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A VERSION OF THE MORSE MULTIGROUP TRANSPORT
CODE FOR FUSION REACTORS BLANKETS AND SHIELDS STUDIES

Magdi M. Ragheb and Charles W. Maynard

August 30, 1975

BROOKHAVEN NATIONAL LABORATORY
UPTON, NEW YORK 11973

INFORMAL REPORT



MASTER

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ABSTRACT

A guide for the use of a new version of the Morse Monte Carlo Multigroup neutron and gamma ray transport code with combinatorial geometry is presented. This version is in current use for neutronics studies of blankets and shields for fusion power reactors designs. A benchmark problem treating a cell calculation for a fusion reactor blanket and shield design , is presented. The modifications introduced to the different routines, the deck organization for the CDC-7600 computer system, a sample problem input and output, and a set of recommendations are given.

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1. Introduction

This work is a documentation for a new version of the MORSE Monte Carlo code ⁽¹⁾ used by the Fusion Technology group at the Department of Nuclear Engineering, the University of Wisconsin, ^(2, 3) and at the Department of Applied Science, Brookhaven National Laboratory ⁽⁴⁾. The version distributed by the Radiation Shielding Information Center (RSIC) is set up for the solution of a sample problem for a point source in an infinite air medium with spherical geometry. After some modifications, the code has been successfully used for general geometries for blanket and shield neutronics studies for fusion power reactor designs. The present work concerns the use of the CDC-7600. A benchmark problem relating to a unit cell computation is used as an illustration for the necessary modifications and the corresponding input and output data for the code.

A discussion of future possible modifications to the code regarding a better implementation on the CDC-7500 and the analysis of geometrical shapes not presently implemented is given.

2. The MORSE Monte Carlo code with Combinational Geometry

The MORSE (Multigroup Oak Ridge Stochastic Experiment) code ⁽¹⁾ is being used in applications for which it was not originally written, and this entails much revision and continuous development of the code. It has the ability to treat the transport of either neutrons or gamma rays or a coupled neutron and secondary gamma ray problem. There are options of the code for the IBM-360, UNIVAC-1108, and CDC-6600, which are distributed by RSIC. The version described in this report has been used on a CDC-7600.

The combinational geometry option of the MORSE code has been recognized to be of great use. It depends on describing general three dimensional configurations by considering unions and intersections of simple bodies such as spheres, parallelepipeds, cylinders; etc.

A preparation of a problem for solution by the code requires: setting up a model, describing it by the combinational geometry technique, checking the geometry input with the PICTURE routine, preparing cross-sections data for input to the MORSEC module, preparing analysis data for input to the SAMBO module, and random-walk data. User routines may be required to be written for each problem.

Either the forward or adjoint mode can be used, and time dependence is provided for shielding and criticality problems, Multigroup cross-sections are accepted in either ANISN or DTF-IV formats, and anisotropic scattering is treated for each group to group transfer by a generalized Gaussian quadrature technique.

3. Modifications introduced to MORSE routines in the present version

The following corrections and modifications were introduced to the latest updated version of MORSE ⁽¹⁾:CCC-203B and distributed by RSIC on July, 1975. Both permanent and problem-dependent modifications are presented:

3.1 Program MORSER

The statements:

COMMON NC (-----)	MAIN1-6
NLFT = -----	MAIN1-29

Should be modified according to the needed size for the blank common area. For moderate size problems, the blank common area fits into the short core memory (SCM) of the CDC-7600, but for larger size problems, since the CDC-7600 lacks the facility of virtual memory, a larger blank common area could be fitted into the large core memory (LCM).

However, this would require modifying the corresponding routines. In addition, overlays could be used to reduce the required field length.

The statement:

```
COMMON/F15BNK/MF1STP (6)           MAIN1-13
```

is modified to:

```
COMMON/F15BNK/MF1STP (6)           MAIN1-13,
```

an apparently punching error.

3.2 ENDRUN Routine

The statement:

```
DATA MMED/20*1/                     ENDRU-27
```

is in the MORSE code for the sample problem of a point fission source in air. For problems with more than one medium, MMED should contain the media numbers corresponding to the different regions.

Preceding the first call to NSIGTA, a statement of the form: IF(NREG.EQ. --.....) GO TO 86 is added to exclude regions containing media without stored cross-sections. Following Statement #85 these two statements are added:

```
GO TO 90
```

```
86 WTS(L) = 0.0
```


Similarly, preceding the second call to NSIGTA,
we add:

```
IF (NREG.EQ. -- ..... ) GO TO 101
```

and the statement 100 is followed by:

```
GO TO 105
```

```
101 WTS(L) = 0.0
```

3.3 GENI Routine

When the MA-array is needed for checking the geometry specifications, its listing can be obtained by deleting the statement:

```
IF (IDBG.EQ.0) RETURN GENI-148
```

3.4 GETMUS Routine

The present coding does not allow the possibility of using P_1 -anisotropy. This can be corrected by inserting the statement:

```
IF (NMOM.EQ.1) GO TO 55
```

just after the computation of:

```
A(1,1) = - MOMENT (1) GETMU-30
```

3.5 GTMED Routine

Replacing the Statement:

```
MDXSEC = 1 GTMED-3
```

```
by: MDXSEC = MDGEOM GTMED-3
```

sets the cross-section medium to be the same as the geometry medium.

