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LOFT MONTHLY PROGRESS REPORT FOR SEPTEMBER 1980

N. C. Kaufman

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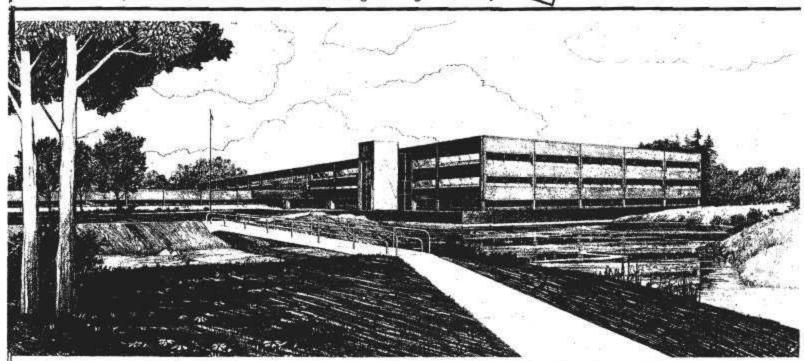
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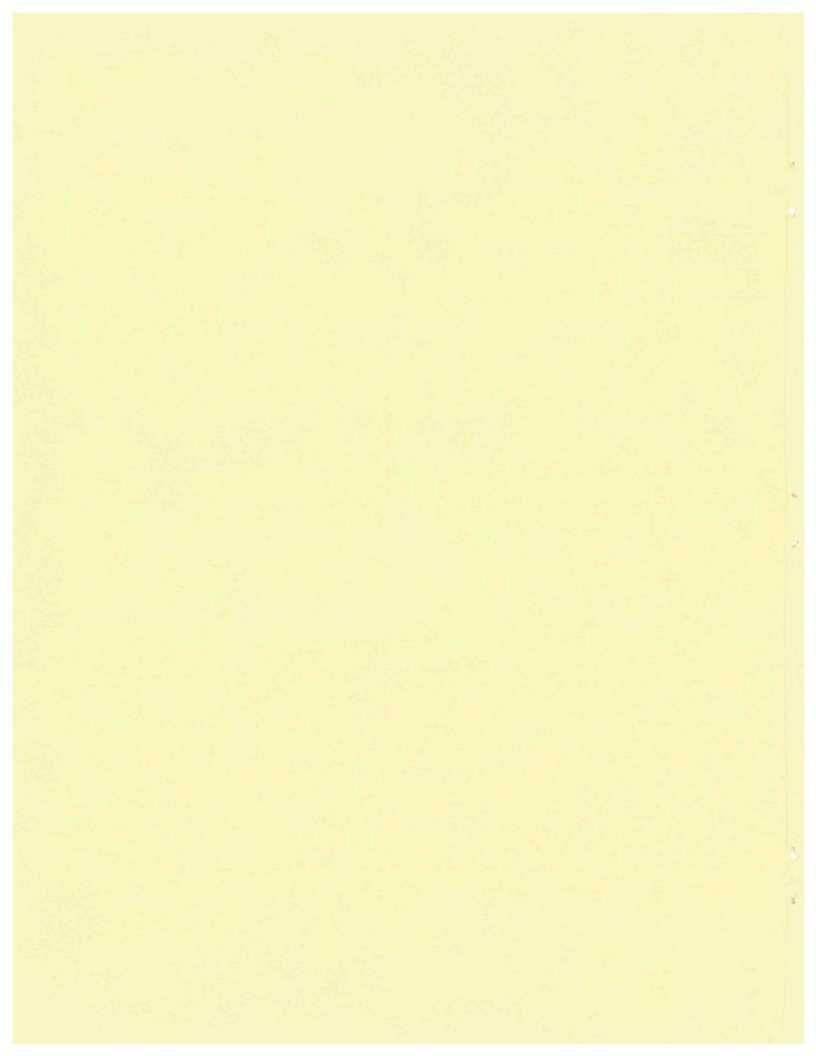
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This is an informal report intended for use as a preliminary or working document

Prepared for the U. S. Nuclear Regulatory Commission Under DOE Contract No. DE-ACO7-76ID01570 FIN No. A6048







#### INTERIM REPORT

Access	ion N	lo	
Report	No	EGG-LOFT-5276	

Contract Program or Project Title:

LOFT

Subject of this Document:

LOFT Monthly Progress Report for September 1980 Type of Document:

Interim Report

Author(s):

N. C. Kaufman

Date of Document:

October 1980

Responsible NRC Individual and NRC Office or Division:

G. D. McPherson, LOFT

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

EG&G Idaho, Inc. Idaho Falls, Idaho 83415

Prepared for the
U.S. Nuclear Regulatory Commission
Washington, D.C.
Under DOE Contract No. DE-AC07-761D01570
NRC FIN No. A6048

INTERIM REPORT

Date:

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#### LOFT MONTHLY PROGRESS REPORT FOR SEPTEMBER 1980

#### DIRECTOR'S MONTHLY SUMMARY

The fourth nuclear powered small break test (L3-5/5A) was conducted on September 29, 1980. The test was initiated from a steady state operating condition wherein the core was generating heat at a maximum rate of approximately 52 kW/m. The test consisted of two parts: L3-5 simulated a 4-in. pipe break in a commercial pressurized water reactor; the second part, L3-5A, was intended to investigate natural circulation and steam generator heat transfer modes and also plan recovery using secondary system control in a situation where the pipe break and the ECCS accumulator are isolated from the primary coolant system. Initial test data indicated that all systems functioned as expected. The several hundred measurements of system coolant and reactor core conditions made during the three hour duration of the test will continue to be analyzed over the next several months.

Preparations were also underway for conduting three tests in the Anticipated Transient Series. These tests, designated L6-1, L6-2, and L6-3 will provide information on plant control systems and operator response to transients in which the initiating event is not a loss-of-primary coolant. The transient tests to be conducted during September and others scheduled in the future will add greatly to understanding responses necessary to a transient condition.

September 1980 marked the successful completion of FY-1980. Final closing values for each of the funding sources are included as part of this report (see Table 7).

NRC and foreign funded tasks closed FY-1980 with underruns documented by identifying committed and uncommitted carryovers.

#### ACCOMPLISHMENTS

#### LOFT TECHNICAL SUPPORT DIVISION

- Two Document Revision Requests (DRRs) (L-3172 and L-4015) were submitted to the Department of Energy-Idaho Office (DOE-ID) for review and approval for updating the LOFT Final Safety Analysis Report (FSAR) to incorporate modifications and additions to the plant design.
- The Experiment Safety Analysis (ESA) for the conduct of test L3-5/L3-5A
  was submitted to DOE-ID and received approval. The supporting analysis
  document, LOFT Technical Report (LTR) L0-11-80-096, has also received
  EG&G Idaho, Inc. management approval and has been issued.
- 3. The following Technical Specifications changes have been submitted to DOE-ID for approval:
  - A. DRR L-3851, Rev. 1 (L3-5 Test)--DOE-ID has approved.
  - B. DRR L-3853 (L6-1, L6-2, and L6-3) DOE-ID is reviewing.
  - C. DRR L-3854 (to correct typographical errors) DOE-ID is reviewing.
    - D. DRR L-3855 (transversing in-core probe system (TIPS) use during L3-5) - DOE-ID has approved.
- 4. The following failure modes effects and consequence analyses (FMECA) were completed:
  - A. System reviews of the primary coolant system, emergency core cooling system, and Loss-of-Coolant Experiment (LOCE) control systems for the conduct of Small Break Test L3-5/L3-5A.
  - B. Overall system review for the conduct of L6-2 and L6-3 (two separate documents).

- A safety issues letter was prepared which identifies concerns for the conduct of the L3-5/L3-5A experiment that are not published in any other formal safety document.
- The L3-5/L3-5A small break experiment was conducted successfully.
- 7. Safety analysis documentation for L6-1, L6-2, and L6-3 was issued.
- The LOFT Technical Support Center (TSC) was placed in operation, manned by Reactor System Branch personnel, and used successfully during the L3-5 small break test.
- Critical boron concentration and expected power distribution for LOCE L3-5 (JBB-1-80) was computed.
  - 10. A design review for the linear table assembly of the isotope detection system has been held, and the integrated planning for the electronic fabrication of interfaces has received a complete review from Quality Division and the Project System Engineer. After the incorporation of minor modifications, the Physics Division started the work.
  - 11. Installation of the snubber test stand and its missile barrier has been completed. The test stand was formally accepted by EG&G after the demonstration. Training of operators and technical support personnel was conducted by an MTS Systems Corporation representative during the second week of September.
- 12. The Inservice Inspection (ISI) evaluation group approved the data packages from L3-5. The ISI manual was completed and has been reviewed by EG&G management. The ISI manual was forwarded to DOE-ID for review four days ahead of schedule.
- 13. Disposal of the spent resin stored in the Hot Cells has been completed. The cask was emptied, cleaned, decontaminated, and reconditioned; and the transfer system has been modified to receive the resin from the blowdown suppression tank (BST) and/or coolant purification system (CPS) ion exchangers.

- 14. The 6000-gallon blowdown header decontamination solution hold tank has been delivered. The tank is equipped with controls and support frame. Inspection by the Quality Division is pending.
- Installation of the intact loop experimental piping configuration (including all supports, hangers, and as-built software) was completed in preparation for L3-5/5A.
  - 16. Several valves were reworked to improve reliability and flow characteristics commensurate with system requirements.
  - 17. An analysis [LOFT Technical Report (LTR) LO-35-80-016] of a potential hydrogen detonation in the containment vessel was prepared. The conservative assumption was made that all the hydrogen generated by a reaction of all the core zirconium with water would mix with air at the optimum ratio in a spherical shape at the containment centerline near the reactor head. The analysis based on this assumption indicated that the pressure wave pulse at the containment vessel wall would be less than 70 psig. No attempt was made to quantify the duration of the pulse wave. This analysis resulted from one of the Three Mile Island (TMI-2) safety issues.
  - 18. The polyethylene neutron shield beneath the mobile test assembly structure was removed. This shield was less effective than the shield that uses dry boric acid. The boric acid shield was installed under the reactor vessel prior to L3-7 and will be retained. Removal of the polyethylene shield reduces the containment combustible inventory.
- The A3 fuel module assembly at Building TAN-615 was completed.
  - 20. Neutron radiography of the poison rods was completed, and the final report will be issued in October. No defects were noted in the rods. Final assembly of the rods to the spider will begin at Building TAN-615 in October.

- 21. Phase III examination (channel spacing probe measurements) was completed. No major anomalies were noted, and a report is being prepared by the Fuels and Materials Division.
- 22. A preliminary design review of the LOFT fuel rod transfer cask was conducted. However, the final review will be delayed until the stress and accident analysis are completed.
- 23. Fabrication of the boron charging system was completed. Design of the flow skirt removal cask was completed, and the drawings are in checking.
- Waste Gas Processing System vault ventilation system instrumentation was installed.
  - 25. Secondary coolant system flow transmitters FT-P4-62A and -62B have been installed; the output signals from these instruments have been patched into the PRIME system.
    - The Heating and Ventilation (HV) System 10 redundant constant air monitors (CAMs) have been installed. A total of four HV System 10 CAMs are now in operation to monitor contamination of air to the LOFT control room under certain accident conditions.
  - 26. A preliminary version of the LOFT instrumentation accuracy analysis LTR has been submitted to LOFT Management for review.
  - 27. A preliminary report, entitled "Reliability Analysis of the LOFT Power Range Nuclear Instrument System," has been completed and distributed for internal review.
  - 28. Specification ES-60337 has been prepared for qualifying the plant protection systems (PPS) A, B, and C distribution panels TPP-1, TPP-3, and PP-5. A work package has been prepared to install the regulators.

- 29. Design was completed of protective covers for the nuclear instrumentation system cables where they exit the shield tank instrument wells, and the covers have been sent to the field for installation. These covers will protect the cables from abuse from work being done on top of the LOFT shield tank.
- 30. Design of an instrument buffer seismic test assembly is complete. Assembly will be used to seismically qualify buffers used in LOFT instrumentation systems.
- 31. The engineering package to route cables between the halon panels in rooms 218 and 219 in Building TAN-630 and the American District Telegraph (ADT) cabinets in room B-104 is ready for field work. The cables will connect the two halon panels to the fire alarm systems.
- 32. A design package was released to upgrade primary coolant flow channels A and B by installing new Rosemount differential pressure flow
  transmitters in the channels. The Rosemount transmitters will replace
  the present Fisher-Porter transmitters and will provide increased
  accuracy, wider range, and higher reliability than the present units.
- 33. Seventeen of the 33 safety-related design descriptions (DDs) were submitted to DOE-IO for approval. The remaining 16 DDs are now in the LOFT approval chain.

## LOFT FACILITY DIVISION

- All Inservice Inspection (ISI) surveillance and plant testing required for L3-5 were completed.
- 2. Test L3-5/5A was completed successfully on September 29, 1980.
- Preparations were begun for performing tests L6-1, L6-2, and L6-3 scheduled for early October.

#### LOFT MEASUREMENTS DIVISION

- The clad surface thermocouples for the NEPTUN Test Program have been shipped to Switzerland.
- 2. Six embedded thermocouples were fabricated for the REBEKA electrically heated test rods. Final calibration and acceptance testing have not been completed. Exxon Nuclear, which has contracted to perform the installation in the test rods, was given weld samples and 0.030-in. diameter zircaloy cabling for coil winding development. Drafts were completed for all related engineering specifications and thermocouple probe fabrication procedures needed to commence qualified production of probes for the LOFT F1 fuel bundle.
  - 3. The two core inlet modular drag disk turbine transducers (MDTTs) planned for installation into the F1 fuel bundle were found to have bearing and design deficiencies to the extent that a decision was made to replace them. Two new units have been initiated, and flow test calibration was performed this month. The authorizing Change Control Board (CCB) requests have been initiated but may not be approved until the scheduled CCB meeting on October 17.
  - 4. Work was continued to establish the upper useful temperature limits and requirements for the clad surface thermocouples. The testing to measure the thermocouple output drift rates and autoclave testing was completed. The results are being analyzed and a report will be written.
  - Kaman Sciences delivered the remaining eight Model A centerline thermocouples.
  - 6. The L3-5 PC-2 MDTT rake was assembled, installed, and operated. All units operated through the initially scheduled test date; however, the A and B turbines failed before the rescheduled test could be run. Operation after the test indicated that the rotors are still intact but may be binding.

- Period electronic drawings were completed for the modular drag disk turbine.
- 8. The acoustic emission system sensors were installed on the L3-5 small-break orifice located immediately downstream of the gamma densitometer spool piece. The data from the sensors will be analyzed later by Technology for Energy Corporation to evaluate the possibility of measuring fluid mass flow through the orifice using acoustic emission techniques.
- Four thermal shields were fabricated for the gamma densitometers. Three
  were installed at PC-1, PC-2, and BL-1 in time for test L3-5; the fourth
  will be installed at BL-2 for test L3-6.
- Data were taken at all installed gamma densitometer locations for the L3-5 experiment.
- The source cask for the nuclear-hardened PC-3 densitometer was fabricated and shipped to Measurement, Incorporated.
- Fabrication and testing of the PC-3 nuclear-hardened densitometer detector preamplifiers were completed.
- Megabyte-stabilized densitometer (MSD) electronics were installed and tested at BL-2A. Preliminary results indicate that the MSD is a viable option for the densitometer system.
- 14. The new small-break pressure transducers were installed. These transducers will provide information on the primary coolant pressure upstream and downstream of the gamma densitometer spool piece orifice and upstream of the drag disk turbine spool piece.
- 15. The pressure transducers for measuring blowdown suppression tank level were upgraded, and an additional differential pressure transducer was installed in parallel with the existing transducer installation.

