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Neptunium Transport Behavior in the Vicinity of Underground Nuclear Tests at the Nevada National Security Site

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Abstract

We used short lived ²³⁹Np as a yield tracer and state of the art magnetic sector ICP-MS to measure ultra low levels of ²³⁷Np in a number of "hot wells" at the Nevada National Security Site (NNSS), formerly known as the Nevada Test Site (NTS). The results indicate that ²³⁷Np concentrations at the Almendro, Cambric, Dalhart, Cheshire and Chancellor sites, are in the range of 3×10^{-5} to 7×10^{-2} pCi/L and well below the MCL for alpha emitting radionuclides (15) pCi/L) (EPA, 2009). Thus, while Np transport is believed to occur at the NNSS, activities are expected to be well below the regulatory limits for alpha-emitting radionuclides. We also compared ²³⁷Np concentration data to other radionuclides, including tritium, ¹⁴C, ³⁶Cl, ⁹⁹Tc, ¹²⁹I, and plutonium, to evaluate the relative ²³⁷Np transport behavior. Based on isotope ratios relative to published unclassified Radiologic Source Terms (Bowen et al., 1999) and taking into consideration radionuclide distribution between melt glass, rubble and groundwater (IAEA, 1998), ²³⁷Np appears to be substantially less mobile than tritium and other non-sorbing radionuclides, as expected. However, this analysis also suggests that ²³⁷Np mobility is surprisingly similar to that of plutonium. The similar transport behavior of Np and Pu can be explained by one of two possibilities: (1) Np(IV) and Pu(IV) oxidation states dominate under mildly reducing NNSS groundwater conditions resulting in similar transport behavior or (2) apparent Np transport is the result of transport of its parent ²⁴¹Pu and ²⁴¹Am isotopes and subsequent decay to ²³⁷Np. Finally, measured ²³⁷Np concentrations were compared to recent Hydrologic Source Term (HST) models. The ²³⁷Np data collected from three wells in Frenchman Flat (RNM-1, RNM-2S, and UE-5n) are in good agreement with recent HST transport model predictions (Carle et al., 2005). The agreement provides confidence in the results of the predictive model. The comparison to Cheshire HST model predictions (Pawloski et al, 2001) is somewhat ambiguous due to the low concentration resolution of the particle transport model.

Introduction

Among all the Np isotopes, ²³⁷Np, an alpha-particle emitter with a half-life of 2.14 million years, is the most stable and long-lived. At the NNSS, the main sources of ²³⁷Np are nuclear device source material (radiochemical tracer) and the decay products of ²³⁷U, ²⁴¹Pu and ²⁴¹Am (Bowen 2001, Smith et al. 2003). The most common oxidation state of ²³⁷Np in natural oxic waters is expected to be +5 (as NpO₂⁺), which sorbs relatively weakly to minerals and other organic and inorganic ligands (Choppin 2006, Assinder et al. 1991, Assinder 1999, Kenna and Sayles 2002, Kenna 2009). Thus, among the actinides, it is expected that Np should be far more mobile than Pu and Am (Lindahl et al. 2005, Yamamoto et al. 1991). ²³⁷Np was identified in the proposed Yucca Mountain repository total system performance assessment to be a key long-term dose contributor due to its long half life and relatively high solubility in the assumed oxygenated

environment. By analogy, ²³⁷Np may also play a role in contaminant boundary estimates at the NNSS.

Only a limited number of Np measurements have been reported for NNSS "hot wells" (radiologically contaminated groundwater wells). Most of these reported values are near instrument detection limits (Finnegan and Thompson 2002), which can lead to large measurement uncertainties. These large uncertainties limit our ability to interpret Np transport behavior and calibrate or validate reactive transport models. For example, at the Cambric site (Figure 1), reported ²³⁷Np concentrations at RNM-1 (Cambric cavity well), RNM-2S (satellite pumping well), and UE-5n (satellite well which intercepted contamination from recharging RNM-2S water) were $3x10^{-13}$ mol/L for all three wells, with a method detection limit on the order of 10⁻¹⁴ mol/L (estimated). Cambric Hydrologic Source Term (HST) reactive transport models predicted ~10⁻¹³ mol/L at RNM-1, ~10⁻¹⁵ mol/L at RNM-2S, and little or no Np at UE-5n (Carle, et al., 2005). It is not clear whether the differences between measured and predicted values can be attributed to measurement uncertainty or inaccurate model predictions. A disagreement also exists between predicted and reported ²³⁷Np concentrations from the Cheshire post-shot hole (U-20n PS#1 DD) (Figure 2). Clearly, validation of modeling results requires more accurate measurement of ²³⁷Np concentration. Reliable and high-quality data are needed to effectively compare model results to ²³⁷Np concentrations in groundwater.

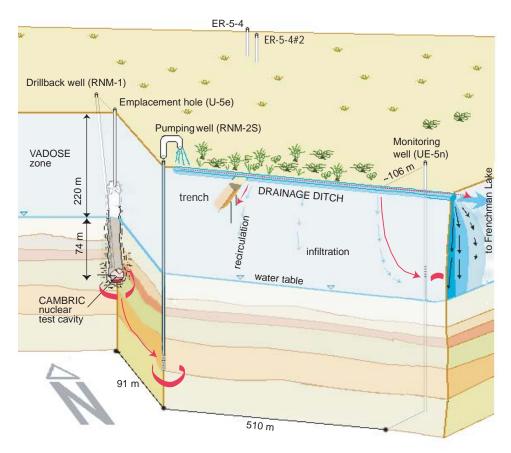


Figure 1. A view of the Cambric site with associated groundwater sampling wells (RNM-1, RNM-2S, and UE-5n) and their locations. From Carle et al. (2005).

