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Formal Report

**DEPLOYMENT OF INNOVATIVE CHARACTERIZATION
TECHNOLOGIES AND IMPLEMENTATION OF THE MARSSIM
PROCESS AT RADIOLOGICALLY CONTAMINATED SITES**

FY 2000 PROGRAM DOCUMENT COMPILATION

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INTRODUCTION

In FY 2000, Brookhaven National Laboratory (BNL) completed an Accelerated Site Technology Deployment (ASTD) project entitled, "Deployment of Innovative *In Situ* Characterization Technologies and Implementation of the MARSSIM Process at Radiologically Contaminated D&D Sites." The work, sponsored by the DOE Office of Science and Technology's Decontamination and Decommissioning Focus Area (DDFA) provided assistance for the characterization of the BNL Brookhaven Graphite Research Reactor (BGRR) that was shut down in 1968 and is currently being decommissioned. Several innovative technologies (the Canberra *In Situ* Object Counting System or ISOCS and the BetaScint Fiber Optic Detector) were deployed to accurately, rapidly, and inexpensively characterize equipment, facilities, and materials to facilitate D&D operations. The MARSSIM process, usually used for final status surveys, was implemented for on-going characterization to better identify the types and number of samples needed. The project successfully demonstrated the value of *in situ* characterization combined with MARSSIM, both in terms of reduced costs and lower worker exposure to radiation hazards. The innovative technologies were also shown to compare favorably on a technical basis with conventional baseline characterization techniques.

Additional information can be obtained at the following web sites:

- <http://www.bnl.gov/bgrr/>
- <http://www.dne.bnl.gov/ewtc>

EXECUTIVE SUMMARY

INTRODUCTION AND BACKGROUND

One of the most significant issues facing planners of decontamination and decommissioning (D&D) projects is the cost associated with characterization of the facility. There is uncertainty concerning the amount of data that needs to be collected and the level of analysis required in all phases of a D&D project, from the initial planning phase through to the closure phase. These uncertainties, which make it difficult to define the full scope of a project at the outset and to prepare with confidence a feasible schedule, can only be reduced by the implementation of an appropriately comprehensive characterization of the facility. Conventional characterization techniques require taking, analyzing, and evaluating a large number of samples; a time-consuming, costly, and potentially hazardous process. Past experiences have shown that characterization tends to be a significant contributor to the total project cost and a pivotal factor in defining and adhering to the project schedule.

This Accelerated Site Technology Deployment (ASTD) project addresses some of the important issues of characterization through deployment of innovative *in situ* characterization technologies and the implementation of the guidelines contained in the Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM). It focuses on the characterization of the Brookhaven Graphite Research Reactor (BGRR), which is currently undergoing an accelerated schedule for D&D. The Environmental Management program of the U.S. Department of Energy (USDOE) includes the D&D of many different types of nuclear facilities. However, the basic issues of characterization are universal and not dependent of site-specific dissimilarities. Thus successful deployment of the technologies included in this project will provide valuable experience and lessons learned that can be shared with facilities throughout the USDOE complex. The project itself involves collaboration between the Brookhaven National Laboratory (BNL) Environmental Sciences Department, BGRR Decommissioning Project (BGRR-DP), the USDOE Environmental Measurements Laboratory (EML), URS Corporation of Ronkonkoma, NY, Canberra Industries of Meriden, CT, Cabrera Services, Inc of East Hartford, CT, and BetaScint, Inc., Kennewick, Washington. Also participating is Bechtel Hanford Inc., which will deploy this approach for characterization as part of the Canyon Disposition Initiative at Hanford. Other DOE sites (e.g., Nevada Test Site) are making plans for additional deployments as part of their D&D mission.

FY 1999 ACCOMPLISHMENTS

The project was initiated mid-year in FY 1999. Program summary documents included a Technology Deployment Plan submitted to and approved by DOE, and a Technology Fact Sheet. Project technical documents included the project Quality Assurance Project Plan, the Standard Operating Procedure for acquiring gamma radiation spectra with the ISOCS instrument, Project Specific Survey Plans and Project Data Reports. Analytical support of the BGRR-DP included characterization campaigns of the Pile Fan Sump, Above Grade Ducts, and the Exhaust Fan House. Specific details have been provided previously in the *FY 1999 Program Document Compilation*, October 1999.

FY 2000 ACCOMPLISHMENTS

The ASTD project was successfully completed in FY 2000. A compilation of project documents is provided in subsequent sections of this document. Program summary documents developed during FY 2000 included a paper appearing in the *Proceedings* of the Waste Management Conference, an invited presentation to the Canberra Users Group conference, and comparability reports on the ISOCS gamma spectrometer and the BetaScint Sr-90 instrument. The ISOCS Comparability Report also contains comprehensive information on costs and regulatory/institutional issues and thus, doubles as an ASTD cost and performance report. Project technical documents developed during FY 2000 included a Standard Operating Procedure for analyzing gamma radiation spectra using the ISOCS software, an operating procedure for sample processing of contaminated soils when using the BetaScint instrument, Project Specific Survey Plans, and Project Data Reports. Analytical support of the BGRR-DP included continued support on characterization campaigns of the Pile Fan Sump and

Above Grade Ducts. New campaigns included characterization of the Duct internal components, the Reactor Graphite Pile, the Fuel Transfer Canal, and the Water Treatment House. Additionally, the project supported the BNL Environmental Restoration Division (ERD) with characterization of soils during the Landscape Soil Remediation project.

SUMMARY and CONCLUSIONS

The success of this ASTD project is measured on several levels. First, the deployment of this innovative approach using *in situ* characterization, portable field laboratory measurements, and implementation of MARSSIM was successfully established for all three phases of D&D characterization, i.e., pre-job scoping, on-going disposition of waste, and final status surveys upon completion of the activity. Unlike traditional D&D projects, since the BGRR-DP is operating on an accelerated schedule, much of the work is being carried out simultaneously. Rather than complete a full characterization of the facility before D&D work begins, specific removal actions require characterization as the activity progresses. Thus, the need for rapid and cost-effective techniques for characterization is heightened.

Secondly, since the approach used for this ASTD project was not thoroughly proven prior to deployment, a large effort was devoted to demonstrating technical comparability to project managers, regulators and stakeholders. During the initial phases, large numbers of replicate samples were taken and analyzed by conventional baseline techniques to ensure that BGRR-DP quality assurance standards were met. ASTD project staff prepared comparisons of data gathered using ISOCS and BetaScint with traditional laboratory methods and presented this information to BGRR-DP staff and regulators from EPA Region II, NYS Department of Environmental Conservation, and the Suffolk County Board of Health. As the results of comparability evaluations became available, approval for these methods was received and the techniques associated with *in situ* characterization, portable field laboratory measurements, and implementation of MARSSIM were gradually integrated into BGRR-DP procedures.

Clearly, a significant metric for success is cost effectiveness. Integration of this approach and displacement of traditional baseline characterization at BGRR-DP resulted in far fewer samples that needed to be taken, packaged for transport to an off-site laboratory and analyzed. In addition, because samples are analyzed rapidly on-site, cost per sample is drastically reduced. For example, cost per sample for ISOCS measurements was calculated to be \$76, compared with \$252 per sample for conventional measurement, a savings of \$175 per sample. Discounting programmatic costs, the relative cost of deploying ISOCS for BGRR characterization in FY 99 and FY 00 was \$82,000. The cost for equivalent conventional baseline characterization was estimated to range from a minimum of \$292,000 to a maximum of \$1,075,000 depending on the number of analyses displaced. This represents net cost savings ranging from \$210,000 to \$993,000. If the total ASTD project cost associated with ISOCS deployment is included, the maximum cost savings range from -\$339,000 (loss) to \$443,000 (gain). While both the cost per sample and overall costs are significantly lower using ISOCS *in situ* and field laboratory characterization, perhaps the greatest cost savings associated with this innovative technology are hidden savings that are more difficult to quantify. These include savings associated with project schedule acceleration, the ability to characterize non-standard systems, and improved health and safety for D&D workers.

Finally, the most significant criterion for success of this ASTD effort is associated with continued deployment as part of the BGRR-DP and other projects at BNL as well as additional deployments at sites throughout the DOE complex. Investment in the initial ASTD project will continue to see dividends as the inherent cost savings continue to be realized. For example, based on proven success at BGRR (both technical and reduced cost), the BNL Environmental Restoration Division (BNL ERD) deployed ISOCS for cleanup of a large volume of contaminated landscape soils. ASTD team members provided both guidance and technical support for this deployment. In FY 01, BNL is engaged in another ASTD project to characterize subsurface soils beneath structures at the BGRR. Both ISOCS and BetaScint have been incorporated into the project plan for that activity, facilitated through cost savings associated with this project. In addition, ISOCS and MARSSIM are currently being deployed for characterization of the Canyon Disposition Initiative at Hanford, based partly on support from this activity. A new effort to deploy ISOCS and MARSSIM is underway at the Nevada Test Site, as part of a large effort to D&D their former nuclear rocket test facilities. Based on expertise developed through this ASTD project, BNL is providing technical support and technology transfer assistance to DOE NV.

ACKNOWLEDGMENTS

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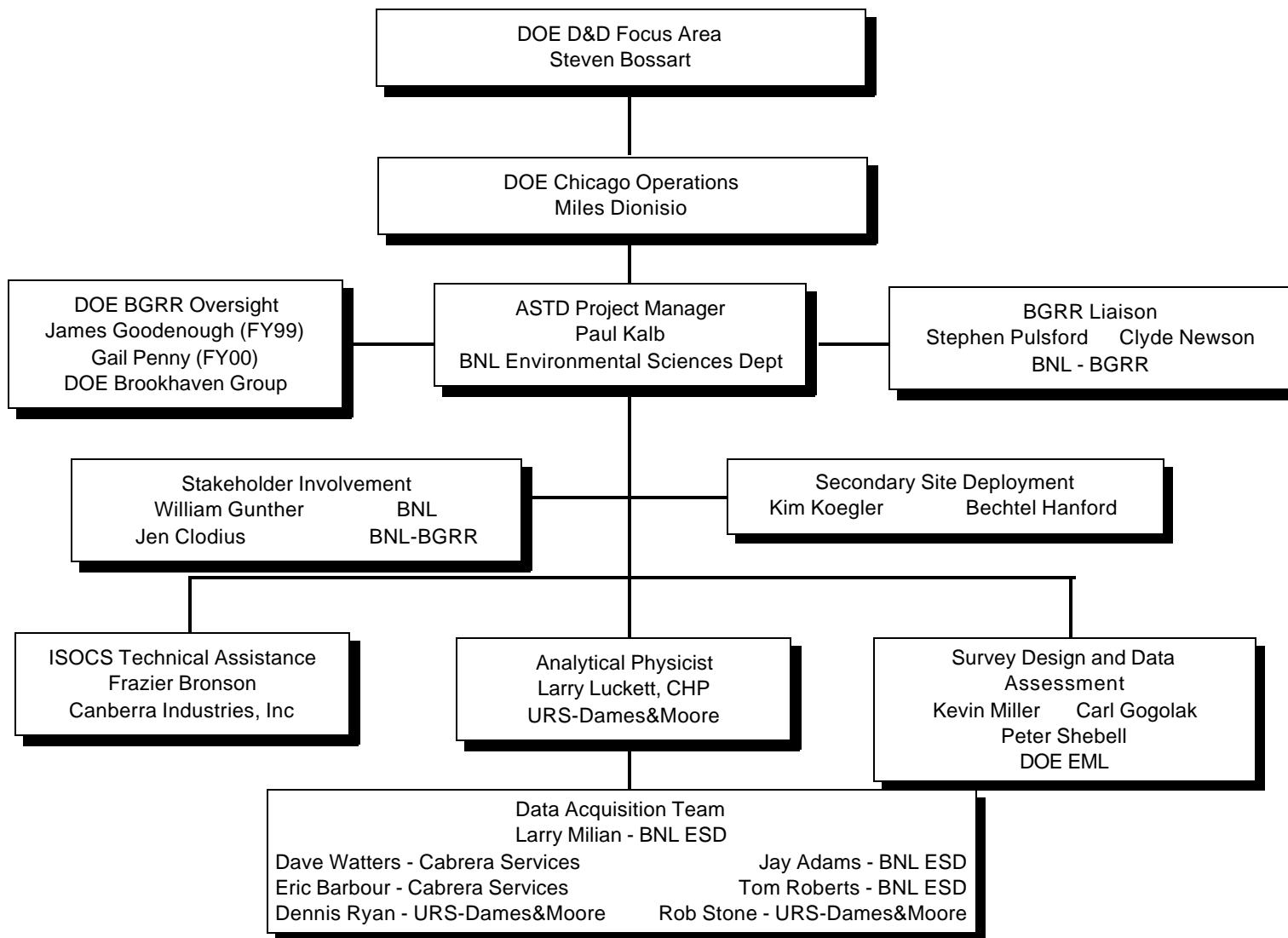
- 1 Paper presented at Waste Management Conference, March 2000, Tucson, AZ
- 2 Briefing Slides presented at Canberra Users Group Meeting,
- 3 ISOCS Comparability (ASTD Cost and Performance) Report, BNL-52607
- 4 BetaScint Comparability Report

Project Technical Documents

- 5 *SOP-DAT2* Analysis of Gamma Spectrum Files Using Canberra ISOCS System [Version 3.0 software], Feb 23, 2000
- 6 *ERD-OPM-4.3* Procedure for ASTD sample Processing to Support BGRR Decommissioning Operations, Rev 0, July 12, 2000
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Accelerated Site Technology Deployment (ASTD) Project Organization Chart



Section 1

**DEPLOYMENT OF *IN SITU* MEASUREMENT TECHNIQUES AND THE MARSSIM
PROCESS FOR CHARACTERIZATION OF THE BROOKHAVEN GRAPHITE
RESEARCH REACTOR**

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ABSTRACT

This paper describes a DOE Accelerated Site Technology Deployment project being conducted at Brookhaven National Laboratory to characterize the Brookhaven Graphite Research Reactor facility, which is currently undergoing decontamination and decommissioning. The MARSSIM process is being implemented to provide guidance for survey planning and data evaluation. Innovative *in situ* analytical techniques are being deployed to quantify the type and extent of radiological contamination including ISOCS (Canberra Industries, Inc.) for gamma emitting radionuclides and BetaScint (BetaScint, Inc.) for Sr-90. These techniques provide a number of advantages compared with conventional characterization methods including near real-time data, ability to evaluate inhomogeneous materials, fewer samples required, and lower radiation dose exposure to personnel. Data has successfully been acquired and evaluated for several BGRR facilities and components including the Pile Fan Sump (PFS), underground piping for the PFS, parking lot areas, Above Ground Ducts, and contaminated cooling fans. Cs-137 is the predominant gamma-emitting radionuclide identified, with smaller quantities of Co-60 and Am-241 detected.

INTRODUCTION

The Brookhaven Graphite Research Reactor (BGRR) is a graphite-moderated, air-cooled, thermal neutron research reactor that operated at Brookhaven National Laboratory (BNL) from 1950 through 1968. Following shutdown, fuel was removed and the facility has been maintained in a safe shutdown mode since then. Many of the major BGRR sub-components are currently scheduled for near-term decontamination and decommissioning (D&D) including the Pile Fan Sump, above and below ground air ducts, and auxiliary buildings that house fans, filters, instruments, fuel transfer canal and water treatment systems. Figure 1 is a photograph of the Above Ground Ducts and Fan House that drew air through the reactor core for cooling. The Canal House used to facilitate removal of spent fuel and equipment is shown in Figure 2. Characterization of these facilities prior to, during, and after

dismantlement is required to minimize worker exposure, plan for appropriate disposition of materials and remaining facilities, and demonstrate compliance with applicable environmental regulations.



Figure 1 Above ground ducts and Fan House at the BNL BGRR



Figure 2 BNL BGRR Canal House structure used for removal of spent fuel and contaminated equipment

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Conventional baseline characterization requires the collection of thousands of surface smear, volumetric, and core samples, sending samples for on and off-site analysis, compiling the information in a database, and reviewing the data for quality assurance. Many of the areas requiring characterization are not readily accessible and/or are highly contaminated, further complicating the process. Thus, in addition to being time consuming and costly, the baseline characterization approach can result in excessive radiation exposures to personnel. This paper describes a project sponsored by the DOE Office of Science and Technology (EM-50) under the Accelerated Site Technology Deployment (ASTD) initiative to deploy state-of-the-art techniques and equipment for improved characterization of the BGRR. The approach includes utilization of the innovative Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM) process and characterization using *in situ* measurement and analyses.

The MARSSIM approach provides guidance on planning, conducting, evaluating, and documenting environmental radiological surveys of soil and building materials to optimize the sampling process and demonstrate compliance with regulations. The final MARSSIM was published in December 1997 and has already attracted considerable interest from the D&D community. However, there is little experience in applying the MARSSIM methodology at actual sites. In those cases where it has been used, the emphasis has been on the final status survey design; its application for D&D characterization is novel. The MARSSIM process involves identifying Data Quality Objectives (DQO) to establish the types of data needed and the confidence levels required. Data validation and verification, as well as data quality assessment, are addressed through implementation of a Quality Assurance Project Plan. Valuable resources and time are saved by focusing on the proper data needed for evaluation.

Conventional gamma spectroscopy requires a major investment for purchase and eventual disposal of a variety of calibration sources that match the geometry and matrix of the expected contaminated medium. For each new geometry, a new calibration standard and hours of instrument calibration are required. This has limited *in situ* gamma spectrum analysis to simple geometries and contamination distributions. Strontium 90 (Sr-90), a fission product commonly associated with nuclear reactors is a pure beta emitter and thus is not directly detected by gamma spectroscopy. Conventional Sr-90 analysis requires chemical separation of the strontium from the sample matrix, followed by in-growth of the Yttrium 90 (Y-90) progeny for analysis, a time consuming procedure that often takes 1 - 4 weeks.

Measurement of gamma emitting radionuclides is being accomplished using a field deployable gamma spectrometer (*In Situ* Object Counting System or ISOCS) manufactured by Canberra Industries, Inc. The battery-operated system provides traditional spectra of counts as a function of gamma energy, which are then converted to radionuclide concentration by applying pre-defined geometry templates in the analysis software. Thus, complex contamination distributions (e.g., an inaccessible contaminated pipe within a wall) and resulting quantification of the contamination therein can be identified.

Detection of Sr-90 is being accomplished by means of a field deployable high energy beta scintillation detector manufactured by BetaScint, Inc. This system, can measure Sr-90 and U-238 at approximately 1 pCi/g above background with a 5-minute count time. Soil samples of 2 -3 kg are collected, analyzed using the BetaScint system and then quantified based on data from a series of known standards prepared using similar media. Measurements are conducted in a field laboratory set up in close proximity to soil removal operations while D&D activities are continuing.

APPLICATION OF THE MARSSIM FOR CHARACTERIZATION

The MARSSIM approach emphasizes the use of statistical planning and data analysis for demonstrating compliance with a final status survey. There are few examples of how to apply the DQO process for other types of surveys where such formal analyses are not necessary, or even appropriate. One of the objectives of this project is to provide a concrete example of how the MARSSIM methodology can be applied to characterization surveys and to develop a framework for the design of characterization survey plans that can be used to implement the MARSSIM at the BGRR and other DOE sites.

The DQO process is the basis for the performance-based guidance in planning MARSSIM surveys. The steps of the DQO process specified in the MARSSIM include:

1. state the problem
2. identify the decision
3. identify the inputs to the decision
4. define the boundaries (spatial and temporal)
5. develop a decision rule
6. specify limits on decision errors
7. optimize the design for collecting data

Through the implementation in this project, the first four steps of the DQO process are common to both characterization and final status surveys. In the final three steps, there is significant difference in interpretation and application to the characterization survey.

The fifth step in the DQO process is the specification of a decision rule. For the final status survey this usually takes the form of a statistical hypothesis test. For a characterization survey such a highly structured rule will not generally be appropriate. However, it should be possible to identify:

- (a) a range of results that clearly indicates that there is no need for remediation in an area,
- (b) a range of results that clearly indicates that there is need for remediation in an area, and
- (c) an intermediate range of results that may indicate the need for more data before a decision is made.

Such a scheme is loosely patterned after sequential testing procedures, but is primarily intended to differentiate the easy decisions from the more difficult ones so that more resources can be devoted to the areas that need it.

Specifying the acceptable limits on decision errors is the sixth step in the DQO process. For final status surveys, this means specifying Type I and Type II error rates for statistical hypothesis tests. Again, such precision is usually neither desirable nor necessary in a characterization survey.

