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# **Results for the First Quarter 2012 Tank 50 WAC Slurry Sample: Chemical and Radionuclide Contaminant Results**

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## EXECUTIVE SUMMARY

This report details the chemical and radionuclide contaminant results for the characterization of the 2012 First Quarter sampling of Tank 50 for the Saltstone Waste Acceptance Criteria (WAC).<sup>1</sup> Information from this characterization will be used by Waste Solidification Engineering (WSE) to support the transfer of low-level aqueous waste from Tank 50 to the Salt Feed Tank in the Saltstone Facility in Z-Area, where the waste will be immobilized. This information is also used to update the Tank 50 Waste Characterization System.

The following conclusions are drawn from the analytical results provided in this memorandum:

- The concentrations of the reported chemical and radioactive contaminants were less than their respective WAC targets or limits unless noted in this section.
- The reported detection limit for <sup>94</sup>Nb is above the requested limit from Reference 2 but below the estimated limit in Reference 3.
- <sup>247</sup>Cm and <sup>249</sup>Cf are above the requested limits from Reference 2. However, they are below the limits established in Reference 3.
- The reported detection limit<sup>4</sup> for Norpar 13 is greater than the limit from Table 4 and Attachment 8.2 of the WAC<sup>1</sup>.
- The reported detection limit for Isopar L is greater than the limit from Table 3 of the WAC<sup>1</sup>.
- Isopar L and Norpar 13 have limited solubility in aqueous solutions making it difficult to obtain consistent and reliable sub-samples. The values reported in this report are the concentrations in the sub-sample as detected by the instrument; however, the results may not accurately represent the concentrations of the analytes in Tank 50.
- The low insoluble solids content increases the measurement uncertainty for insoluble species.

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## LIST OF ABBREVIATIONS

AA	Atomic Absorption (spectroscopy)
AD	Analytical Development
ARP/MCU	Actinide Removal Process/Modular CSSX Unit
CLFL	Composite Lower Flammability Limit
CSSX	Caustic Side Solvent Extraction
CVAA	Cold Vapor Atomic Absorption
DDA	Deliquification, Dissolution and Adjustment
EPA	Environmental Protection Agency
ETP	Effluent Treatment Project
GC/MS	Gas Chromatograph/Mass Spectrometer
HDPE	High Density Polyethylene
HPLC	High Performance Liquid Chromatography
IC	Ion Chromatography
ICP-AES	Inductively coupled plasma – atomic emission spectroscopy
ICP-MS	Inductively coupled plasma – mass spectrometry
L	Liter
LLW	Low Level Waste
LSC	Liquid Scintillation Counting
MDL	Method Detection Limit
MRL	Method Reporting Limit
mg	Milligram
mL	Milliliter
NA	Not Applicable
ND	Not Determined
pCi/mL	Picocurie per milliliter
RSD	Relative Standard Deviation
SC	Shielded Cells (Facility)
SDF	Saltstone Disposal Facility
SFT	Salt Feed Tank
SPF	Saltstone Production Facility
SRNL	Savannah River National Laboratory
SRR	Savannah River Remediation
SRS	Savannah River Site

SVOA	Semi-volatile Organic Analysis
TCLP/UHC	Toxic Characterization Leaching Procedure/Underlying Hazardous Constituent
TIC	Tentatively Identifiable Compound
TIC/TOC	Total inorganic carbon/total organic carbon
TTQAP	Task Technical and Quality Assurance Plan
TTR	Technical Task Request
VOA	Volatile organic analysis
WAC	Waste Acceptance Criteria
WCS	Waste Characterization System
WSE	Waste Solidification Engineering
WT %	Weight percent



## 1.0 Introduction

The Saltstone Facility is designed and permitted to immobilize and dispose of low-level radioactive and hazardous liquid waste (salt solution) remaining from the processing of radioactive material at the Savannah River Site.<sup>1</sup> Low-level waste (LLW) streams from the Effluent Treatment Project (ETP), H-Canyon, and the decontaminated salt solution product from the Actinide Removal Process/Modular Caustic Side Solvent Extraction (CSSX) Unit (ARP/MCU) process are stored in Tank 50 until the LLW can be transferred to the Saltstone Facility for treatment and disposal. The LLW must meet the specified waste acceptance criteria (WAC) before it is processed into saltstone.<sup>1</sup> The specific chemical and radionuclide contaminants and their respective WAC limits are in the current Saltstone WAC.<sup>1</sup>

Waste Solidification Engineering (WSE) requested that the Savannah River National Laboratory (SRNL) perform quarterly analysis on saltstone samples.<sup>5</sup> The concentrations of chemical and radionuclide contaminants are measured to ensure the saltstone produced during each quarter is in compliance with the current WAC.<sup>1,5,6</sup> This report documents the concentrations of chemical and radionuclide contaminants and discusses those results for the 2012 First Quarter samples collected from Tank 50 on January 25, 2012.