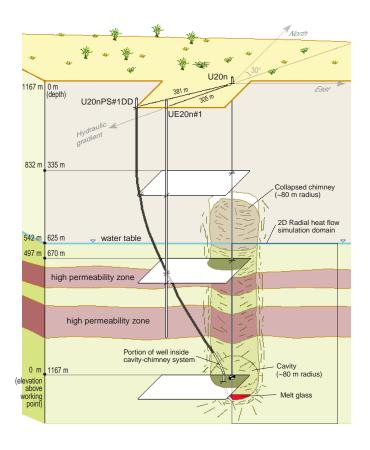


Figure 2. A view of the Cheshire site with associated groundwater sampling wells (U-20n PS#1 DD and UE-20n#1) and their locations. Well UE-20n#1 has not been sampled recently due to pump failure. From Pawloski et al. (2001).

Major challenges in the analysis of ²³⁷Np in environmental samples are its ultra low concentration and lack of a suitable yield tracer. Primary techniques used for determination of ²³⁷Np in environmental samples are alpha spectrometry (Gue´rin 2010, Harvey and Thurston 1988, Lindahl 2004, Prosser et al. 1994, Rosner et al. 1993, Yamamoto et al. 1991), neutron activation analysis (Kalmykov et al. 2004) and the most frequently used ICP-MS (Gue´rin et al. 2010, Kuczewski et al. 2003, Lindahl et al. 2005, Röllin et al. 2009, Truscitt et al. 2001). Other sensitive methods, such as TIMS (Beasley et al. 1998), AMS (Keith-Roach et al. 2001, Wang et al. 2010) and luminescence (Ivanova et al. 1994, Novikov et al. 2009) have also been reported. The advantages of using ICP-MS are high accuracy, speed, and sensitivity, as well as the simultaneous measurement of multiple isotopes. Although isobaric interferences from uranium hydrides such as ²³⁵U¹H₂⁺, ²³⁶U¹H⁺ have been reported (Heras et al. 2002), the most frequently observed interference is the peak tailing (also known as abundance sensitivity) of ²³⁸U. Therefore, a sufficient separation of the uranium from ²³⁷Np is always desired for mass spectrometric analysis. Radionuclide measurement techniques for environmental samples have been reviewed in depth by several authors in recent years (Vajda and Kim 2011, Hou and Roos 2008, Ketterer and Szechenyi 2008, Lariviere et al. 2006).

Suitable yield tracers for ²³⁷Np are limited to ²³⁵Np (half-life of 396 days, Harvey and Sutton 1987), ²³⁶Np (half-life of 154000 years, Efurd et al. 1986, Beasley et al. 1998, Kenna and Sayles 2002, Kenna 2009), and ²³⁹Np (half-life of 2.3 days). Given that isotopes of ²³⁵Np and ²³⁶Np are rare and not readily accessible, the most common yield tracer is the short-lived beta emitter ²³⁹Np (Lindahl et al. 2004, Lindahl et al. 2005, Röllin et al. 2009, Prosser 1994, Harvey and Thurston 1988, Yamamoto et al. 1991). The ²³⁹Np can be "milked" by separating it from its parent isotope ²⁴³Am. The activity of ²³⁹Np tracer can be determined using Gamma spectrometry or Liquid Scintillation Counting (LSC) while the ²³⁷Np concentration can be analyzed using ICP-MS.

In the present study, we set out to measure ultra low levels of ²³⁷Np in a number of ground water samples from hot wells at the NNSS. To achieve this goal, we first developed an effective procedure to separate and purify ²³⁷Np from 1 liter of groundwater. The ²³⁹Np tracer was used to quantify yield. The recent purchase of a state of the art magnetic sector ICP-MS provided the capability to measure ²³⁷Np at ultra-low levels (~0.2 parts per quadrillion (7x10⁻¹⁶ mol/L) for a 1 L groundwater sample). The measured ²³⁷Np concentrations are first compared to Maximum Contaminant Levels established by the EPA for ²³⁷Np (15 pCi/L). The data are also compared to other radionuclides, such as ³H, ¹⁴C, ³⁶Cl, ¹²⁹I, ⁹⁹Tc and plutonium, in order to evaluate ²³⁷Np transport behavior relative to other radionuclides. Finally, the data are compared to recent HST reactive transport models (i.e. Carle et al., 2005; Pawloski et al., 2001) to evaluate the quality of model predictions.

Experimental

1. Sample collection

Eight well samples and one tunnel water sample collected in recent years from all five corrective action units (CAU) at the NNSS were analyzed in the present study (Table 1). Of the samples measured, 5 are from near the working point of tests (Cheshire, Chancellor, Almendro, Dalhart, and Cambric (RNM-1)), three are located downstream of tests (ER-20-5#1, RNM-2S, UE-5n), and one is from a flooded tunnel (T tunnel). A map for the sampling locations is shown in Figure 3. All samples were collected via pumps or bailers directly into new 1L or 20 L plastic containers. The samples were all stored at room temperature at LLNL with no acidification. Samples were not acidified because of an interest in preserving the actinide associations with mineral colloids. However, for the ²³⁷Np analyses, ¹L containers (including colloidal constituents) were acidified prior to sample digestion and the empty sample bottles rinsed with concentrated HNO₃+ HF (5:1) to minimize potential loss of ²³⁷Np to container walls. For samples stored in 20 L containers, containers were shaken vigorously prior to removing 1 L samples for digestion and Np analysis; rinsing of containers was not performed, which leads to possible underestimates of Np concentration due to losses to container walls. This effect is discussed in detail in the text. Only total ²³⁷Np concentrations in groundwater (aqueous and colloid-associated) were investigated.