In a final status survey, the decision errors are used to determine the number of samples it is necessary to collect. The same is true for the characterization survey, except that extensive use of professional judgement must be made to balance the costs of additional measurements against the risk of drawing the wrong conclusion from the data. Optimizing the design of a characterization survey (step seven of the DQO process), involves using all the information available, together with professional judgement, to assess the worth of the information to be gained from additional data in terms of increasing confidence in a remediation decision. This is where the width of the "gray region" expressed by choice (c) of step 5 is used to separate, as efficiently as possible, the easy decisions from the difficult ones. The cost of data

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collected early in the characterization can be balanced against the possibility that new data will be needed. The consequence of incorrectly classifying an area as needing remediation when it does not should be balanced against the cost of discovering during a final status survey that an area thought to be clean actually is not. Remediation costs are also balanced against the cost of characterization measurements.

The seven specific elements of the overall DQO process, outlined above, are addressed by the ASTD project team through development of individual project-specific survey plans (PSSPs) in support of individual BGRR D&D campaigns. The PSSP considers the goals of the intermediate D&D objective, the baseline characterization elements, and the targeted components of the facility, to identify the scope and content of the *in situ* characterization efforts using the DQO process. The PSSP provides details on field of view, shielding, and detection levels necessary for the *in situ* evaluations and identifies sample number designations for items and views of items for tracking and reporting purposes.

Continuity of spectrum analysis and interpretation among the PSSPs is assured by compliance with the ASTD Project *In Situ* Analysis Quality Assurance Project Plan (QAPP).¹ This QAPP provides a description of the individuals, organizational responsibilities, and control measures necessary to achieve, verify and demonstrate compliance with both federal and industry quality assurance requirements. This QAPP has been developed using the guidance in EPA QA/G-5 to ensure that appropriate requirements for project data quality have been adequately addressed.²

***IN SITU* GAMMA SPECTROSCOPY**

In situ gamma spectroscopy has been shown to be cost-effective in almost all applications where field sampling and laboratory analyses are the baseline technologies. Results can be obtained immediately following field acquisitions, thereby reducing the time delays incurred by physical sampling and laboratory analysis. *In situ* measurements can be performed on sealed systems (i.e., without breaching a containment barrier) or remotely (i.e., at a distance from an external radiation source), reducing personnel exposures and/or work hazards. Where independent lab analysis is required prior to free release of materials, *in situ* measurements serve as a screening technique, eliminating the unnecessary analysis of samples above derived concentration guidelines (DCGLs). Large areas or volumes can be assayed with a large field of view to reduce errors arising from non-homogeneity, providing a more accurate estimate of average radionuclide concentrations. These advantages make *in situ* spectroscopy an attractive tool for many characterization applications. The Canberra ISOCS system couples previously proven detector hardware with innovative calibration software to produce an integrated instrument capable of quantified analysis in the field comparable to laboratory-grade analysis.

Germanium Detector: The radiation detector utilizes a high purity germanium crystal for high resolution and high efficiency gamma radiation detection. For this application, a Canberra Broad Energy Germanium (BEGe) detector was selected because it enhances the efficiency below 1 MeV while exhibiting increased transparency to high energy gammas, such as those from naturally occurring K-40 and Tl-208 (thorium series progeny). The detector shape (50 mm diameter by 30 mm thick) is optimized for analysis of real-world objects in the detector's field of view, but it has less sensitivity to a Marinelli beaker sample geometry than a traditional cylindrical-shaped coaxial detector. The enhanced detector efficiency for low energy gammas (from 30-100 keV) provides a field capability for detection of Am-241 and low energy gammas associated with actinide alpha-emitters that greatly exceeds the capability of traditional detectors.

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Modular shields and cart: Useful mechanical components of the ISOCS system include a field deployable mobile cart and a modular system of stainless-steel covered lead shields. Annular side shields of either 19 mm or 44 mm lead thickness effectively reduce the detection of interfering radiation from items in the vicinity of the detector and background radiation, resulting in improved system sensitivity. The field of view can be further restricted, from 180E to 90E or 30E, by installing lead collimators on the cart's mounting rails, so that items adjacent to the object of interest can be significantly reduced or eliminated from the analysis. In addition, a completely shielded sample chamber can be assembled by combining the components of the two thickness annular shield systems to enable timely, low-background analysis of samples in the field.

Analytical Software: The ISOCS efficiency calibration software provides the user with the ability to quantify nuclide activity easily and reliably. This software employs a mathematical calibration technique that includes detector-specific characteristics, accounts for collimators and/or shields, and models the physical object to be assayed. It uses a combination of Monte Carlo calculations and discrete ordinate attenuation computations to derive efficiency curves (fraction of gammas emitted from the object that interact in the detector for an energy interval) for each specific *in situ* analysis. Objects are modeled from one of a set of generic sample shapes, such as boxes, cylinders, planes, spheres, pipes, etc. These basic geometry templates have many parameters that can be modified to create an accurate representation of the sample object and detector geometry. Efficiencies can be generated in a few minutes in the field and can be modified easily if needed.

Technology Application: The versatility of the ISOCS system has been demonstrated in numerous situations during initial characterization and decommissioning efforts at the BGRR. Surface soil detection sensitivities of less than 1 pCi/g have been attained with count times as short as 10 minutes for common gamma emitters such as Cs-137. Final results have been reported the same day, following data review and validation. Lower activities or more difficult to measure objects, such as enclosed systems, buried sources, and low-level surface contamination, can take much longer to measure and evaluate. However, large surface areas or volumes with heterogeneous material distributions can be assayed with a single *in situ* measurement, thus saving time over other, more manual, methods, such as sampling and remote laboratory analysis.

A typical ISOCS application can be illustrated by reviewing the characterization of core-cooling exhaust fans, prior to their removal, volume reduction, and shipment from the site. Each fan is a massive squirrel-cage type blower, nominally 8 ft x 10 ft x 12 ft, and 14,000 lbs. The fans became internally contaminated, likely as a result of fuel element failure, but the identity, extent, and quantity of radioactive material in the fan internals were unknown. External surveys revealed non-uniform internal deposition with highest readings in the vicinity of the fan volutes, where entrained dust particles would have had a higher probability of settling out due to eddies and dead spaces in air flow currents. Three of the five fans had been upgraded/replaced during the operating life of the reactor. Thus, physical configurations, dimensions, and radionuclide quantities were different from those in the other two fans.

The ISOCS was mobilized to the Fan House containing the five contaminated fans and *in situ* gamma spectra were acquired from Fan #5 (representative of Fans #5 and 4) and Fan #3 (representative of Fans #3, 2, and 1). Figure 3 is a photograph of the ISOCS deployed at the BGRR Fan House. Each fan housing was scanned using 44 mm annular shields and 90^E field of view collimators to reduce interference as much as possible from adjacent contaminated structures. Because of the equipment layout, there was no position where gamma spectra could be acquired without structural components

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(concrete supports and carbon steel struts) shielding a portion of the field of view. The ISOCS cart was positioned so that the detector was oriented diagonally downwards at the fan housing volute bottom, where surveys indicated an accumulation of radioactivity. Spectra were accumulated for 15 minutes each from two symmetric positions: NE of the housing facing SW and NW of the housing facing SE. Equipment setup, spectrum acquisition and equipment-breakdown required less than two hours, with only minimal health and safety oversight and no breaching of contamination containment barriers. Radioactivity in the fan housing was modeled as a layer of surface dust, uniformly covering the interior of the carbon steel fan volute (horizontal or diagonal rectangular plane). Due to the complex geometry with intervening structural members, several alternative geometry models were defined. The intervening structural members were adjusted in the models until the results from the symmetric scans were similar. The modeling and analysis of both fans required about six hours.

The results of the analysis are provided in Table 1. The range in the value is representative of the uncertainty in the analysis, and is primarily associated with assumptions on unobserved inner structures of the fan. The table demonstrates that even when using a detector with enhanced low-energy response, the detection level can still be high when the source is shielded by a highly attenuating medium such as this example, inside a carbon steel fan housing. With the use of the ISOCS modeling software, a quantified estimate of the activity in the fan was provided in approximately eight hours, without fabricating a physical radioisotope calibration standard, without breaching contaminated barriers, and without handling and transporting contaminated samples.

Table I Results of *In Situ* Measurements at BGRR Fan House

Fan Unit	Measured Activity, FCi			
	Cs-137 low estimate	Cs-137 high estimate	Co-60	Am-241
Fan No 5	75 ± 3	600 ± 20	# 0.1	# 180
Fan No 3	114 ± 10	330 ± 30	4.8 ± 0.5	# 800

Note: The uncertainties in the table represent ±2s counting error; values expressed as “#” represent the minimum detectable activity and indicate that the radionuclide was not detected.



Figure 3 Using *in situ* ISOCS system to evaluate internal contamination at the BNL BGRR Fan House #5

Another convenient use of the ISOCS technology is the rapid quantification of soil activity in the field. The use of the modular shields to construct a low-background counting chamber on the mobile cart/stand provides a laboratory quality, quantified analysis that is available in almost real time at the point of sampling. The typical detection levels attained by a five-minute count of a soil sample in a one liter high density polyethylene bottle are illustrated in Table 2.

Table II Typical Minimum Detectable Activity for 5-min Assay of 1L Soil Samples

Radionuclide	Co-60	Cs-137	Eu-152	Th-232 ^(a)	Ra-226 ^(b)	U-235	Am-241
MDA (pCi/g)	0.26	0.24	0.32	0.35	0.43	0.20	0.24

^(a) Th-232 activity inferred from Pb-212 assuming secular equilibrium.

^(b) Ra-226 activity inferred from Bi-214 assuming secular equilibrium.

Observations: Data has successfully been acquired and evaluated for several BGRR facilities and components including the Pile Fan Sump (PFS), underground piping for the PFS, parking lot areas, Above Ground Ducts, and contaminated cooling fans. Through deployment of ISOCS to date, the ability to provide individual isotopic identification and quantitative assays in the field quickly and reliably has highlighted several advantages of the technology:

- C reduction of the cost and time delay associated with sampling and laboratory analysis
- C reduction of the potential hazards of contaminated system entry and sampling/measurement
- C the ability to derive efficiencies for objects without purchasing or fabricating radioactive standards
- C enhanced detection sensitivity by using the annular shields and collimators

One design limitation has been identified that affects the efficient use of the BEGe system in the characterization phase of a D&D project. The 30E collimator shields over 90% of the sensitive area of the detector, thus negating any increased sensitivity of the detector. Having to rely on only the 90E collimator results in some assays being more complex and less specific, since adjacent sources cannot be readily screened from the primary object spectrum.

FIELD DEPLOYABLE SR-90 ANALYSIS

Technology Description: The BetaScint system consists of a multi-layer beta scintillation detector array with a beta radiation entrance window measuring 30-cm by 60-cm. Scintillating fibers are fashioned into ribbons, which are stacked vertically. Soil samples are prepared, transferred to large area counting trays, and positioned beneath the detector window for analysis. Beta particles that pass through the detector window excite electrons in the scintillating ribbons resulting in the emission of light pulses, which are counted by photomultiplier tubes. Coincident circuitry to detect simultaneous events in several ribbon layers distinguishes high energy betas (Sr-90) from lower energy contaminants and background.

Initially, the strontium-90 detection efficiency of the BetaScint system is established by measuring its response to site-specific calibration standards. These calibration standards are prepared by spiking clean (non-contaminated) site soils with known quantities of strontium-90. The net system response is directly proportional to strontium-90 activity concentration because it is almost entirely due to strontium-90 beta interactions.

Routine daily operations of the BetaScint system include the performance of daily quality control checks and background measurements. Quality control checks consist of analysis of a calibration standard of known activity. The results of the quality control checks are compared against established acceptance criteria to determine whether the instrument is functioning properly. Background checks are performed by counting with no samples in place (i.e., bare detector.) These measurements, which are performed daily at a minimum, are subtracted from gross sample counts to establish net detector response. For calibration and operation, a 2 - 3 kg sample is typically dried, sieved to remove organic matter and rocks over 6 mm in size, and spread evenly over a large area counting tray. The sample tray is then positioned beneath the window of the detector and counted for five minutes. Following the analysis, the system reports the number of coincident events and the counts are converted to Sr-90 activity concentration in the soil, using the detection efficiency correlation established with the spiked site soils.

Implementation at BNL: The BetaScint system was deployed at Brookhaven for a two week demonstration during the period of December 6 - 17, 1999. Soil samples from the BGRR site and other environmental restoration areas at BNL were collected and analyzed using the system. Figure 4 is a photo of the BetaScint equipment system set up at BNL.

Four calibration standards were prepared from a NIST traceable Sr-90 solution and uncontaminated site soils. Following preparation, the calibration standards were analyzed by the BetaScint system and a Sr-90 detection efficiency correlation was established. It should be noted that the observed efficiency, 0.67 counts/second per pCi/gram, was virtually identical to the efficiency established by BetaScint at other sites.³

During the two-week demonstration, a total of 145 evaluations were performed on 35 samples. The analytical count time for these analyses was 5-minutes. Based on the BetaScint results, Sr-90 activity

concentrations in these samples ranged from non-detectable to approximately 70 pCi/gram. The 5-minute count time yielded a minimum detectable activity concentration of approximately 1.2 pCi/gram at the 95% confidence level, which is considerably less than the BGRR DGCL for Sr-90 of 15 pCi/gram.

The calibration standards were re-analyzed by the BetaScint following system calibration in order to evaluate the precision of the BetaScint analyses,. The results of these analyses (Table 3) are within $\pm 8\%$ of the calibration standard activity concentrations, indicating that the system exhibits acceptable levels of reproducible detection. To assess instrument accuracy, 7 soil samples and aliquots from the 4 calibration standards were sent off-site for conventional baseline Sr-90 analysis to facilitate comparison; the results of these analyses are still pending.

Table III Results of Calibration Standard Measurements

Spiked Sr-90 Concentration (pCi/g)	Measured Sr-90 Concentration ^(a) (pCi/g)	% Difference
7.5	7.9 \pm 1	5
15	16 \pm 2	7
22.5	22.4 \pm 2	1
610	562 \pm 40	8

^(a) Errors reported at the 95% confidence level

Daily, or more frequent, quality control checks were performed (a total of 25) by analyzing the 22.5 pCi/gram calibration standard that had been prepared. The average result of these analyses was 22.2 pCi/gram (see Figure 5.). All results were within $\pm 2 s$ of the average value, with the exception of one result that was slightly less than $2 s$, indicating that the system response is stable over time and exhibits acceptable levels of precision. Duplicate analyses were also performed on most samples to evaluate the precision of the system. In general, the results of duplicate analyses were within acceptable statistical bounds.



Figure 4 Loading soil sample for Sr-90 BetaScint analysis at BNL

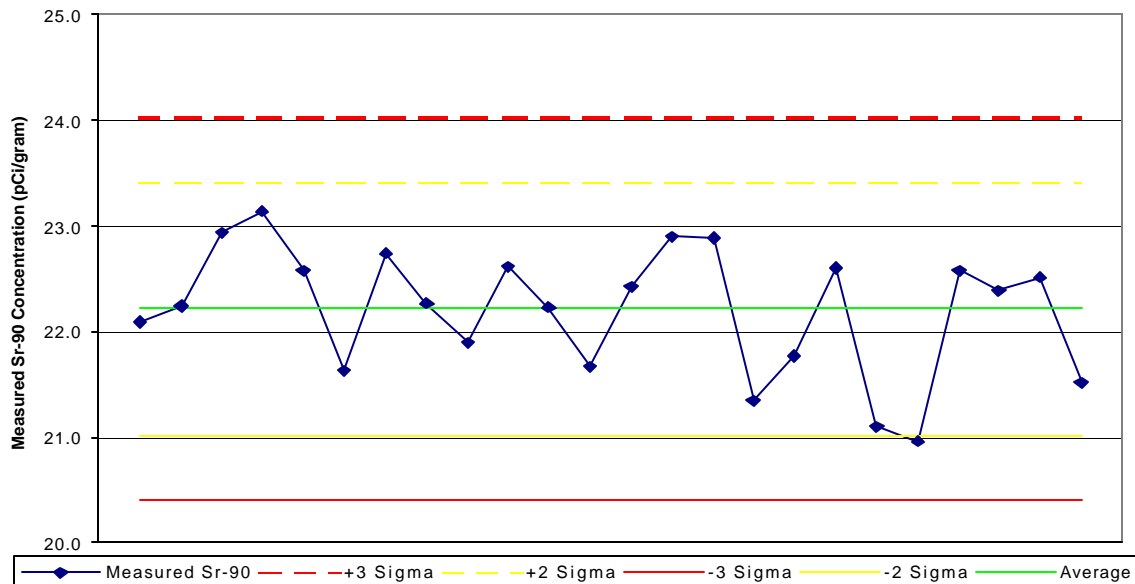


Figure 5 Results of quality control measurements for BetaScint in response to 22.5 pCi/g calibration source

SUMMARY AND CONCLUSIONS

The MARSSIM guidance, with its emphasis on final status survey design, has been applied in the characterization phase through a modification of the final steps of the DQO process. *In situ* data have successfully been acquired and evaluated for several BGRR D&D objectives, and the ability to provide individual isotopic identification and quantitative assays in the field quickly and reliably has highlighted several advantages of the technology. In particular, the Canberra ISOCS system can be used to effectively measure gamma emitting contamination in areas difficult to assay (e.g., pipes and equipment or areas with high radiation levels) or as a field deployable gamma spectroscopy laboratory for volumetric samples. BetaScint provides a field deployable system for near real-time (approx 20 min) evaluation of Sr-90 with good detection limits (1 pCi/g concentrations). Together, the use of MARSSIM with *in situ* characterization techniques is enabling accelerated, accurate, and cost-effective evaluation of equipment, structures, and materials at the BNL BGRR.

ACKNOWLEDGMENTS

The authors gratefully acknowledge the U.S. Department of Energy Office of Science and Technology Decontamination and Decommissioning Focus Area and the Brookhaven BGRR Decommissioning Project for their support of this effort.

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Section 2



Deployment of *InSitu* Measurement Techniques and Implementation of the MARSSIM Process at the BGRR

P.D. Kalb

Collaboration between BNL, DOE EML, Bechtel Hanford

***Presented at the Canberra Users' Group Meeting
Walnut Creek, CA
June 5-9, 2000***





Overview



- **Background**
- **Technology Need**
- **Project Description**
 - ✓ **Goals**
 - ✓ **Technical Approach**
 - ✓ **Technology Description**
- **Technical Progress**
- **Summary and Conclusions**





Background



6 11

THE NEW YORK TIMES, SUNDAY, FEBRUARY 12, 2009

Old B.N.L. Reactor Is Being Dismantled

Continued From Preceding Page

graphite core, picking up radioactive fission products like plutonium-239, which remains dangerous for thousands of years, and depositing them along the ducts, fans and filters of the reactor before blowing out the exhaust tower. The high flux beam reactor was cooled with water in a closed loop system, which meant that the water could be treated and returned to the reactor.

The graphite reactor also had 20 uranium fuel rods, contaminating the cooling air, and dozens of used fuel rod cassettes, which filled the spent fuel casks under the reactor with fission products, some of which turned up in liquid waste discharged from the reactor. Past processing of waste from the graphite reactor is believed to be responsible for the plutonium discovered in the Pacific River in 1998.

Brookhaven workers also discovered radioactive strontium-90 and cesium-137 in the soil beneath a sump that collected rainwater from a 325-foot-high exhaust stack that towers over the graphite reactor site.



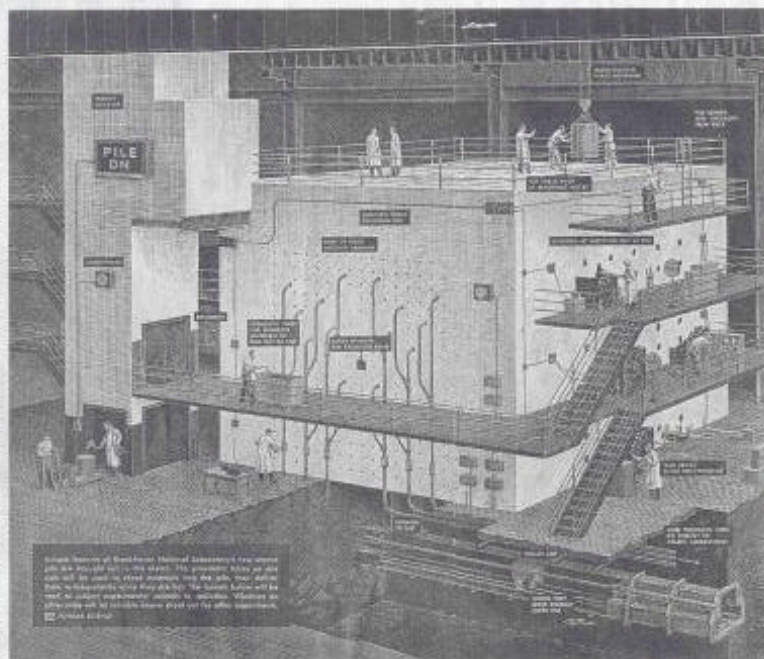
The level of strontium-90 in the groundwater underneath the pile fuel sump was measured at 393 picocuries, well above the allowable 5 picocuries for drinking water. And in a sump underneath the 32-foot-high graphite reactor core, workers discovered 40,000 gallons of water containing uranium, plutonium and other radioactive material.