## 2.0 Experimental

On January 25, 2012, one 200-mL sampler (HTF-50-12-12) and one 1-L sampler (HTF-50-12-11) were collected from Tank 50 for First Quarter 2012 WAC analyses and delivered to the SRNL Shielded Cells (SC). The 200 mL sampler is a dip sample taken six inches below the surface and the one liter sampler was pulled 66 inches from the bottom of the tank with one agitator pump running.

At SRNL, slurry samples (~10 mL each) from HTF-50-12-12 were transferred with glass pipettes to glass vials with Teflon-lined caps. The vials were completely filled to minimize the void space and the volatilization of organics. The aliquots were transferred to the Analytical Development (AD) Organic Analysis Laboratory for semi-volatile and volatile organic analysis (SVOA and VOA, respectively). Four additional 10-mL aliquots (for duplicate analyses) were used for SVOA analysis to determine the concentration of Isopar L and Norpar 13, respectively, in the sample.

After the samples for organic analyses were obtained, the slurries in the steel samplers were combined into a 2-L high density polyethylene (HDPE) bottle. The 200 mL steel sampler was agitated to disperse solids in the slurry and poured into the 2-L HDPE bottle. A slurry pump was used to transfer the contents of the 1-L sampler to the 2-L bottle. The transferred slurry was left to settle in the 2-L bottle. A portion of the clear supernate was returned to each steel sampler, mixed to mobilize any remaining solids, and again returned to the 2-L HDPE bottle. Visual inspection of the inside of sampler indicated there were no visible solids remaining in the samplers. The total weight of the transferred slurry was 1579.32 g

The 2-L HDPE bottle was agitated to thoroughly disperse the solids into the supernate. Aliquots of slurry samples were promptly collected with slurry pipettes to minimize settling effects and placed into HDPE bottles. A three milliliter sample of the slurry was used to determine the density of the slurry using an Anton-Paar DMA 35n portable density meter.

Slurry samples were submitted in triplicate to AD laboratories for the following analyses:

- Six-mL aliquots to the AD Ion Chromatography (IC) Laboratory for soluble anion analyses and soluble cation analyses.
- Three-mL aliquots to the AD Organic Analysis Laboratory for measurement of tetraphenylborate and ethylenediaminetetraacetate by high performance liquid chromatography (HPLC).
- Six-mL aliquots to the AD Wet Chemistry Laboratory for Total Inorganic Carbon/ Total Organic Carbon (TIC/TOC) analyses.
- Approximately 70-mL aliquots were removed from the 2-L HDPE bottle. After each 70-mL aliquot was prepared, it was divided into one 50-mL and one 20-mL sample and sent to AD Radiochemistry Laboratory for radiochemical separations and analyses. Subsamples are historically required in order to stay within the dose limits and hood limits for beta radiation.
- Twelve-mL aliquots of filtered supernate were prepared by filtering aliquots of supernate using a 0.45 micron syringe filter. The filtered supernate samples were then submitted to the AD Wet Chemistry Laboratory for TIC/TOC analyses and Total Base analyses.
- Twelve-mL aliquots were sent to the AD Dissolution Laboratory for digestion using an aqua regia method<sup>7</sup>. Visual inspection of the digested sample by the AD Task Supervisor indicated that all the solids had dissolved. Aliquots of dissolved slurries were analyzed using inductively coupled plasma – atomic emission spectroscopy (ICP-AES), inductively coupled plasma – mass spectrometry (ICP-MS), atomic absorption spectroscopy (AA) for As, K, Na, and Se, and cold vapor atomic absorption spectroscopy (CVAA) for Hg.

### 3.0 Results and Discussion

The following tables contain the results for the First Quarter (1Q12) WAC analyses. Each table provides the analyte of interest, the method used for measuring that analyte, the average concentration of the analyte based on triplicate samples (unless otherwise noted), the %RSD of the average, and, if applicable, the WAC target or limit for the analyte concentration. Several of the contaminants were either not detected in the slurry samples or detected at values below the method reporting limit (MRL). For those analytes, the result is preceded by a “<” which indicates the result is an upper limit based on the sensitivity of the method used to analyze the individual analyte.