2. Reagents and tracers

Reagents used were analytical grade or better and used as received. De-ionized water with resistivity of 18.2 megaohm-cm from a Milli-Q Water (MQW) purification system was used in all procedures and solution preparations. NIST traceable SRM 4341 ²³⁷Np standard was diluted and used for ICP-MS calibration. Anion exchange resin AG1x8 (chloride form, 50-100 and 100-200 mesh, Bio-Rad Laboratories) and UTEVA extraction resin (50-100 micron, Eichrom) was used for ²³⁹Np and ²³⁷Np separation and purification. AG resins were cleaned using HCl and MQW and UTEVA resins were cleaned using MQW prior to use. New Teflon jars/vials (Savillex) were boiled in 1:1 HNO₃ and Milli-Q water prior to use.

The ²³⁹Np isotope was milked from its parent nuclide ²⁴³Am and used as a chemical yield tracer for ²³⁷Np separation and purification. To purify the ²³⁹Np, the ²⁴³Am solution was dried and re-dissolved in <1 mL of conc. HCl and HI (50:1 v/v). The ²⁴³Am solution was warmed on a hotplate for 10 minutes to reduce ²³⁹Np to the +4 oxidation state prior to loading onto a purification column. The column was made from a 1 mL disposable transfer pipette plugged with a porous frit and AG1x8, 50-100 mesh resin. The loaded column was rinsed with 5 column volumes (cv) of conc. HCl + HI (50:1 v/v) and ²³⁹Np was eluted using 6 column volumes (cv) of 6 M HCl + 0.05 M HF. Two AG1x8 resin columns were used to ensure a complete separation of ²³⁹Np from its parent ²⁴³Am. This procedure was necessary to reduce and even eliminate positive errors in ²³⁹Np decay measurements from trace levels of ²⁴³Am over several half-lives of ²³⁹Np. The activities of ²³⁹Np were measured using gamma detector (Broad Energy Ge Detector (BEGe), Canberra). Energy branch at 277 keV was used to quantify ²³⁹Np and other energy branches such as 209, 228, 315 and 334 keV were used for ²³⁹Np quality control.

3. Analytical methods

A 1 liter groundwater sample in a Teflon (Savillex) jar was acidified with 10 mL conc. nitric acid and spiked with ²³⁹Np tracer with an activity of 3000 to 5000 dpm. The sample was gently heated on a hotplate and the sample allowed to evaporate. The resulting sample residue was repeatedly digested in concentrated HCl, aqua regia and/or hydrogen peroxide. If a full digestion could not be achieved, the remaining precipitate was repeatedly washed with concentrated HCl. In preparation for column purification, the digested sample was dissolved in 2 mL concentrated HCl+ HI (50:1 v:v) and warmed up on a hotplate for 10 min. to adjust the oxidation state of neptunium to Np(IV). An anion exchange resin column (2 mL AG1x8, 100-200 mesh Poly-prep column, BioRad) was preconditioned with Milli-Q water, conc. HCl, and conc. HCl+HI (50:1 v:v). The Np(IV) sample was loaded onto the column. Tetravalent Np and U were retained on the resin while trivalent Pu and most other impurities eluted. The loaded column was washed with 5 cv of conc. HCl+HI (50:1) and followed by Np elution using 6 cv of 6 M HCl+0.05 M HF (most of the U was retained on the column). Two columns of AG1x8 resin were used in Np separation. After 2 anion exchange columns, the Np in the eluent was dried 2-3 times in dilute nitric acid to oxidize Np to its +5 state. The sample residue was dissolved into 1 mL of 4 M HNO₃ solution. If further purification of Np from uranium was necessary, the Np sample in 4M HNO₃ solution was passed through a small UTEVA column (0.2 mL column bed made of a small disposable transfer pipet). The uranium was retained while Np(V) eluted. The

Np-containing eluent from the UTEVA column was dried on a hot-plate and the residue was redissolved in 1 mL of 2% HNO₃ as the final sample form. Two 1 mL samples of ²³⁹Np spike solution were preserved without processing as gamma counting references for each batch of sample analyses. The activities of ²³⁹Np in purified samples were measured by Gamma spectrometry, and compared with activities in reference samples to determine the ²³⁷Np yield. After all ²³⁹Np decayed away (~6 half-lives), the ²³⁷Np concentrations was analyzed using a NuPlasma HR ICP-MS.