Department of Energy officials say that the reactor core building was decontaminated and sealed before people were ever allowed inside the museum and that the toxic water had collected deep underground, where it never posed a safety hazard to visitors or workers, but they acknowledge lapses.

"We're not proud of our performance relative to B.G.R.R.," said Frank Crescoston, deputy manager of the Brookhaven group of the Department of Energy. "It really was that kind of thing where it was stabilized long ago, there wasn't an adequate program to monitor and maintain it. When water collected down there, we didn't have a monitoring program to find it."

After the water contamination was discovered in the reactor's sumps, the Department of Energy decided in 1988 to decontaminate and dismantle the graphite reactor. The estimated cost of the decommissioning project starts at \$20.5 million and could increase to as much as \$75 million, depending upon how much of the reactor is removed.

"The cost really depends on the end-state of the machine, which

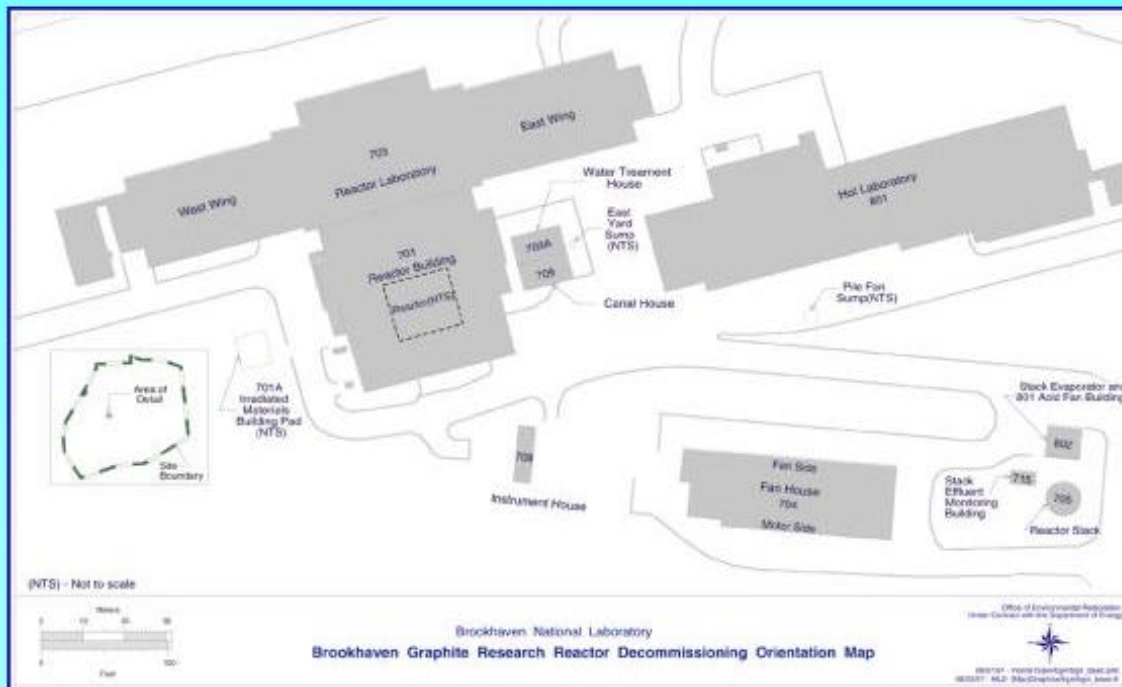


There's a broad range of contaminants - a Brookhaven physicist working on - tv holes. On the west face, blocks - for the scrap





Background

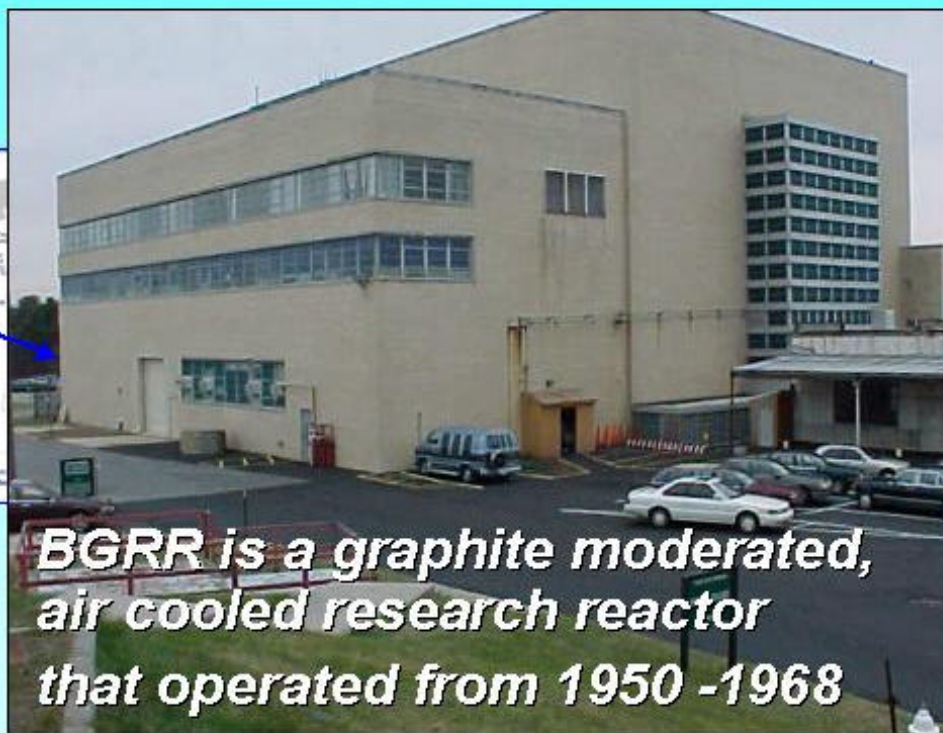


Plan View of BGRR





Background



***BGRR is a graphite moderated,
air cooled research reactor
that operated from 1950 -1968***

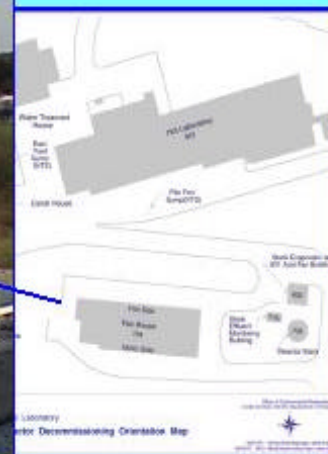




Background



Many BGRR sub-components are currently scheduled for D&D including Above and Below Ground Air Ducts, Fan House,





Background

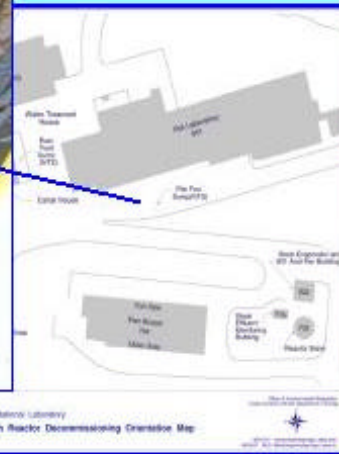




Background



Pile Fan Sump (contaminated liquids collected from floor drains and stack)





Technology Need



- ***D&D characterization is required to:***
 - ✓ *minimize worker exposure*
 - ✓ *plan for appropriate disposition of materials and remaining facilities*
 - ✓ *demonstrate compliance with applicable environmental regulations*
- ***Characterization applied in three stages:***
 - ✓ *Pre D&D scoping*
 - ✓ *During D&D operations*
 - ✓ *Post D&D final status survey*





Baseline Characterization



- **Baseline approach is time consuming and costly:**
 - ✓ collect thousands of surface smear, volumetric, and core samples
 - ✓ ship samples for analysis
 - ✓ wait for results (14 -28 day turn around typical)
- **Many areas difficult to access or highly contaminated**
- **Difficult to measure heterogeneous contamination**





Innovative Characterization



- ***Multi Agency Radiation Survey & Site Investigation Manual (MARSSIM) process:***
 - ✓ *optimize survey design*
 - ✓ *reduce unnecessary sampling*
 - ✓ *save time and money*
- ***In situ techniques:***
 - ✓ *monitor remotely, reducing personnel exposures*
 - ✓ *improved characterization of heterogeneous distribution of contamination*
 - ✓ *faster turn-around time*
 - ✓ *lower analytical costs*





Project Description, Goals and Technical Approach



- *Sponsored by the DOE Accelerated Site Technology Deployment (ASTD) initiative*
- *Collaboration between BNL, EML, Bechtel Hanford and contractors*
- *Direct characterization support of BGRR Decommissioning Project removal actions resulting in:*
 - ✓ *improved D&D planning*
 - ✓ *reduced exposures*
 - ✓ *acceleration of schedule*
 - ✓ *lower cost*





Project Description, Goals and Technical Approach



- *Implement MARSSIM process for D&D characterization as well as final status survey*
- *Deploy innovative in situ characterization techniques:*
 - ✓ *Canberra In situ Object Counting System (ISOCS) for gamma emitting nuclides*
 - ✓ *BetaScint fiber optic sensor for Sr-90*





Multi Agency Radiation Survey and Site Investigation Manual (MARSSIM)



- *MARSSIM process developed for final status surveys - Application for D&D characterization is unique*
- *MARSSIM emphasizes statistical planning and analysis to demonstrate compliance with cleanup guidelines*
- *Incorporates:*
 - ✓ *Data Quality Objective (DQO) process*
 - ✓ *Quality Assurance Project Plan (QAPP)*
 - ✓ *Data Assessment*





MARSSIM Technical Progress

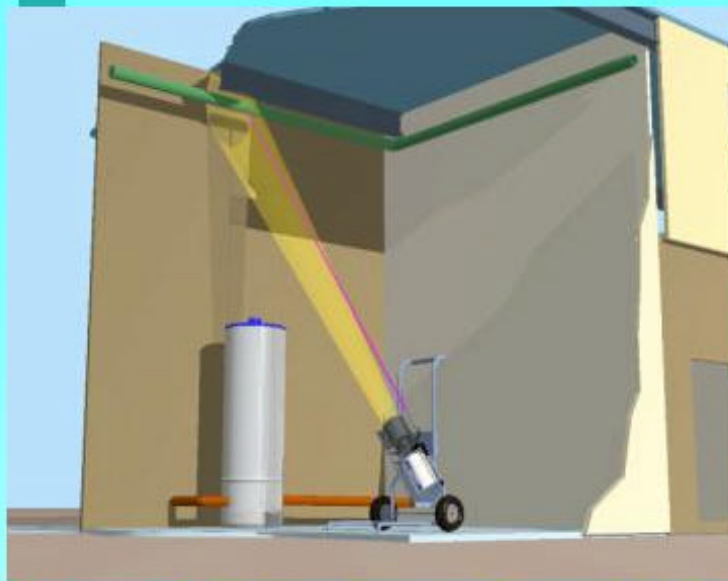


- *QAPP prepared using EPA QA/G-5 guidance and approved by project team*
- *Standard Operating Procedures (SOP) prepared for general measurement activities*
- *Project Specific Survey Plans (PSSP) prepared for specific measurement activities*
 - ✓ *DQO process to identify scope and content of characterization*
 - ✓ *identifies field of view, shielding, and detection levels necessary*
 - ✓ *specific sampling protocol*



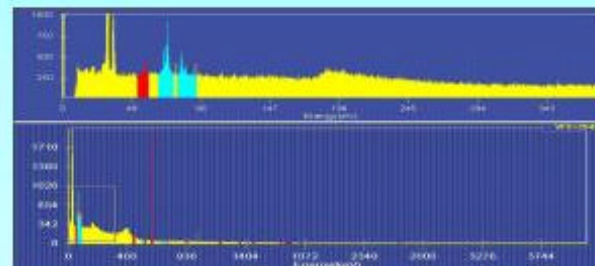


Canberra *In Situ* Object Counting System (ISOCS)



Canberra *In Situ* Object Counting System (ISOCS) measuring contamination within a pipe

- *Field deployable gamma spectroscopy*
- *Broad energy range (30 keV - 3 MeV)*
- *Monte Carlo modeling in place of conventional source calibrations*
- *Ability to model complex geometries*





ISOCS Technical Progress



ISOCS Characterization of BGRR Above Ground Ducts





ISOCS Technical Progress



Characterizing the BGRR Pile Fan Sump





ISOCS Technical Progress



Characterizing BGRR Fan House 5





ISOCS Technical Progress



Characterization of Pile Fan Sump Soils

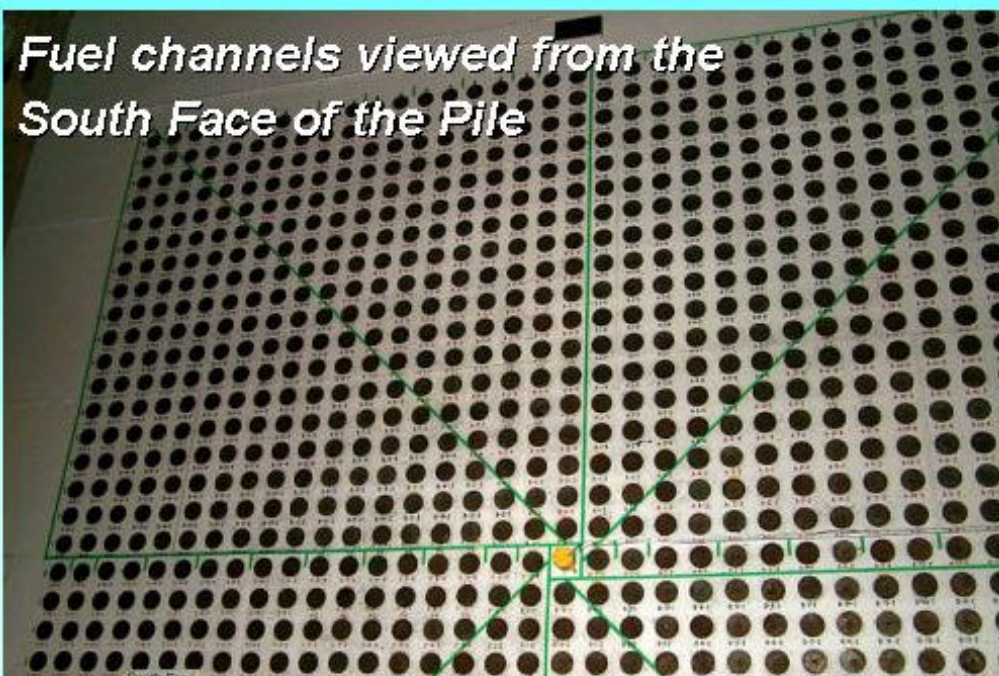
- ~ 600 samples evaluated in 4 weeks
- Samples analyzed immediately upon collection
- Estimated savings of >\$150K
- Estimated schedule acceleration of 2 months

- ISOCS deployed as a field lab to analyze PFS soils during excavation
- Provides field engineers real time, critical info on:
 - ✓ soil disposition
 - ✓ how deep to dig
- Accelerates schedule
- Improves QA
- Saves money





Characterization of the Pile





Characterization of the Pile



Far view of the Reactor Pile West Face





Characterization of the Pile



Close-up view of West Face Experimental Ports

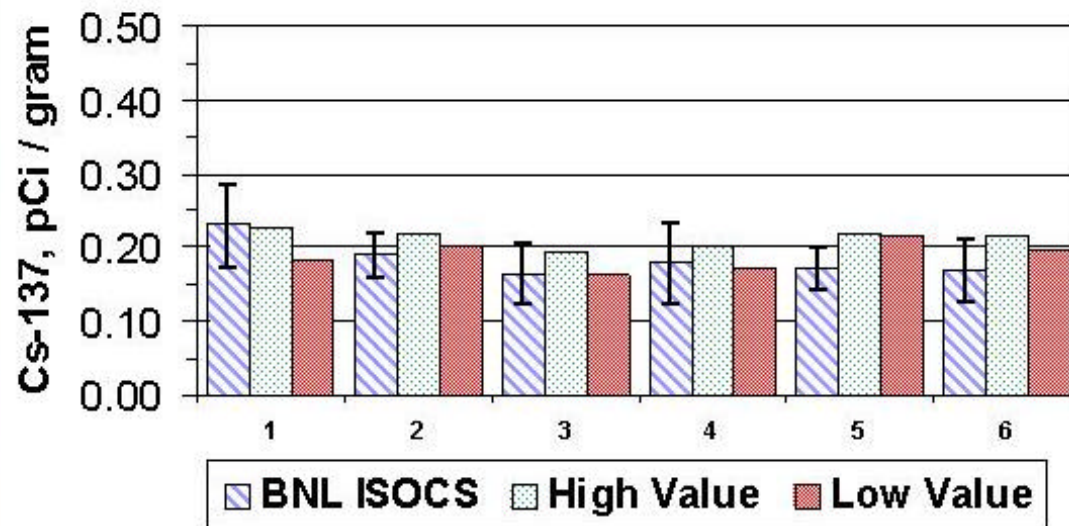




In Situ ISOCS vs. EML Intercomparison Data

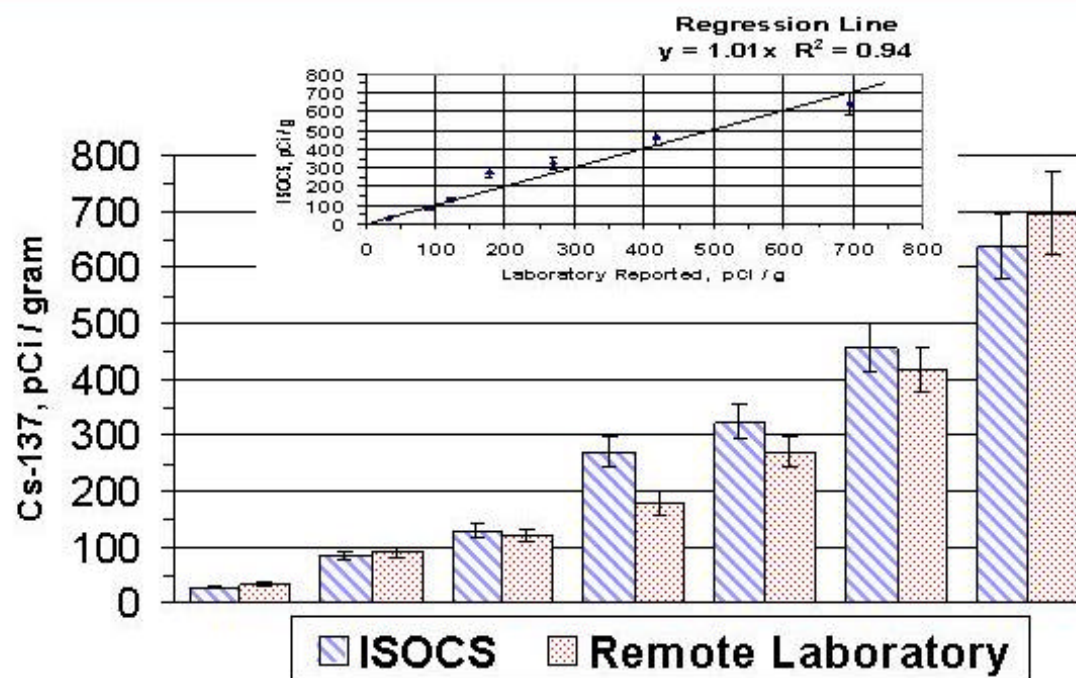


Cs-137 in Uncontaminated BNL Soil





Ex Situ ISOCS vs. Lab Analyses Pile Fan Sump Soils

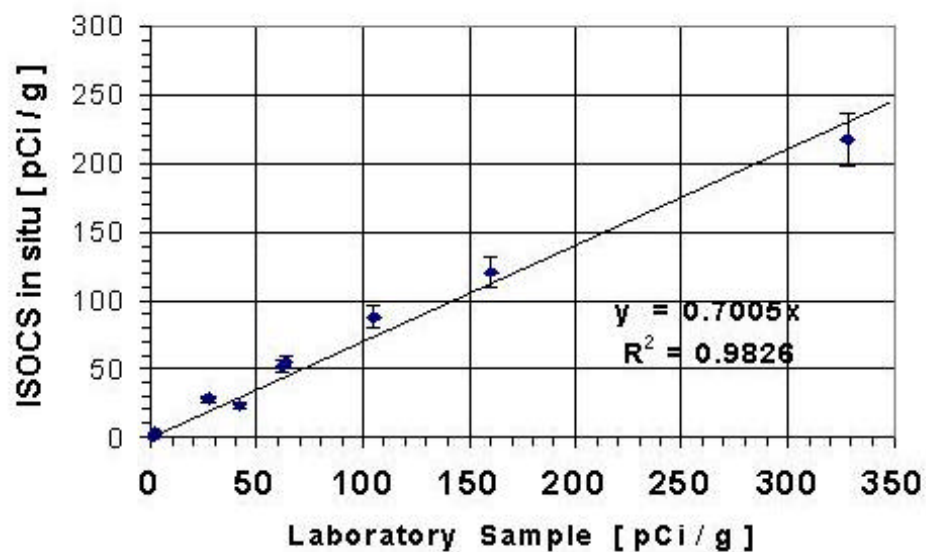




In Situ ISOCS vs. Lab Analyses: Contaminated Landscape Soil



Lab Samples Taken at 0 - 6" depth

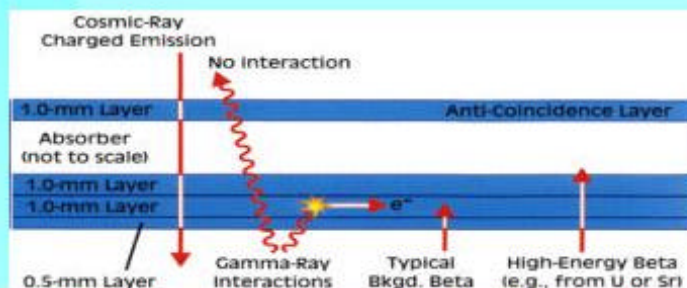




BetaScint Fiber Optic Sensor



- *Field deployable, near real-time analysis for Sr-90 and U-238 @ \$35 - \$60/sample*
- *Conventional techniques require 1 - 4 wks @ \$200 - \$300/sample*
- *1 pCi/g detection limit*