- 16. All spool piece instrumentation operated properly for the L3-5 test, including the turbine, drag screen, pressure and differential pressures, low energy densitometer, and thermocouples.
- 17. Final revisions were made in the densitometer uncertainty analysis, and the report was approved and sent to be printed.
  - 18. Steam generator liquid level discrepancies were resolved, DAVDS coefficients were calculated, and density correction formulas were generated for the L3-5 test.
  - 19. Various test data were extensively analyzed to obtain meaningful calibration coefficients for the MDTT in the L3-5 spool piece.
  - The Exxon test of the in-core MDTT was concluded, and the data will soon be evaluated in detail.
- Software was developed to calculate density and velocity downstream of an orifice, assuming a constant enthalpy and no slip across the orifice.
  - 22. The Al center fuel bundle channel spacing measurements were completed. Preliminary data evaluation indicates all rod-to-rod and rod-to-guide-tube spacings are greater than the minimum spacings specified for new fuel bundles. This bundle has been subjected to two large break LOCAs as well as operation at full power (16 kW/ft).
  - The fuel modules component final design description, CDD 1.1.1.1C, was issued.
  - 24. An evaluation was made of the Exxon Nuclear and EG&G predictions of the capability of fuel rods (which have damaged annealed cladding) in the LOFT peripheral fuel bundle to withstand (a) the power ramp in operational transient L6-3 and (b) the results of power escalations in the LOFT lead rod test. The resulting recommendations were (a) to precondition the fuel at or above the peak power expected in the test for 4 to 8 hours and (b) to relax the power escalation limits of

- the LOFT reactor in a manner that could accelerate the L3-5, L6-2, and L6-3 test sequence by approximately 4 days.
- 25. The A3 center fuel module was completed and is ready for installation. This module features advanced instrumentation including inlet flow meters, fuel centerline thermocouples, and fuel-rod-length measurement.
- 26. Exxon Nuclear completed 1500 hours of flow life testing of the inlet flow meter. No structural damage occurred to the mounting brackets or fasteners. The flow meter turbine bearing failed at 460 hours in the model with first generation graphite bearings and at approximately 1000 hours in the model with improved graphite bearings.

#### LOFT PROGRAM DIVISION

- 1. The initial calculations of noncondensible effects on steam generator heat transfer and natural circulation in the LOFT system during LOCA conditions have been completed. For LOFT small break experiments, the results of the initial calculations indicate that the effect of realizable noncondensible gas concentration on either natural circulation or steam generator heat transfer is small. These results are based on the assumptions that the ECC performs as designed and by doing so prevents significant fuel cladding failure. Further refinements of the calculations and parametric analyses are being done.
- 2. A draft of a paper, titled "A Correlation for Phase Separation in a Tee," was completed. This work is relevant to LOFT Experiments L3-5/5A and L3-6/L8-1 where the small break is created with a tee assembly connected to the intact loop cold leg piping. The draft of the paper is currently in review.
- 3. Development of a model for the leakage through the reflood assist bypass valves (RABVs) was completed and documented by letter. The description of the model (equations) is in a form that can be used by LOCE L3-1 standard problem participants.
- The calibration of the LOCE L3-5 flow spool was completed. Additional
  analysis will be done to reduce the uncertainties in the calibration of
  the drag screen before L3-6.
- 5. The RELAP4/MOD7 code has been used to calculate the ZION large cold leg break transient in essentially the same manner as with RELAP4/MOD6. The RELAP4/MOD6 code was used in a prototypicality study between LOFT and ZION for LOCE L2-3. The RELAP4/MOD7 code is being used to recalculate the ZION transient using data on ZION supplied by Westinghouse. Agreement between RELAP4/MOD7 and RELAP4/MOD6 is essential to this calculation.
- 6. The Experimental Prediction Report for LOFT LOCA L3-5/L3-5A was issued.

- Calculations and a draft report for tests L6-1, -2, -3, and -5
   Experimental Prediction were completed.
- A preliminary posttest analysis letter on test L6-5 using RETRAN was issued.
- The Experiment Operation Specification (EOS) was issued for tests L6-1,
   L6-2, and L6-3 after comments received during the August review were resolved. These tests are scheduled for early October.
- The EOS for tests L3-5/L3-5A was issued. This test was successfully performed on September 29, 1980.
- Test support of Experiment Operating Procedure (EOP) L3-5/L3-5A was provided by participating in Data Integrity Review Committee (DIRC) activities to qualify the test data.
- 12. The results of a study undertaken to review existing fuel-cladding test data were issued to provide a comparison of cladding burst temperature versus internal fuel rod pressure. These data suggest that fuel cladding failure should not occur until performance of test L2-6 using 750 psig pressurized fuel rods. Test L2-6 is currently scheduled for May 1982.
- 13. A draft of the Experiment Definition Document (EDD) for test L3-6/L8-1, providing a preliminary discussion of test initial conditions and operating requirements, was prepared and submitted for review. Test L3-6 L8-1 is currently scheduled for December 1980.
- 14. The results of a study performed to determine the effect of various initial blowdown suppression tank pressures on the break characteristics of test L5-1 (scheduled for August 1981) was listed. There were no observable effects on break characteristics from varying initial suppression tank pressures.

## FOREIGN-FUNDED TASK SUMMARIES

Foreign-funded and in-kind LOFT support projects are summarized in this section.

## SUMMARY OF JAPANESE-FUNDED (JAERI) TASKS

## 1. Task 5F8C1 -- JAERI Management

The new work breakdown structure was approved this month, and all JAERI tasks were rebudgeted as appropriate.

# 2. Task 5F8C4 -- Advanced DTT

Testing was performed on the LOFT pressure balanced drag turbine using the LOFT Technical Support Facility (LTSF) Blowdown Facility. Data review and analysis are underway, and preliminary assessment of that data indicates the existing design is not suited for transient blowdown testing as might be expected in LOFT.

## 3. Task 5F8C6 -- Reevaluation of LOFT Experiments

No progress was achieved this month.

# Task 5F8C7 -- Miscellaneous Code Studies

This task was inactive this month.

# 5. Task 5F8C8 -- LTSF Suppression Tank

A final task report was issued. This task is complete, except for final cost resolution.

# 6. Task 5F8CA -- PC-3 and Small-Break Densitometers

## A. PC-3 Gamma Densitometer

- The source cask was fabricated by L&S Machine. The source was loaded by Gamma Industries and shipped to Measurements, Inc. (MI).
- (2) All document submittals by MI have been approved by EG&G with exception of the stress analysis. Fabrication release has been given.
- (3) Fabrication and testing of the detector preamplifiers was completed. Four of the eight photomultiplier tubes have been received.

# B. Small-Break Instruments

Test L3-5 was conducted using the small break instruments developed by this task. Preliminary data from the small break gamma densitometer, turbine, drag screen, and thermocouples appear clear with low uncertainties. All instruments functioned properly, and high reliability is expected to continue. This task is complete.

# 7. Task 5F8CB -- Post-CHF Heat Transfer

A conceptual design and sketches of the test section and planning for installation into the blowdown loop at LTSF are nearly complete. A list of required hardware has been compiled. Several sources for a new low-voltage, high-current power supply have been investigated.

# SUMMARY OF GERMAN-FUNDED (FRG) TASKS

# Task 5F7C1 -- FRG Management

The new work breakdown structure was approved, and appropriate tasks were rebudgeted. A new three-year FRG-NRC LOFT agreement was approved.

# Task 5F7C4 -- Miscellaneous Tasks

This task was inactive in September.

## 3. Task 5F7C5 -- Steam Probe

This task was inactive in September.

## Task 5F7C8 -- LOFT State Vector Cost Estimate

No progress was reported in September.

#### SUMMARY OF JAERI/FRG-SHARED TASKS

# 1. Task 5FC94 -- Two-Phase Loop Boiler Building

Construction of the building was completed. The first construction inspection found several items improperly performed by the contractor. The contractor has agreed to correct the deficiencies by October 15.

# SUMMARY OF NETHERLANDS-FUNDED (ECN) TASKS

# 1. Task 5FNC2 -- Program Development and Analysis

A new work package, "Internal Cladding Thermocouple Design (in REBEKA Heater Rod)," was approved and work is in progress at Exxon Nuclear.

The "Wyle Data Analysis" work package has been completed and two LOFT Technical Reports written. The major conclusions of this work are as follows:

A. The densitometer and drag disk rake combination gives quite accurate mass flow measurements, with errors in the total (time integrated) mass flow of not more than 6.3% of reading.

- B. The densitometer and turbine rake combination gives consistently low estimates of the mass flow, with errors as large as 22% of reading in the total mass flow.
- C. The densitometer and pitot tube rake combination gave results that varied substantially from one test to the next with no discernible, consistent cause. Unlike the drag disk and turbine results, the results for the pitot tubes depended strongly on what kind of symmetry was assumed when extrapolating the pv<sup>2</sup> (density times velocity squared) profile from the local pitot tube measurements. The assumptions that the pv<sup>2</sup> profile symmetry should match the density profile symmetry usually gave substantially greater mass flow estimates than did the assumption that the pv<sup>2</sup> profile was symmetrical about the rake and independent of the density profile. The lower mass flow values resulting from the second assumption were usually more accurate than the values obtained with the first assumption. Even with the better assumption, there were errors as large as 27% of reading in the total mass flow estimate.
- D. The turbine and drag disk combination usually gave low estimates for the total mass flow, with errors as large as 34% of reading. These tests did not use in-place calibrations, which have been found to substantially enhance the accuracy of turbine flow meters in LOFT. Thus, it seems that, without in-place calibration, the drag disk-densitometer combination gives better mass flow measurements, than other instrument combinations give.

Progress on the "Critical Flow Scaling Studies" work package continued with examination of LTSF calibration data on Semiscale 0.11- and 0.028-in, nozzle due to low water quality during testing. Effort is proceeding to analyze remaining data to provide Henry-Fauske, Moody and modified Burnell critical flow data base.

Under "Analysis of PNA Techniques" work package the analytical study of the N-16 transport from the tagging to the detector positions has

been initiated under Task II. A computer program is being developed which will simulate the detector counting spectrum versus time for an irradiated single-phase laminar flow.

The simulation is done by dividing radially and axially the 112 irradiated water regions into smaller control volumes. These control volumes have a velocity that depends on their radial positions (the parabolic velocity profile for laminar flow is being considered), and an initial N-16 activity that depends on their radial and axial position from the source(s). Therefore, through this transport the program couples together the axial and radial profiles of the induced N-16 activity at the source(s) position with the axial and radial profiles at the detector position. The analysis of the mass-weighted velocity from the resultant time profile is also included.

The program is being tested for a single-phase laminar flow when water is flowing in a 14-in. schedule 160 pipe at a mean velocity of 0.05 cm/s (to ensure Re < 2000). The distance from the midplane of the neutron source to the midplane of the detector is assumed to be 127 cm.

This program will also be used for the transport of the laminar liquidphase of a two-phase stratified flow.

The Rensselaer Polytechnic Institute (RPI) work package on modeling critical flow in an orifice has been completed and documented. One of the original objectives of the research contract was to develop a model of critical flow through an orifice such that with only measurements of the upstream stagnation conditions  $(P, T, \rho)$ , the model could be used to predict the discharge mass flow through an orifice. The results obtained indicate that Dr. Gay has gone a long way toward achieving the objective.

# Task 5FNC3 -- Component Development

The two-phase loop platform and stairs addition was completed. A new task to develop a conceptual design and cost estimate for mounting

internal zircaloy-clad thermocouples in a heater rod zircaloy clad was approved, and work is underway at Exxon Nuclear.

# SUMMARY OF FRENCH-FUNDED (CEA) TASKS

## 1. Task 5FFC2 -- Program Development and Analysis

A new French Commissariat a l'Energie Atomique (CEA) - USNRC LOFT agreement was approved in September. This agreement is for a three-year period, and two CEA delegates are expected to join the EG&G staff soon in the LOFT Program Office.

## SUMMARY OF AUSTRIAN-FUNDED (FZS) TASKS

# Task 5FAC2 -- Program Development and Analysis

Minor information was exchanged between LOFT and the Austrian staff during this period. The Austrian Studiengesellschaft fuer Atomenergie (SGAE) has been renamed to Austrian Forschungszentrum Seibersdorf Gesellschaft mbH (FZS). This reflects the need for FZS to consider projects other than nuclear-related ones.

# 2. FZS In-Kind Support to LOFT

The FZS staff have performed some preliminary autoclave tests to ensure that they can perform the desired tests on EG&G-provided optic window and insulator samples. The LOFT chemistry oxygen low limits cannot be achieved, so the tests will be conducted using the least oxygen content achievable by FZS. Production testing is scheduled to start October 7.

FZS questions related to design of a steam-water mixer were resolved between LOFT and FZS. A fabrication for the mixer has been identified, and FZS is in the process of funding the task.

# SUMMARY OF SWITZERLAND IN-KIND (EIR) SUPPORT

## 1. NEPTUN Reflood Test Program

The Inconel thermocouples, dummy segments, and spare thermocouple material were shipped to Switzerland. The Swiss were asked to evaluate if zircaloy clad heater rods could be fabricated for LOFT-NEPTUN support.

The NEPTUN test program is not firm at this time, and tests to uncover the core are being considered for the first major test series.

#### FOREIGN COOPERATIVE SUPPORT TO LOFT

No progress was reported during September on the FRG programs being conducted to provide added insight on LOFT clad thermocouple behavior.

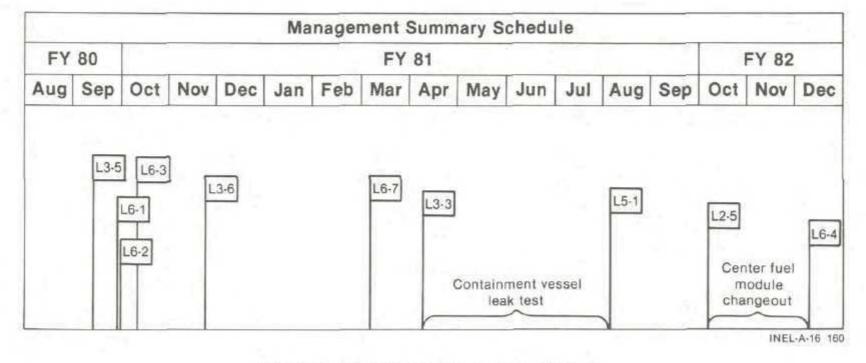
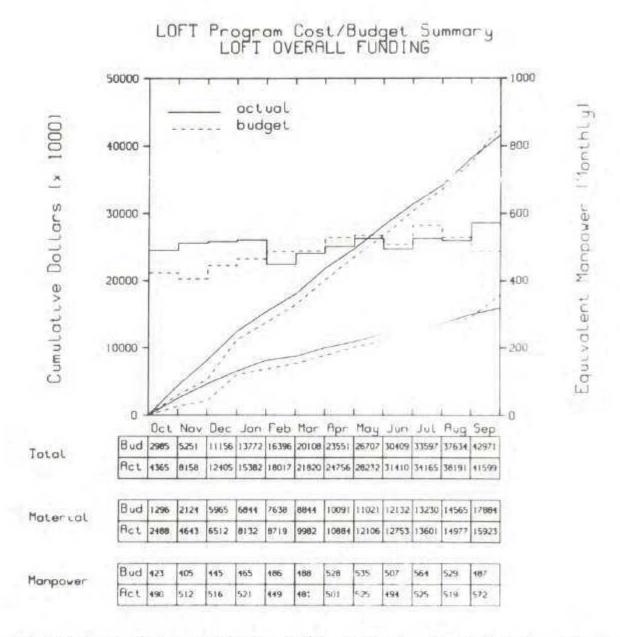
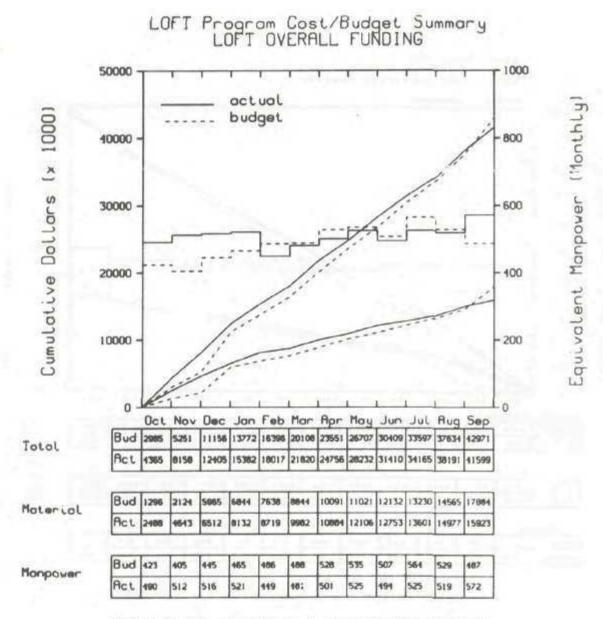


Figure 1. LOFT management summary schedule.