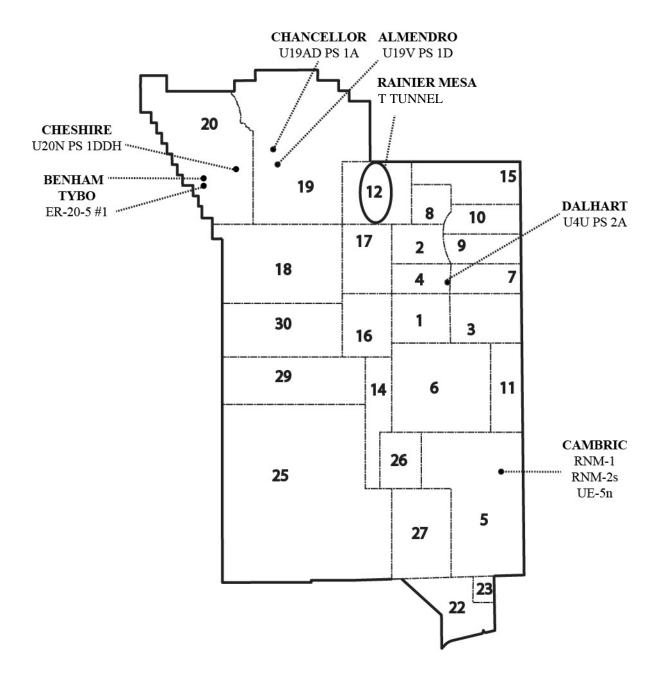


Figure 3. Map of NNSS and sampling locations.

Results and Discussion

1. Separation and purification procedures

The 243 Am "cow" which was used as a 239 Np source contained ~ 10% (by activity) of 241 Am, which decays to 237 Np. Due to its long half-life, 237 Np continues to accumulate in the 243 Am "cow" while 239 Np activity reaches secular equilibrium with 243 Am. This results in an ever increasing 237 Np/ 239 Np ratio as a function of time. As a result, the 239 Np spike can introduce a significant amount of 237 Np into samples and increase the background 237 Np signal. In order to minimize the 237 Np background in the 239 Np spike solution, the 243 Am stock solution was purified from 237 Np using AG1x8 50-100 mesh resin 1-2 days prior to milking 239 Np. The spiked activity was also limited to the lowest possible level that would accommodate the instrument detection limits. Based on the 243 Am/ 241 Am isotope ratio of the 243 Am "cow" and the two-day ingrowth time used, the amount of 237 Np introduced into samples from the 239 Np spike is $< 5x10^{\circ}$ pCi/sample. In theory, the amount of 237 Np introduced by the spike is the limiting factor for the procedure blank. To reduce 237 Np background, simpler and quicker separation procedures combined with a more sensitive, yet accurate counting method is desirable. Alternatively, a stock solution with a higher 243 Am/ 241 Am ratio should be used.

In 237 Np analysis using ICP-MS, the major interference comes from the tailing of 238 U, which is undoubtedly present at much greater quantities compared to 237 Np. Therefore, a good chemical separation of 238 U is required (Hou and Roos 2008). The NuPlasma magnetic sector, multi-collector HR ICP-MS can tolerate low ppb levels of 238 U for quantitative measurements of 237 Np at ppq levels. However, due to the interferences from unknown impurities in the groundwater samples, the AG1x8 resin column was insufficient to reduce uranium concentration to the desirable low ppb or sub ppb levels. Solvent extraction has been used in conjunction with anion exchange columns to achieve a better separation of Np from its interfering impurities (Efurd et al. 1986, Harvey and Thurston 1988) for sub-nanogram levels of 237 Np. However, solvent extraction has not been applied to ultra trace Np separation. In our study, the AG1x8 column was followed by a UTEVA extraction resin column to effectively separate uranium from Np. A quadrupole ICP-MS was used to survey uranium concentrations in 5 μ L Np samples to ensure that sufficient removal of 238 U was achieved. The typical overall Np chemical recovery yields among the samples ranged from 50% to 90%, which was considered acceptable for ultra low level analyses.

2. Neptunium data

The 237 Np data are grouped by CAU and listed in Table 1. The detectable 237 Np concentrations ranged from 2.7×10^{-5} in well RNM-1 (Cambric cavity) to 7.1×10^{-2} pCi/L in well U-19ad PS1a (Chancellor cavity). The measured 237 Np concentrations in all samples were well below the maximum contaminant level (MCL, 15 pCi/L) for alpha emitters identified by the U.S. EPA (2009). The Np concentrations in groundwater samples from Almendro (U-19v PS1ds) hot well and two Cambric satellite wells (RNM-2s and UE-5n) were below our detection limit of 1×10^{-5} pCi/L. Duplicate samples were run at Cambric (RNM-2s), Dalhart (U4u PS2a),

Chancellor (U19ad PS1a), and Almendro (U-19v PS1ds) and reproducible results within uncertainties of 10% or less (when detectable) were obtained.

Cheshire samples collected on 11/15/05 in two different containers analyzed in different batches differ by an order of magnitude. This may have been caused by the long storage times and the fact that the 20 L container from which one sample was taken was not acidified during sample processing. Loss of Np to container walls may have been a problem. Further investigation is needed to positively identify the cause. Analysis of acidified samples with shorter hold times is always preferable.

Regardless of sampling location, all samples except the high concentration Cheshire sample resulted in ²³⁷Np concentrations up to 3 orders of magnitude lower than the previously reported values (Finnegan and Thompson, 2002). The ²³⁷Np data previously reported was obtained from groundwaters sampled at the same locations but on different dates. Nevertheless, the differences between these two sets of data suggest the existence of a systematic potential error in historical data.