Schematic cross-section of Sr-90 sensor





BetaScint Technical Progress

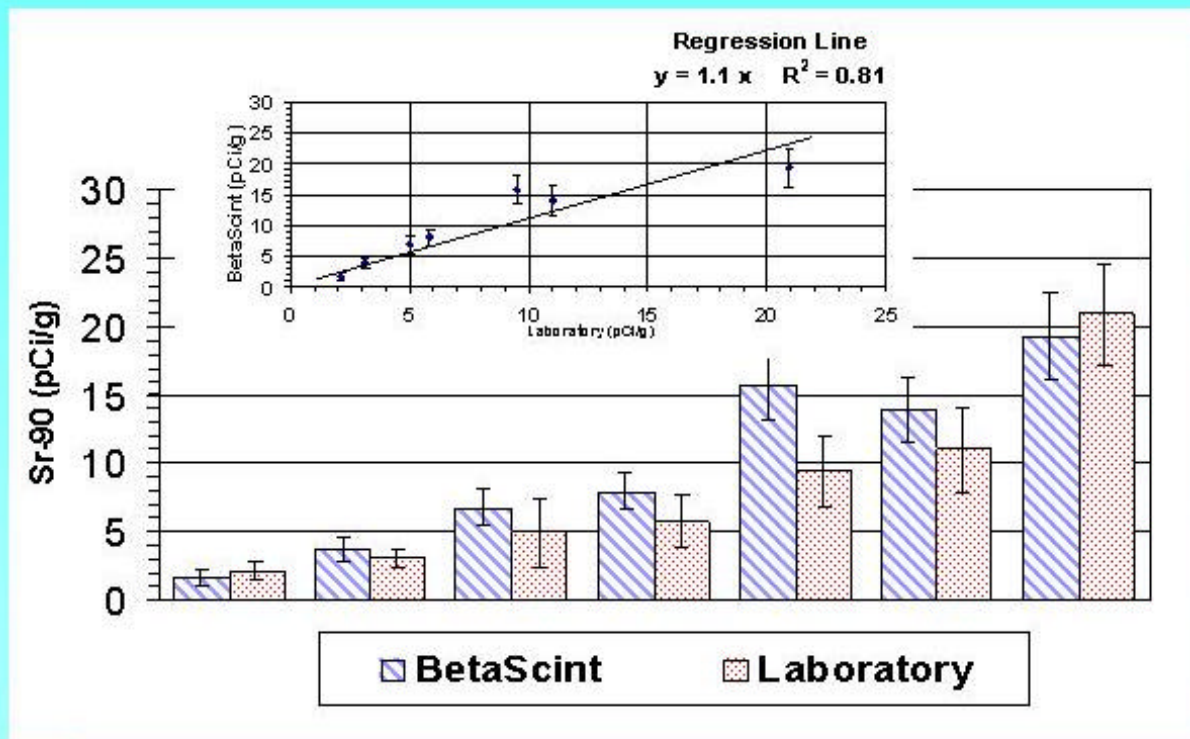


BetaScint Field Lab deployed at BGRR





BetaScint vs. Baseline Analyses





Future Plans



- *Complete in situ characterization of facilities (e.g., Instrument House, Filter, Canal House, Pile)*
- *Complete in situ characterization of soils and waste as generated*
- *Issue Final Report including technical and cost comparisons of in situ and conventional techniques*
- *BGRR Decommissioning Project to fund continued deployment in FY 01 and beyond*
- *Assist Hanford for secondary deployment at Canyon Disposition Initiative and other interested sites (e.g., Nevada Test Site D&D)*





Summary and Conclusions



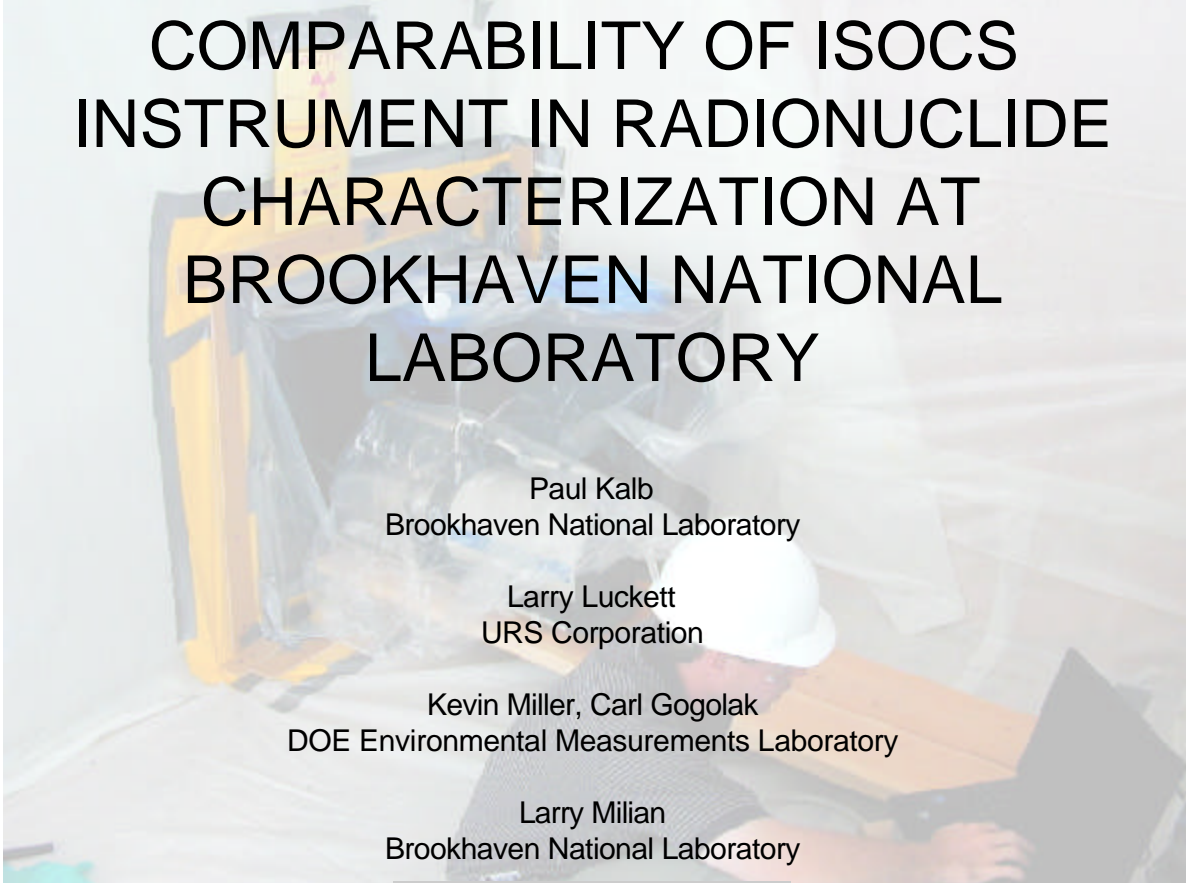
- *MARSSIM approach can be effectively applied for D&D characterization*
- *ISOCS is a versatile and accurate characterization tool for both in situ and field lab applications*
- *BetaScint provides faster, better, cheaper results for measurement of Sr-90 in soils*
- *Through implementation of MARSSIM and deployment of in situ techniques, this ASTD project is resulting in:*
 - ✓ *lower characterization costs*
 - ✓ *accelerated project schedule*
 - ✓ *improved data quality*
 - ✓ *reduced personnel exposure*



Section 3

Accelerated Site Technology Deployment

Cost and Performance Report



COMPARABILITY OF ISOCS INSTRUMENT IN RADIONUCLIDE CHARACTERIZATION AT BROOKHAVEN NATIONAL LABORATORY

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Under Contract No. DE-AC02-98CH10886 with the
UNITED STATES DEPARTMENT OF ENERGY

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COMPARABILITY OF ISOCS INSTRUMENT IN RADIONUCLIDE CHARACTERIZATION AT BROOKHAVEN NATIONAL LABORATORY*

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ABSTRACT

This report describes a DOE Accelerated Site Technology Deployment project being conducted at Brookhaven National Laboratory to deploy innovative, radiological, *in situ* analytical techniques. The technologies are being deployed in support of efforts to characterize the Brookhaven Graphite Research Reactor (BGRR) facility, which is currently undergoing decontamination and decommissioning.

This report focuses on the deployment of the Canberra Industries *In Situ* Object Counting System (ISOCS) and assesses its data comparability to baseline methods of sampling and laboratory analysis. The battery-operated, field deployable gamma spectrometer provides traditional spectra of counts as a function of gamma energy. The spectra are then converted to radionuclide concentration by applying innovative efficiency calculations using monte carlo statistical methods and pre-defined geometry templates in the analysis software. Measurement of gamma emitting radionuclides has been accomplished during characterization of several BGRR components including the Pile Fan Sump, Above Ground Ducts, contaminated cooling fans, and graphite pile internals. Cs-137 is the predominant gamma-emitting radionuclide identified, with smaller quantities of Co-60 and Am-241 detected.

The Project used the *Multi-Agency Radiation Survey and Site Investigation Manual* guidance and the Data Quality Objectives process to provide direction for survey planning and data quality assessment. Analytical results have been used to calculate data quality indicators (DQI) for the ISOCS measurements. Among the DQIs assessed in the report are sensitivity, accuracy, precision, bias, and minimum detectable concentration. The assessment of the *in situ* data quality using the DQIs demonstrates that the ISOCS data quality can be comparable to definitive level laboratory analysis when the field instrument is supported by an appropriate Quality Assurance Project Plan. A discussion of the results obtained by ISOCS analysis of objects that could not be analyzed readily by conventional methods demonstrates a powerful application of the instrument. In conclusion, a comparison of costs associated with the analysis on the ISOCS instrument to the costs of conventional sampling and laboratory analysis is presented.

*This work was performed under the auspices of the U.S. Department of Energy.

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ACRONYMS AND ABBREVIATIONS

AIF	Atomic Industrial Forum	keV	kilo electronVolts (energy)
ALARA	As Low As Reasonably Achievable	MARSSIM	Multi-Agency Radiation Survey and Site Investigation Manual
ANSI	American National Standards Institute	MCA	Multi-channel Analyzer
ASTD	Accelerated Site Technology Deployment	MDA	minimum detectable activity
BEGe	Broad energy germanium [detector]	MDC	minimum detectable concentration
BGRR	Brookhaven Graphite Research Reactor	MCNP	Monte Carlo Nuclear particle
BNL	Brookhaven National Laboratory	NIST	US National Institute of Standards and Technology
CERCLA	Comprehensive Environmental Response, Compensation and Liability Act	NRC	US Nuclear Regulatory Commission
D&D	Decontamination and Decommissioning	PARCC	QA parameters of precision, accuracy, representativeness, completeness and comparability
DOE	US Department of Energy	PBMS	performance-based measurement system
DQA	Data Quality Assessment	PLF	Productivity Loss Factor
DQI	Data Quality Indicator	PSSP	Project Specific Survey Plan
DQO	Data Quality Objective	QA	Quality Assurance
EML	US DOE Environmental Measurements Laboratory	QC	Quality Control
EPA	US Environmental Protection Agency	QAPP	Quality Assurance Project Plan
<i>ex situ</i>	out of position, a sample removed and relocated for analysis	RPD	Relative percent difference
FOV	Field of View	RCRA	Resource Conservation and Recovery Act
HPGe	hyper-pure gemanium [detector]	SOP	Standard operating procedure
<i>in situ</i>	in position, usually without disturbing or sampling the media	WAC	Waste Acceptance Criteria
ISOCS	In Situ Object Counting System		

COMPARABILITY OF ISOCS INSTRUMENT IN RADIONUCLIDE CHARACTERIZATION AT BROOKHAVEN NATIONAL LABORATORY

1.0 OVERVIEW

1.1 Introduction

This report describes a project sponsored by the DOE Office of Science and Technology (EM-50) under the Accelerated Site Technology Deployment (ASTD) initiative to deploy state-of-the-art techniques and equipment for improved characterization of nuclear facilities during characterization, decommissioning, and surveys for final status certification. Measurement of gamma emitting radionuclides is being accomplished using a field deployable gamma spectrometer (*In Situ* Object Counting System or ISOCS) manufactured by Canberra Industries, Inc. This report assesses the operational capabilities of the ISOCS instrument and the comparability of the field instrument results to results generated through the laboratory analysis of physical samples.

1.2 Description of ASTD Project

This Accelerated Site Technology Deployment (ASTD) project addresses some of the important issues of radioactive material characterization through deployment of an innovative *in situ* characterization technology. This project focuses on the characterization of the Brookhaven Graphite Research Reactor (BGRR), which is currently undergoing stabilization and near-term D&D. While the Environmental Management program of the U.S. Department of Energy (DOE) includes the D&D of many different types of nuclear facilities, the basic issues of characterization are universal and not dependent on site-specific dissimilarities.

The Brookhaven Graphite Research Reactor (BGRR) is a graphite-moderated, air-cooled, thermal neutron research reactor that operated at Brookhaven National Laboratory (BNL) from 1950 through 1968. Following shutdown, fuel was removed and the facility has been maintained in a safe shutdown mode since then. Many of the major BGRR sub-components are currently scheduled for near-term decontamination and decommissioning (D&D) including the Pile Fan Sump, above and below ground air ducts, and auxiliary buildings that house fans, filters, instruments, fuel transfer canal and water treatment systems. Characterization of these facilities prior to,

during, and after dismantlement is required to minimize worker exposure, plan for appropriate disposition of materials and remaining facilities, and demonstrate compliance with applicable environmental regulations. Due to the 30 years interval since shutdown, short-lived radionuclides have undergone considerable decay. Cs-137 is the predominant gamma-emitting radionuclide identified, with smaller quantities of Co-60, Eu-152, Eu-154, and Am-241 detected.

The project execution involved collaboration between BNL, the U.S. DOE Environmental Measurements Laboratory (EML), URS Corporation of Ronkonkoma, NY, Canberra Industries of Meriden, CT, and Cabrera Services, Inc of East Hartford, CT. Also participating in the plan was Bechtel Hanford Inc., which plans to deploy this approach for characterization of nuclear facilities at Hanford. Successful demonstration of comparability of the *in situ* technology provides an additional capability in nuclear characterization for DOE D&D undertakings. This deployment project also provides valuable experience and "lessons learned" that can be shared with facilities throughout the USDOE complex.

1.3 Report Format

This report describes the Canberra ISOCS instrument and the mathematical characterization (calibration) of the detector. Then a description of instrument response to field of view, depth of source, point source response and extended source response are described. This paper then discusses data quality indicators the basis for analytical comparability and demonstrates the comparability of the ISOCS instrument analysis to laboratory sample analysis. A discussion of the results obtained by ISOCS analysis of objects that could not be analyzed readily by conventional methods demonstrates a powerful application of the instrument. A comparison of costs associated with the analysis on the ISOCS instrument to the costs of conventional sampling and laboratory analysis is presented. Appendices to this report provide details of studies performed, project organization, cost assumptions, and a glossary of terms.

2.0 STUDY DESIGN AND METHODS

In situ gamma spectroscopy has been shown to be cost-effective in almost all applications where field sampling and laboratory analyses are the baseline technologies. Results can be obtained immediately following field acquisitions, thereby reducing the time delays incurred by physical sampling and laboratory analysis. *In situ* measurements can be performed on sealed systems (i.e., without breaching a containment barrier) or remotely (i.e., at a distance from an external radiation source), reducing personnel exposures and/or work hazards. When analysis by an independent laboratory is required by the project regulator prior to free release of materials, *in situ* measurements serve as a screening technique, eliminating the unnecessary analysis of samples above the derived concentration guideline level (DCGL). Large areas or volumes can be assayed with a large field of view to reduce errors arising from non-homogeneity, providing a more accurate estimate of average radionuclide concentrations. These advantages make *in situ* spectroscopy an attractive tool for many characterization applications.

2.1 Recent *In Situ* Studies

a. The DOE Fernald Area Office performed a study of comparability of traditional *in situ* gamma spectroscopy to the results of laboratory analysis of samples [ref 14]. The study was performed to demonstrate the comparable decision on disposition of remediated land parcels derived from either field or laboratory analysis. Due to heterogeneously contaminated surface soils and difficult to detect contaminants that also appeared in the background, the study was required to obtain as many as 15 samples from the field of view of the *in situ* spectrometer for correlation to the single *in situ* measurement. The study concluded that HPGe measurements of total uranium and thorium could meet certain of the QC acceptance criteria established by the project QAPP. Measurements of Ra-226 could only meet the QC criteria if corrected for disequilibrium caused by radon emanation.

b. The DOE Office of Science and Technology sponsored a demonstration of the ISOCS technology at the Argonne CP-5 Research Reactor [ref 15]. This demonstration, limited to performance over three days, determined that:

- The ISOCS can provide rapid, real time information on the type of radionuclides and the magnitude of the radiological hazard.
- As the ISOCS assay system is relatively new, it will be necessary to demonstrate the accuracy of

the ISOCS system in relation to the standard baseline analysis.

It is this comparison of the ISOCS to baseline analysis technology that is addressed in this report.

2.2 ASTD Project Design

The ASTD project plan is built around the guidance contained in the *Multi-Agency Radiation Survey and Site Investigation Manual* (MARSSIM), published in December 1997 under the auspices of the U.S. Environmental Protection Agency (EPA) but compiled in collaboration with the USDOE, the U.S. Department of Defense, and the U.S. Nuclear Regulatory Agency [ref 13]. The MARSSIM was developed to provide a nationally-consistent consensus approach to conducting radiation surveys and investigations at potentially contaminated sites. The approach adopted in the MARSSIM is scientifically rigorous and yet flexible enough to be applied to a variety of site cleanup conditions. The MARSSIM provides information on planning, conducting, evaluating, and documenting environmental radiological surveys of surface soil and building surfaces for demonstrating compliance with regulations. Its focus is on the final status survey that is used in judging if a remediated site meets the applicable release criteria.

To date, the MARSSIM has been used primarily in determining the post-remediation status of a site or facility, not the condition prior to the initiation of remediation or decommissioning activities. However, the technical guidance contained within MARSSIM regarding the conduct of radiation surveys and site investigations has generic application, and has the potential for use in any situation involving radioactive contamination, whether or not a release criterion is to be applied. Using the MARSSIM to guide the characterization process in this project helped to optimize the survey design and to reduce or eliminate unnecessary samples, saving both time and money.