Tables 3-1, 3-2, 3-3 and 3-4 are based directly on attachments 8.1, 8.2, 8.3, and 8.4, respectively, of the WAC.<sup>1</sup>

**Table -3-1. Results for the 1st Quarter 2012 Tank 50 Slurry Samples for Chemical Contaminants Listed in Attachment 8.1 of the Saltstone WAC, Revision 10**

<u>Chemical Name</u>	<u>Method</u>	<u>Average Concentration (mg/L)</u>	<u>% RSD</u>	<u>WAC Limit (mg/L)</u>
Ammonium (NH <sub>4</sub> <sup>+</sup> )	IC	<1.00E+01	NA	7.13E+03
Carbonate (CO <sub>3</sub> <sup>2-</sup> )	TIC	1.40E+04 <sup>a</sup>	15.5	1.45E+05
Chloride (Cl <sup>-</sup> )	IC	1.15E+02	15.1	9.68E+03
Fluoride (F <sup>-</sup> )	IC	<1.00E+02	NA	4.94E+03
Free Hydroxide (OH <sup>-</sup> )	Total Base	3.23E+04	0.61	1.91E+05
Nitrate (NO <sub>3</sub> <sup>-</sup> )	IC	1.08E+05	2.45	5.29E+05
Nitrite (NO <sub>2</sub> <sup>-</sup> )	IC	1.86E+04	13.9	2.59E+05
Oxalate (C <sub>2</sub> O <sub>4</sub> <sup>2-</sup> )	IC	4.84E+02	4.74	3.30E+04
Phosphate (PO <sub>4</sub> <sup>3-</sup> )	IC/ICP-ES	4.34E+02	13.2	3.56E+04
Sulfate (SO <sub>4</sub> <sup>2-</sup> )	IC	3.68E+03	0.83	6.89E+04
Arsenic (As)	AA	<9.76E-02	NA	7.50E+02
Barium (Ba)	ICP-ES	<4.62E-01	NA	7.50E+02
Cadmium (Cd)	ICP-ES	<6.21E-01	NA	3.75E+02
Chromium (Cr)	ICP-ES	4.24E+01	0.60	1.50E+03
Lead (Pb)	ICP-MS	2.18E-01	11.1	7.50E+02
Mercury (Hg)	CVAA	3.31E+01	2.87	3.25E+02
Selenium (Se)	AA	<1.96E-01	NA	4.50E+02
Silver (Ag)	ICP-ES	<1.30E+00	NA	7.50E+02
Aluminum (Al)	ICP-ES	3.03E+03	0.80	1.41E+05
Potassium	AA	2.84E+02	4.22	3.67E+04
Nickel Hydroxide	ICP-ES	<2.25E+00 <sup>d</sup>	NA	3.95E+03
n-Butanol	VOA	<5.00E-01 <sup>b</sup>	NA	2.25E+03
Isobutanol	VOA	<5.00E-01 <sup>b</sup>	NA	2.25E+03
Isopropanol	VOA	<2.50E-01 <sup>b</sup>	NA	2.25E+03
Phenol	SVOA	<1.00E+01 <sup>b</sup>	NA	7.50E+02
Isopar L	SVOA	<2.71E+01 <sup>b,c</sup> ppm	NA	1.50E+02 ppm
Total organic carbon	TOC	4.04E+02 <sup>a</sup>	14.6	5.00E+03
Tetraphenylborate (TPB anion)	HPLC	<5.00E+00	NA	7.50E+02

a. Measurement performed on filtered supernate samples.

b. Measurement performed on duplicate samples rather than triplicate samples.

c. Result is calculated from the reported concentration of < 33 mg/L and the density of the slurry sample listed in Table 3-8.

d. Result is calculated from the measured Ni concentration assuming all of the Ni is present as the hydroxide compound.

**Table -3-2. Results for the 1st Quarter 2012 Tank 50 Slurry Samples for Chemical Contaminants Listed in Attachment 8.2 of the Saltstone WAC, Revision 10**