3.6 INSCOR Routine

The routine is modified from its original use for spherical geometry to treat a general geometry. The following comment is inserted:

```
C THAT VERSION FOR THE GENERAL GEOMETRY READS THE REGION
C DETECTORS VOLUMES AND COMPUTES THEIR RECIPROCAL S
C NDC      IS LAST BOUNDARY CROSSING DETECTOR
C VV (I), I=1, ND  ARE REGION DETECTOR VOLUMES
```

The vector VV(I) is dimensioned:

```
DIMENSION VV(21)
```

and the VV(I) are read, after NDC is read, by the statements:

```
READ (I1, 102) (VV(I), I = 1, ND)
102 FORMAT (8E10.4)
```

The statements from 5 to 12 are deleted, and replaced by:

```
5 BC(LF) = 1./VV(ID-1)
```

3.7 NSIGTA Routine

Inserting the statement:

```
IF (MED. EQ. 1000. OR.MED.EQ.0) GO TO 11
```

after the call to GTMED, results in excluding the computation of TSIG and PNAB when treating internal and external void regions.

Also, the following statements are inserted after statement 55:

GO TO 12

11 TSIG = 0.0

PNAB = 0.0

and the three statements following the EQUIVALENCE:

IF (PNAB + 1.0) 3,1,3 NSIGT-18

1 MED = JMED NSIGT-19

GO TO 5 NSIGT-20

are deleted. The RETURN statement is changed to:

12 RETURN NSIGT-45

3.8 SOURCE Routine

This routine samples from an input frequency distribution function for the angular distribution of the source, and determines the direction cosines U,V,W. It also determines the coordinates of the source X,Y,Z when needed to follow a specified distribution. The routine has thus to be modified for each problem. As listed below, the routine uses the rejection method to sample for the source coordinates and the angular distribution. More efficient sampling can be achieved by construction of cumulative distributions. The following routine was used to sample from a discrete angular distribution of

the neutron wall loading for a typical tokamak with a circular cross-section in the neutronics calculations of a modular blanket unit cell. Details of the problem will be included in a future publication.

```

SUBROUTINE SOURCE(IG,U,V,W,X,Y,Z,WATE,MED,AG,ISOUR,ITSTR,NGPQT3,
 1 DDF,ISBJAS,NMTG)
C
C IF ITSTR=0, MUST PROVIDE IG,X,Y,Z,U,V,W,WATE AND AG IF DESIRED TO BE
C DIFFERENT FROM CAND VALUES (WHICH ARE THE VALUES INPUT TO SOURCE)
C IF (ITSTR=1), IG IS THE GRP NO, CAUSING FISSION, MUST PROVIDE NEW IG
C THIS VERSION OF SOURCE SELECTS INITIAL GROUP FROM THE INPUT SPEC
C DIMENSION FDF(50)
C
COMMON /USER/ DUM(9),IO,II,IOUM(12)
COMMON WTS(1)
DATA ICALL/1/
DATA PYE/3,14159/
IF (ICALL) 10,10,5
5 ICALL = 0
WRITE (10,1000)
1000 FORMAT(94H YOU ARE USING THE DEFAULT VERSION OF SOURCE WHICH SETS
1 WATE TO DDF AND PROVIDES AN ENERGY IG. )
C
C READING OF SOURCE DISTRIBUTION FUNCTION DATA
C LOWER LIMIT=AA UPPER LIMIT=BB N.OF SUBDIVISIONS=NN
C
HEAD(11,1002) AA,BB,NN
1002 FORMAT(2E14,8,15)
C
C HEAD UNNORMALIZED F.O.F.
C
HEAD(11,1003) (FDF(I),I=1,NN)
1003 FORMAT(10F5,3)
WRITE (10,1004) AA,BB,NN
1004 FORMAT(2X,12HLOWER LIMIT=E14,8,5X,12HUPPER LIMIT=E14,8,5X,24H NU
MBER OF SUBDIVISIONS=,15,/,2X,3HFDF//)
WRITE (10,1005) (FDF(I),I=1,NN)
1005 FORMAT(2X,5E14,8//)
UPYE=2.0*F E
XNNENN
HNG=BB-AA
DELTA=RNG/XNN
C
C FIND MAX OF FDF (FMAX)
C
DO 1201 I=1,NN
IF (1.E0.) GO TO 1202
II=I-1
IF (FDF(I).GT.FDF(II)) FMAX=FDF(I)
GO TO 1201
1202 FMAX=FDF(I)
1201 CONTINUE
C
CMCROSSING COORDINATES OF SOURCE PARTICLES
C
10 Z=40.0
103 X=RANF(0)*36.9504
Y=RANF(0)*32.0
IF (X.GT.18.4752) GO TO 101
GO TO 102
101 XDUM=64.0-1.732*X
IF (Y.GT.XDUM) GO TO 103
C
CMCROSSING ANGULAR PARAMETERS FOR SOURCE PARTICLE
C
102 R1=RNG*FLTNMF(0)
R2=FMAX*FLTNMF(0)

```

```

C      DETERMINE INTERVAL NUMBER FOR COS THETA
C
C      I=(R1/DELTA)+.99
      IF (R2.LE.FOF(I)) GO TO 1203
      GO TO 102
1203 W=AA+R1
      XDUM=ACOS(W)
      XDUM=5IN(XDUM)
      R2=DPYE*FLTRNF(0)
      U=XDUM*COS(R2)
      V=XDUM*5IN(R2)
C 10 IF (ISOUR) 15,15,60
15  WATE=DDF
      IF (ISBIAS) 20,20,25
20  NMT = 2*NMTG
      GO TO 30
25  NMT = 3*NMTG
30  R = FLTRNF(R)
      DD 35 I=1,NGPOT3
      IF (R = WTS(I*NMT)) 40,40,35
35  CONTINUE
40  IG=1
      IF (ISBIAS) 60,60,45
45  IF (I=1) 60,50,55
50  WATE = WATE+WTS(2*NMTG+1)/WTS(3*NMTG+1)
      GO TO 60
55  WATE = WATE+WTS(2*NMTG+1)-WTS(2*NMTG+1-1))/WTS(3*NMTG+1)-WTS(3*N
      MNTG+1-1))
60  RETURN
      END

```

4. The PICTURE Routine

This routine is used as an aid in preparing correct input for the combinational geometry package. PICTURE⁽¹⁾ displays two-dimensional cuts through the specified three-dimensional geometry. Media, regions, or zones may be displayed according to debugging needs. PICTURE uses two related subroutines: PRINT and MESH. Other subroutines from the MORSE package are also called as shown in the hierarchy of subroutines in Figure 1. Figures 2 and 3 show a two-dimensional cut through the geometry of the blanket unit cell for the benchmark problem treated in that report.