The Data Quality Objective (DQO) process is the basis for the performance-based guidance in planning MARSSIM surveys. Because the MARSSIM emphasizes the use of statistical planning and data analysis for demonstrating compliance with a final status survey, there are few examples of how to apply the DQO process for other types of surveys where such formal analyses are not necessary, or even appropriate. For example, data are collected during characterization surveys in order to determine the extent, but not necessarily the amount, of contamination. This does

not mean that the data do not meet the objectives of compliance demonstration, but it may mean that formal statistical tests would be of little or no value because the data have not been collected for that purpose. However, all analytical data should be of a quality, demonstrable through the DQO process, to support the determination or decision needed.

2.3 Baseline Analytical Methods

Conventional or baseline characterization in nuclear facility D&D requires the collection of thousands of surface smear, volumetric, and core samples, sending samples for on and off-site analysis, compiling the information in a database, and reviewing the data for quality assurance. Many of the areas requiring characterization are not readily accessible and/or are highly contaminated, further complicating the process. Thus, in addition to being time consuming and costly, the baseline characterization approach can result in excessive radiation exposures to personnel.

2.3.1 Conventional Gamma Spectrometry. The conventional laboratory gamma spectrometry system consists of a germanium detector connected to a dewar or cryostat of liquid nitrogen, a high voltage power supply, a spectroscopy grade amplifier, an analog to digital converter, and a multi-channel analyzer (MCA). When a gamma ray interacts with a germanium crystal, it produces electron-hole charge pairs which are collected rapidly. The total charge collected is proportional to the deposited energy.

The spectrometer system is energy calibrated using isotopes that emit at least two known gamma ray energies, so the MCA data channels are correlated to an energy equivalence. A curve of gamma ray energy versus counting efficiency is generated using known concentrations of mixed isotopes. The center of each gaussian-shaped peak corresponds to the gamma ray energy that produced it, the combination of peaks identifies each radionuclide, and the area under selected peaks is a measure of the amount of that radionuclide in the sample. Since the counting efficiency depends on the distance from the sample to the detector, each geometry must be given a separate efficiency calibration curve.

Samples are placed in containers and tare weighed. Standard practice is to dry solids and homogenize using a ball mill process prior to analysis. Plastic petri dishes sit atop the detector and are useful for small volumes or low energies, while Marinelli beakers fit around the detector and provide exceptional counting efficiency for volume samples. For environmental levels of contaminants, the sample and germanium detector are

usually placed within a lead-shielded counting cavity, to lower interference from radionuclides in the surroundings. Counting times of 1000 seconds to 1000 minutes are typical. Each peak is identified manually or by gamma spectrometry analysis software. The counts in each peak or energy band, the sample weight, the efficiency calibration curve, and the isotope's decay scheme are factored together to calculate the concentration of radionuclide in the sample. The system accurately identifies and quantifies the concentrations of multiple gamma-emitting radionuclides in samples like soil, water, and air filters when a reference standard of known activity is available in a similar matrix and geometry [ref 13]. The availability of the reference standard requires preparation of a radioactive source that eventually must be disposed, thus increasing cost and generating secondary waste.

2.3.2 Conventional *In Situ* Analysis. The advantages of *in situ* measurements over traditional sampling methods have been known and appreciated for some time. These advantages include:

- reducing the potentially large errors associated with random sampling of non-homogeneous source distributions;
- reducing costs and improving safety by minimizing the sampling process; and
- essentially eliminating the delay time between sample collection and availability of nuclide-specific analysis results.

There have been many advances in gamma detection hardware and analysis software during the past 10 years, which now make it much more practical to perform *in situ* gamma spectroscopy. These advances include: large high-purity germanium detectors which provide the required resolution and sensitivity, rugged multi-attitude cryostats allowing the detector to be aimed in any direction and ensuring adequate liquid nitrogen holding times, laboratory-quality battery-powered portable MCAs, portable laptop-size computers with tremendous processing power and data storage capacity, and sophisticated and easy-to-use spectral analysis software.

In order to use the acquired pulse height spectrum for quantitative assessment of radioactivity, an efficiency calibration must be performed. This is normally done with the use of known quantities of radioactive materials in fixed distributions. Previous techniques used, involving uniform mixtures of radionuclides or large numbers of small sources in inert matrices, are very expensive. The user must purchase radioactive sources of the proper range of activity and energy, distribute the

source appropriately, and finally dispose of the source as radioactive waste.

For each new geometry, a new calibration standard and one to several hours of instrument calibration are required. This has limited *in situ* gamma spectrum analysis to simple geometries and contamination distributions. The mathematical detector characterization of the Canberra ISOCS is the innovative response to this problematic aspect of conventional *in situ* analysis.

2.4 ISOCS Description

ISOCS is a complete *In Situ* Object Counting System developed by Canberra for use in a wide variety of measurement applications [ref 2, ref 4]. The battery-operated system provides traditional spectra of counts as a function of gamma energy, which are then converted to radionuclide concentration by applying pre-defined geometry templates in the analysis software. The ISOCS software overcomes the limitations of traditional (tedious and expensive) efficiency calibration techniques, and allows practical modeling and accurate assay of almost any object in the workplace. Thus, complex contamination distributions (e.g., an inaccessible contaminated pipe within a wall) can be identified, and resulting quantification of the contamination therein can be performed.

2.4.1 Germanium Detector. The gamma radiation detector utilizes a high purity germanium crystal for high resolution and high efficiency gamma radiation detection. For the ASTD project, a Canberra Broad Energy Germanium (BEGe) detector was selected because it enhances the efficiency of gamma radiation detection below 100 keV while exhibiting increased transparency to high energy gammas, such as those from naturally occurring K-40. Typical energy response curves for the BEGe and the conventional Coaxial detectors are illustrated in Figure 2-1.

The wide, squat shape of the BEGe detector (active volume of 80 mm diameter by 30 mm thick) is optimized for analysis of objects in front of the detector. However, it has less sensitivity to a Marinelli beaker sample geometry than a traditional co-axial detector whose diameter is smaller than its thickness. The enhanced BEGe detector efficiency for low energy gammas (from 30-100 keV) provides a field capability for detection of Am-241 and low energy gammas associated with actinide alpha-emitters that greatly exceeds the capability of traditional detectors.

There is a drawback to using the BEGe detector. The ISOCS 30E field of view (FOV) collimator is designed for

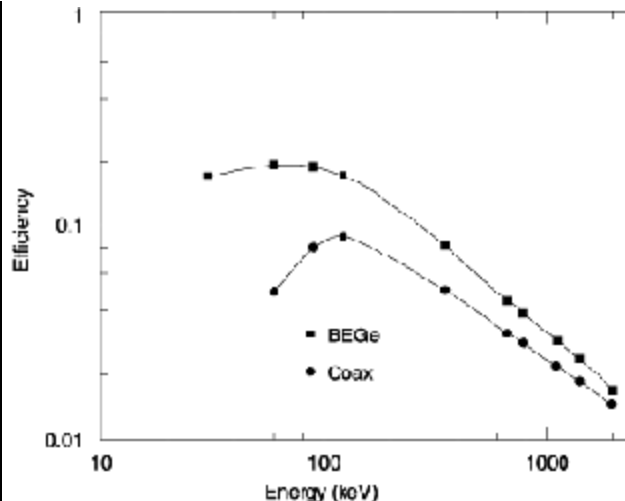


Figure 2-1. Comparison of BEGe detector response to coaxial germanium detector (Canberra, 1999)

use with a standard narrow co-axial detector, and its use is impractical with the BEGe detector. The wide, flat shape of the BEGe results in the 30E FOV collimator shielding over 80% of the active BEGe detector region. This shielding decreases the sensitivity of the detector, which increases the time to acquire a spectrum with enough counts to provide minimal statistical uncertainty. This limitation could be rectified by the manufacturer producing a re-designed collimator for use with the BEGe detector.

2.4.2 Modular Shields and Cart. Mechanical components of the ISOCS system, including a field deployable mobile cart and a modular system of stainless-steel covered lead shields are shown in Figure 2-2. Annular side shields of either 19 mm (0.75 in) or 44 mm (1.75 in) lead thickness effectively reduce the detection of interfering radiation from items in the vicinity of the detector and from background radiation, resulting in improved system sensitivity. The detector's field of view can be further restricted, from 180E to 90E or 30E, by installing lead collimators on the cart's mounting rails, so that interference from items adjacent to the object of interest can be significantly reduced or eliminated from the analysis. In addition, a completely shielded sample chamber can be assembled by stacking the components of the two thickness annular shield systems to enable timely, low-background analysis of samples in the field.

2.4.3 ISOCS Detector Characterization. Previous attempts at simplified mathematical calibrations have had accuracy shortcomings due to assumptions that the

detector was a point detector, and due to limitations in sample shapes accommodated. With ISOCS, however, each individual detector has a unique set of characteristics that are used to generate the calibration data [Ref 2]. This process employs a two-phase, mathematical computation technique that includes detector-specific characteristics, accounts for collimators and/or shields, and models the physical object to be assayed. It uses a combination of Monte Carlo calculations and discrete ordinate attenuation computations to derive efficiency curves for quantitative spectral analysis.

Canberra uses the Monte Carlo Neutron-Particle (MCNP) code for the detector characterization phase of the process [Ref 1]. To accurately represent the Ge detector response, the MCNP model must be rather complex, and typically requires approximately 25 different physical elements. Even with fast 64 bit 300 MHz computers, and special biasing procedures, these efficiencies can take days to compute. To ensure the maximum accuracy and to minimize subsequent analysis times, this phase of the detector characterization is performed on each detector by Canberra at the factory before it is delivered.

The output of the detector characterization process is a series of equations that defines the detector response (in terms of fraction of gammas emitted from the object that interact in the detector):

- at any distance from the end-cap, from 0 to 50 meter;
- at any energy from 3 - 7000 keV; and
- at any angle in all 4-pi directions.

The results of this individual detector characterization are incorporated as a part of the calibration software. Individual detector characterization is recalled by the user when quantifying an individual gamma spectrum.

2.4.4 Source Geometry Modeling. The user phase of the efficiency computation allows accurate efficiency calibrations to be performed rapidly for a wide variety of sample shapes, sizes, densities and distances between the sample and the detector. Objects are modeled from one of a set of generic sample shapes, such as boxes, cylinders, planes, spheres, pipes, etc. These basic geometry templates have many parameters that can be modified to create an accurate representation of the sample object and detector geometry.

Photon attenuation effects due to collimators and shielding components (if present) can be included in the efficiency calibration process. Attenuation effects due to the sample material itself, the container walls (if any), and the air between the sample and the detector are also



Figure 2-2. ISOCS cart with detector, cryostat and modular shield system

included in the calculations. For typical objects and energy ranges of interest, an experienced user can complete the entire efficiency calibration process in several minutes or less. Exposure parameters and efficiencies can be generated in a few minutes in the field and can be modified easily if needed.

At this point, the data is presented and stored just as it would be using the conventional process, that is as if the user had prepared a multiple energy calibration source in the appropriate geometry, counted it, analyzed the spectrum, and computed the efficiency based upon the data in the calibration source certificate file. The resulting ISOCS efficiency calibration functions can then be used to analyze acquired spectral data files with the standard spectrometry analysis software.

2.4.5 Analytical Software. The output of the ISOCS process is a set of energy/efficiency/error triplets. Upon exiting the ISOCS user interface, this data is converted into the energy-efficiency curve format and is displayed for the user as shown in Figure 2-3. The user can manipulate Canberra's standard efficiency response curve interface to determine the best equation that

represents the energy vs. efficiency function. After the calibration curve is accepted, it is stored as an efficiency file, and is available for re-analysis of previous spectra or for newly acquired spectra from sources that are described by the same geometry parameters.

2.4.6 Validation of the ISOCS Computation. Given the uniqueness of the ISOCS mathematical calibration, Canberra Industries has performed a series of internal consistency tests and efficiency validations of the ISOCS methodology [ref 3]. These evaluations were conducted by Canberra to ensure that the ISOCS version 3.0 software handles the physics correctly for different source shapes and sizes, and for different source-detector geometries. The following discussion summarizes the validation performed by Canberra as reported in the validation document.

2.4.6.1 Internal Consistency Tests. Internal consistency tests were designed to demonstrate that the ISOCS software treated source geometries in a consistent manner when defined using several of the available geometry templates. Eight shaped objects (point, sphere, box, etc) were modeled using from 4 to 11 geometry templates of equivalent geometrical shape. If a given source geometry can be configured using different templates, then ISOCS should give the same efficiency

values for different templates. The tests indicated that for the eight shapes, the percent difference in efficiency at a given energy value ranged:

- between 0.10% to 1.12% for energies below 150 keV; and
- between 0.05% to 0.56% for energies greater than 150 keV.

Thus the detector characterization and template definition scheme is internally consistent across the 11 geometry templates.

2.4.6.2 Validation Tests. Validation tests were performed by Canberra to demonstrate the accuracy of the ISOCS efficiency calibrations when compared to actual, physical sources. The tests involved 119 different, multi-energy sources in three categories of spectrum acquisition,

- field counting geometries, involving large sources (> 1 m³ in volume) and/or large source-to-detector distances (> 1 m);
- laboratory counting geometries, involving small volume sources located within 1 meter of the detector;
- collimated geometries, similar to the field counting geometries, using annular shields and 180E, 90E, or 30E FOV collimators.

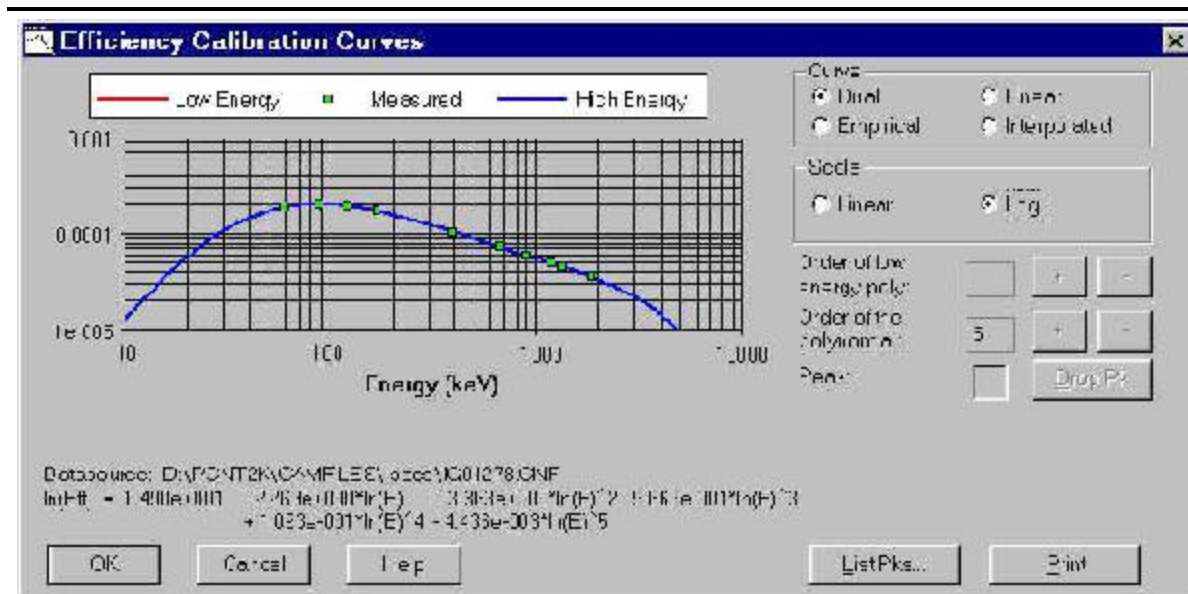


Figure 2-3. ISOCS efficiency response curve interface

The tests indicated that for the three conditions, the percent difference in efficiency at a given energy value ranged:

- between -1% to -2% for energies below 150 keV;
- between +0% to +6% for energies greater than 150 keV; and
- between +0% to +2% for all energies, pooled data.

The validation tests demonstrated that for all three categories, the average ISOCS calculated efficiency to true efficiency ratios were very close to unity. The greatest deviation (+6%) occurred for the higher energy gammas in the laboratory geometry.

2.4.6.3 Propagated Uncertainty. In concluding the software validation, Canberra presented the values in Table 2-1 for uncertainty in the efficiency calculation. These values are present as defaults in the analysis software code and are used with the counting uncertainty to develop a propagated total uncertainty for the measurement result.

The software allows these default values to be changed

Table 2-1. ISOCS Efficiency Uncertainties

Geometry Condition	Energy Range	Rel Std Dev (%)
Laboratory Sources	50 - 100 keV	7.1
	100 - 400 keV	6.0
	400 - 7000 keV	4.3
Field Sources and Collimated Geometry	50 - 100 keV	10.6
	100 - 400 keV	7.5
	400 - 7000 keV	4.4

by the analyst, when other sources of error are known or eliminated. For instance, Canberra recommends that under conditions of heavy attenuation (transmission less than 1% due to an absorber between the source and the detector), an additional multiplication of the error by a factor of 1.5 - 2.0 be applied.

3.0 BASIS FOR ASSESSING DATA QUALITY LEVEL

This ASTD Project is designed to demonstrate that *in situ* gamma spectrum analysis can be used in lieu of laboratory analysis in one or more of the following situations during the D&D process:

- Defining worker protection requirements in the design of work packages;
- Controlling work progress, excavation advancement and waste segregation;
- Waste characterization to demonstrate compliance with waste acceptance criteria; and
- Performing final status surveys for remediated facilities and land areas.

Since each of these phases uses the data for a different purpose, with different consequences for level of precision, accuracy and timely acquisition of results, the data quality requirements are not identical.

The Data Quality Objective (DQO) process has been developed to address the differing issues and requirements on the data use, in order to optimize the return of useful, relevant data for the collection and analytical efforts. The following discussion of data quality is distilled from federal agency guidance provided in EPA data quality documents [ref 10, ref 11, and ref 12] and in the *Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM)* [ref 13].

3.1 Data Quality Level

Despite a number of successful applications of *in situ* spectrometry over the years, issues have arisen regarding the level of data quality that is obtained with field measurement techniques for the purposes of demonstrating compliance with the Resource Conservation and Recovery Act (RCRA), the Comprehensive Environmental Response, Compensation and Liability Act (CERCLA), the Waste Acceptance Criteria (WAC) at disposal facilities, and other regulatory compliance mandates. In the past, EPA defined different levels of data quality, termed “analytical support levels,” by the types of technology and documentation used, and the degree of analytical sophistication [ref 17]. Notwithstanding this intent, the actual titles provided to the analytical levels by the EPA guidance tended to associate the level of quality with the location of the analysis. The relevant levels are:

- Level IV – “Contract Laboratory Program Routine Analytical Services” – characterized by rigorous QA/QC protocols and documentation, providing qualitative and quantitative analytical data.
- Level II – “Field Analysis” – characterized by the use of portable analytical instruments which can be used on-site, or in mobile laboratories stationed near a site (close support labs). Depending on the types of contaminants, sample matrix, and

personnel skills, qualitative and quantitative data can be obtained.

- Level I – “Field Screening” – characterized by the use of portable instruments which can help provide real-time data to assist in the optimization of sampling point locations and in health and safety support.

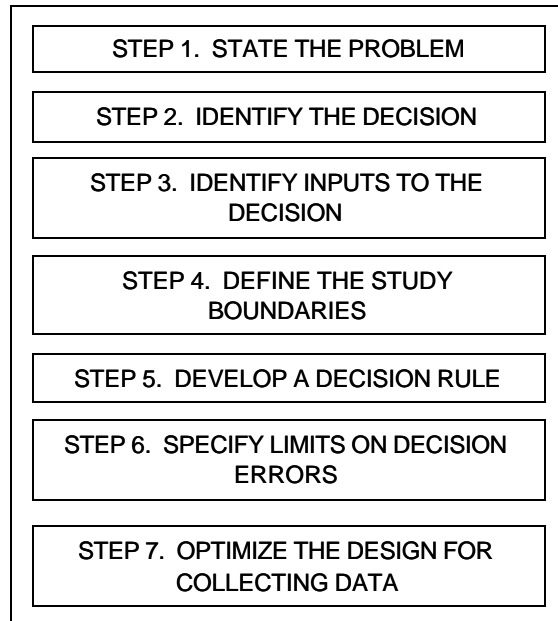
Thus, field measurements, by definition and common usage, have been considered not to possess the quality control that needs to be established to match data quality from the laboratory.