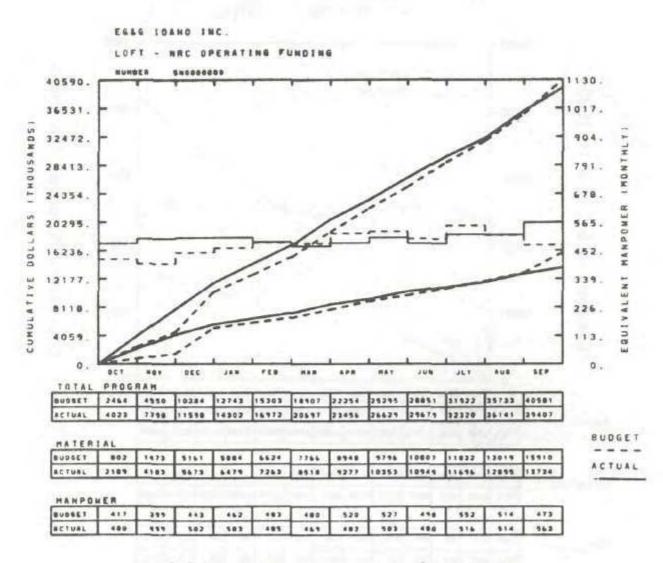
LOFT Overall Funding 5xxxxx



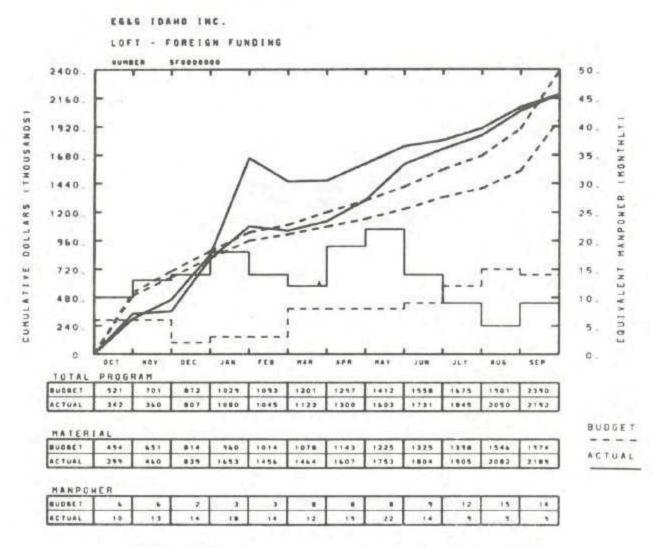
The Nuclear Regulatory Commission (NRC) and foreign-funded budgets reflect the LOFT Q80-5 Rev 0/4 baseline approved in September 1980. Refer to the Director's Monthly Summary for comments. 5N--NRC Operating Funding 5F--Foreign Funding



Refer to the summary cost accounts for comments.

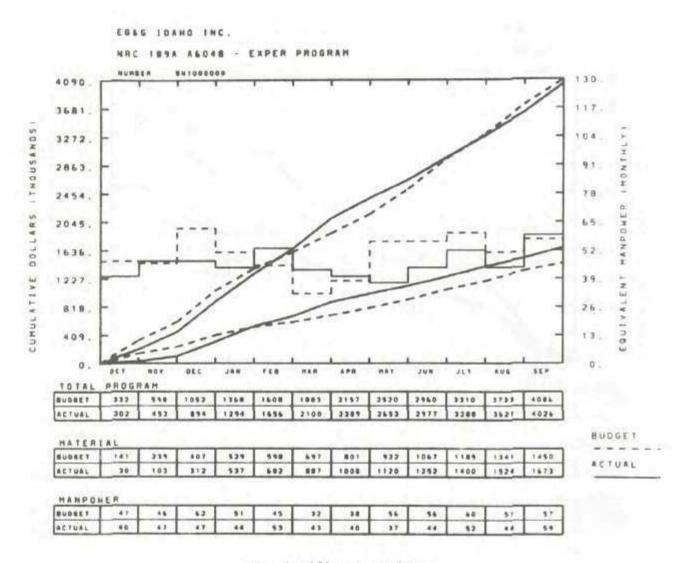


Refer to summary cost accounts for comments.

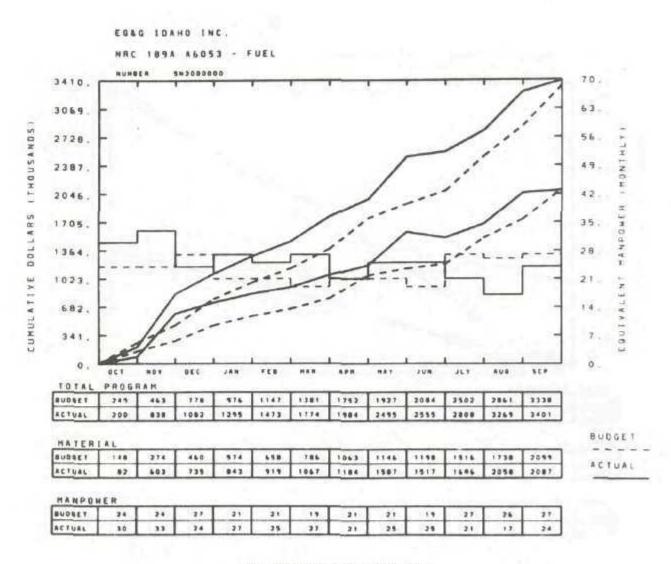


Refer to the summary cost accounts for components.

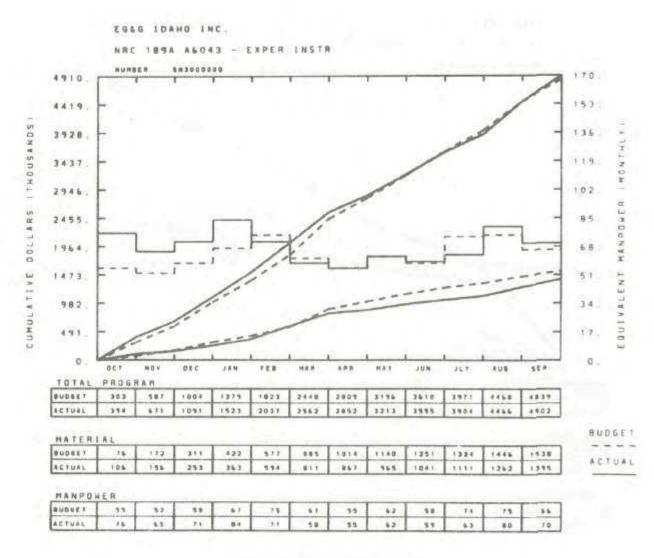
LOFT 189a Summary 5NX--NRC 189a 5FXX--Foreign 189a



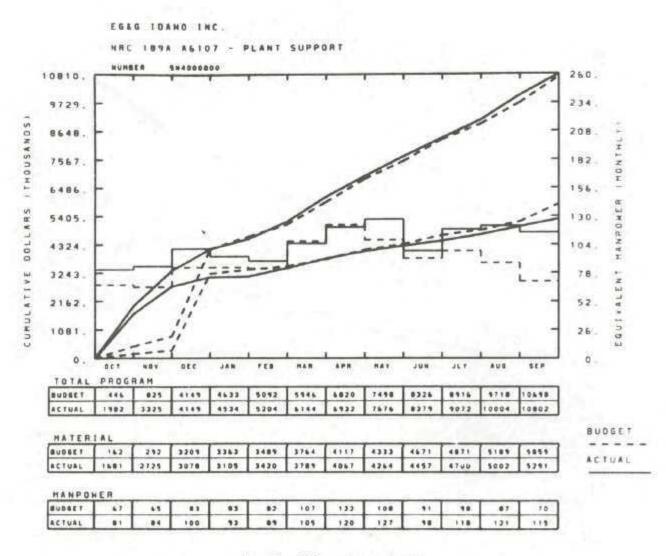
No significant variance.



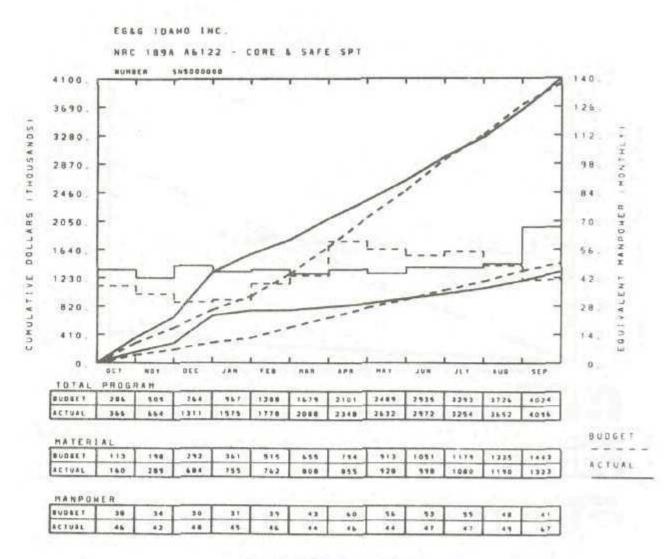
No significant variance.



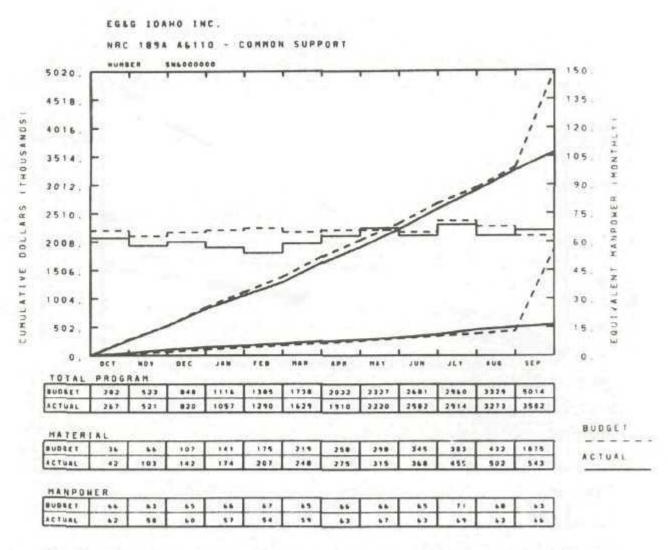
No significant variance.



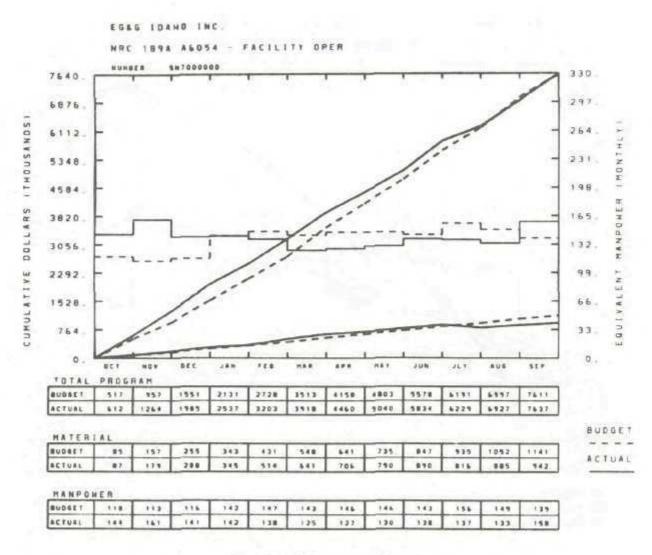
No significant variance.



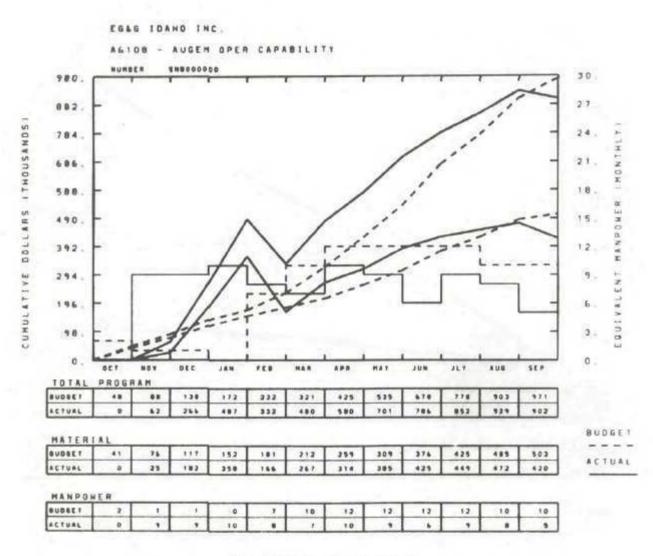
No significant variance.



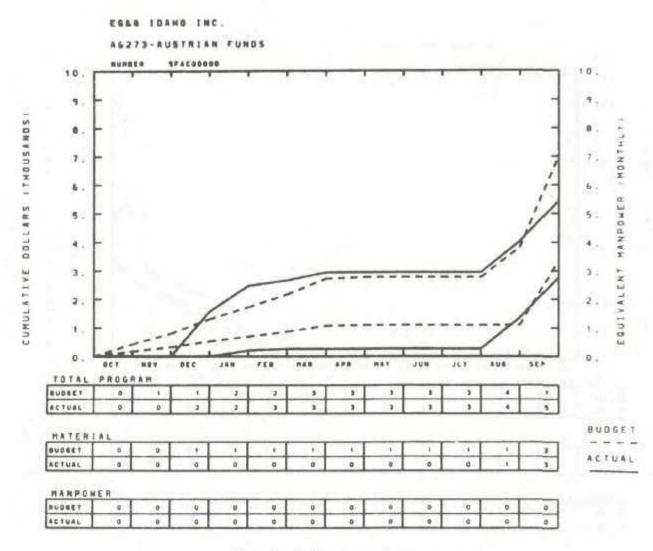
No significant variance. Budget includes management reserve of \$1409K.