<u>Chemical Name</u>	<u>Method</u>	<u>Average Concentration (mg/L)</u>	<u>% RSD</u>	<u>WAC TARGET (mg/L)</u>
<b>Boron (B)</b>	ICP-ES	8.73E+01	0.50	<b>9.00E+02</b>
<b>Cobalt (Co)</b>	ICP-MS	<7.55E-01	NA	<b>9.00E+02</b>
<b>Copper (Cu)</b>	ICP-ES	<6.48E-01	NA	<b>9.00E+02</b>
<b>Iron (Fe)</b>	ICP-ES	9.54E+00	5.16	<b>6.00E+03</b>
<b>Lithium (Li)</b>	ICP-ES	1.48E+01	6.89	<b>9.00E+02</b>
<b>Manganese (Mn)</b>	ICP-ES	2.79E+00	1.53	<b>9.00E+02</b>
<b>Molybdenum (Mo)</b>	ICP-ES	1.13E+01	6.99	<b>9.00E+02</b>
<b>Nickel (Ni)</b>	ICP-ES	<1.42E+00	NA	<b>9.00E+02</b>
<b>Silicon (Si)</b>	ICP-ES	4.52E+01	2.46	<b>1.29E+04</b>
<b>Strontium (Sr)</b>	ICP-ES	<4.44E-02	NA	<b>9.00E+02</b>
<b>Zinc (Zn)</b>	ICP-ES	5.62E+00	3.37	<b>9.75E+02</b>
<b>Benzene</b>	VOA	<1.50E-01 <sup>a</sup>	NA	<b>3.75E+02</b>
<b>Methanol</b>	VOA	b	NA	<b>2.25E+02</b>
<b>Toluene</b>	VOA	<1.50E-01 <sup>a</sup>	NA	<b>3.75E+02</b>
<b>Tributyl Phosphate (TBP)</b>	SVOA	<7.50E-01 <sup>a</sup>	NA	<b>3.00E+02</b>
<b>EDTA</b>	HPLC	<1.00E+02	NA	<b>3.75E+02</b>
<b>Norpar 13</b>	SVOA	<7.50E-01 <sup>a</sup>	NA	<b>1.00E-01</b>

a. Measurement performed on duplicate samples rather than triplicate samples.

b. Currently, a routine method for detecting this species does not exist in AD.

As indicated in Tables 3-1 and 3-2, all of the contaminants are within the WAC limits with the exception of Norpar 13. In October 2010, AD reviewed the MRL's for the organic constituents in Tank 50. All of the MRL's are at or below the WAC limits for the organics with the exception of Norpar 13 which has an MRL of 0.75 mg/L, which is above the WAC limit.<sup>4</sup> Isopar L and Norpar 13 have negligible solubility in aqueous solutions, which makes it difficult to obtain reliable sub-samples of the original sample. The values reported in these tables are the concentrations as detected by the GC/MS but may not necessarily be an accurate representation of the concentrations of these analytes in Tank 50.

**Table -3-3. Results for 1st Quarter 2012 Tank 50 Slurry Samples and WAC Limits for Radionuclide Contaminants Listed in Attachment 8.3 of the Saltstone WAC, Revision 10**

<b>Radionuclide</b>	<b>Method</b>	<b>Average Concentration (pCi/mL)</b>	<b>% RSD</b>	<b>WAC LIMIT (pCi/mL)</b>
<b>Tritium (<sup>3</sup>H)</b>	Tritium counting	3.85E+02	8.86	<b>5.63E+05</b>
<b>Carbon-14 (<sup>14</sup>C)</b>	C-14 Liquid scintillation	6.73E+02	15.8	<b>1.13E+05</b>
<b>Nickel-63 (<sup>63</sup>Ni)</b>	Ni-59/63	<8.47E+00	NA	<b>1.13E+05</b>
<b>Strontium-90 (<sup>90</sup>Sr)</b>	Sr-90 Liquid scintillation	2.31E+03	14.3	<b>2.25E+07</b>
<b>Technetium-99 (<sup>99</sup>Tc)</b>	Tc-99 Liquid scintillation	2.67E+04	6.93	<b>4.22E+05</b>
<b>Iodine-129 (<sup>129</sup>I)</b>	I-129 (w/ separation) Liquid scintillation	6.05E+00	3.75	<b>1.13E+03</b>
<b>Cesium-137 (<sup>137</sup>Cs)</b>	Gamma Scan	1.63E+06	3.21	<b>4.75E+07</b>
<b>Uranium-233 (<sup>233</sup>U)</b>	ICP-MS	<3.44E+02	NA	<b>1.13E+04</b>
<b>Uranium-235 (<sup>235</sup>U)</b>	ICP-MS	2.70E-01	4.90	<b>1.13E+02</b>
<b>Plutonium-241 (<sup>241</sup>Pu)</b>	Pu238/241 Liquid scintillation	<4.48E+00	NA	<b>8.38E+05</b>
<b>Total Alpha</b>	Liquid Scintillation Counting	<9.77E+02	NA	<b>2.50E+05</b>

**Table -3-4. Results for 1st Quarter 2012 Tank 50 Slurry Samples and WAC Targets for Radionuclide Contaminants Listed in Attachment 8.4 of the Saltstone WAC, Revision 10**