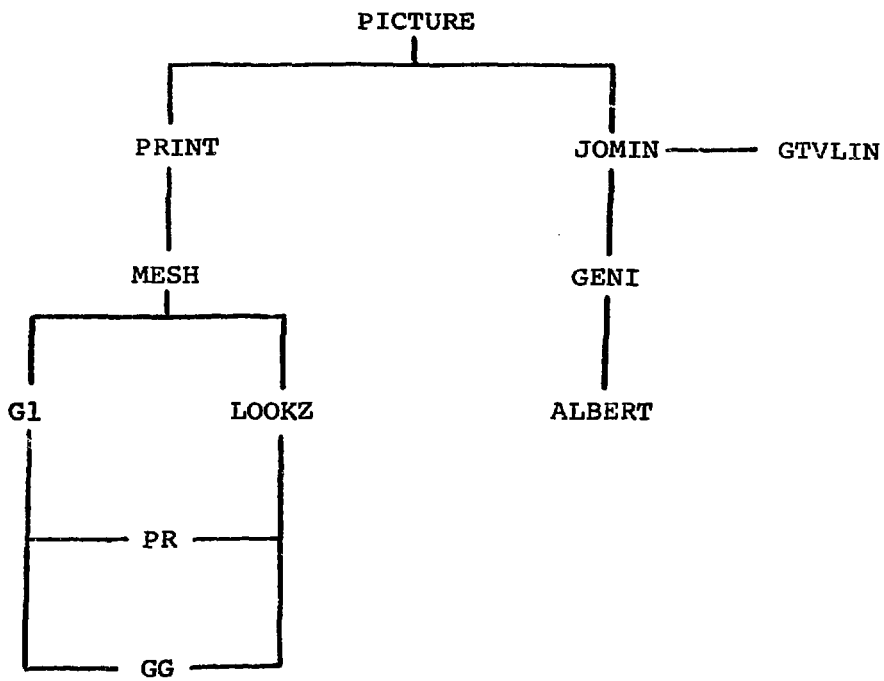
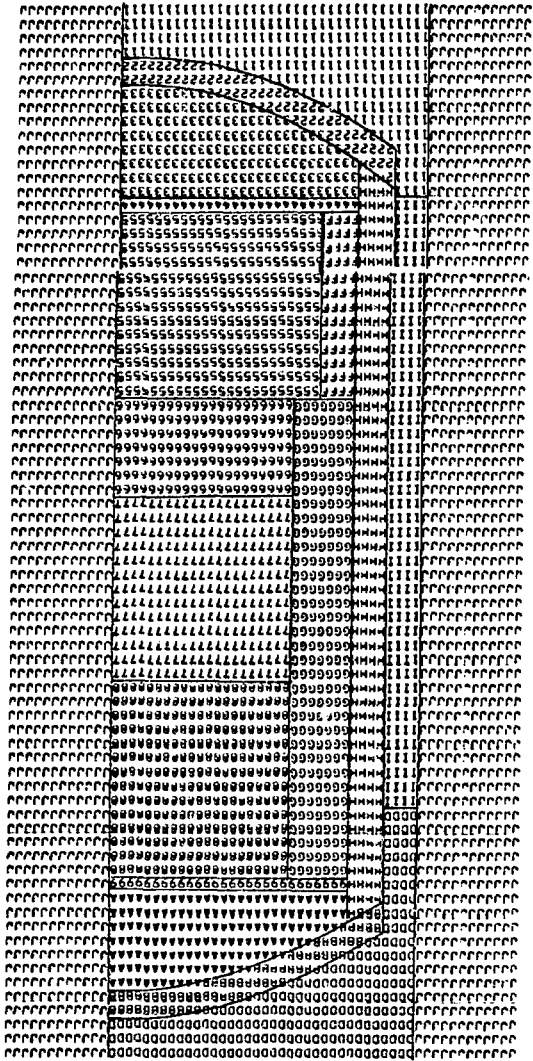


Figure 1 . Hierarchy of subroutines in the PICTURE program

Figure 2. Output of PICTURE for a vertical cut through the blanket and shield unit cell.