The distinction between screening level and higher quality measurements is based on factors relating to data quality which should be demonstrable. In principle, the rigorous QA/QC protocols and documentation required for definitive analysis using EPA Contract Laboratory Program (CLP) procedures could be applied to radionuclide specific field measurements. Using field techniques at a higher analytical level is also in keeping with the latest EPA proposals for performance-based measurement systems (PBMS). PBMS is a process in which data quality needs, mandates, or limitations of a program or project are specified and serve as a criterion for selecting appropriate analytical methods. Under the PBMS framework, the performance of the method employed is emphasized rather than the specific technique or procedure [or location] used in the analysis. Equally stressed in this system is the requirement that the performance of the method be documented by the laboratory that appropriate QA/QC procedures have been conducted to verify the performance. PBMS applies to physical and chemical techniques of analysis performed in the field as well as in the laboratory [ref 11].

Thus, data quality is assured by adherence to a quality assurance program, regardless of whether analysis occurs in the field or in the laboratory. The quality assurance program establishes the required data quality indicators, procedures and operations. Data quality assessment determines the validity and performance of the data collection design, determines the adequacy of the data set for its intended use, and ultimately determines whether the *in situ* analysis can be used (is “comparable” or not).

From this discussion, it is evident that quality data may be generated in the field, as long as the project DQOs and QA/QC requirements are satisfied. It is the assessment of the total data quality, not the identification of which specific method or instrument was used, that establishes the confidence in the analysis and determines the data quality level.

Table 3-1. The Data Quality Objective Process



3.2 *In Situ* Analysis Quality Assurance

The BNL-ASTD project used the DQO process as the basis for the performance-based guidance in planning characterization and final status surveys. The steps of the DQO process identified in the EPA guidance are shown in Table 3-1.

In the implementation of this project, the first four steps of the DQO process are common to both characterization and final status surveys. In the final three steps, there is significant difference in interpretation and application to the characterization survey.

The fifth step in the DQO process is the specification of a decision rule. For the final status survey Step 5 usually takes the form of a statistical hypothesis test. For a characterization survey such a highly structured rule will not generally be appropriate. However, in characterization surveys it should be possible to identify:

- a range of results that clearly indicates that there is no need for remediation in an area;
- a range of results that clearly indicates that there is need for remediation in an area; and
- an intermediate range of results that may indicate the need for more data before a decision is made.

Such a scheme is loosely patterned after sequential testing procedures, but is primarily intended to differentiate the easy decisions from the more difficult ones so that more resources can be devoted to the areas that need it.

Specifying acceptable limits on decision errors is the sixth step in the DQO process. For final status surveys, Step 6 means specifying decision error rates for Type I (false positive or false rejection) errors and Type II (false negative or false acceptance) errors for statistical hypothesis tests. Again, such precision is usually neither desirable nor necessary in a characterization survey. In a final status survey, the decision errors are used to determine the number of samples it is necessary to collect. The same is true for the characterization survey, except that extensive use of professional judgement is made to balance the costs of additional measurements against the risk of drawing the wrong conclusion from the data.

Optimizing the design of a characterization survey (step seven of the DQO process), involves using all the information available, together with professional judgement, to assess the worth of the information to be gained from additional data in terms of increasing confidence in a remediation decision. This is where the width of the “gray region” expressed by choice (c) of Step 5 is used to separate, as efficiently as possible, the easy decisions from the difficult ones. The cost of data collected early in the characterization can be balanced against the possibility that new data will be needed. The consequence of incorrectly classifying an area as needing remediation when it does not should be balanced against the cost of discovering during a final status survey that an area thought to be clean actually is not. Remediation costs are also balanced against the cost of characterization measurements.

The seven specific elements of the overall DQO process, outlined above, are addressed by the ASTD project team through development of individual project-specific survey plans (PSSPs) in support of individual BGRR D&D campaigns. The PSSP considers the goals of the intermediate D&D objective, the baseline characterization elements, and the targeted components of the facility, to identify the scope and content of the *in situ* characterization efforts using the DQO process. The PSSP provides details on field of view, shielding, and detection levels necessary for the *in situ* evaluations and identifies sample number designations for items and views of items for tracking and reporting purposes.

3.3 Project Quality Assurance

Continuity of spectrum analysis and interpretation among the PSSPs is assured by compliance with the ASTD Project *In Situ* Analysis Quality Assurance Project Plan (QAPP). This QAPP provides a description of the individuals, organizational responsibilities, and control measures necessary to achieve, verify and demonstrate

compliance with both federal and industry quality assurance requirements. This QAPP has been developed using the guidance in EPA QA/G-5 [ref 11] to ensure that appropriate requirements for project data quality have been adequately addressed. The incorporation of EPA QAPP guidance into the ASTD Project QAPP is demonstrated in Table 3-2.

In addition to the Project QAPP, instrument operations and spectrum analysis were standardized by the use of written procedures. These procedures included:

- SOP - DAT1, *Standard Operating Procedure for Gamma Spectrum Acquisition Using Canberra ISOCS System*, version 1, August 19, 1999. This procedure is used to acquire a gamma radiation spectrum for determining qualitatively, the gamma emitting radionuclides *in situ* and in samples. This procedure describes the steps necessary for routine operation of the Canberra gamma spectroscopy system, GENIE-2000 and ProCOUNT. Instructions are provided for identifying hardware components, proper equipment setup, routine instrument operation, in-field spectrum acquisition (sample counting), and spectrum file management.
- SOP - DAT2, *Analysis of Gamma Spectrum Files Using Canberra ISOCS System [Software ver 3.0]*, version 3, February 23, 2000. This procedure describes the steps necessary for quantitative gamma spectrum analysis and reporting using version 3.0 of the Canberra In Situ Object Counting System (ISOCS) Software. Instructions are provided for software environment setup, routine spectrum analysis, software modifications for specific acquisition and analysis requirements, and spectrum file management. The procedure covers the computation steps following spectrum acquisition (accomplished using SOP-DAT1), the development of a quantitative result from the gamma radiation spectrum and acquisition geometry parameters.
- ERD-OPM-4.3, *Procedure for ASTD Sample Processing to Support BGRR Decommissioning Operations*, revision 0, July 12, 2000. This is a work flow procedure to ensure proper, safe, and consistent handling and processing of potentially contaminated samples by instrument operators.

Table 3-2. EPA Requirements to Project QAPP Crosswalk

QAPP ELEMENT (defined in EPA QA/R-5)	Location in ASTD Project QAPP
A. Project Management	
A-1 Title and Approval Sheet	Cover Page
A-2 Table of Contents	Page ii
A-3 Distribution List	Page i
A-4 Project/Task Organization	§ 2.1
A-5 Problem Definition/Background	§ 1.1
A-6 Project/Task Description	§ 1.2
A-7 Quality Objectives and Criteria for Measurement	§ 3.1
A-8 Special Training Requirements / Certifications	§ 2.3
A-9 Documentation and Records	§ 6
B. Measurement / Data Acquisition	
B-1 Sampling Process Design	§ 3.2
B-2 Sampling Methods Requirements	§ 3.3, SOP-DAT1
B-3 Sample Handling and Custody Requirements	§ 3.4
B-4 Analytical Methods Requirements	§ 3.5, SOP-DAT2
B-5 Quality Control Requirements	§ 4.1
B-6 Instrument/Equipment Testing, Inspection and Maintenance Requirements	§ 4.2
B-7 Instrument Calibration and Frequency	§ 4.3
B-8 Inspection/Acceptance Requirements for Supplies and Consumables	§ 5
B-9 Data Acquisition Requirements (Non-direct measurements)	§ 5
B-10 Data Management	§ 6
C. Assessment / Oversight	
C-1 Assessment and response Actions	§ 7.1
C-2 Reports to Management	§ 7.2
D. Data Validation and Usability	
D-1 Data Review, Validation and Verification Requirements	§ 8.1
D-2 Validation and Verification Methods	§ 8.2
D-3 Reconciliation with Data Quality Objectives	§ 8.3

Ref: U. S. EPA, *EPA Guidance for Quality Assurance Project Plans (EPA QA/G-5)*, EPA/600/R-98/018, U. S. Environmental Protection Agency, Washington, DC. February 1998.

3.4 Data Quality Indicators

Data Quality Indicators (DQIs) are qualitative and quantitative descriptors used in interpreting the degree of acceptability or utility of data. The principal DQIs are precision, bias, representativeness, comparability, and completeness. Secondary DQIs include sensitivity, recovery, memory effects, limit of detection, repeatability, and reproducibility. Establishing acceptance criteria for the DQIs sets quantitative goals for the quality of data generated in the analytical measurement process. Quantitative DQIs will be discussed in Section 4, below. The non-quantitative aspects of DQIs are addressed here.

3.4.1 Representativeness. Representativeness refers to the degree to which a measurement reflects the condition at a location or whether a group of measurements reflects the conditions in a particular area. Generally, one desires that measurements (or samples) provide an estimated value of a mean radionuclide concentration that in turn yields a dose estimate (and thus risk) to the average member of a critical group for a particular scenario. In order to achieve representativeness, a number of samples or measurements in a given area would be required in order to achieve a given confidence level or power using a statistical test.

Representativeness is affected by the heterogeneity of the contaminants in the media under investigation. Perhaps more than any other factor, field and laboratory measurements may differ at any particular measurement location due to the effects of heterogeneity. Heterogeneity can exist in both the lateral and depth distribution of a contaminant and can take the form of changes in concentration across various distances:

- a centimeter or less, as would result from hot particles;
- meters, as might occur from dumping and localized spills; and
- tens or hundreds of meters, as from up-wind airborne sources.

Survey designs incorporate techniques and sample/measurement densities to accommodate these variations. The number of measurements and the standard deviation about the mean are fundamental parameters to judge whether the mean concentration that is measured is within a certain confidence limit. These parameters can be used to compute the “t” statistic or applied to other statistical tests.

Where variations in concentration occur on a scale of tens of meters or more, it can be expected that either field measurements or soil sampling will give similar results. It is where variations on the scale of a few meters or less occur that agreement in the results between any pair of

measurements (i.e., two soil sample results or a field measurement and a soil sample result) might suffer. However, if the mean concentration in an area must be determined, a sufficient number of measurements or samples can ultimately yield the same average result, regardless of where the measurements or samples are taken within the area under investigation.

Depending upon the objectives of a measurement program, a field method could inherently have an advantage over discrete sampling. If the viewing area of a field instrument is significantly larger than the area of a soil sample, a set of field measurement results would tend to show a smaller standard deviation as compared to a set of soil sample data in a heterogeneous area. The mean obtained for a given number of field instrument measurements would then be more representative of the true mean than the mean obtained from a similar number of discrete samples. A wide measurement area represented by a field method could also be consistent with the assumptions of a dose model such as RESRAD, which uses the average concentration over a large contaminated area.

The *in situ* measurement might also be more representative of actual radioactivity concentrations. Since samples in the laboratory are often screened to remove rocks, sticks and non-soil matter and are almost always dried before analysis, the laboratory concentration result is consequently biased high. The *in situ* analysis measures the soil “as found”, which is more representative of the actual conditions, but is usually lower in magnitude than the laboratory result.

Because the *in situ* field of view of the ISOCS is greater than the area of an individual sample, the ISOCS measurements will generally be more representative of the average contamination in an area than a single sample with much smaller support. In general, up to ten or more samples may be required to determine the average concentration comparable to a single ISOCS wide-area measurement.

3.4.2 Comparability. Comparability is the qualitative term that expresses the confidence that two data sets can contribute to a common analysis and interpretation [ref 11]. Comparability is one of the principal Data Quality Indicators (DQIs) identified by the US EPA. The DQIs are quantitative and qualitative descriptors used in interpreting the degree of acceptability or utility of data.

Comparability is a critical factor that readily establishes the validity of a field technique. It can be established by performing a study where field measurement results are compared to those given by an independent technique

such as sampling and lab analysis. In some situations, it may be possible to compare two different field techniques. Numerical criteria proposed to determine an analytical method data quality level [ref 9] are provided in Table 3-3.

In performing a direct comparison study, it is important to establish that the two techniques are measuring the same thing. For instance, a technique that measures a contaminant concentration in the surface soil may compare poorly to one that is integrating down to greater depths. This situation would result where there is a non-uniform concentration depth profile of the contaminant. Where comparisons are made to soil samples, core depths can be adjusted to better match the effective viewing depth of the field measurement. The lateral distribution of the contaminant concentration across the ground could also be a factor. In this situation, compositing samples may be required to yield a better average with which to compare a field technique.

Table 3-3. Criteria for Categorizing Data Quality Level

Data Quality Level	Coefficient of Variation	Relative Percent Difference
Definitive Level	$0.85 < R^2 < 1.0$	RPD < 10%
Quantitative Screening Level	$0.70 < R^2 < 1.0$	RPD < 20%
Qualitative Screening Level	$R^2 < 0.70$	20% < RPD; False Negative rate < 10%

ref: U. S. EPA, *EPA Environmental Technology Verification Report: Field Portable X-ray Fluorescence Analyzer*, EPA/600/R-97/150, Washington, DC. March 1998.

4.0 COMPARABILITY DEMONSTRATIONS

An objective of this ASTD project is to document the comparability and quality of the ISOCS system field measurements. However, data “quality” or “comparability” as a concept is meaningful only when it relates to the intended use of the data. Data quality indicators, yardsticks for judging whether or not the data set is adequate, are essential criteria for ensuring that data fulfill the overall DQOs for the project. The context of the use of the data set is the basis for establishing data quality indicators during the planning phase of the DQO process.

The data quality assessment process is the statistical and scientific evaluation of data to determine if the data are of the right type, quality, and quantity to support their intended use. The DQA process addresses two fundamental questions [ref 12]:

- Can the decision (or estimate) be made with the desired confidence, given the quality of the data set?
- How well can the sampling (in this case the *in situ* analysis) design be expected to perform over a wide range of possible outcomes?

Note that the first question does not require that data from two measurement methods produce the same numerical result. It is the decision drawn from the data

that must be the same. During a characterization phase project, less correspondence in reported results may be acceptable, while for the determination of unrestricted release less variation in the data sets might be required. The degree of acceptable correspondence or variation required is established in the DQO process. The second question recognizes that the measurement method performance must be understood over a wide range, so that its application for various decisions may be evaluated in the DQO planning process.

In this section, the data quality indicators are assessed in the context of the second question: If the ISOCS *in situ* analysis is used in an environmental/D&D study, would the data be expected to support the intended use for various decisions and with what desired level of confidence?

Initially, the quantitative data quality indicators of sensitivity, precision, accuracy, and detection limit are assessed. Then the issue of comparability is addressed by reviewing the implementation of ISOCS analysis for two data sets:

- ISOCS analysis of *ex situ* samples, in a field laboratory set-up; and
- ISOCS analysis of *in situ* surface soil, the most common use for *in situ* gamma spectrometry.

Finally a powerful use of the ISOCS system, for objects that cannot be evaluated by conventional methods, is discussed.

4.1 Benchmarking Data Quality Assessment

As part of the deployment of the ISOCS, the BNL ASTD Project needed to demonstrate that this system provides data of sufficient quality for their intended use, compared to the baseline technology of collecting samples for laboratory analysis. The demonstration of data quality involves the assessment of data quality indicators, indicators that provide quantitative and qualitative measures of the degree of acceptability or utility of the data.

One aspect of comparability of the ISOCS measurements relies on the validity and reproducibility of the ISOCS mathematical efficiency computation. Another aspect is the stability and precision of repeated measurements. The sensitivity of the measurement to the interaction between size of the *in situ* field of view (FOV) and the size of the object are important to understand. The assessment of these indicators of data quality form the basis for confidence in the comparability of the analysis method or system.

4.1.1 Assessment 1: Instrument Sensitivity to Source Width. For analysis performed in the laboratory, the sample is finite and well defined by the physical container dimension. For analysis performed *in situ*, it is necessary to identify the lateral boundaries of the detector FOV in order to define what “sample” is actually being analyzed. This can be easily accomplished with the ISOCS efficiency computation code by calculating the efficiency response to a series of virtual contamination sources.

To assess the influence of source diameter, the detector efficiency response was calculated under the following simulated geometry conditions:

- a circular contaminated soil layer, 1.6 g/cm³ density, 15 cm (6 inches) thick, lying on the surface, centered on the detector axis,
- a uniformly mixed contaminated layer with normalized activity concentration of 1 pCi/g for gamma energies of 59.5 keV (Am-241), 661.6 keV (Cs-137), 1173.2 keV (Co-60), and 1460.8 keV (K-40),

- a BEGe detector oriented to look vertically downward from a distance of 1 meter (39.4 inches) above the soil surface, and
- detector shielded by 44 mm (1.7 in) lead annular side shields.

The source diameter was increased step-wise from 0.5 m (18.7 in), and the efficiency was re-calculated at each diameter until the difference in detector efficiency with a change in diameter varied less than $\pm 0.5\%$. The calculated efficiency was then normalized by dividing by the maximum calculated efficiency for that energy.

Calculations were performed for two cases:

- Without a collimator, providing a 180E or 2 pi FOV; and
- With the 90E FOV collimator.

The results of the simulation are shown in Figure 4-1 for the 180E FOV and in Figure 4-2 for the 90E FOV.

Many studies of *in situ* analysis of large areas assume an area of approximately 10 meters diameter is “seen” by an unshielded detector [ref 6, ref 14]. The use of the annular shields illustrates that 95% of the BEGe response is achieved from radionuclides in a diameter of approximately 6.4 m (21 ft) for 59.5 keV (Am-241) to 7.6 m (25 ft) for 1460 keV (K-40). The use of the annular shields reduces the influence of adjacent, interfering sources.

With the 90E FOV collimator, the area “seen” by the BEGe detector varies little with gamma ray energy, as shown in Figure 4-2. What should be noted for the 90E FOV is the penumbra effect of the collimator for all energies. For a “90E FOV”, at 1 meter distance the field should have a 2 meter diameter. The curves in Figure 4-2 show that fully 22% to 27% of the instrument response comes from radioactive material beyond the assumed edge at 2.0 meters. The 95% response level occurs for a field diameter of approximately 2.75 meters, which corresponds to a “108E FOV.”

The Canberra “30E FOV” collimator is designed for use with a standard “co-axial” HPGe detector, and it is incompatible with the BEGe detector. The flat shape of the BEGe results in the 30E FOV collimator shielding over 80% of the active BEGe detector region. This shielding decreases the sensitivity of the detector which increases the time to acquire a spectrum with enough counts to provide minimal statistical uncertainty. The collimator also increases the ISOCS efficiency calculation time dramatically.