<b>Radionuclide</b>	<b>Method</b>	<b>Average Concentration (pCi/mL)</b>	<b>%RSD</b>	<b>WAC TARGET (pCi/mL)</b>
<b>Aluminum-26 (<sup>26</sup>Al)</b>	Gamma scan (Cs removed)	<1.50E-01	NA	<b>2.88E+03</b>
<b>Cobalt-60 (<sup>60</sup>Co)</b>	Gamma scan (Cs removed)	4.58E-01	22.5	<b>1.13E+06</b>
<b>Nickel-59 (<sup>59</sup>Ni)</b>	Ni-59/63	<1.65E+01	NA	<b>1.13E+05</b>
<b>Selenium-79 (<sup>79</sup>Se)</b>	Se79	1.65E+01	35.7	<b>1.90E+04</b>
<b>Niobium-93m (<sup>93m</sup>Nb)</b>	ICP-MS	<1.31E+02	NA	<b>2.85E+06</b>
<b>Niobium-94 (<sup>94</sup>Nb)</b>	Gamma scan (Cs removed)	<3.73E-01	NA	<b>1.53E+04</b>
<b>Molybdenum-93 (<sup>93</sup>Mo)</b>	ICP-MS	<5.86E+04	NA	<b>1.18E+07</b>
<b>Ruthenium-106 (<sup>106</sup>Ru)</b>	Gamma scan (Cs removed)	<2.59E+00	NA	<b>1.13E+06</b>
<b>Antimony-125 (<sup>125</sup>Sb)</b>	Gamma scan (Cs removed)	3.31E+02	2.83	<b>2.25E+06</b>
<b>Tin-126 (<sup>126</sup>Sn)</b>	Gamma scan (Cs removed)	1.05E+02	3.45	<b>1.80E+04</b>
<b>Cesium-134 (<sup>134</sup>Cs)</b>	Gamma Scan	<4.36E+01	NA	<b>1.13E+06</b>
<b>Cesium-135 (<sup>135</sup>Cs)</b>	ICP-MS	<6.13E+01	NA	<b>1.13E+06</b>
<b>Cerium-144 (<sup>144</sup>Ce)</b>	Gamma scan (Cs removed)	<4.04E+00	NA	<b>1.13E+05</b>
<b>Promethium-147 (<sup>147</sup>Pm)</b>	Pm147/Sm151 Liquid scintillation	<2.27E+01	NA	<b>5.63E+06</b>
<b>Samarium-151 (<sup>151</sup>Sm)</b>	Pm147/Sm151 Liquid scintillation	2.51E+01	NA	<b>2.25E+04</b>

**Table -3-4(continued). Results for 1st Quarter 2012 Tank 50 Slurry Samples and WAC Targets for Radionuclide Contaminants Listed in Attachment 8.4 of the Saltstone WAC, Revision 10**

<u>Radionuclide</u>	<u>Method</u>	<u>Average Concentration (pCi/mL)</u>	<u>%RSD</u>	<u>WAC TARGET (pCi/mL)</u>
Europium-154 ( <sup>154</sup> Eu)	Gamma scan (Cs removed)	2.34E+00	21.1	2.25E+06
Europium-155 ( <sup>155</sup> Eu)	Gamma scan (Cs removed)	<2.24E+00	NA	1.13E+04
Radium-226 ( <sup>226</sup> Ra)	Gamma scan (Cs removed)	<1.40E+01	NA	7.97E+03
Thorium-229 ( <sup>229</sup> Th)	ICP-MS	<7.55E+03	NA	1.63E+05
Thorium-230 ( <sup>230</sup> Th)	ICP-MS	<7.49E+02	NA	6.26E+03
Thorium-232 ( <sup>232</sup> Th)	ICP-MS	<3.90E-03	NA	2.88E+03
Uranium-232 ( <sup>232</sup> U)	U232	3.13E+00	34.0	1.71E+05
Uranium-234 ( <sup>234</sup> U)	ICP-MS	<2.22E+02	NA	1.13E+04
Uranium-236 ( <sup>236</sup> U)	ICP-MS	<2.30E+00	NA	1.13E+04
Uranium-238 ( <sup>238</sup> U)	ICP-MS	4.95E+00	1.42	1.13E+04
Neptunium-237 ( <sup>237</sup> Np)	ICP-MS	<2.50E+01	NA	2.50E+05
Plutonium-238 ( <sup>238</sup> Pu)	Pu238/241 Pu alpha PHA	1.16E+03	17.7	2.50E+05
Plutonium-239 ( <sup>239</sup> Pu)	Pu238/241 Pu alpha PHA	1.60E+02	3.94	2.50E+05
Plutonium-240 ( <sup>240</sup> Pu)	Pu238/241 Pu alpha PHA	1.60E+02	3.94	2.50E+05
Plutonium-242 ( <sup>242</sup> Pu)	ICP-MS	<1.36E+02	NA	2.50E+05
Plutonium-244 ( <sup>244</sup> Pu)	ICP-MS	<6.30E-01	NA	7.02E+04
Americium-241 ( <sup>241</sup> Am)	Gamma scan (Cs removed)	2.18E+01	87.0	2.50E+05
Americium-242m ( <sup>242m</sup> Am)	Am/Cm	<1.73E-01	NA	4.50E+05
Americium-243 ( <sup>243</sup> Am)	Am/Cm	<8.20E-01	NA	2.50E+05
Curium-242 ( <sup>242</sup> Cm)	Am/Cm	<1.43E-01	NA	1.13E+04
Curium-244 ( <sup>244</sup> Cm)	Am/Cm	3.30E+02	134	2.50E+05
Curium-245 ( <sup>245</sup> Cm)	Am/Cm	<2.08E+00	NA	2.25E+05