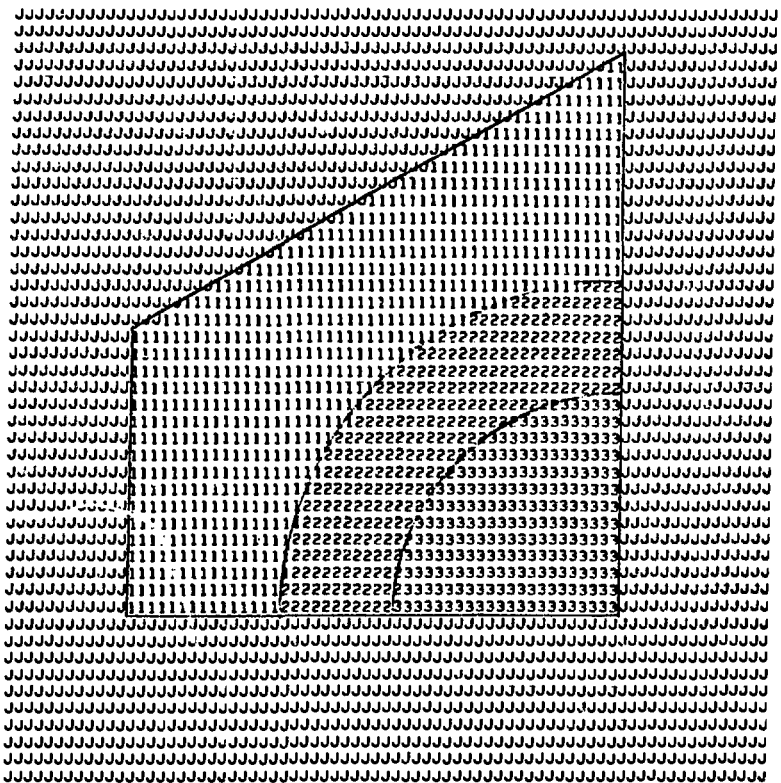


Figure 3 . Output of PICTURE for a horizontal cut through the blanket and shield unit cell .

5. Deck Organization for the CDC-7600

For this Benchmark problem, the blank common area is considered as fitting into short core memory (SCM). A file MRS is attached containing the source routines, and changes to different routines are introduced by use of UPDATE ⁽⁵⁾. A program library MORSELIB containing the object decks, of routines which were not updated, is then attached to the run. The UPDATE correction cards follow, and then the input data to MORSE. The CDC-7600 control cards, and the UPDATE correction cards are listed below, according to the modifications explained in section 4 for the benchmark problem.

```
MAGUI,CP70,T100,P666.
ACCOUNT(LAZARETH,128)
MAP(ON)
ATTACH,OLDPL,MRS,ID=LAZARETH.
UPDATE.
FIN(I,A)
ATTACH( LIB,MORSELIB,ID=LAZARETH)      FILE MORSELIB(CATALOGUED)
LIBRARY(LIB)
LGO.
?
*ID,CORRECT
*U,MAIN1.6
C      COMMON NC(22000)
      COMMON NC(33000)
*U,MAIN1.13
C      COMMON/F15BNK/MF1STP(6)
      COMMON/F15BNK/MF1STP(6)
*U,MAIN1.29
C      NLFT=22000
      NLFT= 33000
*U,ENDMU.27
C      DATA MMED/20*1/
      DATA MMED/1000,1,2,3,4,4,4,4,3,7,1,1,5,5,6,6,1,1000,8,0/
*1,ENDMU.104
      IF(NNEG.EG. 1.0M,NNEG.GE.18)                GO TO 86
*1,ENDMU.111
      GO TO 90
      86 MTS(L)=0.0
*1,ENDMU.117
      IF(NNEG.EG. 1.0P,NNEG.GE.18)                GO TO 101
*1,ENDMU.124
      GO TO 105
      101 MTS(L)=0.0
```

```

•D,GENI,148
C   IF(1DBG,EG,0) RETURN
•I,GETMU,30
   IF(NMOM,EG,1) GO TO 55
•D,GTMEU,3
C   MDXSEC=1
   MDXSEC=MUGEUM
•I,INSCO,2
C   THAT VERSION FOR THE GENERAL GEOMETRY READS THE REGION
C   DETECTORS VOLUMES AND COMPUTES THEIR RECIPROCAL
C   NDC IS LAST BOUNDARY CROSSING DETECTOR
C   VV(I),I=1,ND ARE REGION DETECTOR VOLUMES
•I,INSCO,13
   DIMENSION VV(21)
•I,INSCO,15
   READ(11,102) (VV(I),I=1,ND)
   102 FORMAT(8E10,4)
•U,INSCO,21,INSCO,29
   5 BC(LF)=1./VV(1D-1)
C
C   NINE CARDS WERE DELETED HERE
•D,NSIGT,18,NSIGT,20
C   IF(PNAB+1.,13,1,3
C   1 MED=JMED
C   GO TO 5
•I,NSIGT,21
   IF(MED,EG,1000,CH,MED,EG,0) GO TO 11
•I,NSIGT,44
   GO TO 12
   11 TSIG=0.0
   PNAH=0.0
•U,NSIGT,45
C   RETURN
   12 RETURN
•U,RELCO,3
C   SPHERICAL GEOMETRY
C   GENERAL GEOMETRY
•D,RELCO,16,RELCO,29
C   FIFTEEN CARDS WERE DELETED HERE
   CALL NSIGTA(JGO,NMED,TSIG,PNAB)
   CON=WBTC/TSIG
   ID=NREG+1
•INSEXT,SOURC,8
   DIMENSION FDF(50)
•INSERT,SOURC,12
   DATA PYE/3.14159/
•INSEXT,SOURC,17
C
C   HEADING OF SOURCE DISTRIBUTION FUNCTION DATA
C   LOWER LIMIT=AA UPPER LIMIT=BB N,OF SUBDIVISIONS=NN
C
   READ(11,1002) AA,BB,NN
   1002 FORMAT(2E14,8,15)
C
C   HEAD UNNORMALIZED F,U,F.
C
   READ(11,1003) (FDF(I),I=1,NN)
   1003 FORMAT(10F5,3)
   WRITE(10,1004) AA,BB,NN
   1004 FORMAT(2X,12MLOWER LIMIT=,E14,8,5X,12MUPPER LIMIT=,E14,8,5X,2M NU
   MBER OF SUBDIVISIONS=,I5,/,2X,3MFDF//)
   WRITE(10,1005) (FDF(I),I=1,NN)
   1005 FORMAT(2X,5E14,8/)
   UPYE=2.0*PYE
   XNN=NN
   HNG=BB-AA
   UELTA=HNG/XNN
C
C   FIND MAX OF FDF (FMAX)
C
   DO 1201 I=1,NN
   IF(1.EQ.1) GO TO 1202
   II=I-1
   IF(FDF(II),GT,FDF(I)) FMAX=FDF(II)
   GO TO 1201
   1202 FMAX=FDF(I)
   1201 CONTINUE