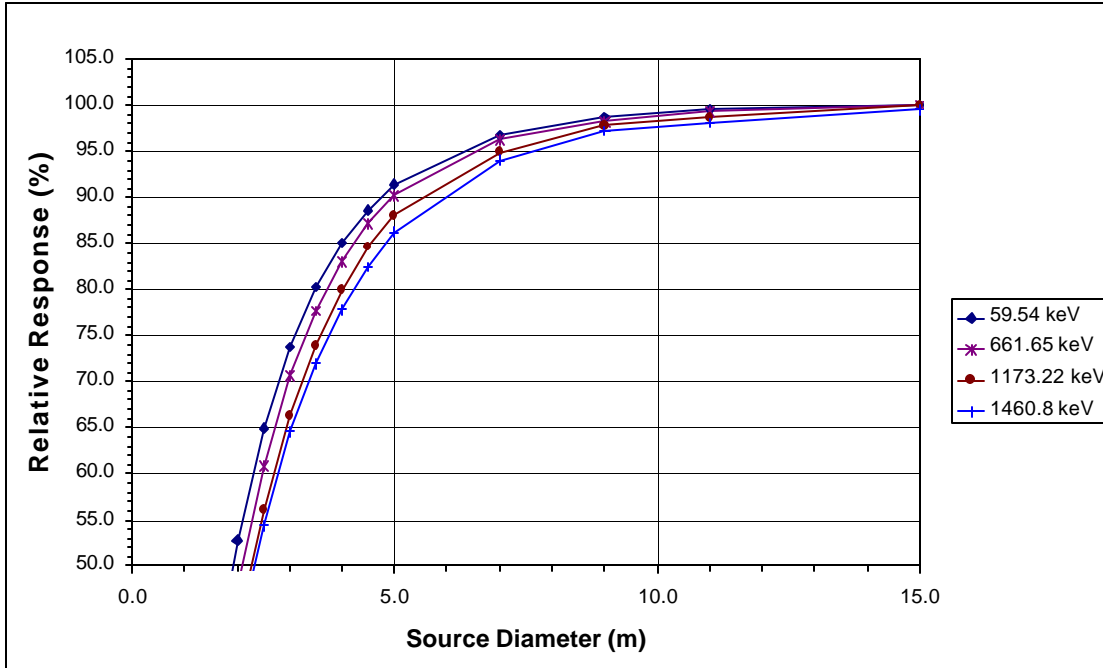


Figure 4-1. ISOCS detector response to surface soil contamination layer diameter at 1 meter with 180E FOV collimator and 44 mm annular shields

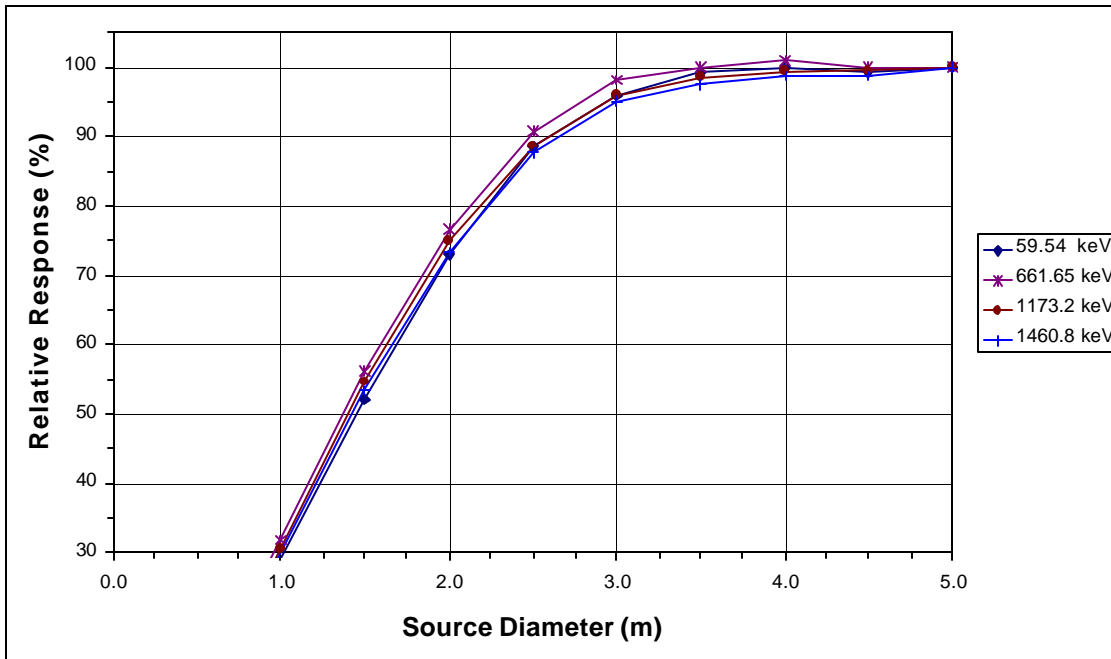


Figure 4-2. ISOCS detector response to surface soil contamination layer diameter at 1 meter with 90E FOV collimator and 44 mm annular shields

When using any collimator to reduce interference from an adjacent radionuclide source, the effect of penumbra penetration should be considered. The “effective” diameter for the 90E FOV collimator with the BEGe detector is approximately 40% larger than would be expected by “sharp” geometry alone. While the software correctly handles the penumbra penetration effect, when positioning or aiming the detector, the operator should be aware that adjacent objects may influence the measurement when near the “edge” of the FOV.

There are “effective” source dimensions, based on parameters of source distribution and FOV collimators. The use of greater dimensions in the geometry model enables the exact mathematical solution, but contributes little to the effective activity measurement reported by the ISOCS calculation. The code run time necessary to determine the detector-geometry efficiency can be reduced by choosing “effective” source dimensions, with impact on the analytical results that is insignificant when compared to other uncertainties in the analysis.

4.1.2 Assessment 2: Instrument Sensitivity to Source Thickness. As with the diameter, for analyses performed *in situ* there is an uncertainty in the thickness of the radiation source that is being measured. For physical samples, the depth of the sample layer is usually well defined and recorded when the sample is collected. Gamma photons of different energies are attenuated differently for the same thickness of a medium, so the detector response would be expected to differ also. Thus to have confidence in the *in situ* analysis, it is necessary to assess the influence of the source thickness in order to define what “sample” is actually being analyzed. This can be easily accomplished with the ISOCS efficiency computation code by calculating the efficiency response to a series of virtual contamination sources.

The detector efficiency response was calculated under the following simulated geometry conditions:

- a circular contaminated soil layer, 1.6 g/cm³ density, 10 m (394 inches) diameter, lying on the surface, centered on the detector axis;
- a uniformly mixed contaminated layer with normalized activity concentration of 1 pCi/g for gamma energies of 59.5 keV (Am-241), 661.6 keV (Cs-137), 1173.2 keV (Co-60), and 1460.8 keV (K-40);

- a BEGe detector oriented to look vertically downward from a distance of 1 meter (39.4 inches) above the soil surface; and
- detector shielded by 44 mm (1.7 in) lead annular side shields.

The source thickness was increased step-wise from 1.0 cm (0.4 inch), and the efficiency was re-calculated with each thickness until the difference in detector efficiency with a change in thickness varied less than $\pm 0.5\%$. The calculated efficiency was then normalized by dividing by the maximum calculated efficiency for that energy.

Calculations were performed for two cases:

- Without a collimator, providing a 180E or 2 pi FOV; and
- With the 90E FOV collimator.

The results of the simulation are shown in Figure 4-3 for the 180E FOV and in Figure 4-4 for the 90E FOV.

The results in Figure 4-3 and Figure 4-4 disclose that there is little difference in detector response with source thickness between the two FOVs evaluated. The figures do demonstrate the magnitude of the difference on response depending on the energy of the gamma ray. For many *in situ* applications, the depth of the field of view is assumed to be 10-15 cm (4-6 inches), usually with the wording like “...average thickness for medium to high energy gamma rays” [ref 6, ref 7, ref 14].

For low energy gammas like 59.5 keV (Am-241), 50% of the detector response is to radioactivity in the top 1.2 cm (0.5 inch) of soil and 95% of the response is from approximately 6.5 cm (2.5 inches). Conversely, for a high energy gamma emitter, such as 1173.2 keV (Co-60), 15% of the response is from activity deeper than 15 cm (6 inches).

As with the source diameter discussed in Section 4.1.1 above, there are “effective” source thicknesses, based on source parameters and FOV collimators. The use of greater dimensions in the model contributes to the exact mathematical calculation, but contributes little to the effective activity measurement of the ISOCS calculation. The code run time necessary to determine the detector-geometry efficiency can be reduced by choosing “effective” source dimensions, with insignificant impact on the analytical results.

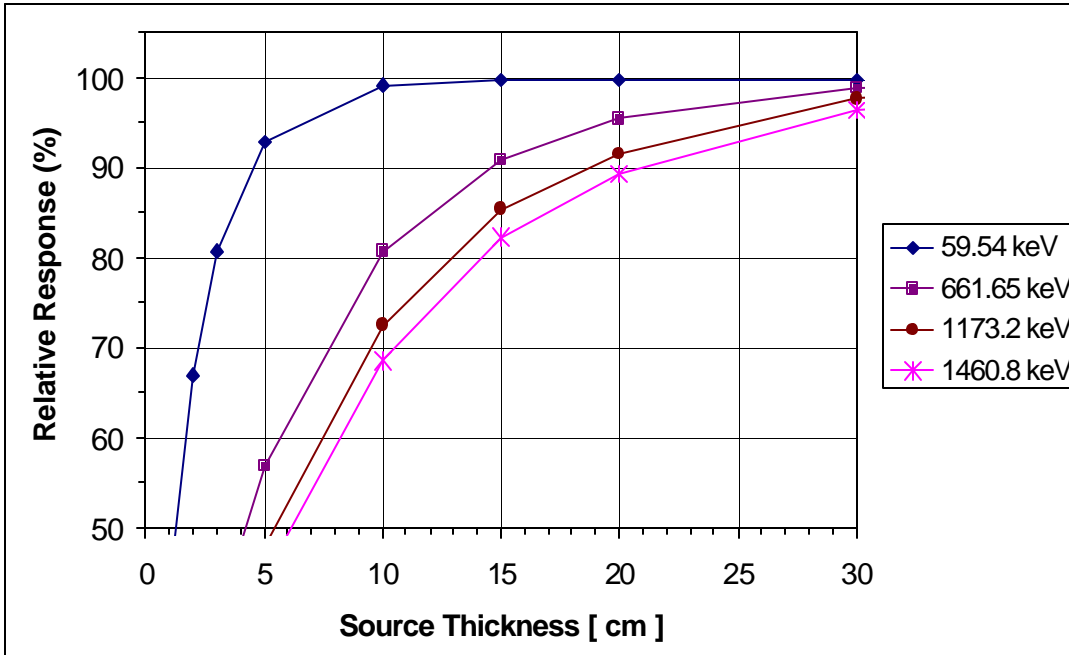


Figure 4-3. ISOCS detector response to surface soil contamination layer thickness at 1 meter with 180E FOV collimator and 44 mm annular shields

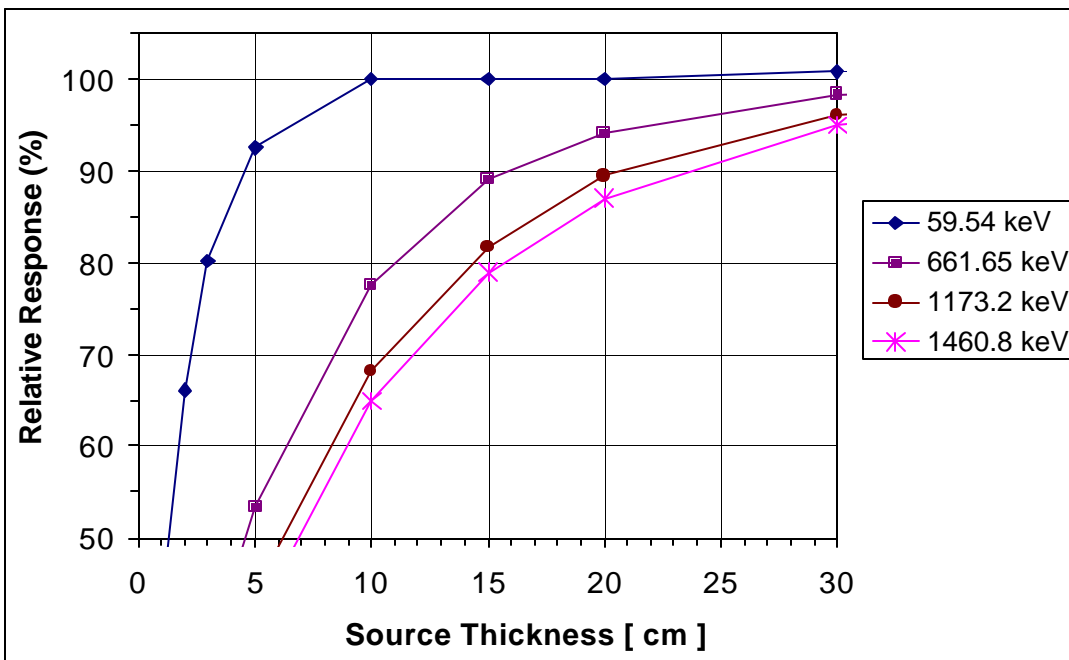


Figure 4-4. ISOCS detector response to surface soil contamination layer thickness at 1 meter with 90E FOV collimator and 44 mm annular shields

4.1.3 Assessment 3: Analytical Accuracy to a Point Source. Analytical accuracy is an important, critical data quality indicator. The ISOCS code verification by Canberra Industries, discussed in Section 2.4.6 above, demonstrated internal consistency and accuracy of the analysis software itself. A benchmark for use of the system is that local operators should demonstrate the capability of interpreting and using the ISOCS computation methods to achieve accurate results.

For this assessment a spectrum of an NIST traceable Eu-152 source in a 0.5 mm capsule was acquired for 10 minutes. The source was positioned perpendicular and 30.5 cm away from the center of the detector end-cap, with the 47 mm annular shields around the detector. This geometry corresponds to the “laboratory counting” *in situ* geometry category identified in the Canberra Industries validation tests, a relatively high activity close to the detector (section 2.4.6.2 above). The quantification accuracy was calculated by using the simple cylinder ISOCS efficiency geometry model. An ambient background spectrum was accumulated with the source absent and demonstrated that Eu-152 was not present in the detector field of view. The results of the analysis are provided in the Table 4-1.

As shown in Table 4-1, the ISOCS system measurement is in excellent agreement with the manufacturer-specified source activity. Performance of this benchmark demonstrated the capability to use the ISOCS geometry templates to accurately model a physical exposure geometry and come up with accurate activity determinations.

4.1.4 Assessment 4: Analytical Accuracy to an Extended Source. For an assessment of analytical accuracy for the “field counting” *in situ* geometry category, *in situ* spectra were accumulated at an agricultural test pad at BNL. The accuracy of the *in situ* measurements is assessed by comparison to laboratory

Table 4-1. Assessment of Point Source Accuracy

Radio-nuclide	Point Source Activity (FCi)		Per cent Difference
	Manufacturer Specified	ISOCS Measured	
Eu-152	0.714 ± 0.036 ⁽¹⁾	0.699 ± 0.022 ⁽²⁾	-2.1 %

- (1) Activity uncertainty of 5% at 99.7% confidence level, as specified by source manufacturer.
- (2) Measurement errors reported at the 95% confidence level and represent only counting error and ISOCS-generated efficiency errors.

analysis of soil samples collected from the same area. The area is the location designated “Site X”, where EML conducted an *in situ* intercomparison study in the fall of 1997 [ref 7].

Site X, an approximately 100 m by 100 m area, has been used by EML as a field baseline study area. Gamma-ray-emitting radionuclides are present in the soil at ambient levels of naturally occurring radionuclides (background) and the nuclear weapons test fallout product Cs-137. The prior use of this site and the tilling operations that have taken place over the years make it a fairly homogenous area in terms of the lateral and depth distributions of the radionuclides. In the 1997 study at this site, six organizations participated in an *in situ* gamma ray spectrometer intercomparison, acquiring measurements at each apex of a regular hexagon of 5 m sides. After the *in situ* measurements, the EML collected 19 soil samples on a 5-m triangular grid overlying the hexagon for laboratory analysis to use in evaluating the *in situ* determinations.

The ASTD Project deployed to Site X in the fall of 1999 to acquire *in situ* spectra for comparison with the previous study results. Gamma ray spectra were accumulated *in situ* for 20 minutes at the six locations used in the prior study. The BEGe detector was mounted on the mobile cart and oriented to look vertically downward from a distance of 1 meter (39.4 inches) above the soil surface. The detector was shielded by 44 mm (1.7 in) lead annular side shields for a 180° FOV. Surface soil activity at each spectrum accumulation location was quantified using the uniform, circular plane geometry template, (contaminated layer 15 cm thick, 10 m diameter, with soil density of 1.6 g/cm³). For data comparison, the ASTD ISOCS measurement results were corrected for radioactive decay occurring during the period between the original intercomparison in Fall 1997 and the ASTD measurements in Fall 1999. Results of the *in situ* measurements and the soil sample analysis are provided in Table 4-2.

In situ measurements with the ISOCS instrument were marginally lower than the results reported by the laboratory (percent difference of the means is 10 %). The low response of the ISOCS instrument can be partially explained by considering the following points that apply to any *in situ* measurement:

- The *in situ* result is for “as found” soil, which contains a non-uniform distribution of uncontaminated rocks and organic material. This material is removed from the sample before laboratory analysis. The extraneous material adds uncontaminated mass to the *in situ* soil that

decreases the reported radionuclide concentration (activity per gram).

- The *in situ* result is for “as found” soil, which contains a non-uniform moisture content, while a laboratory sample is dried before analysis. The uncontaminated moisture adds mass to the *in situ* soil that decreases the reported activity per gram.

For the intercomparison study, the soil activity concentration, reported in Table 4-2 above, was adjusted to account for soil moisture at the time of collection in 1997, i.e., the reported results are for moist soil, not dry. Since the ASTD readings were performed in 1999, some of the difference may be related to different soil moisture, but the values were close enough that further investigation of moisture effects were not deemed worthwhile.

4.1.5 Assessment 5: Analytical Accuracy through Intercomparison. One characteristic of quality sample analysis is the appropriate performance in an intercomparison program. For the baseline of laboratory analysis, intercomparison involves measuring blind samples of calibrated activity and submitting analytical results to the comparison organizer. Both the EPA and the EML coordinate a program of blind sample distribution and intercomparison of laboratory analysis results. For an *in situ* intercomparison, participants mobilize their portable instruments to a common site that has been characterized by the organizer. They then perform the indicated measurements, and submit analytical results for comparison with other participants.

The ASTD project participated in a modified intercomparison by performing measurements at a field site on BNL, where EML conducted an *in situ* intercomparison study in the fall of 1997 [ref 7]. The accuracy of the ASTD *in situ* measurements for the “field counting geometry” category was assessed by comparison to *in situ* measurements performed by other organizations in the same area. In the 1997 study at this site, measurements were performed on a 5 m triangular grid by different pairs of the six participants at each apex of a regular hexagon of 5 m sides. Thus each of the six positions had two reported analyses, a “higher” value and a “lower” value.

The ASTD project accumulated *in situ* spectra for 20 minutes at each of the six locations used in the prior study. The BEGe detector was mounted on the mobile cart and oriented to look vertically downward from a

Table 4-2. Analytical Accuracy for a Large Area Source

Cs-137 Activity in Surface Soil (pCi/g)	ASTD Project <i>in situ</i> measurements at six locations	EML analysis of 19 soil samples
Mean	0.194	0.216
Median	0.188	0.211
Standard Deviation	0.026	0.027
Maximum Observed	0.243	0.281
Minimum Observed	0.174	0.181

Note: Laboratory analysis results from the EML study report [ref 7].

distance of 1 meter (39.4 inches) above the soil surface. The detector was shielded by 44 mm (1.7 in) lead annular side shields for a 180E FOV. Surface soil activity at each spectrum accumulation location was quantified using the uniform, circular plane geometry template, (contaminated layer 15 cm thick, 10 m diameter, with soil density of 1.6 g/cm³). For data comparison, the ASTD ISOCS measurements were corrected for decay during the time elapsed between the original intercomparison study in Fall 1997 and the ASTD measurements in Fall 1999. Figure 4-5 displays for each of the six positions, the BNL ISOCS measurement as well as the two measurements reported by the EML study.

The BNL-ASTD results were greater than the higher measurement at one position, between the two measurements for three positions, and less than the lower measurement for two positions. For the area as a whole, the ASTD *in situ* measurements determined a mean concentration (and 2 sigma uncertainty) of 0.19 ± 0.05 pCi/g, while the intercomparison participants average concentration (and 2 sigma uncertainty) was 0.20 ± 0.04 pCi/g. The 5% lower response of the ASTD measurement could be due to differences in the soil moisture due to the time elapsed between measurements. The ASTD performance is in line with other participants performance and demonstrates the capability to perform *in situ* analysis of a large source geometry.

Comparison of Cs-137 Analysis *in situ* Analysis vs *in situ* Analysis

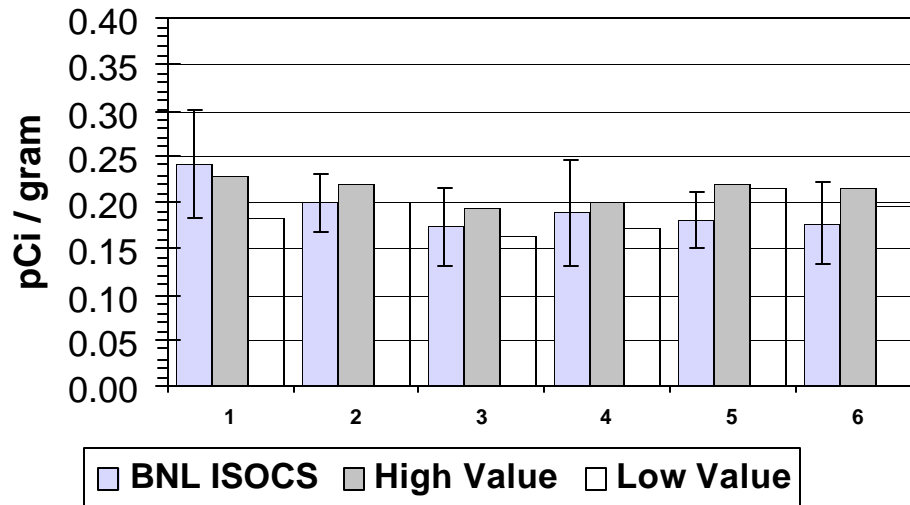


Figure 4-5. Comparison of ISOCS *in situ* analysis to inter-comparison study

4.1.6 Assessment 6: Analytical Precision Over an Extended Period. An important QC indicator of data quality is analytical precision over an extended time period. Often referred to as “reproducibility”, this aspect of precision evaluates the day-to-day stability of the instrument and/or the analysis method for periods of months to years. The data is often graphed as a control chart with bounds indicated for awareness and intervention.