The RNM-1 ²³⁷Np data obtained in the present study was about an order of magnitude lower than the Np concentrations predicted in HST models (1.5×10⁻⁴ pCi/L, Carle et al. 2005). Uncertainties in the radiologic source term (based on an average source term for all tests in Frenchman Flat from Bowen et al., 2001), in the partitioning behavior of Np (IAEA, 1998), and in nuclear melt glass dissolution which releases Np into groundwater are likely to limit accuracy of unclassified source term models to within an order of magnitude or larger. Thus, the measured values at RNM-1 should be considered in good agreement with model predictions. Modeling results also suggested that insignificant concentrations of ²³⁷Np would reach the RNM-2s and UE-5n wells, which is consistent with the non-detectable levels determined in our analyses.

The Np concentrations in groundwater from the cavity/chimney area of the Cheshire test were found to be 3×10^{-4} and 6×10^{-3} pCi/L in two separate analyses. As stated earlier, the relatively large difference between the two analyses may be the result of losses of Np to container walls during the long sample storage time, particularly for the sample stored in the 20L container which was not acidified. The Np concentration previously reported was 3×10^{-2} pCi/L, substantially higher than the values measured in this analysis. HST modeling (Pawloski et al., 2001) using particle transport methods did not accurately resolve Np distribution at low concentrations; Np concentrations in the cavity chimney 23.5 years after the test ($t_0 = 1976$) were predicted to range from 10^{-8} mol/L (1600 pCi/L) in the glass zone to below 10^{-13} mol/L (1600 pCi/L) in the upper cavity and chimney. The high 100 pCi/L in the cavity chimney area are in qualitative agreement with predicted values for the upper cavity and chimney zones. However, the poor resolution of Np concentrations and various assumptions included in the model limits the applicability of this comparison.

Table 1. ²³⁷Np, ³H and total Pu activities in NNSS Groundwater Samples (all data were decay corrected to 9/23/92)

CAU ¹	Test Name	Sampling well/Location	Sampling date	This method		Previously Reported Value		
				²³⁷ Np	+/-	²³⁷ Np	^{3}H	Pu-total
						pCi/L		
Central - Pahute Mesa -	Almendro	U-19v PS1ds; Cavity/chimney	4/18/2006	< 1E-5		-	2.32E8	< 0.004
			4/18/2006	< 1E-5		-	2.32E8	< 0.004
			8/17/1999	-		< 0.0108	2.18E8	9.5
	Camembert	U-19q PS1d; Chimney	10/20/1998	=		0.0216	2.94E7	0.019
	Chancellor	U19ad PS1A; Cavity	5/1/2008	7.1E-2	1E-3	-	3.09E7	23.5
			5/1/2008	7.0E-2	1E-3	-	3.09E7	23.5
	Cheshire	U-20n PS1ddh; Cavity	11/15/2005	$3.3E-4^2$	3E-6	-	6.98E7	0.46
			11/15/2005	$5.5E-3^3$	3E-5	-	6.98E7	0.46
			10/12/1999	-		0.027	7.58E7	0.51
Western Pahute Mesa	Tybo/Benham —	ER-20-5 #1; 280/1300 m from	11/30/2004	3.3E-4	1E-6	-	7.48E7	0.42
		Tybo/Benham	7/09/1998	-		0.149	8.60E7	0.59
		ER-20-5 #3; 280/1300 m from Tybo/Benham	4/30/1998	-		0.0211	2.14E5	NA
Ranier Mesa	Hot Tunnels	U-12t gas sealed plug	11/2/2006	3.3E-4	3E-6	-	1.06E8	54
Yucca Flat	Dalhart	U-4u PS2a; Chimney	2/6/2008	6.6E-5	3E-7	-	5.71E7	0.19
			2/6/2008	5.9E-5	5E-7	-	5.71E7	0.19
Frenchman Flat	Cambric	RNM-1, cavity	3/6/2007	2.7E-5	1E-6	-	1.95E3	0.006
			6/28/2000	-		0.054	4.40E4	NA
		RNM-2s; 91 m from Cambric	3/7/2007	< 1E-5		=	2.26E5	< 0.02
			3/7/2007	< 1E-5		-	2.26E5	< 0.02
			6/14/2000	-		0.054	2.93E5	NA
		UE-5n; 510 m from RNM-2s	2/12/2004	< 1E-5		-	2.79E5	< 0.02
			4/19/2001	-		0.054		

CAU: Corrective Action Unit
Groundwater sample stored in 20-liter container prior to analysis.
Groundwater sample stored in 1-liter container.

3. Neptunium transport behavior

As an anthropogenic radionuclide, the main sources of ²³⁷Np at NNSS are from the original test-derived source material and as a decay product of short lived ²³⁷U, which was produced through neutron capture reactions of device components ²³⁵U and ²³⁶U (Bowen 2001, Smith et al. 2003). The ²³⁷Np is also produced via the ²⁴¹Am and ²⁴¹Pu decay chain. Previous studies of environmental transport of ²³⁷Np have shown that the mobility of ²³⁷Np conspicuously depended on its speciation and redox conditions of the surroundings (Dozol and Hagemann 1993, Choppin 2006). Neptunium was found to be very mobile in surface waters such as Irish Sea (Assinder 1999), Lake Karachai (Novikov et al. 2009) and Swedish-Danish waters (Lindahl et al. 2005) due to the oxygenated water conditions. Even if the initial distribution of Np is similar, if not the same, as Pu, the former radionuclide tends to become pentavalent and mobile as it migrates away from its origins over time (Lindahl et al. 2004& 2005, Novikov et al. 2009). The groundwater conditions in NNSS hot wells are relatively isolated and the redox conditions can vary (Hu et al. 2008, Nimz and Thompson 1992). As a result, oxidizing conditions cannot be assumed for all NNSS groundwaters. Thus, we cannot assume that ²³⁷Np transport behavior will be governed by the behavior of the oxic +5 cation.