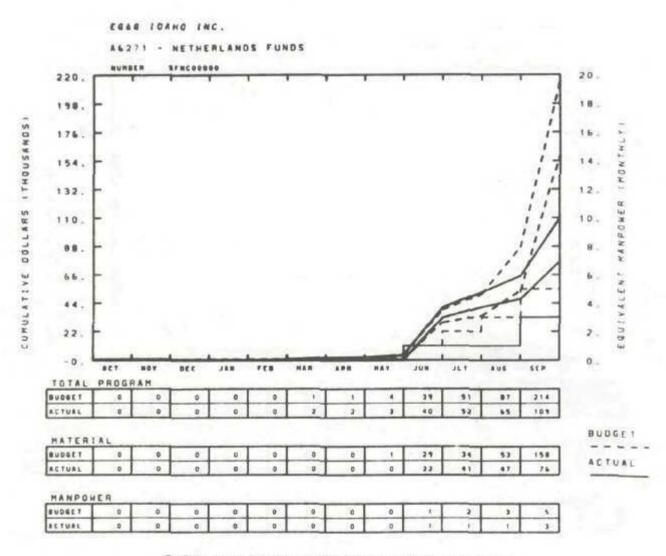
No significant variance.



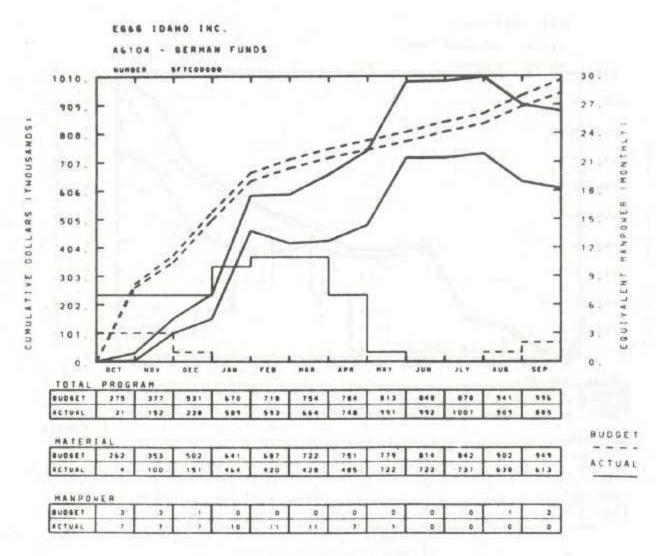
No significant variance.



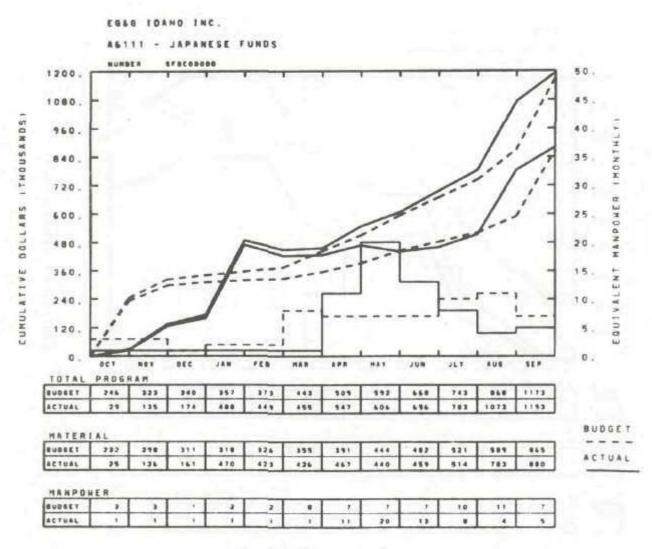
No significant variance.



Refer to summary cost accounts for comments.

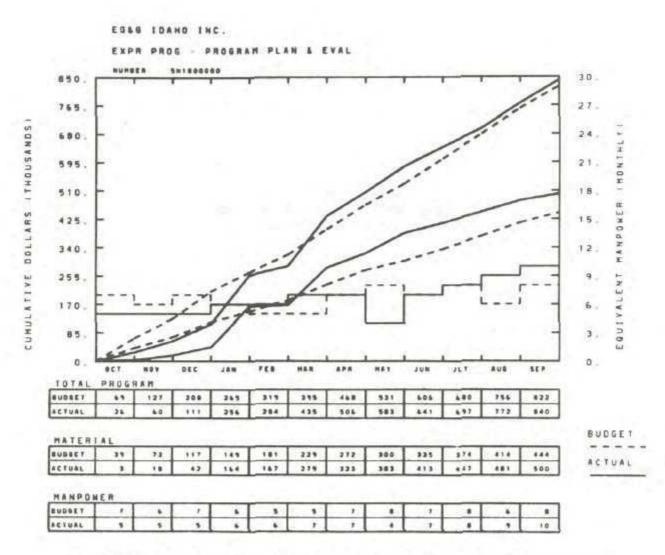


No significant variance.

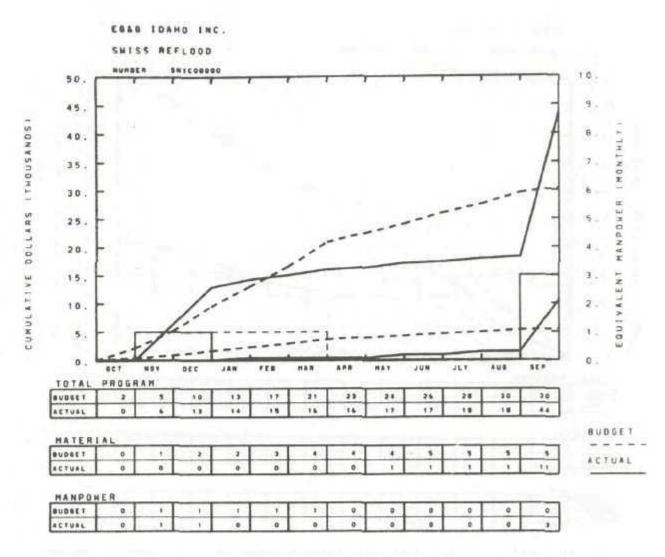


No significant variance.

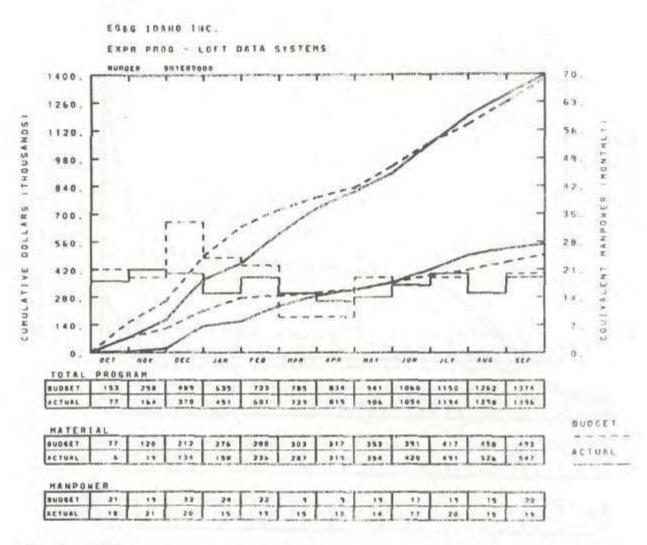
Summary Cost Accounts 5Nxx--NRC Summary Cost Accounts 5Fxxx--Foreign Summary Cost Accounts



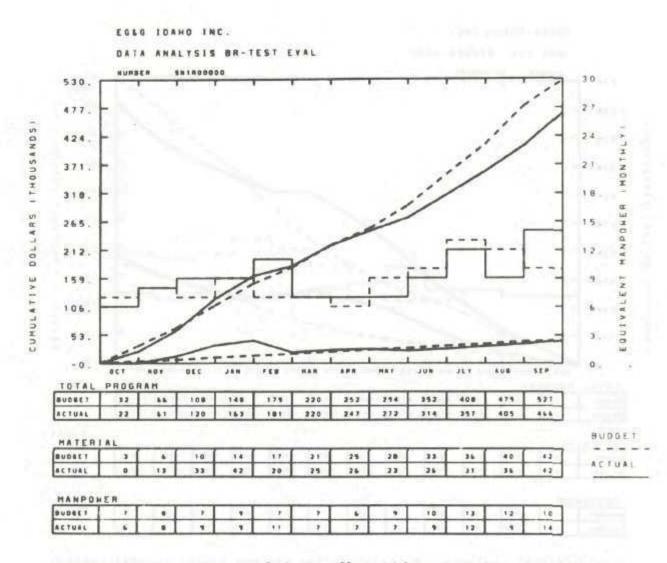
No significant variance. Material dollars overrun due to increased computer usage in preparation of tests.



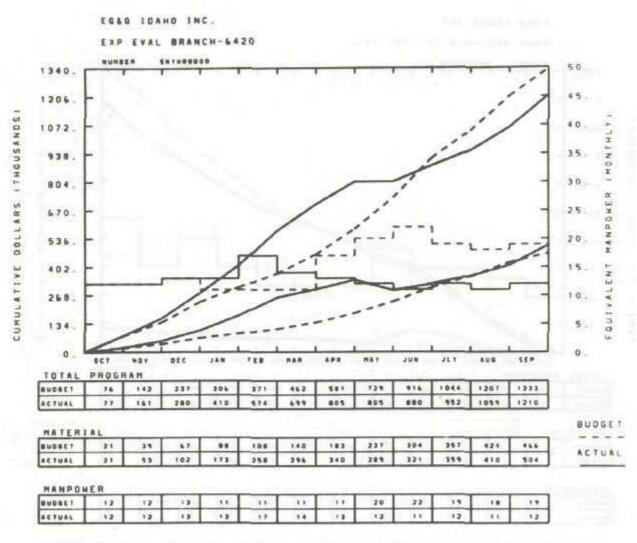
No significant variance.



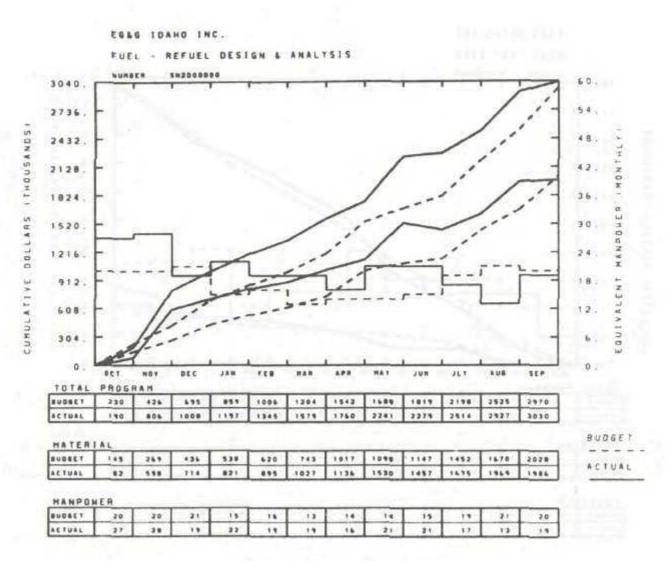
No significant variance. Unplanned large volumes of densitometer data and disk storage charges resulted in computer overruns.



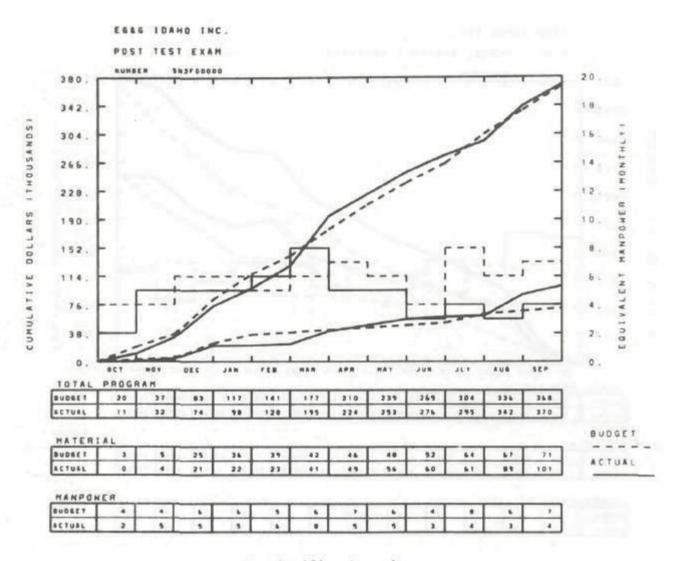
No significant variance. Budget reflects L6 test series scheduled for September 1980 but accomplished in October 1980. Year end closing will adjust FY-1981 budget to reflect current performance and status.



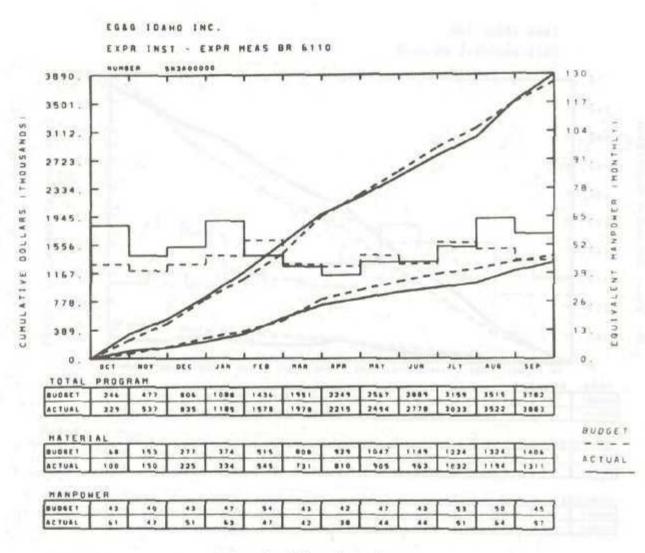
No significant variance. Material dollar overrun caused by additional computer calculations for L3-5/5A.



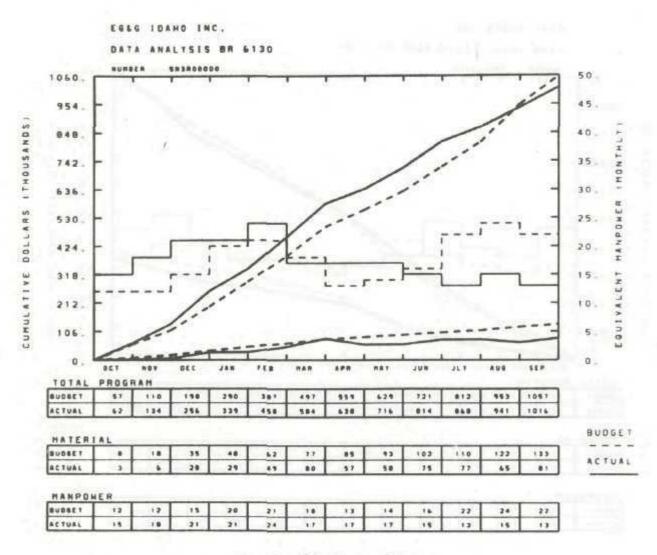
No significant variance.



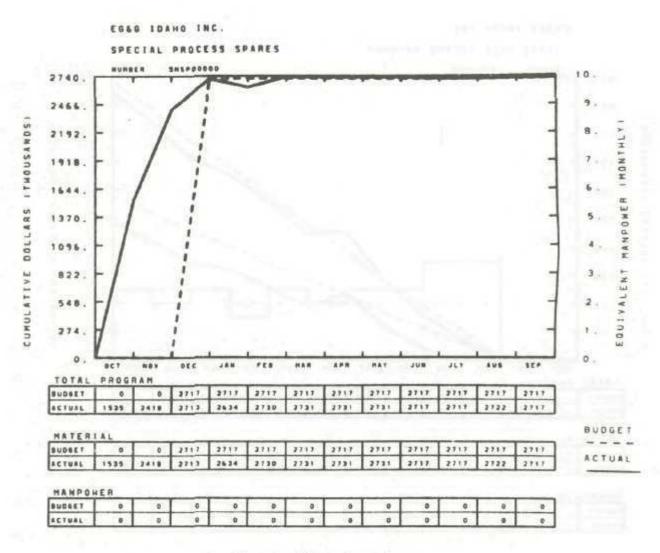
No significant variance.



