</sup>lt;sup>°</sup>Density measurements were made at ANL's Idaho Facility, Analytical Laboratory.

Pin	Sample Type	T;* (°C)	т <sub>f</sub> † (°С)	Fluence (n/m <sup>2</sup> , E >0.1 MeV)	$\frac{(\rho_0^-\rho_i)}{(\%)}$	$\frac{\Delta \nabla}{(\nabla - \Delta V)}$ (%)	$ ho_V$ (V/m <sup>3</sup> )	d (nm)
А	Cylinder	500	530	$3.8 \ge 10^{26}$	$-0.1 \pm 0.2$			
в	Cylinder	600	620	$3.8 \ge 10^{26}$	$+0.1 \pm 0.2$			
C	Cylinder	500	. 500	$\sim 8 \times 10^{26}$	+0.1 ± 0.2			
D	Cylinder	600	535	$\sim 8. \times 10^{26}$	-1.0 ± 0.2			
E	Cylinder	500	545	$\sim 8 \times 10^{26}$	$+0.0 \pm 0.2$			
F	Cylinder	600	600	$\sim 8 \times 10^{26}$	$-3.3 \pm 0.2$			
A	Foil	500 ʻ	520	$3.8 \times 10^{26}$	-	-0.01 ± 0.01	$4 \ge 10^{19}$	15.4
в	Foil	600	630	$3.8 \ge 10^{26}$		-0. <b>l</b> ± 0.03	$2 \ge 10^{19}$	31.7
C	Foil	500	500	$\sim 8 \times 10^{25}$		$-2.0 \pm 0.7$	$3.8 \ge 10^{20}$	39.5
F	Foil	600	· 60Ò	$\sim 8 \times 10^{25}$	-	-7 ± 2	$6.4 \ge 10^{20}$	53.0
				•				1

TABLE 3 IMMERSION DENSITY AND TEM SWELLING RESULTS

- ${^{*}T_{i}}$  = Initial temperature  ${^{\dagger}T_{f}}$  = Final temperature

¢.(

AI-ER DA -13164 8

#### III. RESULTS AND DISCUSSION

### A. SWELLING

Swelling results from immersion density and TEM measurements are summarized in Table 3. The errors associated with the immersion-density results are one standard deviation, while those for the TEM results are  $\pm 30\%$ , due to foil thickness uncertainties.

Garner et al.,<sup>(5)</sup> folded the immersion density data for the high fluence samples irradiated at 500 and 600°C, Pins C and F, into their revised design correlation equation. They found that the 600°C data is best represented by taking a shorter incubation period for the onset of swelling compared to the other data; the 500°C data fit the revised equation without modification.

Swelling data for samples cold rolled 20 to 27% and irradiated to a fluence greater than  $7.0 \ge 10^{26} \text{ n/m}^2$  have been plotted in Figure 1. Data from Sub-assemblies X098, X100, and X157 above 600°C appear to be in good agreement and sufficient to define the swelling curve in this region. Below 600°C there are





AI-ERDA-13164

and the state of the second state of the

insufficient data to interpret the shape of the curve and it is, therefore, shown as dashed in that region. The French data, obtained from Dupouy,<sup>(6)</sup> are peaked about 75°C lower in temperature than the U.S. data. This discrepancy is probably due to differences in the models and gamma heating rates used to calculate the temperature profile in a subassembly. Although it cannot be stated which data are correct, it was decided to arbitrarily shift the French data 75°C higher in temperature (shown as solid symbols) for comparison with the U.S. data. The correlation is quite good. From the available U.S. data, the peak swelling would be placed between 590 and 600°C.

The data from samples irradiated in Pins D and E are not included here because of the large temperature shift during irradiation. However, it should be noted that the sample in Pin D, which had a large temperature decrease, had a swelling of 1.0%, whereas the sample in Pin E, which had a temperature increase, did not show any change in density. Assuming that (1) the samples were always between their initial temperatures and their final temperatures, and (2) the change in temperature was a function of the swelling of the capsule and/or surrounding subassembly components, then the results can be explained with reference to Figure 1. The Pin D sample was always in a temperature region conducive to swelling, 600 to 535°C. The sample in Pin E, however, probably remained in the region indicating no change for most of its radiation history, and what small increment of swelling due to voids that may have taken place was only sufficient to offset densification caused by precipitation.

#### B. METALLOGRAPHY

Metallographic observations for samples irradiated to  $3.8 \times 10^{26} \text{ n/m}^2$  were reported earlier.<sup>(1)</sup> Irradiation to the higher fluence of about  $8 \times 10^{26} \text{ n/m}^2$  resulted in considerably more recovery and precipitation. The precipitates that were examined by selected area diffraction were identified as  $M_{23}C_6$  type. Although considerable recovery occurred in both samples irradiated to the higher fluence, no obvious cell structure was evident after irradiation at 500°C. After irradiation at 600°C, however, polygonization and cell walls were evident. Some grain boundary migration was evident in all the irradiated samples. Although rare, small helium bubbles were observed in the sample irradiated at 600 to 620°C to  $3.8 \times 10^{26} \text{ n/m}^2$ ;<sup>(1)</sup> no helium bubbles were observed in the sample

Ξ

AI-ERDA-13164

irradiated to about  $8 \ge 10^{26} n/m^2$  at 600°C. These observations of helium bubbles are in good agreement with the work of Brager and Straalsund.<sup>(7)</sup>

Void formation after irradiation at 500°C to a fluence of about  $8 \times 10^{26} n/m^2$  occurs primarily between the remains of original deformation bands, and is still somewhat inhomogeneous. After irradiation at 600°C to the same fluence, the voids approach a homogeneous distribution. This is probably due to the greater degree of recovery that has taken place in the sample. Figure 2 shows the pre-irradiation microstructure of a foil sample while Figures 3 to 6 show the microstructure following irradiation at 500 or 600°C to a fluence of 3.8 x  $10^{26} n/m^2$  and about  $8 \times 10^{26} n/m^2$ .