Activity ratios can be used to evaluate the mobility of Np relative to other radionuclides. For example, if the ratio of Np to a non-sobing tracer in the mobile phase is low relative to its initial source term ratio, it suggests that Np is retarded or otherwise immobilized. However, in the case of underground nuclear tests, one must also take into account that a fraction of initial source term will be deposited in nuclear melt glass, effectively isolating it from groundwater. The IAEA (1998) has published partitioning estimates (i.e. estimates of groundwater accessible source terms) for many radionuclides associated with underground nuclear tests. Thus, we can compare isotope ratios relative to the expected groundwater-accessible radionuclide ratios using the following equation:

$$R = \frac{\left(\frac{237Np}{RN}\right)_{measured}}{\left(\frac{237Np}{RN}\right)_{source} \times \left(\frac{f_{237Np}}{f_{RN}}\right)_{IAEA}}$$

where ²³⁷Np and RN are the measured concentration and source term (Bowen et al. (2001)) inventory of ²³⁷Np and reference radionuclide in a particular hot well and CAU, respectively, and *f* is the fraction of the inventory that is groundwater accessible based on IAEA (1998). The fraction that is groundwater accessible is 100% for tritium and ¹⁴C, 50% for ³⁶Cl and ¹²⁹I, 20% for ⁹⁹Tc, and 5% for ²³⁷Np, and 2% for Pu. If the ratio, R, is less than 1, it indicates a relatively smaller fraction of neptunium in the mobile phase relative to the reference radionuclide (i.e. ²³⁷Np depletion). The uncertainty in the source term and groundwater accessible fraction limit the efficacy of this comparison. Nevertheless, values <<1.0 indicate that Np is substantially depleted relative to the reference radionuclide. In most cases, this depletion can be attributed to its greater sorption (lower mobility) relative to the reference radionuclide.

Figure 4 is a plot of activity ratio of ²³⁷Np relative to ³H as a function of tritium concentration for all wells with detectable ²³⁷Np concentrations. In all cases, tritium has been decay corrected to September 23, 1992, as reported in Bowen et al. (2001). For reference, we also identify the Maximum Contaminant Level as defined by the EPA (2009) for tritium. As stated earlier, all ²³⁷Np concentrations are well below the MCL for that radionuclide. However, most samples are well above the MCL for tritium (20,000 pCi/L). Importantly, the R values for all hot well samples except RNM-1 in Frenchman Flat are << 1, suggesting that Np is much less mobile than ³H. Well RNM-1 is unique in that nearly all the ³H was pumped out of the cavity during the course of a 20 year pumping experiment at RNM-2s (Bryant 1992, Nimz and Thompson 1992). At RNM-1, the relatively high ²³⁷Np concentrations relative to tritium results from the fact that the source term tritium has been effectively removed from the cavity while ²³⁷Np has not.

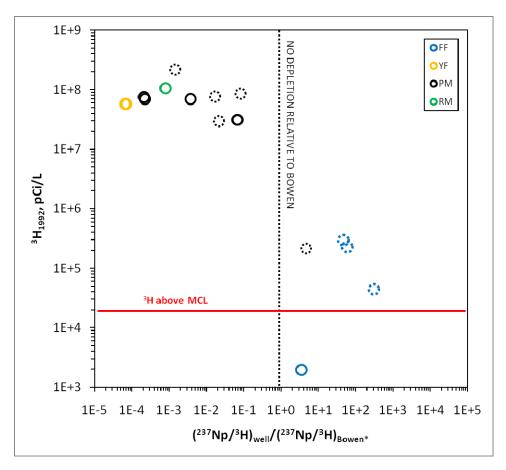


Figure 4. ²³⁷Np/³H isotope ratio in groundwater relative to source term ratio based on Bowen et al. (2001) and IAEA (1998). Values <<1 are indicative of retardation. FF data point is from well RNM-1. Dashed circles represent historical data from the UGTA geochemistry database. Color scheme indicates the CAU of sampled well locations with: 1) FF = Frenchman Flat, 2) YF = Yucca Flat, 3) PM = Pahute Mesa, 4) RM = Ranier Mesa.

The data in Figure 4 clearly indicate that ²³⁷Np is retarded relative to ³H. We can qualitatively simulate the effect of retardation in the cavity/chimney of a test by fitting a Continuous Stirred-Tank Reactor (CSTR) model to the cavity/chimney hot well samples observational data. The model treats the cavity/chimney system as an equilibrium CSTR for which the concentration evolution of each radionuclide is predicted by the following equation:

$$C_i = C_0 \times e^{\left(\frac{J \times t}{R \times V}\right)}$$

where the concentration of species i is a function of the initial aqueous concentration (C_0), the groundwater flux (J), time (t), retardation (R), and reactor volume (V). For this model, an average radionuclide initial concentration in the cavity/chimney region was calculated based on the total radiologic inventory for each CAU (Bowen et al., 2001), the sum of all exchange volumes for each test calculated using the following: $70.2 *Y^{1/3}/[\rho_{ob} *WP]^{1/4}$ (Pawloski, 1999) where Y is the announced yield or maximum of announced yield range from DOE/NV-209 rev.15 (2000) (kt), ρ_{ob} is an approximate overburden density (2.0 Mg/m^3), and WP is the working point depth (meters). The exchange volume was assumed to be spherical with a radius of 1.5 Rc, fully saturated, and with a porosity of 30%. The groundwater accessible fraction was based on IAEA (1998) partitioning estimates.