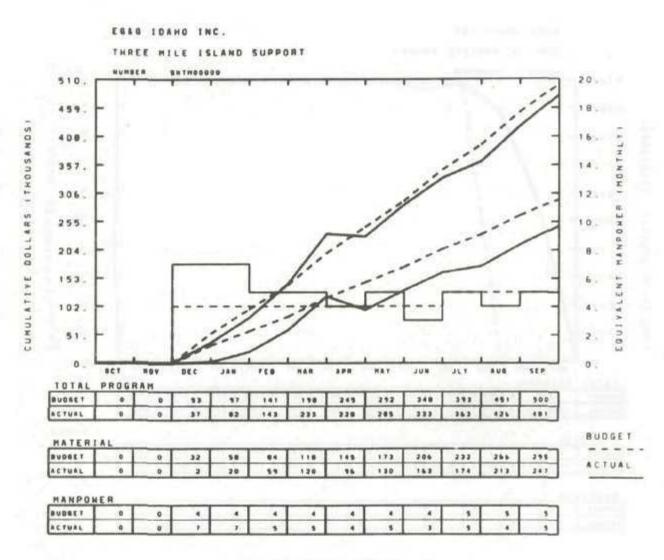
No significant variance.



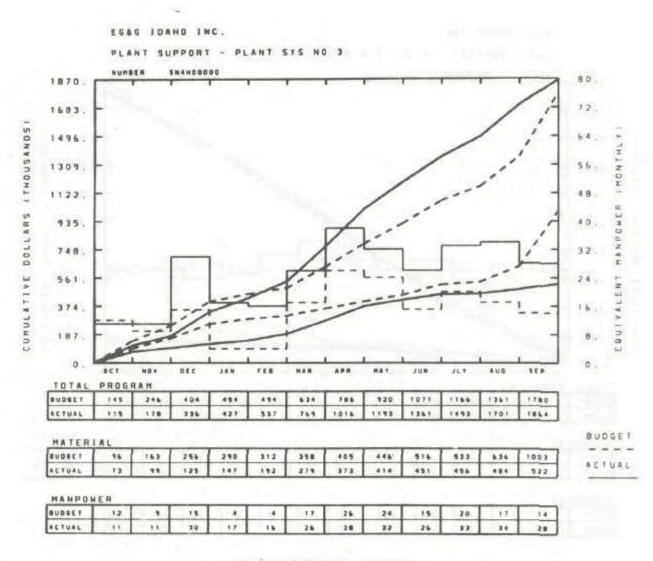
No significant variance.



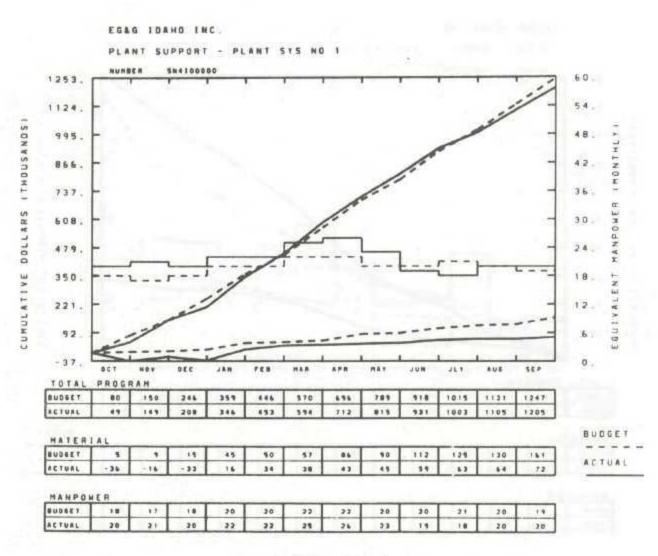
No significant variance.



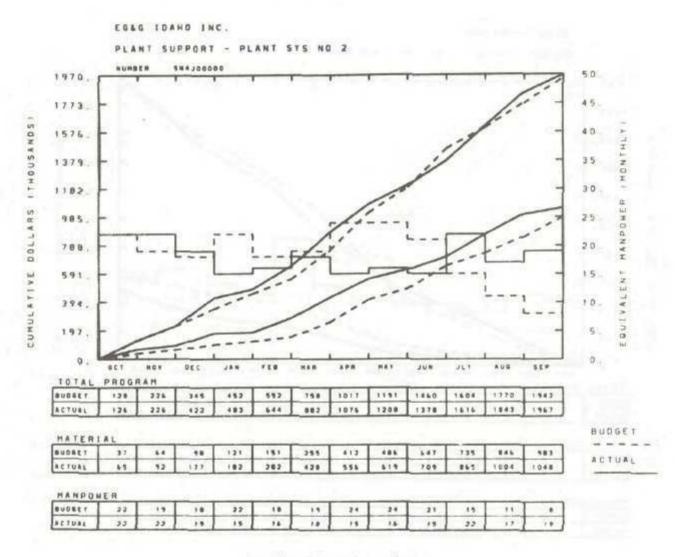
No significant variance.



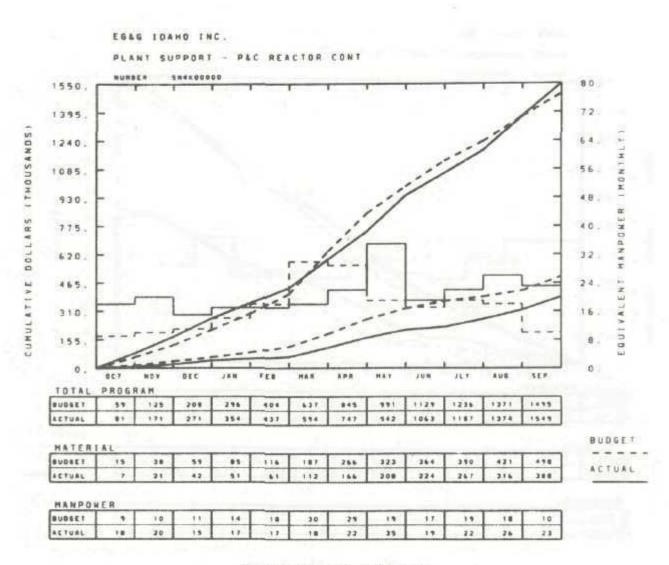
No significant variance.



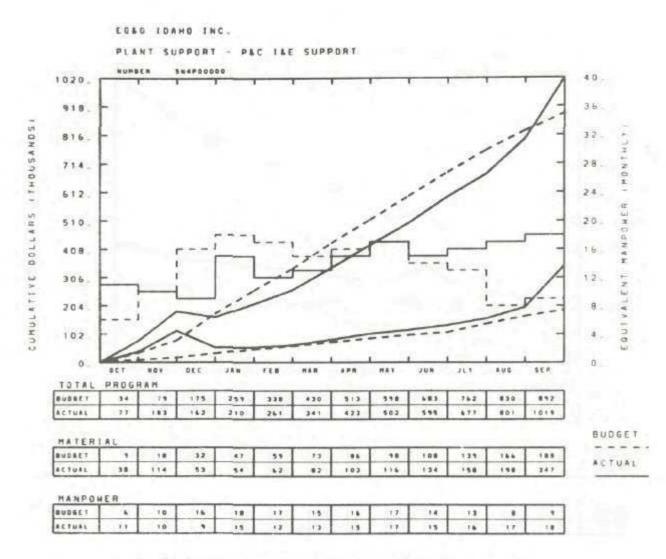
No significant variance.



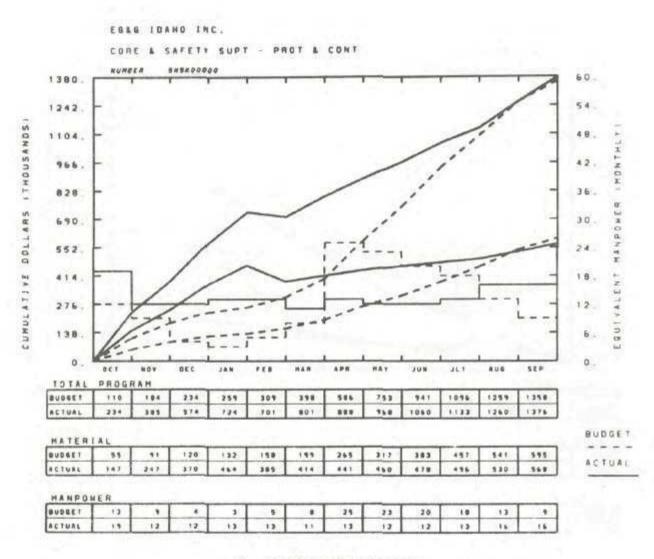
No significant variance.



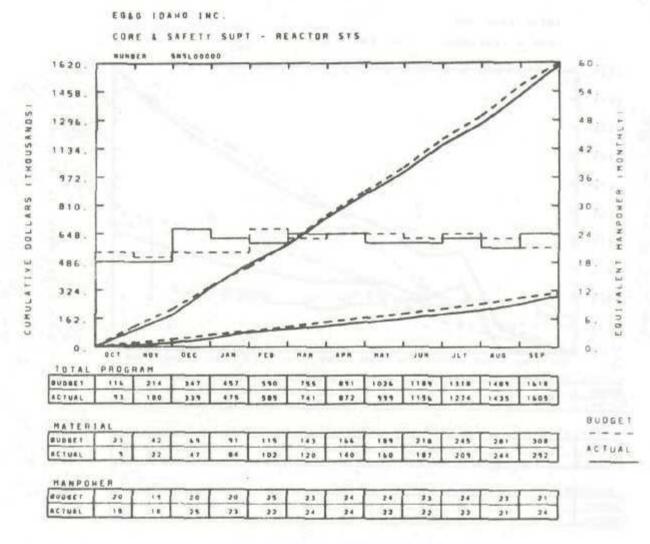
No significant variance.



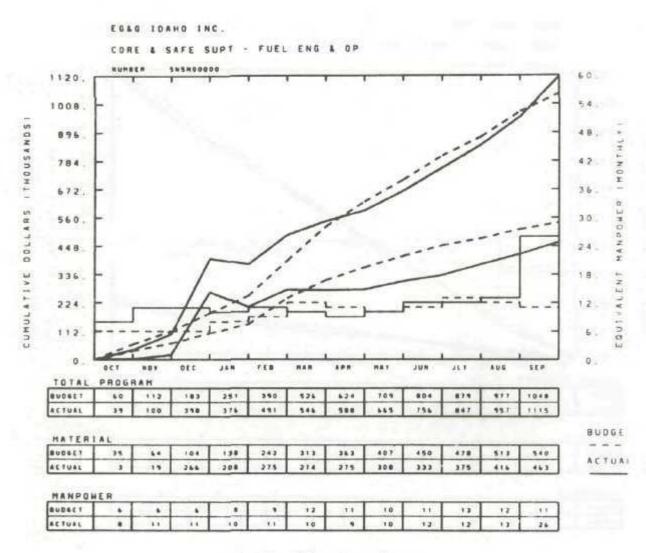
Deviation is due to realignment of funding sources.



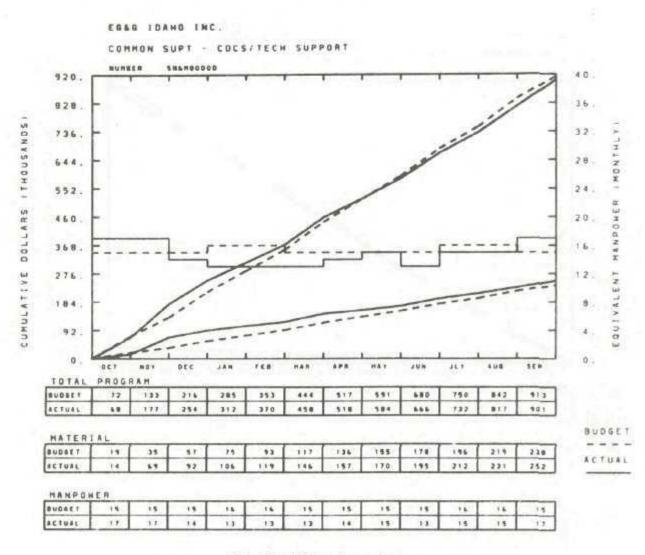
No significant variance.



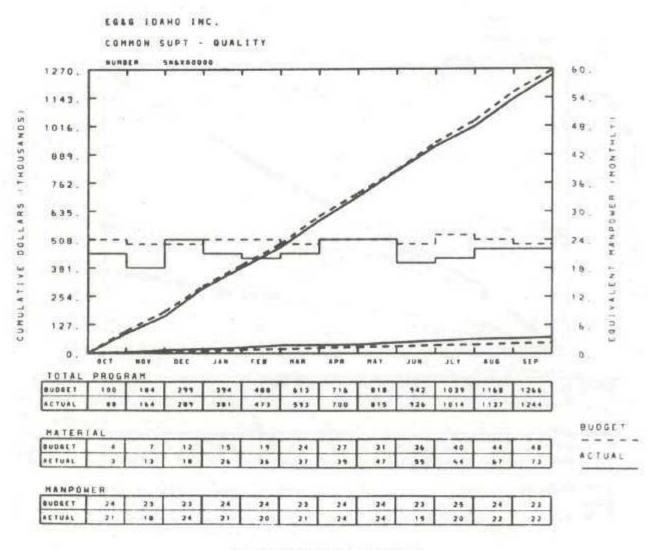
No significant variance.



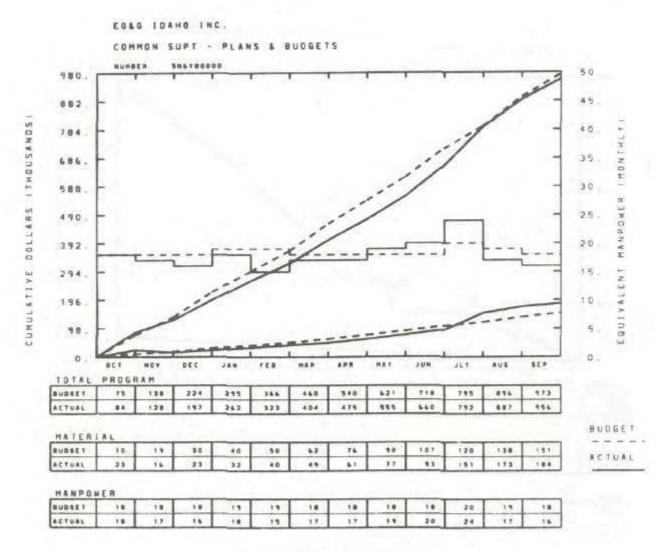
No significant variance.