```

```

C
C=COSING COORDINATES OF SOURCE PARTICLES
C
  10 Z=0.0
  103 X=RANF(0)*36.9564
      Y=RANF(0)*32.0
      IF(X.GT.18.4752) GO TO 101
      GO TO 102
  101 XDUM=64.0-1.732*X
      IF(Y.GT.XDUM) GO TO 103
C
C=CHOOSING ANGULAR PARAMETERS FOR SOURCE PARTICLE
C
  102 R1=RNG*FLTRNF(0)
      R2=FMAX*FLTRNF(0)
C
C      DETERMINE INTERVAL NUMBER FOR COS THETA
C
      I=(R1/DELTA)+.95
      IF(R2.LE.FDF(1)) GO TO 1203
      GO TO 102
  1203 W=AA*R1
      XDUM=ACOS(W)
      XDUM=SIN(XDUM)
      R2=DPYE*FLTRNF(0)
      U=XDUM*COS(R2)
      V=XDUM*SIN(R2)
*DELETE,SOURC.18
      IF(ISOUM)15,15,60
C  10 IF(ISOUM) 15,15,60

```

6. Input data for the benchmark calculation

Table I shows the listing of the input cards. The MORSE data comprise a problem of 5 batches each of 150 histories. Six groups of neutrons are used. Both splitting and Russian roulette are allowed. The geometry data comprise 25 bodies defining 20 regions in the unit cell. Seven cross-section media are used, the albedo medium is numbered 8 and the internal and external void regions are designated by 1000 and 0 respectively. The regions volumes are shown as unity.

Table I. Input data for the benchmark problem

MORSE	Uw-BNL	BNL	MINAC	MEX	CELL	MAGDI	RAGMEB					
150	250	5	1	6	0	6	6	0	0	5	7	8
1	1	0	0.10000E+01	0.00250E+00	0.00000E+00	0.00000E+00	0.22000E+06	0.0	0.0	0.0000E+00	0.0000E+00	0.0000E+00
0.0	0.0	0.0	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00	0.0000E+00
.1492E+08	.1350E+08	.1221E+08	.1105E+08	.1000E+08	.9048E+07							
00006472261267317161												
1	1	0	0	20	006							
0	1	0	0	1	0.10000E+02	1.0000E-01	1.0000E+00	0.00000E+00				
-1												
0	0	0	0									
3	0											
ARR	10.0	0.0	-70.0	36.9504	0.0	-70.0						
	18.4752	32.0	-70.0	0.0	32.0	-70.0						
	0.0	0.0	250.0	36.9504	0.0	250.0						
	18.4752	32.0	250.0	0.0	32.0	250.0						
	1562.	2673.	3784.	4851.	1234.	5876.						
RPP	2	-40.0	+72.0	-40.0	+72.0	-110.0	+250.0					
HPP	3	-50.0	+82.0	-50.0	+82.0	-120.0	+260.0					
HCC	4	0.0	0.0	48.5	0.0	0.0	106.5					
		32.0										
HCC	5	0.0	0.0	47.5	0.0	0.0	110.0					
		29.25										
RCC	6	0.0	0.0	65.0	0.0	0.0	25.0					
		26.75										
HCC	7	0.0	0.0	85.0	0.0	0.0	60.0					
		24.25										
RCC	8	0.0	0.0	151.0	0.0	0.0	15.0					
		7.5										
KPP	9	-2.5	30.0	-2.5	30.0	137.0	138.0					
RPP	10	-5.0	35.0	-5.0	35.0	117.0	138.0					
RPP	11	-10.0	37.5	-10.0	37.5	97.0	138.0					
RPP	12	-2.5	30.0	-2.5	30.0	67.0	68.0					
RPP	13	-5.0	35.0	-5.0	35.0	67.0	77.5					
RPP	14	-7.5	37.5	-7.5	37.5	67.0	87.0					
RPP	15	-5.0	37.5	-5.0	37.5	190.0	250.0					
HPP	16	-20.0	45.0	-20.0	45.0	102.0	250.0					
SPH	17	0.0	0.0	140.0	50.0							
SPH	18	0.0	0.0	100.0	47.25							
SPH	19	0.0	0.0	104.0	50.0							
SPH	20	0.0	0.0	104.0	47.25							
RPP	21	-12.5	40.0	-12.5	40.0	87.0	138.0					
HPP	22	-15.0	52.5	-15.0	52.5	67.0	130.0					
SPH	23	0.0	0.0	100.0	50.0							
HPP	24	-15.0	52.5	-15.0	52.5	67.0	130.0					
HPP	25	-15.0	52.5	-15.0	52.5	67.0	130.0					
END												
1	200R	1	-17	-16	-220R	1	23	-4	-16	-24		
2	20	1	17	-18	-16	4						
3	20	1	18	5	-22	-16						
4	20	1	12	5								
5	20	1	6	14	-12	-21						
6	20	1	7	21	-11	-14						
7	20	1	7	11	-10							
8	20	1	7	10	-9							
9	20	1	9	5								
10	20	1	20	5	16	-21	-8					
11	20	1	19	-20	16	-8	4					
12	20	1	8									
13	200R	1	-19	16	-15	-80R	1	19	16	-22		
		-4										
14	20	1	15									
15	20	1	5	-6	14	-12						
16	20	1	5	-7	21	-14	-4					
17	200R	1	4	-5	180R	20	1	4	-5			
18	20	1	25	-4								
19	20	2	-1									
20	20	3	-2									
END												