As shown in Table 3-4, none of the radionuclide contaminants exceed the targets listed in the latest revision of the WAC. In a memo from Savannah River Remediation (SRR), the requested detection limits for several radionuclides were lowered in order to accommodate future inventory reporting requirements.<sup>2</sup> The reported detection limit of <sup>94</sup>Nb is above the limit requested by SRR (2.00E-03 pCi/mL)<sup>2</sup> but below the limit set by AD (4.38E-01 pCi/mL).<sup>3</sup>

The values for <sup>93m</sup>Nb and <sup>93</sup>Mo in Table 3-4 are estimated from the ICP-MS result for mass 93. The entire signal at mass 93 is assigned to <sup>93</sup>Zr, and since it is in secular equilibrium with <sup>93m</sup>Nb, the maximum activity of the <sup>93m</sup>Nb is equal to that of the <sup>93</sup>Zr. The specific activity of <sup>93</sup>Zr (2.51E-03 Ci/g) is used when calculating the activity concentration of <sup>93m</sup>Nb. Similarly, <sup>93</sup>Mo is estimated by assigning all of mass 93 to <sup>93</sup>Mo and using the specific activity of <sup>93</sup>Mo to calculate

the concentration. The concentration of  $^{135}\text{Cs}$  is calculated by assigning all of the mass at 135 to cesium. It is assumed all the mass detected at mass 244 is  $^{244}\text{Pu}$ . The Pu alpha Pulse Height Analysis (PHA) method does not resolve the alpha activities of  $^{239}\text{Pu}$  and  $^{240}\text{Pu}$ . To determine the maximum concentration of each radionuclide, the total activity is assigned to each radionuclide separately. As shown in Table 3-4, the reported activity is below the WAC limit for each radionuclide.

The relatively low insoluble solids content of the 1Q12 Tank 50 WAC sample,  $0.17 \text{ wt}\% \pm 0.12 \text{ wt}\%$  (see Table -3-5) results in higher uncertainties for insoluble radionuclide species such as Am-241 and Cm-244 as shown in Table -3-4. The low insoluble solids content makes subsampling difficult; therefore the reported results are not necessarily representative of the concentration of these analytes in the Tank 50 sample received by SRNL.

Tables 3-5 and 3-6 list the chemical contaminants that impact vault flammability. These chemicals must be monitored to ensure flammable gases do not contribute more than 10% of the Composite Lower Flammability Limit (CLFL).<sup>1</sup>

**Table -3-5. Results for the 1st Quarter 2012 Tank 50 Slurry Samples for Acceptance Criteria Limits for Chemical Contaminants Impacting Vault Flammability, Listed in Table 3 of the Saltstone WAC, Revision 10**

<u>Chemical Name</u>	<u>Method</u>	<u>Average Concentration (mg/L)</u>	<u>% RSD</u>	<u>WAC Limit</u>
Isopar L	SVOA	<2.71E+01 ppm <sup>a</sup>	NA	1.10E+01 ppm
Tetraphenylborate (TPB anion)	HPLC	<5.00E+00	NA	5.00E+00 mg/L
Ammonium ( $\text{NH}_4^+$ )	IC	<1.00E+01	NA	2.12E+02 mg/L

a. Result is calculated from the reported concentration of < 33 mg/L and the density of the slurry sample.

**Table -3-6. Results for the 1st Quarter 2012 Tank 50 Slurry Samples for Concentrations of “Other Organics” Impacting Vault Flammability, Listed in Table 4 of the Saltstone WAC, Revision 10**

<u>Chemical Name</u>	<u>Method</u>	<u>Average Concentration (mg/L)</u>	<u>% RSD</u>	<u>WAC Concentrations</u>
n-Butanol	VOA	<5.00E-01	NA	0.75 mg/L
Tributylphosphate	SVOA	<7.50E-01	NA	1.0 mg/L
Isopropanol	VOA	<2.50E-01	NA	0.25 mg/L
Methanol	a	NA	NA	0.25 mg/L
Norpar 13	SVOA	<7.50E-01	NA	0.1 mg/L

a. Currently, a routine method for detecting this species does not exist in AD.