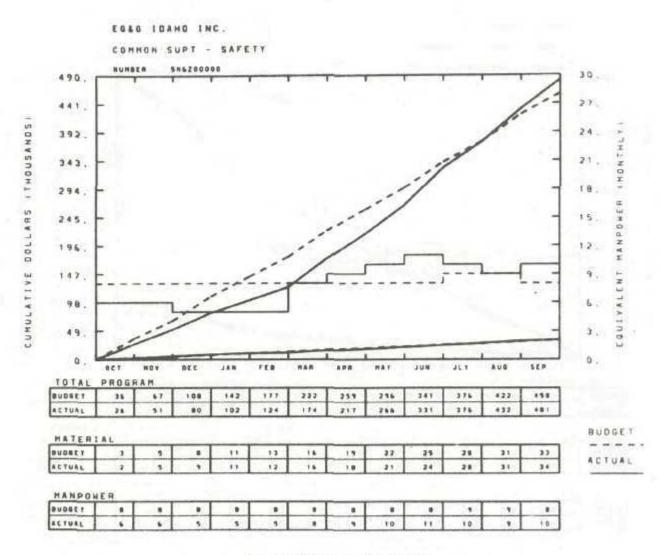
No significant variance.



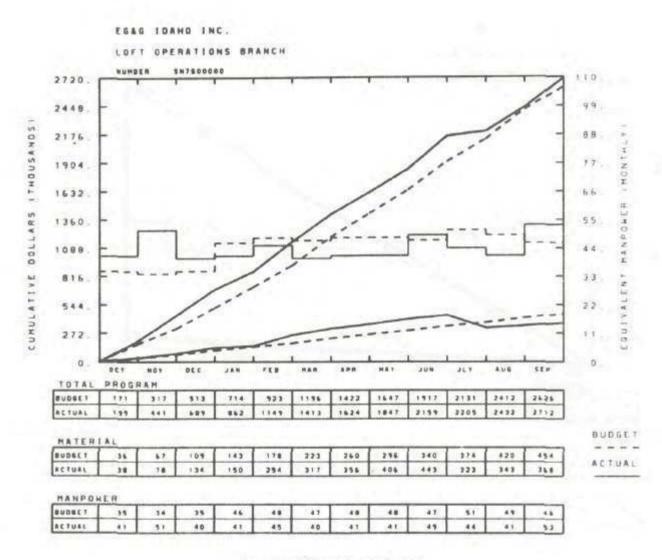
No significant variance.



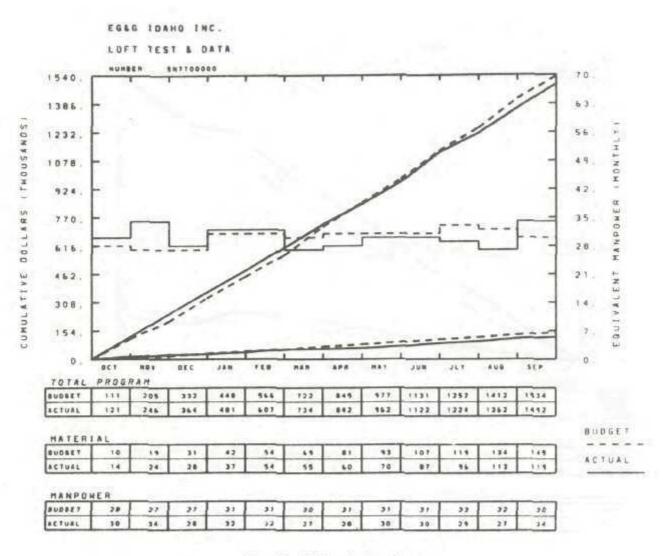
No significant variance.



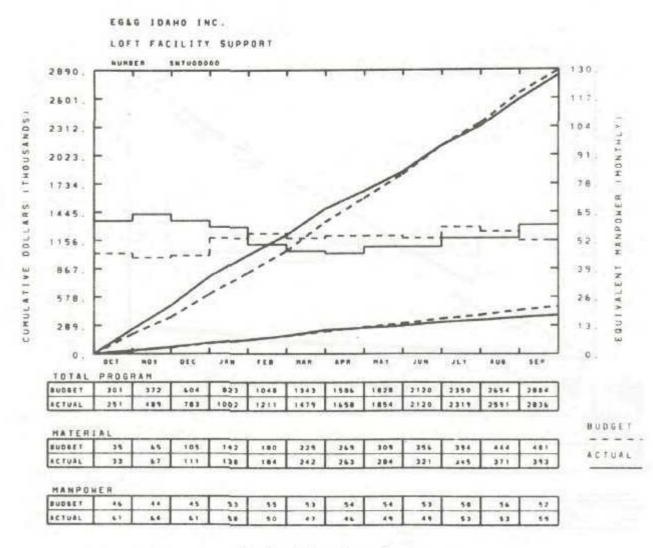
No significant variance.



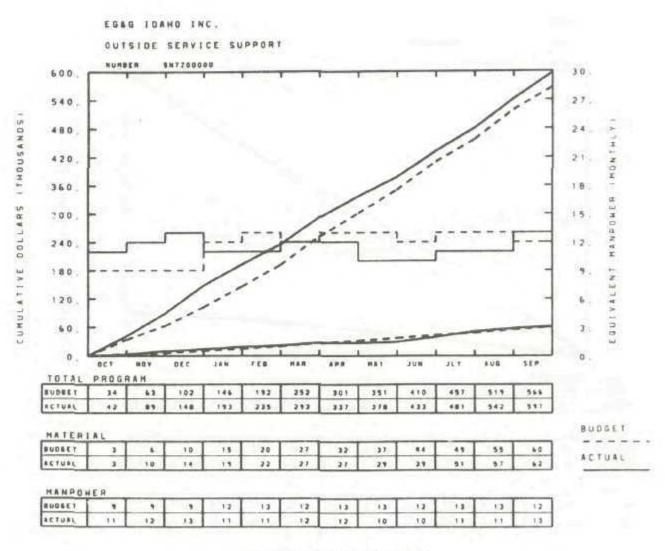
No significant variance.



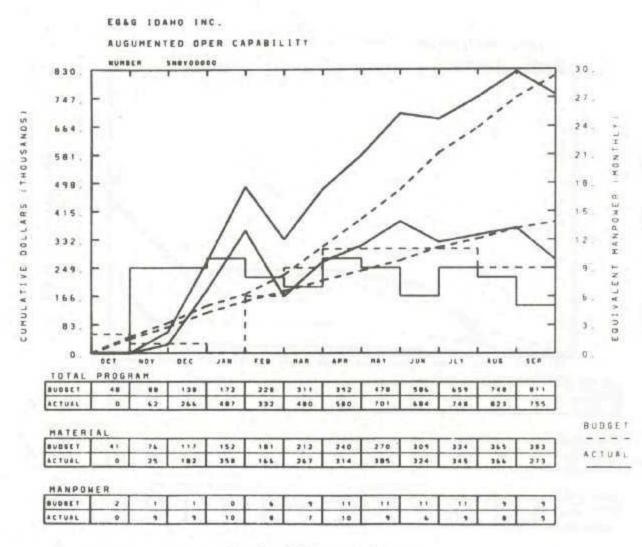
No significant variance.



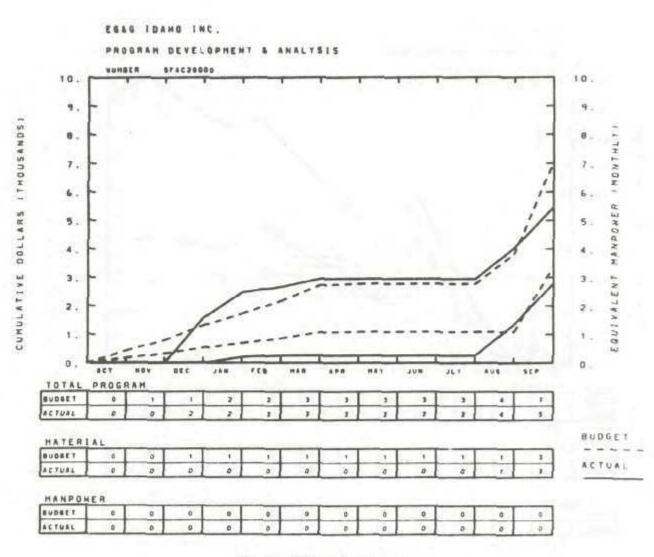
No significant variance.



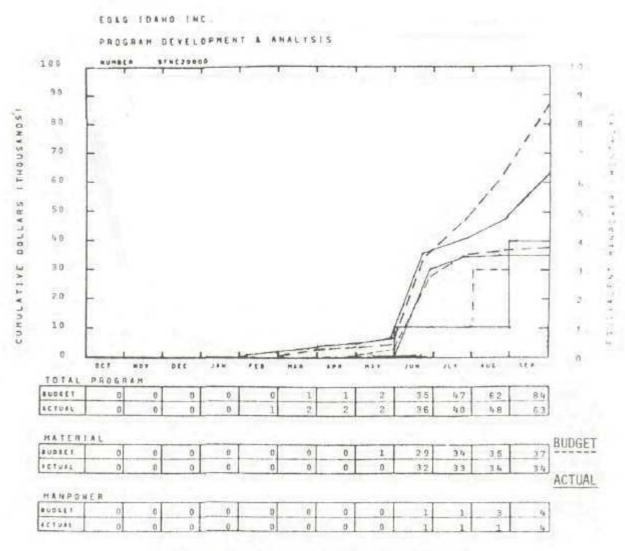
No significant variance.



No significant variance.



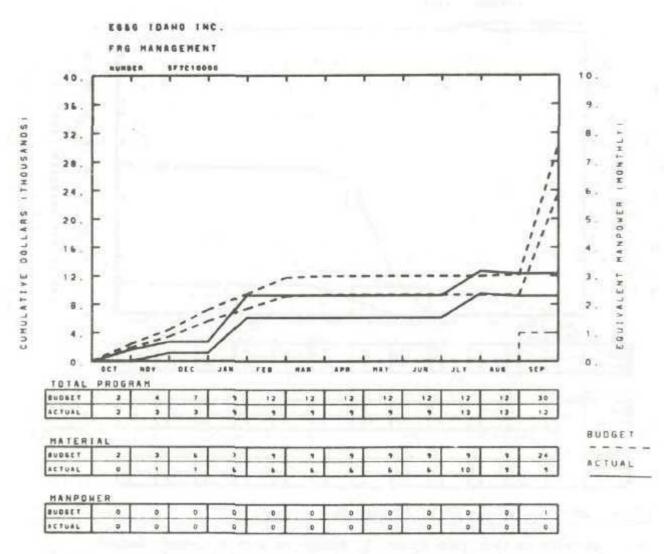
No significant variance.



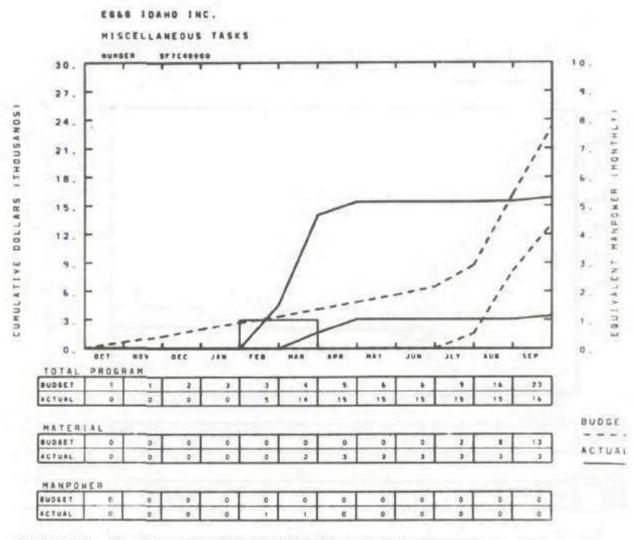
Variance due to manpower unavailability.

EG&G Idaho, Inc. Component Development Number 5FNC30000 50 45. CUMULATIVE DOLLARS ITHOUSANDS! 40. 35 30. 25. 20. 15. 10. 5 0 . oct OFC TOTAL PROGRAM supse: 0 o. à. 0 0 ø , 5 25 4.7 ACTUAL 46 BUDG MATERIAL ..... ė 0 0 a ACTU ACTUAL 0 HANPOHER \*\*\*\*\*\* 0 ACTUAL

No significant variance.



FRG reserves are budgeted in September and this causes the discrepancy. Reserve funds will be transferred into FY-1981.

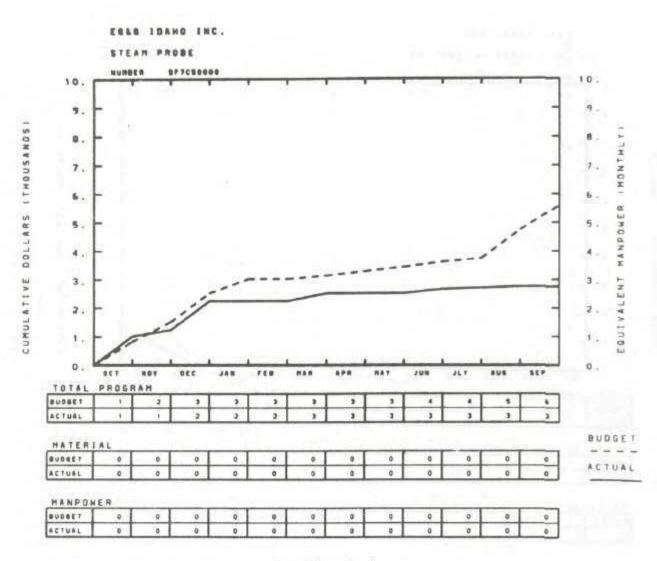


### 5F7C40100: Development of Pitot Rake Measurement

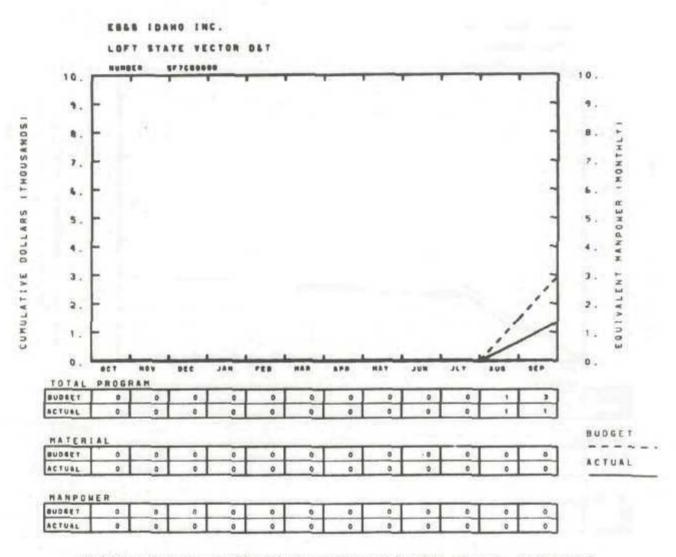
Work completed on this task by Dr. S. Bannerjee with a summary letter received in early August. This completed contractual obligation.

## 5F7C40200: Storage Tank Analysis

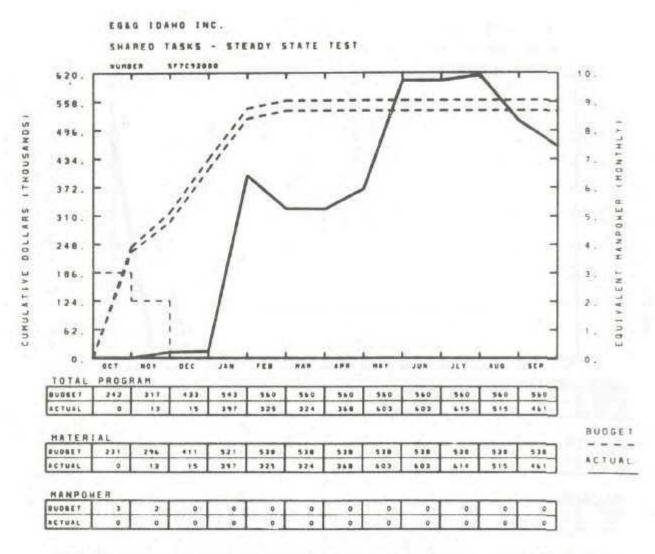
Work on this activity has been delayed due to manpower allocation to higher priority tasks. A CCF is required to allow work to extend into December.