Figure 5 compares the cavity/chimney data to CSTR model predictions based on a retardation factor (R) of 1000 which is equivalent to a Kd of 170 mL/g. The CSTR model produces a straight line on a log-log plot which represents the decrease in tritium concentration and the increase in the ²³⁷Np/³H isotope ratio over time. The qualitative match to the data suggests that effective Kds for Np lie broadly in a range from 10 mL/g to values >100 mL/g. These retardation values are higher than typically reported for Np sorption to tuff and alluvium (YMP, 2004). Most likely, they reflect conservative estimates of Np partitioning to groundwater, variability between individual test radiologic source terms, and processes other than sorption (e.g. coprecipitation/sequestration of Np in secondary mineral phases). Regardless of the processes involved, the modeling results indicate that overall Np retardation appears to be substantial and provides an explanation for the low ²³⁷Np concentrations observed in hot well cavity/chimney samples (the source of radiologic contamination from underground nuclear testing). We do not anticipate that ²³⁷Np concentrations will evolve over time to approach concentrations that would surpass MCL limits identified by the EPA (2009) for alpha-emitting radionuclides.

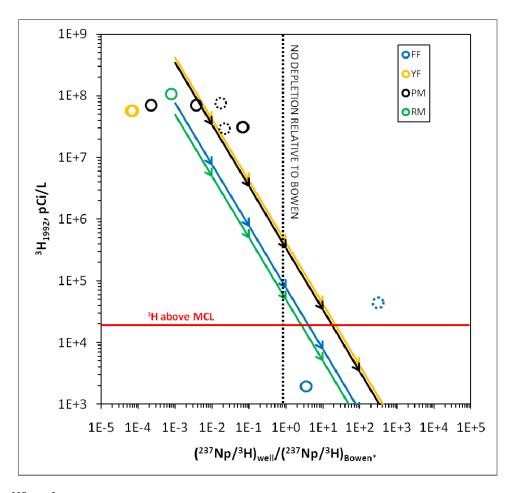


Figure 5. ²³⁷Np/³H isotope ratio in cavity/chimney groundwater samples relative to initial ratio based on Bowen et al. (2001) and IAEA (1998) and comparison to model prediction of CSTR with a retardation factor of 1000 (lines and arrows represent CSTR-predicted evolution of isotope ratio in the cavity as a result of groundwater flow). Dashed circles represent historical data from the UGTA geochemistry database. Color scheme indicates the CAU of sampled well locations with: 1) FF = Frenchman Flat, 2) YF = Yucca Flat, 3) PM = Pahute Mesa, 4) RM = Ranier Mesa.

Figure 6 contains plots of activity ratios of ²³⁷Np to other presumed mobile radionuclides (¹⁴C, ³⁶Cl, ⁹⁹Tc and ¹²⁹I). The results compare favorably with the tritium ratios, suggesting that ²³⁷Np in the analyzed NNSS hot wells is generally less mobile than the commonly considered non-sorbing radionuclides. Importantly, historical ²³⁷Np concentration measurements are consistently higher than those measured here, possibly resulting from the higher detection limits or other artifacts. In some cases (e.g. ³⁶Cl data), this results in an apparent Np mobility that is greater than non-sorbing radionuclides (ratio >>1 in high activity samples). ³H and ³⁶Cl have been shown to exhibit unretarded migration over a wide range of environmental conditions (Hu et al. 2008). Thus, the greater mobility of ²³⁷Np compared to ³⁶Cl is unlikely. While ¹⁴C, ¹²⁹I and ⁹⁹Tc can exhibit some retardation depending upon the type of geological media and redox conditions of their surroundings (Hu et al. 2008), it appears that ²³⁷Np is relatively less mobile than these radionuclides. Importantly, the uncertainties and assumptions made in calculating these ratios lend themselves to only a qualitative assessment of the relative mobility of ²³⁷Np.

Figure 7 shows relative mobility of neptunium to total plutonium in the studied wells. Surprisingly, the ratios are centered near 1 for most hot well data, suggesting that neptunium mobility is similar to plutonium at these locations. The historical data appear to be biased towards greater Np mobility. The observations are suggestive of two possible transport phenomena: 1) the predominance of Np(IV) in the mildly reducing groundwater conditions exhibits transport behavior similar to Pu(IV) or 2) an apparent transport of Np is the result of transport of its parent ²⁴¹Pu and ²⁴¹Am isotopes and subsequent decay to ²³⁷Np. In the first case, Np and Pu transport behavior is similar because both actinides are predominantly in the +4 state and exhibit similar sorption/retardation. In the second case, observed ²³⁷Np is a product of the decay of ²⁴¹Pu and ²⁴¹Am which have similar transport behaviors as other Pu isotopes (²³⁹Pu and ²⁴⁰Pu). The available data cannot be used to distinguish between these two possibilities. The high ratio observed at RNM-1 suggests that oxygenated waters introduced into the cavity/chimney as a result of ~20 years of pumping at this location may have remobilized Np by oxidizing it to Np(V). However, this explanation is not unique and other hypotheses may be equally plausible.