Isopar L and Norpar 13 are the only species considered in Tables 3-5 or 3-6 with reported values above the WAC limit. Although the reported detection limit for Isopar L is greater than the WAC limit for vault flammability, it is below the WAC limits for accident analysis as shown in Table 3-1. It should be noted that the detection limit for Isopar L was expected based on current AD capabilities.<sup>4</sup> The reported detection limit for Norpar is above the WAC limit for both accident analysis (Table 3-2) and vault flammability (Table 3-6) but it is the lowest achievable MRL for this analyte.<sup>4</sup> As previously discussed, the insolubility of Isopar L and Norpar 13 makes sub-

sampling difficult, therefore the reported results are not necessarily representative of the concentration of these analytes in the Tank 50 sample received by SRNL.

Table 3-7 provides results for the processing criteria for transfers into the Saltstone Facility.

**Table -3-7. Results for the 1st Quarter 2012 Tank 50 Slurry Samples for Saltstone Processing Criteria WAC Limits, Listed in Table 5 of the Saltstone WAC, Revision 10**

<u>Processing Criterion</u>	<u>Method</u>	<u>Value</u>	<u>Value</u>
<b>pH &gt; 10</b>	Calculated	> 13	NA
<b>2.5 M &lt; [Na<sup>+</sup>] &lt; 7.0 M</b>	ICP-ES	4.87 M	0.49 %RSD
<b>Total Insoluble Solids &lt;15 wt%</b>	Calculated	0.17 wt%	± 0.12 wt% <sup>a</sup>

a. This is the 95% uncertainty calculated from the measured results for the Wt % total solids and Wt % dissolved solids in quadruplicate samples of the slurry.

All of the results contained in Table 3-7 fall within the general processing criteria. The pH was calculated using the free base concentration (OH<sup>-</sup>). The value for the total insoluble solids was calculated from experimentally determined values for total solids and dissolved solids in the slurry supernate.

Table 3-8 provides constituents listed in the Technical Task Request but not contained in the WAC.

**Table -3-8. Requests for Constituents for TCLP/UHC Support as well as from the TTR for the 1st Quarter 2012 Tank 50 Slurry Samples; Results Not Contained in Previous Tables**

<u>Constituent</u>	<u>Method</u>	<u>Average Value (mg/L, unless stated otherwise)</u>	<u>%RSD</u>
<b>Antimony (Sb)</b>	ICP-ES	<9.47E+00	NA
<b>Beryllium (Be)</b>	ICP-ES	<7.10E-02	NA
<b>Cyanide (CN)</b>	a.	NA	NA
<b>Thallium (Tl)</b>	ICP-MS	<5.04E-02	NA
<b>Density (slurry)</b>	Measured (21.2°C)	1.2159	0.12
<b>Total Beta</b>	LSC	2.10E+06 pCi/mL	1.73
<b>Total Solids</b>	Measured	26.15 wt%	0.19

a. Currently, a routine method for detecting this species does not exist in AD.

The results from Table 3-8 are used to support TCLP/UHC testing by a certified laboratory.<sup>8</sup> The density of the slurry was measured at 21.2 °C. Natural Tl is composed of two isotopes, <sup>203</sup>Tl and <sup>205</sup>Tl with fractional abundances of 0.295 and 0.705, respectively. The concentration of each isotope was divided by its fractional abundance and the reported concentration of Tl is that determined from the lowest detection limit value for the two isotopes.



The tank corrosion species listed in Table 3-9 were requested by Waste Solidification Engineering (WSE).<sup>a</sup> Specific gravity was calculated by dividing the measured density of the slurry (given in Table 3-8 at 21.2 °C) by the density of water at the same temperature.<sup>9</sup>

**Table -3-9. Requests from the WSE for Corrosion Species from the 1st Quarter 2012 Tank 50 Slurry Samples; Results Not Contained in Previous Tables**

<u>Constituent</u>	<u>Method</u>	<u>Average Value</u>	<u>%RSD</u>
Specific Gravity	a	1.2184	0.12
Ba-137m	b	1.54E+06 pCi/mL	3.21
Total Gamma	c	1.54E+06 pCi/mL	NA

a. Calculated from the measured density of slurry and density of water at 21.2 °C<sup>9</sup>.

b. Calculated from the measured concentration of Cs-137.

c. Calculated from the sum of measured gamma emitters.