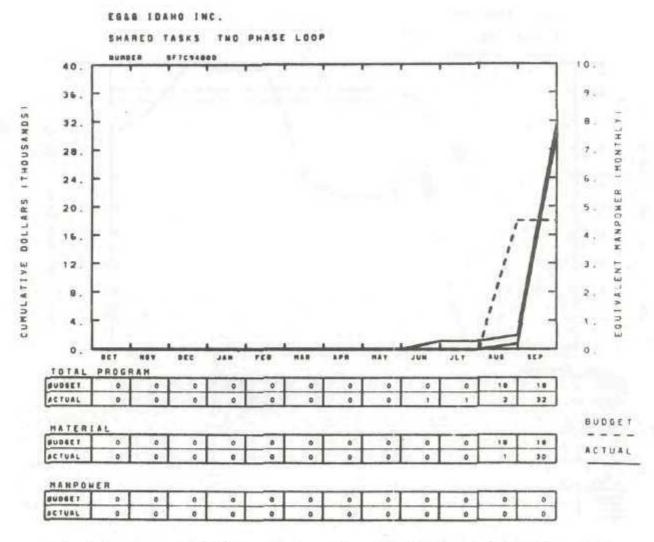
Inactive task.



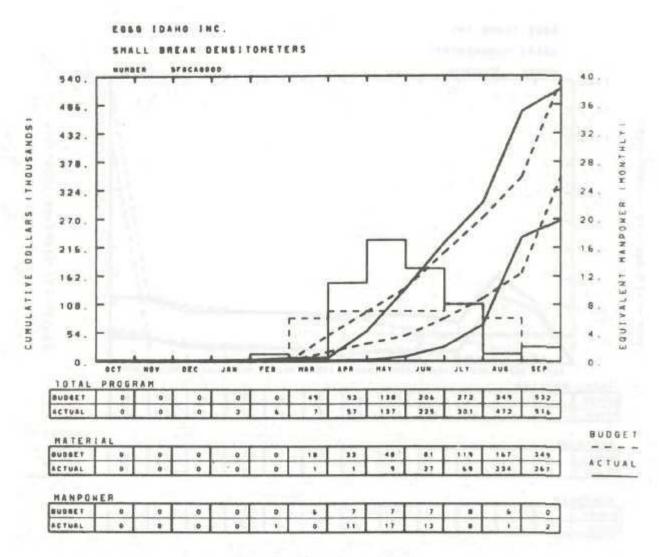
Budget has been realigned to perform majority of task in FY-1981.



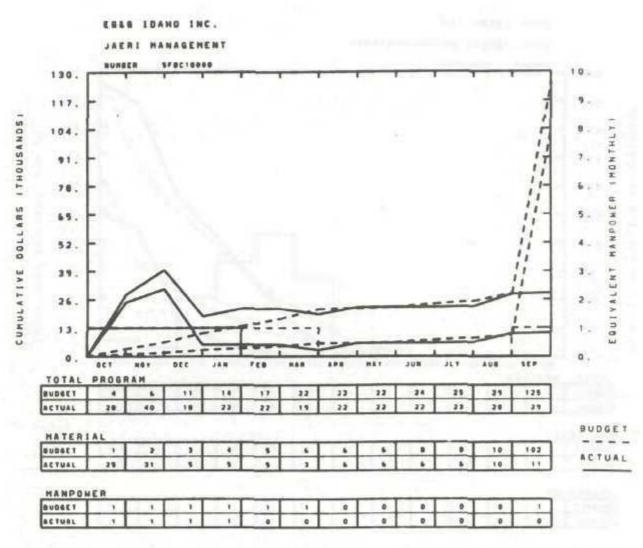
\$99K variance due to cost transfer correcting a prior year allocation error. The budget was not revised to correspond to the new value.



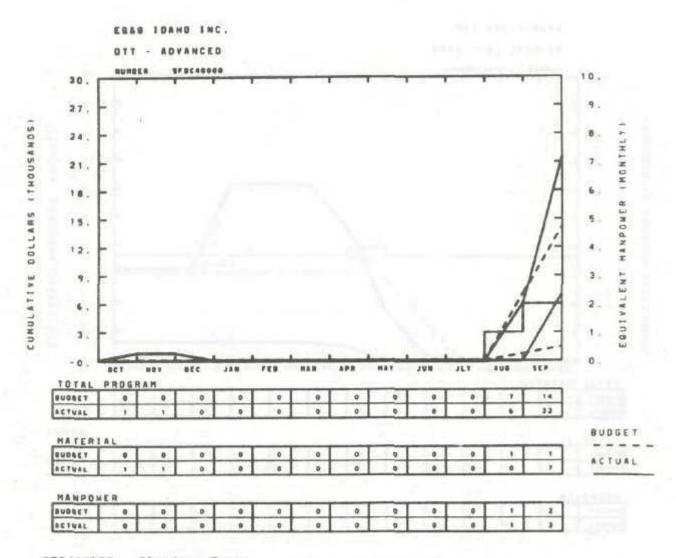
Work is near completion. An error in charging has led to all costs appearing in this account instead of being split with 5F8C94000.



No significant variance.

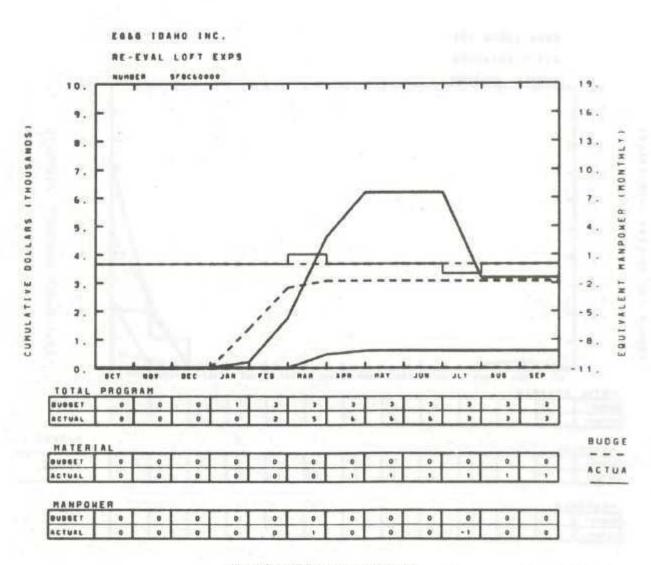


JAERI reserve funds are budgeted in September and will be transferred into FY-1981.

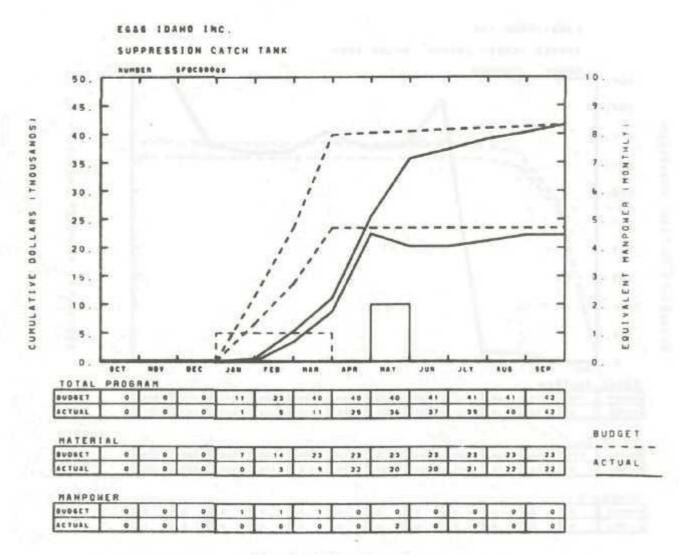


#### 5F8C40100: Blowdown Tests

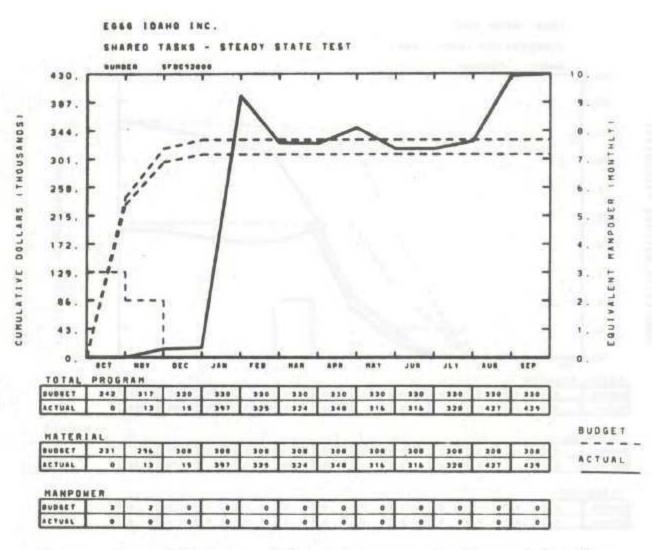
Budget has been respread to allow work to continue into FY-1981. However, testing was completed in September and a report is in preparation. Deviation from budget was caused by accelerating work to complete in FY-80. Overrun of 2K is due to higher craft support required than expected for installation of the instrument.



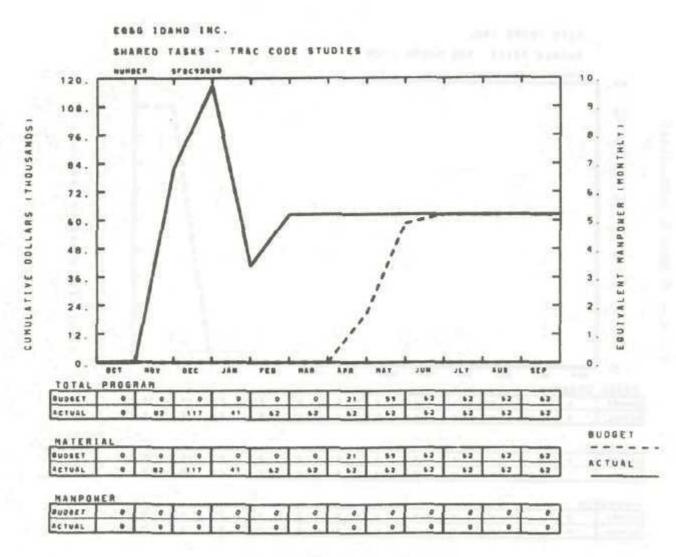
No significant variance.



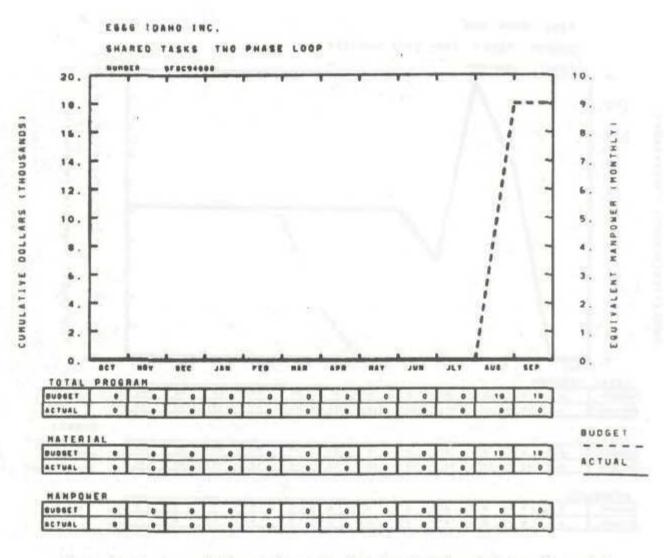
No significant variance.



Variance due to \$99K cost transfer made to correct prior year allocation error.

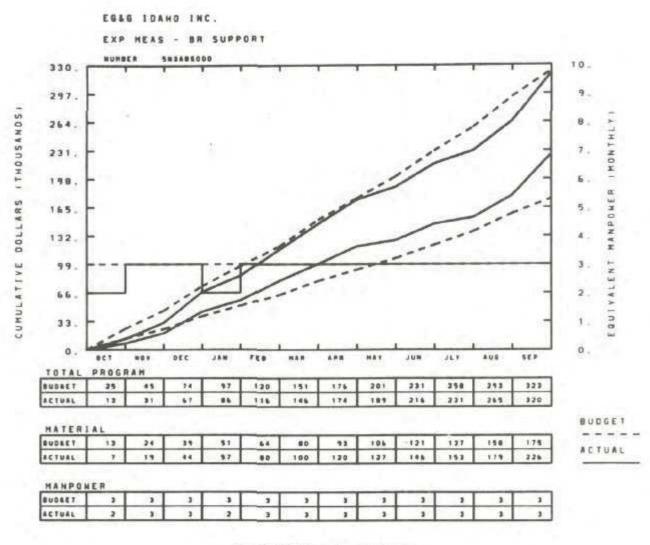


No significant variance.

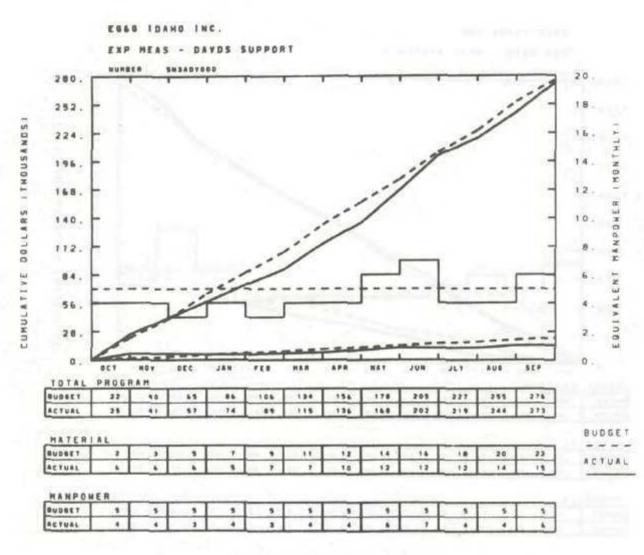


Work is near completion. An error in charging has led to all costs appearing in 5F7C94000.

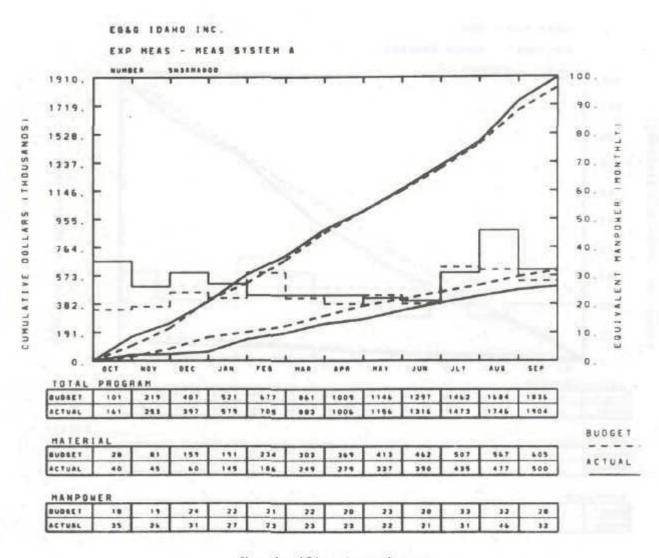
LOFT Cost Accounts 5N3Axx--NRC Cost Accounts



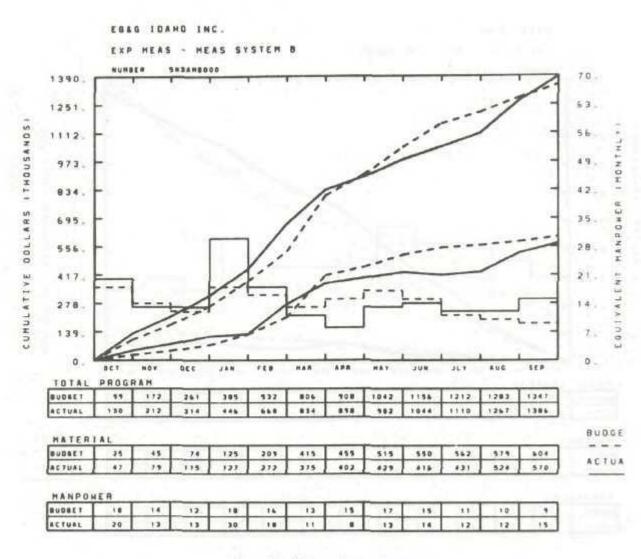
No significant variance.