Table II . Sample output for the benchmark problem

MURSE UM=BNL BNL MINAC , HEX.CELL

NSTRT= 130 NMUST= 250 NITS= 5 NQUIT= 1 NGPOTN= 6
 NGPUTG= 0 NMGP= 6 NMTG= 6 NCOLTP= 0 IADJM= 0
 MAXIMUM EXECUTION TIME = 5 MINUTES MEDIA= 7 MEUALB= 8

ISOUR= 1 NGPFS= 1 ISBIAS= 0 NHESP= 0
 WTSTRT=3.0000E+00 EBOTN=2.5000E-03 EBOTG=0. TCUT=0. VELTH=2.2000E+05

XSTRT= 0. YSTRT= 0. ZSTRT= 0. AGSTRT= 0.
 UINP= 0. VINP= 0. WINP= 0.

GROUP PARAMETERS, GROUP NUMBERS GREATER THAN 6 CORRESPOND TO SECONDARY PARTICLES

GROUP	UPPER EDGE (EV)	VELOCITY (CM/SEC)
1	1.4920E+07	5.2134E+09
2	1.3500E+07	4.9586E+09
3	1.2210E+07	4.7164E+09
4	1.1050E+07	4.4868E+09
5	1.0000E+07	4.2681E+09
6	9.0480E+06	2.2000E+05

INITIAL RANDOM NUMBER = 20001057152522261031

NSPLT= 1 NKILL= 1 NPAST= 0 NOLEAK= 0 IEBIAS= 0 MXREG= 20 MAXGP= 6

WEIGHT STANDARDS FOR SPLITTING AND RUSSIAN ROULETTE AND PATHLENGTH STRETCHING PARAMETERS

NGP1	NDG	NGP2	NRG1	NRG	NRG2	WTHI1	WTLOW1	WTAVE1	XNU
0	1	0	0	1	0	1.0000E+01	1.0000E-02	1.0000E-01	0.

NSOUR= 0 MFISTP= 0 NKCALC= 0 NORMF= 0

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Table II. (Continued)

IVCPT = 3

IDBG = 0

		BODY DATA							
AHB	1	0.	0.	-.700000E+02	.369504E+02	0.	-.700000E+02	3	
		.184752E+02	.320000E+02	-.700000E+02	0.	.320000E+02	-.700000E+02		
		0.	0.	.250000E+03	.369504E+02	0.	.250000E+03		
		.184752E+02	.320000E+02	.250000E+03	0.	.320000E+02	.250000E+03		
		.156200E+04	.267300E+04	.378400E+04	.485100E+04	.123400E+04	.587600E+04		
		0.	0.	.100000E+01	0.	0.	0.		
		-.866025E+00	-.499999E+00	0.	0.	.319999E+02	0.		
		0.	-.100000E+01	0.	0.	.320000E+02	0.		
		.100000E+01	0.	0.	0.	0.	0.		
		0.	0.	0.	.100000E+01	.700000E+02	0.		
		0.	0.	0.	-.100000E+01	.250000E+03	0.		
		.184752E+02	.600000E+01	0.	0.	0.	0.		
KPP	2	-.400000E+02	.720000E+02	-.400000E+02	.720000E+02	-.110000E+03	.250000E+03	35	
KPP	3	-.500000E+02	.820000E+02	-.500000E+02	.820000E+02	-.120000E+03	.260000E+03	43	
RCC	4	0.	0.	.485000E+02	0.	0.	.106500E+03	51	
		.320000E+02	0.	0.	0.	0.	0.		
RCC	5	0.	0.	.475000E+02	0.	0.	.110000E+03	60	
		.292500E+02	0.	0.	0.	0.	0.		
KCC	6	0.	0.	.650000E+02	0.	0.	.250000E+02	69	
		.267500E+02	0.	0.	0.	0.	0.		
KCC	7	0.	0.	.850000E+02	0.	0.	.600000E+02	78	
		.242500E+02	0.	0.	0.	0.	0.		
KCC	8	0.	0.	.151000E+03	0.	0.	.150000E+02	87	
		.750000E+01	0.	0.	0.	0.	0.		
KPP	9	-.250000E+01	.300000E+02	-.250000E+01	.300000E+02	.137000E+03	.138000E+03	96	
KPP	10	-.500000E+01	.350000E+02	-.500000E+01	.350000E+02	.117000E+03	.138000E+03	104	
KPP	11	-.100000E+02	.375000E+02	-.100000E+02	.375000E+02	.970000E+02	.138000E+03	112	
KPP	12	-.250000E+01	.300000E+02	-.250000E+01	.300000E+02	.670000E+02	.680000E+02	120	
KPP	13	-.500000E+01	.350000E+02	-.500000E+01	.350000E+02	.670000E+02	.775000E+02	128	
KPP	14	-.750000E+01	.375000E+02	-.750000E+01	.375000E+02	.670000E+02	.870000E+02	136	
KPP	15	-.500000E+01	.375000E+02	-.500000E+01	.375000E+02	.190000E+03	.250000E+03	144	
KPP	16	-.200000E+02	.450000E+02	-.200000E+02	.450000E+02	.102000E+03	.250000E+03	152	
SPH	17	0.	0.	.100000E+03	.500000E+02	-0.	-0.	160	
SPH	18	0.	0.	.100000E+03	.472500E+02	-0.	-0.	168	
SPH	19	0.	0.	.104000E+03	.500000E+02	-0.	-0.	176	
SPH	20	0.	0.	.104000E+03	.472500E+02	-0.	-0.	184	
KPP	21	-.125000E+02	.400000E+02	-.125000E+02	.400000E+02	.870000E+02	.138000E+03	192	
KPP	22	-.150000E+02	.525000E+02	-.150000E+02	.525000E+02	.670000E+02	.130000E+03	200	
SPH	23	0.	0.	.100000E+03	.500000E+02	-0.	-0.	208	
KPP	24	-.150000E+02	.525000E+02	-.150000E+02	.525000E+02	.670000E+02	.130000E+03	216	
KPP	25	-.150000E+02	.525000E+02	-.150000E+02	.525000E+02	.670000E+02	.130000E+03	224	
END	26	-0.	-0.	-0.	-0.	-0.	-0.	232	
NUMBER OF BODIES			25						
LENGTH OF FPD-ARRAY			237						