The radionuclide <sup>137m</sup>Ba is the radioactive daughter of 94.6% of the beta decay of <sup>137</sup>Cs. 5.3% of the <sup>137</sup>Cs decays to stable <sup>137</sup>Ba. The half-life of the parent radionuclide, <sup>137</sup>Cs, is five times longer than its daughter, <sup>137m</sup>Ba, therefore the two radionuclides are in secular equilibrium. Radionuclides in secular equilibrium have the same activity associated with their decay. Thus the activity of <sup>137m</sup>Ba is 94.6% of the activity of the <sup>137</sup>Cs or 1.54E+06 pCi/mL. The activities calculated for total gamma and <sup>137m</sup>Ba are expected to be close for this sample because the total gamma activity is dominated by <sup>137m</sup>Ba, the radioactive daughter of <sup>137</sup>Cs. The total gamma activity was calculated by summing the measured gamma activity of the major gamma emitters: <sup>60</sup>Co, <sup>125</sup>Sb, <sup>126</sup>Sb, <sup>126</sup>Sn, <sup>137</sup>Cs (via <sup>137m</sup>Ba), <sup>154</sup>Eu, and <sup>241</sup>Am.

Table 3-10 provides results for additional radionuclides not listed in the WAC but which now require quantification in order to support inventory reporting requirements.

**Table -3-10. Additional Radionuclides Requested for Inventory Reporting Requirements**

<u>Radionuclide</u>	<u>Method</u>	<u>Average Concentration (pCi/mL)</u>	<u>%RSD</u>	<u>REQUESTED TARGET (pCi/mL)</u>
Potassium-40 ( <sup>40</sup> K)	Gamma scan (Cs removed)	<2.47E+00	NA	1.00E+04
Silver-108m ( <sup>108m</sup> Ag)	Gamma scan (Cs removed)	<4.95E-01	NA	1.00E+04
Barium-133 ( <sup>133</sup> Ba)	Gamma scan (Cs removed)	<8.60E-01	NA	1.00E+04
Bismuth-207 ( <sup>207</sup> Bi)	Gamma scan (Cs removed)	<4.33E-01	NA	1.00E+04
Actinium-227 ( <sup>227</sup> Ac)	Gamma scan (Cs removed)	<8.78E+00	NA	1.00E+04
Radium-228 ( <sup>228</sup> Ra)	Gamma scan (Cs removed)	<1.07E+00	NA	1.00E+04
Thorium-228 ( <sup>228</sup> Th)	Gamma scan (Cs removed)	<1.35E+01	NA	1.00E+04
Protactinium-231 ( <sup>231</sup> Pa)	Gamma scan (Cs removed)	<2.16E+01	NA	1.00E+04

<sup>a</sup> Requested in an electronic mail message from S. D. Hevel on December 20, 2007. (See page 30 of WSRC-NB-2007-00189.)

**Table -3-11 (continued). Additional Radionuclides Requested for Inventory Reporting Requirements**

<b>Curium-247 (<sup>247</sup>Cm)</b>	Am/Cm	<3.04E+00	NA	1.43E-11
<b>Californium-249 (<sup>249</sup>Cf)</b>	Am/Cm	<3.26E+00	NA	1.33E-10
<b>Californium-251 (<sup>251</sup>Cf)</b>	Am/Cm	<2.37E+00	NA	1.00E+02

#### 4.0 Conclusions

The following conclusions are drawn from the analytical results provided in this report:

- The concentrations of the reported chemical and radioactive contaminants were less than their respective WAC targets or limits unless noted in this section.
- The reported detection limit for <sup>94</sup>Nb is above the requested limit from Reference 2 but below the estimated limit in Reference 3.
- <sup>247</sup>Cm and <sup>249</sup>Cf are above the requested limits from Reference 2. However, they are below the limits established in Reference 3.
- The reported detection limit<sup>4</sup> for Norpar 13 is greater than the limit from Table 4 and Attachment 8.2 of the WAC<sup>1</sup>.
- The reported detection limit for Isopar L is greater than the limit from Table 3 of the WAC<sup>1</sup>.
- Isopar L and Norpar 13 have limited solubility in aqueous solutions making it difficult to obtain consistent and reliable sub-samples. The values reported in this report are the concentrations in the sub-sample as detected by the instrument; however, the results may not accurately represent the concentrations of the analytes in Tank 50.
- The low insoluble solids content increases the measurement uncertainty for insoluble species.

## 5.0 References

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9. Lide, D. R., *Handbook of Chemistry and Physics*. 90th ed.; CRC Press: Boca Raton, FL, 2009; p 6-4.