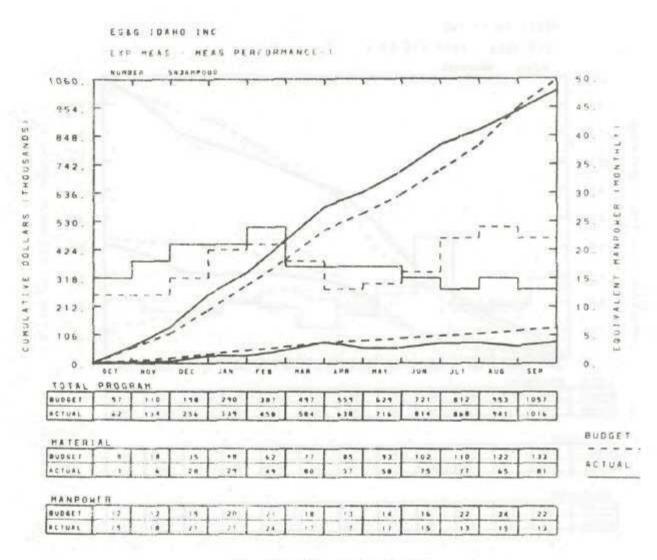
No significant variance.



No significant variance.



No significant variance.



No significant variance.

TABLE 1. FOREIGN FUNDS AVAILABILITY AT END OF SEPTEMBER 1980 (In Thousands of Dollars)

			-
Participant	Actual Reserve	Contingency	
JAER I	24	57	
FRG	4	0	
CEA	0	0	
ECN	57	27	
FZS	_0	0	
Total	85	84	

TABLE 2. FOREIGN FUNDED TASK SUMMARY AT END OF SEPTEMBER 1980

	Project Description	Total Proposal Est. Inc. Contingency (SK)	Total Spending Auth. by CCB (\$K)	Funds Spent to Date (\$K)	Expected Task Completion Date
JAERI T	ASKS				
5F8C1 5F8C2 5F8C4 5F8C5 5F8C6 5F8C7 5F8C8 5F8C8 5F8C8 5F8C92 5F8C93 5F8C93	JAERI Management Completed Tasks Advanced DTT PBF/LOFT Lead Rod Reevaluation of LOFT L1 Exper. Misc. Code Studies LTSF Suppression Caten Tank Small Break Densitometers Post CHF Heat Transfer Shared Two-Phase Steady-State Loop Shared-TRAC Code Studies Two-Phase Loop Boiler Building	262 846 154 1881 25 16 43 592 200 782 83 18	262 845 154 1881 25 16 41 660 177 782 83	183 846 136 1882 25 10 42 516 64 878a 83 0b	Sept. 81 Done Sept. 80 July 80 Oct. 80 Jan. 81 Oct. 80 Oct. 80 Jan. 82 Sept. 80 June 80 Sept. 80
FRG TAS	KS				
5F7C1 5F7C2 5F7C4 5F7C5 5F7C7 5F7C8 5F7C9 5F7C93 5F7C93	FRG Management Completed Tasks Miscellaneous Tasks Steam Probe Ultrasonic Density Detectors LOFT State Vector Small Break Inst. Shared Two-Phase Steady-State Loop TRAC Code Studies Two-Phase Loop Boiler Building	170 2570 58 30 78 10 206 1012 83	170 2570 58 30 78 10 206 1012 83 13	156 2570 43 22 78 1 206 912ª 83 32 <sup>b</sup>	Sept. 81 Done Dec. 80 Aug 81 May 80 Feb. 81 May 80 Sept. 80 June 80 Sept. 80

TABLE 2. (continued)

	Project Description	Total Proposal Est. Inc. Contingency (\$K)	Total Spending Auth. by CCB (\$K)	Funds Spent to Date (\$K)	Expected Task Completion Date
ECN TASK	<u>S</u>				
5FNC11 5FNC21 5FNC221 5FNC222 5FNC223 5FNC224 5FNC225 5FNC321 5FNC311	Completed Tasks Management and Delegate Support Wyle Data Analysis Critical Flow Studies PNA Techniques RPI Subcontract INEL Support Two-Phase Loop Platform Internal Clad Thermocouple Design	92 27 22 53 38 117 4 59	92 27 20 48 33 114 4 47	92 8 20 7 32 112 5 46 0	Done Sept. 81 Sept. 80 Dec. 80 Sept. 80 Sept. 80 Dec. 80 Oct. 80 Nov. 80
FZS TASK	3				
5FAC11 5FAC2	Completed Tasks Program Development	123 24	123 24	123 12	Done Sept. 81

a. \$99,000 prior year cost transfer has been made to correct error between FRG and JAERI accounts, but is under review.

b. \$16,000 cost shall be transferred from 5F7C941 to 5F8C941 (Shared Task).

# BUDGET STATUS REPORT

## TABLE 3. LOFT FY-80 SUMMARY STATUS REPORT NUCLEAR REGULATORY COMMISSION (In Thousands of Dollars)

WBS#_	189	Q80-5-0/3	Approved CCBs	Current PMB # Q80-5-0/4	Current BAC
5N1XX	A6048	4,086	0	4,086	4,086
5N2XX	A6053	3,187	151	3,338	3,336
5N3XX	A6043	5,102	<263>	4,839	4,839
5N4XX	A6107	10,919	<221>	10,698	10,698
5N5XX	A6122	4,069	<45>	4,024	4,024
SN6XX	A6110	3,610	0	3,610	3,610
5N7XX	A6054	7,610	0	7,610	7,610
5N8XX	A6108	971	0	971	971
	A6308				
5NXXX		39,554	<378>	39,176	39,176
	New Pro	gram			125
	Suppler	entary progra	ms		5,253
	FY-1981	commitment			152
	NRC dis	cretionary re	serves		50
	NRC man	agement reser	ves		1,359
	Total N	RC funding (F		46,115	

TABLE 4. LOFT FUNDING SUMMARY FOR FY-80 (In Thousands of Dollars)

Funds	Current FIN Plan No. 10	Current Budget File (Q80-5-0/4)
LOFT Foreign Funds LOFT Lead Rod Tests	2,845 192	2,390 192
Total	3,037	2,582
NRC Operating Funds	45,990	40,710
Electric Heat Rod Evaluation Computer Code Support TC-2 Tests LTSF PWR/BWR Task Group Standard Problem Analysis Advanced Instrumentation TC-3 Tests		237 233 118 2,587 700 150 973 255
Total	45,990	46,115
Total LOFT Fundinga	49,027	48,697

a. Excludes C.E., GSO, and overhead.

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TABLE 5. LOFT FY-80 SUMMARY BUDGET STATUS REPORT OF LOFT FOREIGN FUNDS (In Thousands of Dollars)

LOFT	189	Q80-5-0/3	Approved CL.I CCBs	Current PBM # Q80-5-0/4	Current FY-80 Budget	Total Authorized Spending Limit
5FAXX	A6273	15	8	7	7	147
5FNXX	A6271	166	<36>	130	130	396
5F7XX	A6104	993	<1>	992	992	4,229
5F8XX	A6111	1,337	<53>	1,284	1,284	4,919 <sup>a</sup>
5F9XX	A6104S	0	0	0	0	0
5FXXX		2,511	98	2,413	2,413	9,691
	Foreign	contingency re	serves		84	84
	Foreign o	management res	erves		85	85
	Total FY	-80 LOFT forei	gn funds		2,582	9,860
	Foreign	funds spent th	rough FY-79		6,854	
	Foreign	funds budgeted	in FY-81		424	0
	Total for	reign funds re	ceived to date		9,860	9,860

a. Includes LOFT Lead Rod.

## TABLE 6. LOFT CAPITAL EQUIPMENT STATUS REPORT THROUGH SEPTEMBER

Schedule 189a	Title	Prior Year Uncosted	Current Year Funds	Total Available to Cost	Current Year Costs	Outstanding Commitments	Balance Less Costs and Commitments	Estimate to Complete	Balance
4CA101	Integral System Design & Fab.	111,731	(2,256)	109,475	34,766	23,312	51,397	72,492	2,217
4CA102	LOFT Operations	194,419	(73,041)	121,378	142,003	1,749	<22,374>	1,961	<22,586>
4CA103	UT & Requalification Program	140,034	75,297	215,331	215,291	33,3243	<33,290>	0	34
	Total DOE	446,184	0	446,184	392,066	58,385	<4,267>	74,453	<20,335>
A-6061	Experimental MeasurementsD	788,769	789,000	1,577,769	1,118,137	207,034	252,598	456,813	2,819
A-6048	Integral System Design & Fab.	689,139	1,422,000	2,111,139	947,611	317,381	846,147	1,162,493	1,035
A-6088	LOFT Operations	18,091	89,000	107,091	72,413	14,752	19,926	33,959	719
	Total NRC	1,495,999	2,300,000	3,795,999	2,138,161	539,167	1,118,671	1,653,265	4,573
	Total LOFT	1,942,183	2,300,000	4,242,183	2,530,227	597,552	1,114,404	1,727,718	<15,762>

a. This outstanding commitment is being moved to an MRC account for the same item.

o. Includes A-6085, A-6086, and A-6089.

TABLE 7. FINAL CLOSING VALUES FOR FUNDING SOURCES

Program	Funding Office	1891	Title	FY-1980 Funding (\$)	FY-1980 Actuals (\$)	FY-1980 Carryover (\$)	FY-1981 Funds (\$)	Additional Anticipated Funding (S)	Total FY-1981 (\$)	GS0 (\$)
LOFT LOFT LOFT LOFT	R SR R SR R SR R SR	A6043 A6048 A6053 A6055 A6055	LOFT Instrumentation LOFT Plant & Analysis LOFT Fuels Des & Fab LOFT Operations LOFT in-Serv. Inspection	8,810,919 5,120,275 3,755,086 7,635,682	8,434,170 4,821,811 3,400,658 7,535,871	376,749 298,464 354,430 48,811	5,790,000 4,880,000 3,180,000 9,070,000	0 0 0	7,166,749 5,178,464 3,534,430 9,118,811	90.000 90.000
LOFT LOFT LOFT LOFT	RSR RSR RSR RSR RSR	A6074 A6107 A6108 A6110 A6121	LOFT Advanced Instru. LOFT Plant Support LOFT Ennanced Operator C LOFT Common Support	37.795 11,175,284 771,000 3,614,362 2,622	37.795 10,802,384 755,479 3,582,032 2,822	373,100 15,521 32,330	3,353,000 940,000 5,520,000	00000	7,733,100 955,521 5,552,330	0 0
LOFT LOFT LOFT LOFT	RSR RSR RSR RSR	A6122 A6275 A6308 A6351	LOFT Exp. Support Testing LOFT Core Safety LOFT Muc/Elec. Heater LOFT CTR Display Des/Ana LOFT Core Melt Mitigation	4,132,398 69,819 200,000 125,000	4,095,797 69,819 146,198 60,019	35,601 0 53,802 64,981	4,550,000 500,000 500	500,000	4,696,601 0 553,802 664,981	5 5 5 0
	SUBTOTAL			45,500,644	43,845,855	1,654,789	42,900,000	600,000	45,154,789	235,000
LOFT FOREIGN LOFT LOFT LOFT LOFT	R SR R SR R SR R SR R SR	A6104 A6111 A6271 NE4 A6273	LOFT FRGResearch Prog. LOFT Gapan Research Prog. LOFT Coop-Netherlands LOFT French Funds LOFT Coop-Austria	925,199 1,827,134 268,035 0 16,573	884,952 1,386,111 109,085 0 5,457	40,247 441,023 158,950 0	1,000,000 1,000,000 150,000 1,000,000	0 0 0 0	1.040,247 1.441,023 318,950 1,000,000 11,116	00000
	SUBTOTAL			3,036,941	2,385,605	651,336	3,160,000	0	3,811,336	0
LOFT LOFT LOFT LOFT LOFT LOFT	R SR R SR R SR R SR R SR R SR	A6061 A6084 A6085 A6086 A6088 A6088	LOFT instrumentation LOFT Plant Support LOFT Exp. Program LOFT Advanced Instru. LOFT Operations LOFT fuel Des/Fab	1,014,253 2,111,139 1,398 373,873 107,091 188,245	806,848 947,615 0 294,606 72,412 16,596	207,405 1,163,524 1,398 79,267 34,679 171,549	945,000 1,079,000 0 76,000	0000	1,152,405 2,242,524 1,398 79,267 110,679 171,549	0000
	SUBTOTAL	NRC		3,795,999	2,138,177	1,657,822	2,100,000	.0	3,757,822	2
LOFT CAPITAL LOFT LOFT	DOE DOE DOE	4CA101 4CA102 4CA103	LOFT Internal Sys. Des/Fab. LOFT Operations LOFT VT & Requal.	101,731 126,419 218,034	34,765 142,003 215,296	56,965 <15,584> 2,738	0 0	0 0 0	66.965 <15.584> 2,738	0 0
	SUBTOTAL	DOE		446,184	392,065	54,119	0	0	<54,119>	0
LOFT	RSR RSR	A60488 A60480	Standard Analysis PMR/BWR Task Force	150,000 700,000	149,725 700,001	275 <1>	0 8	0	0	0
	TOTAL			53,629,768	49,611,428 (A60488&C)	4,018,340 -274 4,018,066	48,150,000	600,000 (A6048B&C)	52,778,340 -274 52,778,066	0 0 235,000

TABLE 7. (continued)

Program	Funding Office	189a	Title	FY-1980 Funding (\$)	FY-1980 Actuals (\$)	FY-1980 Carryover (\$)	FY-1981 Funds (3)	Additional Anticipated Funding (S)	Total FY-1981 (\$)	650 (\$)
MEMO Supplemen	ntal Programs									
(Included in totals above)	(42810) (42920) (4281E)	A6048 A6048	TC-2 Tests TC-3 Tests	95,831 338,169	95.831 272.844	0 55,325	0	0	0 65,325	0.0
	(51BEP01R5) (4292C123) (412X) (47ARN5) (42LXXX)	A6048 A6043 A6043 A6043	Electric Heater Rod LTSF Advanced Instrumentation LOFT Lead Rod Support	233,000 328,000 2,587,000 973,000 192,000	247,355 249,943 2,612,940 957,661 193,318	<14,355> 78,057 <25,940> 15,339 <1,318>	200,000 1,720,000 750,000	0 0 0	278,057 1,694,060 765,339 0	0 0
	SUBTOTAL			4,747,000	4,629,892	117,108	2,670,000	0	2,802,781	0

a. Excludes from LOFT's funding source.