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Table II. (Continued)

INPUT ZONE DATA												
1	200W	1	-17	-16	-22UR	1	23	-4	-16	-24	2	1
2	20	1	17	-18	-16	4	-0	-0	-0	-0	2	3
3	20	1	18	5	-22	-16	-0	-0	-0	-0	2	4
4	20	1	12	5	-0	-0	-0	-0	-0	-0	2	5
5	20	1	6	14	-12	-21	-0	-0	-0	-0	2	6
6	20	1	7	21	-11	-14	-0	-0	-0	-0	2	7
7	20	1	7	11	-10	-0	-0	-0	-0	-0	2	8
8	20	1	7	10	-9	-0	-0	-0	-0	-0	2	9
9	20	1	9	5	-0	-0	-0	-0	-0	-0	2	10
10	20	1	20	5	16	-21	-8	-0	-0	-0	2	11
11	20	1	19	-20	16	-8	4	-0	-0	-0	2	12
12	20	1	8	-0	-0	-0	-0	-0	-0	-0	2	13
13	200W	1	-19	16	-15	-80W	1	19	16	-22	2	14
	-0	-4	-0	-0	-0	-0	-0	-0	-0	-0	2	16
14	20	1	15	-0	-0	-0	-0	-0	-0	-0	2	16
15	20	1	5	-6	14	-12	-0	-0	-0	-0	2	17
16	20	1	5	-7	21	-14	-9	-0	-0	-0	2	18
17	200W	1	4	-5	180W	20	1	4	-5	-0	2	19
18	20	1	25	-4	-0	-0	-0	-0	-0	-0	2	21
19	20	2	-1	-0	-0	-0	-0	-0	-0	-0	2	22
20	20	3	-2	-0	-0	-0	-0	-0	-0	-0	2	23
END	-0	-0	-0	-0	-0	-0	-0	-0	-0	-0	2	24

NUMBER OF INPUT ZONES 20
 NUMBER OF CODE ZONES 23
 LENGTH OF INTEGER ARRAY 1379

MORSE REGION IN INPUT ZONE (I) ARRAY MRIZ(I), I=1,20)

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20

MORSE MEDIA IN INPUT ZONE (I) ARRAY MMIZ(I), I=1,20)

1000 1 2 3 4 4 4 4 3 7 1 1 5 5 6 6 11000 8 0

SIX GROUPS CROSS-SECTIONS FOR CODE MORSE

NUMBER OF PRIMARY GROUPS (NGP) 6
 NUMBER OF PRIMARY DOWNSCATTERS (NUS) 6
 NUMBER OF SECONDARY GROUPS (NGG) 0
 NUMBER OF SECONDARY DOWNSCATTERS (NUSG) 0
 NUMBER OF PRIM-SEC GROUPS (INGP) 6
 TABLE LENGTH (ITBL) 9
 LOC OF WITHIN GROUP (SIG GG) (ISGG) 4
 NUMBER OF MEDIA (NMED) 7
 NUMBER OF INPUT ELEMENTS (NELEM) 4
 NUMBER OF MIXING ENTRIES (NMIX) 11
 NUMBER OF COEFFICIENTS (NCOEF) 4
 NUMBER OF ANGLES (NST) 2
 RESTORE COEFF (ISTAT) 0
 ADJOINT SWITCH (FROM MORSE) 0

CROSS SECTIONS AS READ FOR ELEMENT 1 COEFFICIENT 1

GP	ABS XSEC	NU*FISS	TOTAL	FROM GROUP TRANSFER PROBABILITIES									
1	1.366E-03	0.	1.045E-02	3.496E-03									
2	1.392E-03	0.	1.055E-02	3.598E-03	1.003E-03								
3	1.317E-03	0.	1.051E-02	3.364E-03	1.017E-03	2.279E-04							
4	1.199E-03	0.	1.041E-02	3.296E-03	9.925E-04	2.321E-04	2.512E-04						
5	1.050E-03	0.	1.032E-02	3.349E-03	9.956E-04	2.393E-04	2.664E-04	1.235E-04					
6	1.865E-04	0.	1.785E-02	1.766E-02	5.917E-03	4.922E-03	6.597E-03	4.044E-03	3.985E-03				