#### C. TENSILE TESTS

Results of the tensile tests are shown in Table 4. Also included are results of samples irradiated to a fluence of  $3.8 \times 10^{26} \text{ n/m}^2$ .<sup>(1)</sup> These results indicate that irradiation in the temperature region of 500 to 600°C leads to a reduction in both the yield and ultimate strengths and an increase in both the uniform and total elongations.

Fahr et al.,<sup>(8)</sup> and Fish et al.,<sup>(9)</sup> have reported similar observations on irradiated 20% cold-rolled Type 316 stainless steel tested in the same temperature region. Both also reported that the yield strength of samples that were cold rolled and aged, but not irradiated, was essentially that of the cold-rolled and irradiated samples.

Metallographic examination of 20% cold-rolled foils, irradiated in the same capsules as the tensile samples, revealed an obvious loss of cold-rolled structure in all cases except those irradiated at 500°C to a fluence of  $3.8 \times 10^{26} \text{ n/m}^2$ . Thus, the reduction in strength appears to be due to thermal processes, independent of irradiation, in the temperature range of 500 to 600°C.





×



Figure 3. Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 500 to 530°C to a Fluence cf  $3.8 \times 10^{26} \text{ n/m}^2$  (E >0.1 MeV)

AI-ERDA-13164 13 =



2-325

K

8 6

Figure 4. Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 600 to 620°C to a Fluence of 3.8 x 10<sup>26</sup> n/m<sup>2</sup> (E >0.1 MeV)





20-380

K

Figure 5. Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 500°C to a Fluence of  $\sim 8 \ge 10^{26} n/m^2$  (E>0.1 MeV)

And Bridge Barney Sand States



10-369

Figure 6. Microstructure of 20% Cold-Rolled Type 316 Stainless Steel after Irradiation at 600°C to a Fluence of  $\sim 8 \ge 10^{26} \text{ n/m}^2$  (E>0.1 MeV)

~ •	Fluence	Irradiated	Test	Str	ength	Elongation		
Sample	(E > 0.1  MeV) x 1026 (n/m <sup>2</sup> )	Temperature (°C)	Temperature (°C)	Yield (MPa)	Ultimate (MPa)	Uniform (%)	Total (%)	
Control			500	825	863	2.9	3.1	
Irradiated	3.8	500-530	500	595	667	3.2	3.6	
Control	<del>-</del> -		600	698	756	2.6	2.9	
Irradiated	3.8	600-620	600	438	. 51,9	4.5	5.0	
Control			525	796 ,	831	2.6	2.9	
Irradiated	~8	500-545	525	463	547	5.0	5.4	
Control	·		575	719	786	2.6	2.8	
Irradiated	~8	600-535	575	407	478	4.0	4.3	

SUMMARY OF TENSILE DATA FOR 25% COLD-ROLLED TYPE 316 STAINLESS STEEL

# TABLE 4

# AI-ERDA-13164 17

3.0

#### IV. CONCLUSIONS

Significant amounts of recovery occur in 20% cold-rolled Type 316 stainless steel irradiated at 500 or 600°C to a fluence of about  $8 \times 10^{26} n/m^2$  (E >0.1 MeV).

Swelling occurred in the samples irradiated at 600°C while none occurred in the samples irradiated at 500°C, as measured by immersion density.

Voids were observed in samples irradiated at both temperatures.

Irradiation at 500 to 545°C and 600 to 535°C results in a reduction in yield strength and an increase in ductility.

n T

#### REFERENCES

- K.R. Garr, A.G. Pard, and D. Kramer, "The Effect of Neutron Irradiation on Types 316, 321, and Sandvik 12R72 Stainless Steels," AI-AEC-13130 (June 28, 1974)
- 2. E. E. Bloom, ORNL, Sample Temperatures (private communication)
- 3. H. Farrar and C. H. Knox, "Determination of Very Low Levels of Helium in Metals," Transactions, American Nuclear Society, <u>11</u>, p 503 (1968)
- 4. J.A. Ulseth, Fast Reactor Materials Dosimetry Center, HEDL, Sample Fluence (private communications)
- 5. F.A. Garner, G.L. Guthrie, and T.K. Bierlein, "Evaluation of 20% Cold-Worked 316 Swelling Correlation," HEDL-TME-74-4, Vol 1, p D-4
- 6. J. M. Dupouy, C. E. N. Saclay, Information Exchange at Atomics International, (October 1973)
- 7. H.R. Brager and J.L. Straalsund, "Defect Development in Neutron Irradiated Type 316 Stainless Steel," J. Nuc Mat 46, 134 (1973)
- D. Fahr, E.E. Bloom, and J.O. Stiegler, "Post-Irradiation Tensile Properties of Annealed and Cold-Worked Type 316 Stainless Steel," Proceedings BNES Conference, British Nuclear Energy Society, 9-10, p 167 (November 1972)
- R. L. Fish, A. J. Lovell, H. R. Brager, and J. J. Holmes, "Tensile and Creep Behavior of Cold-Worked Type 316 Stainless Steel After EBR-II Irradiation," Proceedings BNES Conference, British Nuclear Energy Society, 9-10, p 187 (November 1972)