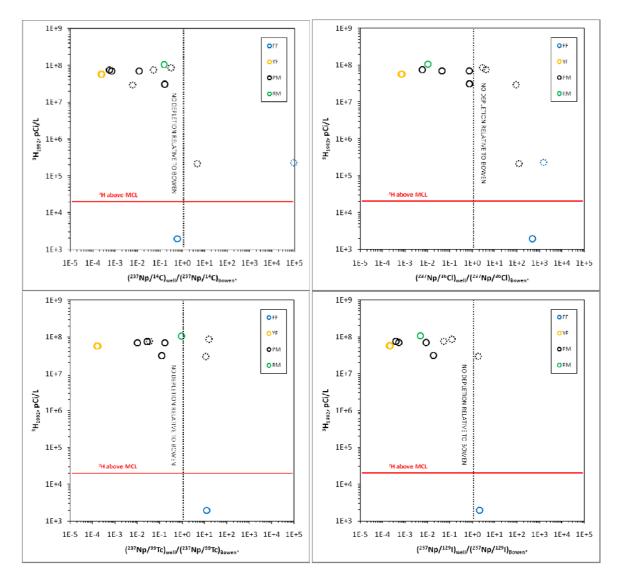


Figure 6. Isotope ratios of ²³⁷Np/¹⁴C, ²³⁷Np/³⁶Cl, ²³⁷Np/⁹⁹Tc, and ²³⁷Np/¹²⁹I in groundwater relative to initial ratio based on Bowen et al. (2001) and IAEA (1998). Values <<1 are indicative of retardation. Dashed circles represent historical data from the UGTA geochemistry database. Color scheme indicates the CAU of sampled well locations with: 1) FF = Frenchman Flat, 2) YF = Yucca Flat, 3) PM = Pahute Mesa, 4) RM = Ranier Mesa.

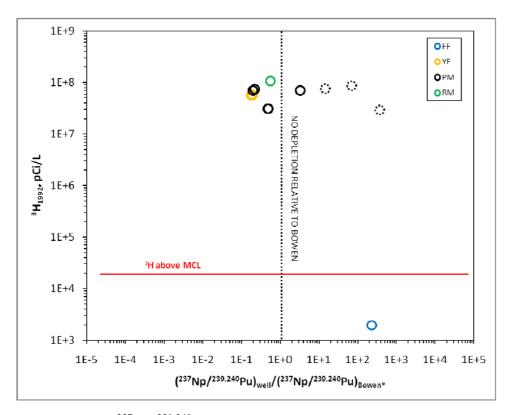


Figure 7. Isotope ratios of 237 Np/ 239,240 Pu in groundwater relative to initial ratio based on Bowen et al. (2001) and IAEA (1998). Dashed circles represent historical data from the UGTA geochemistry database. Color scheme indicates the CAU of sampled well locations with: 1) FF = Frenchman Flat, 2) YF = Yucca Flat, 3) PM = Pahute Mesa, 4) RM = Ranier Mesa.

Conclusions

We have successfully measured ultra trace levels of ²³⁷Np in groundwater samples collected from NNSS hot wells and tunnels in five different CAUs using an isotope dilution method. A short lived ²³⁹Np yield tracer in conjunction with a state of the art magnetic sector HR ICP-MS were used for the ²³⁷Np analyses. The measured ²³⁷Np concentrations in all samples were below the maximum contaminant level (MCL) of ²³⁷Np proposed for drinking water by the U.S. EPA (2009). The analyses of transport behavior of ²³⁷Np in these NNSS groundwaters suggest that ²³⁷Np was substantially less mobile than tritium and other conservative nuclides as expected. Surprisingly, its mobility appeared to be similar to that of plutonium (^{239,240}Pu) either as a result of its reduction to Np(IV) and chemical similarity to Pu(IV) or as an artifact of its ingrowth from parent ²⁴¹Pu and ²⁴¹Am.

The measured ²³⁷Np concentrations by the present work were generally lower than previously reported. The difference may be, in part, a result of the high detection limits in historical data. However, additional data collection activities are needed to verify and quantify the measurement uncertainties and variabilities reported here.

The measured ²³⁷Np concentrations in Cambric and Cheshire hot wells were compared to recent HST modeling efforts. The RNM-1 ²³⁷Np data obtained in present study, about an order of magnitude lower than the values predicted by HST transport modeling (1.5×10⁻⁴ pCi/L, Carle et al., 2005), is considered an acceptable validation of HST transport models. Modeling results also suggested that insignificant concentrations of ²³⁷Np would reach the RNM-2s and UE-5n wells, which is consistent with the non-detectable levels determined in our analyses. At the Cheshire site, HST modeling (Pawloski et al., 2001) using particle transport methods did not accurately resolve Np distribution at low concentrations; Np concentrations in the cavity chimney 23.5 years after the test ($t_0 = 1976$) were predicted to range from 1600 pCi/L in the glass zone to below 0.02 pCi/L in the upper cavity and chimney. The high ²³⁷Np concentration predicted for the glass zone is the result of very high simulated glass dissolution rates. The measured 237 Np concentrations in the cavity chimney area (3×10^{-4} and 6×10^{-3} pCi/L) are in qualitative agreement with predicted values for the upper cavity and chimney zones. However, the poor resolution of Np concentrations and various assumptions included in the model limits the applicability of this comparison. While ²³⁷Np remains an important indicator for radionuclide transport rates at the NNSS, it does not appear to pose a significant hazard in relation to U.S. EPA maximum contaminant levels for alpha-emitting radionuclides.

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