CROSS SECTIONS FOR MEDIA 1

GROUP	SIGT	SIGST	PNUP	PNAHS	GANGEN	NU*FIS	DOWNSCATTER PROBABILITY					
1	7.898E-04	6.866E-04	0.0000	.8693	0.0000	0.0000	.3848	.1104	.0251	.0276	.0136	.4385
2	7.970E-04	6.919E-04	0.0000	.8681	0.0000	0.0000	.3929	.1110	.0253	.0291	.4416	
3	7.944E-04	6.950E-04	0.0000	.8748	0.0000	0.0000	.3662	.1079	.0260	.0260	.4998	
4	7.867E-04	6.961E-04	0.0000	.8649	0.0000	0.0000	.3577	.1081	.0260	.0260	.4998	
5	7.795E-04	7.002E-04	0.0000	.8982	0.0000	0.0000	.3614	.1081	.0260	.0260	.4998	
6	1.349E-03	1.334E-03	0.0000	.9895	0.0000	0.0000	1.0000					

ANALYSIS DATA

Table II. (Continued)

ND= 21, NNE= 0, NE= 0, NT= 0, NA= 0, NHESP= 4, NEX= 22, NEXND= 0

GROUP	HESP(1)	RESP(2)	RESP(3)	HESP(4)
1	1.0000E+00	3.2924E-01	2.5580E-02	9.2200E-03
2	1.0000E+00	3.5914E-01	2.8350E-02	8.0300E-03
3	1.0000E+00	3.8763E-01	3.2290E-02	6.3800E-03
4	1.0000E+00	4.0802E-01	3.5900E-02	4.9100E-03
5	1.0000E+00	4.1632E-01	4.0120E-02	3.6200E-03
6	1.0000E+00	4.2086E-01	4.5290E-02	2.5800E-03

TIME REQUIRED FOR INPUT WAS 1
YOU ARE USING THE DEFAULT VERSION OF STRUM WHICH DOES NOTHING.

***START BATCH 1 RANDOM=20001057152522201031

SOURCE DATA
YOU ARE USING THE DEFAULT VERSION OF SOURCE WHICH SETS WATE TO ODF AND PROVIDES AN ENERGY IG,
LOWER LIMIT=0. UPPER LIMIT= .10000000E+01 NUMBER OF SUBDIVISIONS= 40

PDF

0. 0. 0. 0. 0.
0. 0. 0. 0. .10000000E+01
.20000000E+01 .30000000E+01 .40000000E+01 .60000000E+01 .70000000E+01
.90000000E+01 .11000000E+02 .12000000E+02 .14000000E+02 .16000000E+02
.17000000E+02 .18000000E+02 .19000000E+02 .21000000E+02 .22000000E+02
.23000000E+02 .24000000E+02 .24000000E+02 .26000000E+02 .26000000E+02
.28000000E+02 .28000000E+02 .29000000E+02 .27000000E+02 .25000000E+02
.23000000E+02 .22000000E+02 .22000000E+02 .20000000E+02 .20000000E+02

SWATE IAVE UAVE VAVE WAVE XAVE YAVE ZAVE AGEAVE
1.500E+02 1.00 .0476 .0327 .6624 1.406E+01 1.469E+01 4.000E+01 0.
YOU ARE USING DEFAULT VERSION OF ALBDO WHICH PERFORMS SPECULAR REFLECTION

NUMBER OF COLLISIONS OF TYPE NCOLL												
SOURCE	SPLIT(0)	FISHN	GAMGEN	REALCOLL	ALBEDO	BDYX	ESCAPE	E-CUT	TIMEKILL	R R KILL	R R SURV	GAMLOST
150	0	0	0	19	1887	3013	150	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 2

***START BATCH 2 RANDOM=20005016326400052701

SOURCE DATA
SWATE IAVE UAVE VAVE WAVE XAVE YAVE ZAVE AGEAVE
1.500E+02 1.00 -.0685 -.0522 .7018 1.418E+01 1.385E+01 4.000E+01 0.

NUMBER OF COLLISIONS OF TYPE NCOLL												
SOURCE	SPLIT(0)	FISHN	GAMGEN	REALCOLL	ALBEDO	BDYX	ESCAPE	E-CUT	TIMEKILL	R R KILL	R R SURV	GAMLOST
150	0	0	0	26	1792	2965	150	0	0	0	0	0

TIME REQUIRED FOR THE PRECEDING BATCH WAS 2

The cross-section data consists of a six-group set with a P_3 -anisotropy in ANISN-format card images. Only the cross-sections for two elements are displayed. The mixing table consists of 11 mixing operations using 4 elements to define 7 cross-section media. The points detectors locations are input as unities since they are not used. The region detector volumes are input and the number of the last boundary crossing detector are also set to unity. The analysis input data consist of three characteristic response functions used in blanket and shield studies. The input to the subroutines INSCOR, RELCOL and SOURCE complete the data.

7 . Sample Output

Table II shows major parts of the output obtained for the cell benchmark problem. However, part of the input data is only used, and do not represent a real problem situation. The reader is referred to another publication ⁽⁴⁾ for realistic data input.

8 . Recommendations and Conclusions

No complete documentation has been published before as a guidance on how to implement the MORSE code to the solution of practical problems. We have tried in this work to document that aspect in relation to Monte-Carlo

calculations of blankets and shields for fusion blanket studies using the CDC-7600 computer system. The version has been used for cell calculations at Brookhaven National Laboratory.

Due to the lack of the virtual memory facility on the CDC-7600, the code as set here requires the use of the SCM (Short Core Memory): for that case a group collapsing procedure by ANISN code is necessary to get a few-group cross-section set. If a fine group structure is needed, the LCM (large core memory) should be used with the subsequent modifications to the coding of the different routines.

Major code modifications for wider versatility in the future should include the implementation in the combinational geometry package of geometrical shapes not presently implemented, especially the circular and ellipsoidal toroids, the general ellipsoid shape, the right elliptical cylinder (for non-circular Tokomaks), and the wedge shape.

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