To assess ISOCS system stability the spectrum from a nominal 1 microCurie source of Eu-152 is acquired in a fixed geometry each day of operation. The DQIs tracked and the limits on each are identified in the QAPP [ref 8] and are listed in Table 4-3. The energy calibration and detector resolution elements track the electronic stability. The detector efficiency element is a higher order indicator that includes the analysis software operation, as well as electronic stability of the instrument. An electronic control chart for each of the QC elements is produced following the daily QC procedure, and is reviewed on line at the laptop computer. An example of a typical control chart is provided in Figure 4-6.

The control chart in Figure 4-6 illustrates the stability and precision of the ISOCS system. The chart shows response in a narrow band (average = 1.117 FCi, standard

deviation = 0.008 FCi) and an absence of trends. The outliers are few and occur randomly, and were usually corrected by repeating the QC check. Note that the ± 3 sigma intervention limit is only a $\pm 2.1\%$ variation in the activity measurement, indicating that the ISOCS system response is very stable.

Table 4-3. *In Situ* Quality Control Indicators and Limits

QC Element	Investigation Limit	Action Limit
Energy Calibration Low – 122 keV	± 1 keV	± 2 keV
Energy Calibration High – 1408 keV	± 1 keV	± 2 keV
Detector Resolution Low – 122 keV	± 1 keV	± 2 keV
Detector Resolution High – 1408 keV	± 1 keV	± 2 keV
Detector Efficiency Low – 122 keV	± 2 sigma	± 3 sigma
Detector Efficiency High – 1408 keV	± 2 sigma	± 3 sigma

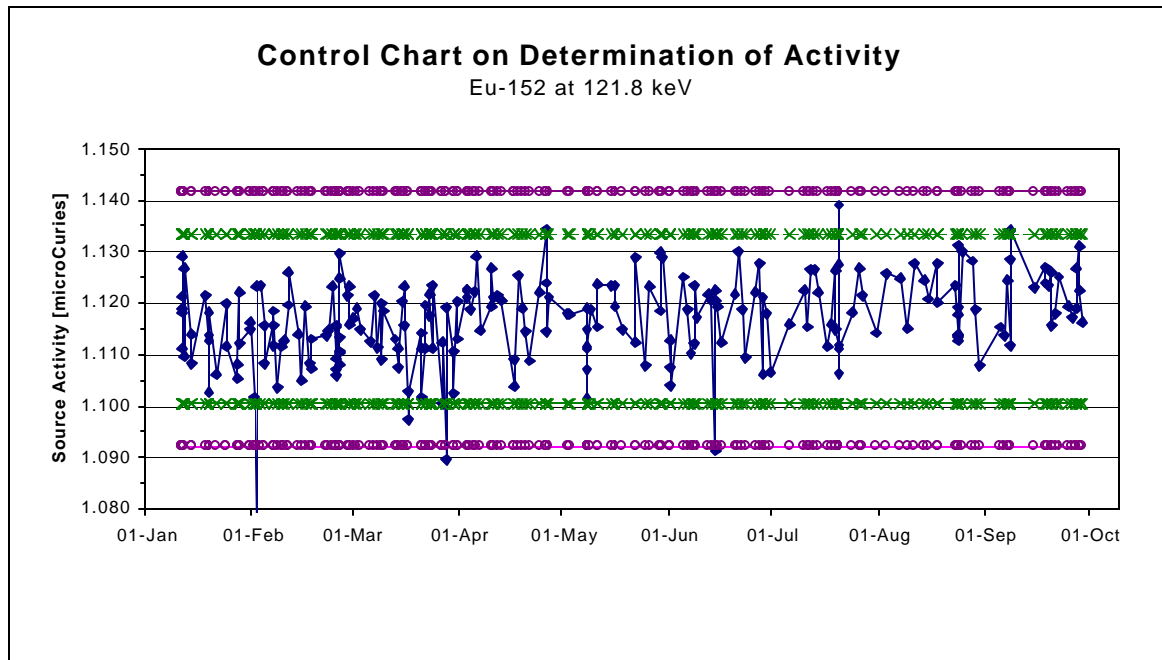


Figure 4-6. ISOCS analytical precision over extended time period

4.1.7 Assessment 7: Analytical Precision in Duplicate Analyses. Precision is a measure of agreement among replicate measurements of the same property, under prescribed similar conditions. Intralaboratory precision represents the agreement expected when a single laboratory uses the same method to make repeated measurements of the same sample. The precision may be expressed as a percentage of the mean of the measurements, the relative percent difference (RPD) [ref 12]. The BNL-ASTD Project QAPP stipulates a repeat analysis at least once per every 20 scans, with the QC action level of an RPD greater than 20%.

The analytical precision of the ISOCS instrument is illustrated by the results of repeat analyses performed during a remedial excavation of a contaminated sump. The ISOCS system deployed to the excavation site and provided rapid analysis on soil contamination, allowing the excavation manager to make real-time decisions on advancing the excavation and on disposition of spoils. To expedite the excavation, samples of suspect soil in 1 liter bottles were analyzed adjacent to the construction site, rather than moving the instrument in and out of the active excavation zone. The samples were modeled prior to the operation using the simple cylinder geometry template for four sample sizes (1/4 full, 2/4 full, 3/4 full and 4/4 full). Gamma spectra were acquired for 5 minutes, the proper model of bottle fullness selected, and the activity

concentration was calculated immediately, available on the screen and stored for later printout.

During the excavation support, 600 samples were analyzed of which 25 were QC repeat analyses. Of the 25 repeat samples, 9 were “Non Detect” on both measurements, one was borderline (“Non Detect” vs 0.3 ± 0.2 pCi/g), and 15 samples had measurable activity. The original and repeat measurements for the 15 samples are provided in Table 4-4.

The results in the table indicate that the ISOCS system is capable of measurements as precise as those performed in a laboratory situation. All values of RPD were within the criteria of the Project QAPP. The largest values of RPD occurred when the activity was small (5~6 pCi/g or less), otherwise the RPD was less than 10% for the sample set.

Table 4-4. Analytical Precision in Repeat Sample Analyses by ISOCS

ASTD Analysis ID	Measured Activity Concentration (pCi/gram)		Relative percent difference
	Cs-137		
	Results	MDC	
SAM00181	93.0 ± 8.7	0.6	1.07%
SAM00202	94.0 ± 8.8	0.7	
SAM00183	19.9 ± 2.0	0.4	0.50%
SAM00203	19.8 ± 2.0	0.4	
SAM00185	3.5 ± 0.5	0.2	12.12%
SAM00201	3.1 ± 0.5	0.2	
SAM00328	204.7 ± 15.9	1.2	2.92%
SAM00335	198.8 ± 15.4	0.9	
SAM00336	50.2 ± 4.1	0.5	0.80%
SAM00358	49.8 ± 4.1	0.8	
SAM00356	2169 ± 165	3.8	0.23%
SAM00357	2164 ± 165	4.0	
SAM00368	5.7 ± 0.6	0.3	15.09%
SAM00391	4.9 ± 0.6	0.3	
SAM00548	356 ± 27.3	1.3	2.77%
SAM00561	366 ± 28.0	1.1	
SAM00563	33.9 ± 2.8	0.6	6.39%
SAM00577	31.8 ± 2.7	0.5	
SAM00562	325 ± 25	1.4	0.61%
SAM00578	327 ± 25	1.6	
SAM00605	75.1 ± 6.0	0.5	2.84%
SAM00608	73.0 ± 5.8	0.6	
SAM00619	0.5 ± 0.2	0.1	0.00%
SAM00625	0.5 ± 0.2	0.2	
SAM00645	0.5 ± 0.2	0.2	0.00%
SAM00646	0.5 ± 0.2	0.2	
SAM00715	14.1 ± 1.3	0.3	5.84%
SAM00716	13.3 ± 1.2	0.3	
SAM00680	88.8 ± 7.1	0.6	4.62%
SAM00684	93.0 ± 7.4	0.6	

4.1.8 Assessment 8: Analytical Minimum Detectable Concentration. An indicator that typically substantiates the data quality of a laboratory method is the limit of detection. The limit of detection is the minimum concentration of an analyte in a specific matrix that can be identified with high probability (usually 95%) when it is present at that concentration. The ISOCS software

uses an algorithm based on the Currie method of determining minimum detectable activity (MDA). The Currie method is widely accepted and used as the basis for much of the laboratory analysis software used in industry [ref 6, ref 4].

The Currie method is based on treating the number of counts, *B*, in the Compton continuum beneath a gamma radiation photo-peak as a Poisson random variable, and the standard deviation of this number is then the square root of *B*. The minimum number of counts above the Compton continuum which results in an interpretation of detected radioactivity is then a function of the standard deviation (typically about four times the square root of *B*). Applying the instrument efficiency calibration curve, for the matrix and geometry of the spectrum acquisition, to this minimum count produces the MDA or the minimum detectable concentration (MDC), depending on the definition of the efficiency calibration.

Typical MDCs for the ISOCS system are provided in Table 4-5. These values are for common ISOCS uses or geometries that occurred during the program duration. The tabulated values are the averages of MDCs reported from 5 different positions/samples in each category in which none of the four radionuclides was detected.

Table 4-5. ISOCS Minimum Detectable Concentrations for Various Geometries

Spectrum Acquisition Conditions and Area or Volume Analyzed	MDC (pCi/g)			
	Co-60	Cs-137	Am-241	K-40
<i>in situ</i> , 90E FOV @ 0.5 m, 1200 sec 4.9 m ² or 735 liters of surface soil	0.09	0.12	0.71	0.78
<i>in situ</i> , 180E FOV @ 1.0 m, 1200 sec 20 m ² or 3000 liters of surface soil	0.04	0.07	0.30	0.40
<i>in situ</i> , 180E FOV @ 1.0 m, 600 sec 20 m ² or 3000 liters of surface soil	0.04	0.08	0.36	1.5
sample in shielded cavity, 300 sec 1 liter of soil	0.30	0.30	0.40	2.0

The results in the table are indicative of the detection limits of the ISOCS in typical *in situ* and sample counting geometries that occurred in the D&D project. The magnitude of the MDCs are small, and much less than typical levels of clean-up goals. The MARSSIM recommends that measurement techniques should be capable of measuring levels below the established clean up goals, and 10-50% of the clean up goal should be the target [ref 13]. The results in the Table 4-5 illustrate that the ISOCS can achieve this target.

It should be noted that the MDC values in Table 4-5 are for samples that have no activity detected. Due to the incomplete interaction in the detector, scattered or partially absorbed high energy gamma rays are detected in the Compton continuum at lower energies, raising the Currie method detection limit for radionuclides with low energy gamma rays. For example, for a soil sample measured in a 1 liter bottle for 300 second acquisition, the MDC for Am-241 shifts from ~0.4 pCi/g when Cs-137 is "not detected" to ~1.0 pCi/g when Cs-137 is present at 100 pCi/g. This phenomena is common to any gamma spectrometer, both ISOCS and laboratory systems.

4.2 Assessment of Comparability

Comparability is the qualitative term that expresses the confidence that two data sets can contribute to a common analysis and interpretation. Comparability must be carefully evaluated to establish whether two data sets can be considered equivalent in regard to the measurement of a specific variable or groups of variables. In a laboratory analysis, the term comparability focuses on method type comparison, holding times, stability issues, and aspects of overall analytical quantitation.

There are a number of issues that can make two data sets comparable, and the presence of each of the following items enhances their comparability:

- two data sets should contain the same set of variables of interest;
- units in which these variables were measured should be convertible to a common metric;
- similar analytic procedures and quality assurance should be used to collect data for both data sets;
- time of measurements of certain characteristics (variables) should be similar for both data sets;
- measuring devices used for both data sets should have approximately similar detection levels;
- rules for excluding certain types of observations from both samples should be similar;

- samples within data sets should be selected in a similar manner;
- sampling frames from which the samples were selected should be similar; and
- number of observations in both data sets should be of the same order or magnitude.

These characteristics vary in importance depending on the final use of the data. The closer two data sets are with regard to these characteristics, the more appropriate it will be to compare them. Large differences between characteristics may be of only minor importance, depending on the decision that is to be made from the data.

Two data sets obtained during the performance of the BNL-ASTD Project can be used to demonstrate the assessment of comparability:

- ISOCS analysis of *ex situ* samples, in a field laboratory set-up; and
- ISOCS analysis of *in situ* surface soil, the most common use for *in situ* gamma spectrometry.

4.2.1 ISOCS Sample Analysis vs Laboratory Sample Analysis. A convenient use of the portable gamma spectrometer is in the on-site, *ex situ* measurement of radioactivity in physical samples. In many locations or areas of interest, an *in situ* measurement is impractical:

- the material of interest is in an area with adjacent radiation sources that interfere with an *in situ* measurement,
- the material of interest is from an area, such as an excavation or in a sump, where moving the *in situ* instrument requires strenuous efforts and/or impractical delays, or
- the material of interest is in an area where *in situ* operators do not have the training, medical evaluation, or security clearance to enter.

In these situations, on site field laboratory analysis of *ex situ* samples provides a rapid, economical alternative to shipping to a remote or off-site laboratory. The ASTD project performed a direct comparison of on site field laboratory sample analysis to remote laboratory analysis to demonstrate comparability of the ISOCS system.

4.2.1.1 Sample Collection. A total of twenty-five samples of surface soil were collected from four contaminated locations on the BNL site. The locations were selected for historical knowledge of the wide range of contamination level in the soil. Each sample consisted of 2 liters of material from the surface layer (0-15 cm) which

was sieved to remove organic material, rocks and items larger than 6 mm (1/4 in). Each sample component of sieved soil was combined, thoroughly mixed together and used to fill a 1 liter container. The liter bottles of sample were analyzed on the ISOCS instrument and then packaged and sent off-site for analysis at an analytical laboratory.

4.2.1.2 ISOCS Sample Analysis. The analysis was performed by accumulating *in situ* a gamma radiation spectrum using an ISOCS BEGe detector attached to a computer-based, multi-channel gamma spectroscopy system. The detector, portable cryostat and lead shielding collimators were mounted on the mobile cart support, allowing consistent sample positioning in a vertically upward orientation. The detector was configured using 44 mm lead annular side shields around the detector and 19 mm annular shields above the detector, creating a low-background counting cavity for the 1 liter sample bottles. The sample analysis geometry is illustrated in Figure 4-7. The gamma spectrum from each sample was evaluated using the ISOCS software,

Figure 4-7. ISOCS instrument configured for sample analysis



modeling the sample as a uniformly-contaminated, simple cylinder of soil, with density of 1.6 g/cm³. Results were reported as picoCuries/gram (pCi/g).

4.2.1.3 Laboratory Analysis. Analysis of samples by the analytical laboratory was performed in a low-background shielded cavity, with a Hyperpure germanium detector attached to a computer-based, multi-channel gamma spectroscopy system. Prior to analysis, samples were dried, tumbled with steel shot to homogenize the matrix, and a 150-gram aliquot was sealed in an aluminum/tin can to isolate contaminants and facilitate sample handling. Results were reported as picoCuries/gram (pCi/g).

4.2.1.4 Results. Analysis results for Cs-137 in the samples by the ISOCS instrument and the off site laboratory are provided in Table 4-6. The only contaminant observed was Cs-137. A plot of the ISOCS results versus the laboratory results is provided in Figure 4-8.

4.2.1.6 Discussion. The graph in Figure 4-8 demonstrates the excellent agreement of BNL ISOCS sample analysis with the laboratory sample analysis results. Indicated on the graph of Figure 4-8 is the least-squares linear regression line with slope 1.00 and correlation coefficient of 0.99. The regression slope indicates that the ISOCS result is equivalent to the laboratory reported result (no bias), with a very strong correlation ($R^2 = 0.99 \approx 1.0$) reflecting little non-random error.

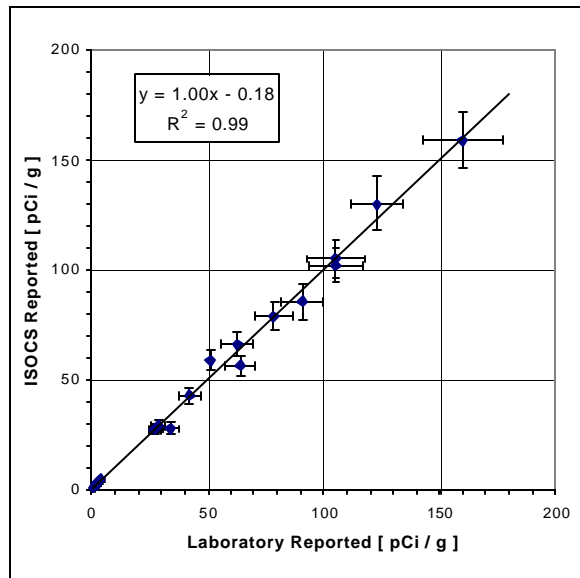


Figure 4-8. Correlation between ISOCS sample and laboratory sample analysis

The absence of bias reflects the conditioning of the samples before analysis:

- extraneous non-contaminated biomass was separated from the matrix by sieving in the field, so both systems analyzed only soil;
- Samples were surface soil from open fallow fields, collected in early March before spring rains, so moisture content was very low; soil drying at the remote lab did little to alter the mass of the sample from what was analyzed on site; and
- Ball milling by the remote laboratory was effective in homogenizing the soil.

Table 4-6. Precision in Sample Analysis: ISOCS to Laboratory

Cs-137 Activity Concentration (pCi/g)					
Sample No.	ISOCS		Remote Lab		Percent Difference
	activity	2 sigma	activity	2 sigma	
1	0.65	0.4	0.76	0.09	-14.5 %
2	0.80	0.3	0.85	0.10	-5.7 %
3	1.8	0.3	1.73	0.23	4.0 %
4	2.4	0.4	1.99	0.20	20.6 %
5	3.3	0.5	2.69	0.29	22.7 %
6	4.5	0.6	4.01	0.41	12.2 %
7	27.1	2.3	27.2	2.68	-0.4 %
8	27.6	2.8	34.0	3.30	-18.8 %
9	28.9	2.5	28.6	3.08	1.0 %
10	42.8	3.6	42.1	4.70	1.7 %
11	56.2	4.6	64.0	6.67	-12.2 %
12	58.6	4.8	51.3	6.08	14.2 %
13	66.1	5.3	62.6	7.18	5.6 %
14	78.7	6.3	78.4	8.1	0.4 %
15	85.3	7.9	90.8	9.3	-6.1 %
16	102	8.1	105	11.4	-2.9 %
17	105	8.3	105	12.4	0.0 %
18	130	12.1	123	11.4	5.7 %
19	159	12.5	160	17.4	-0.6 %
20	256	19.7	317	30.8	-19.2 %
21	307	23.7	328	44.0	-6.4 %
22	324	29.8	270	26.4	20.0 %
23	457	41.9	418	39.2	9.3 %
24	494	38.0	441	47.6	11.9 %

4.2.2 ISOCS *In Situ* Analysis vs Laboratory Sample Analysis. An optimum use of the *in situ* technique is in the evaluation of large areas of surface soil, prior to or following remediation. Because of the field of view and ability to average heterogeneous distributions, the *in situ* technique provides a rapid, economical alternative to conventional sampling methods. The ASTD project performed a direct comparison of *in situ* analysis to laboratory analysis to demonstrate comparability.

4.2.2.1 Area Evaluated. A 90 m by 40 m irregular area of landscape soil adjacent to the BNL medical facility parking area known to be contaminated with Cs-137 was selected for the comparison. Ten positions were chosen to provide a variety of contamination levels across the range of interest, see Figure 4-9.

4.2.2.2 ISOCS Instrument. The analysis was performed by accumulating *in situ* a gamma radiation spectrum using an ISOCS BEGe detector attached to a computer-based, multi-channel gamma spectroscopy system. The detector, portable cryostat and lead shielding collimators were mounted on the mobile cart support, allowing consistent 1 meter distance and vertical orientation at each position. The detector was configured using 44 mm lead annular side shields and 180° field of view, with 5-20 minute accumulation. The gamma spectrum from each position was evaluated using the ISOCS software, modeling the surface as a uniformly-contaminated, circular plane of surface soil, 10 m diameter and 15 cm (6 in) thick. Results were reported as pCi/g.

4.2.2.3 Samples for Laboratory Analysis. At each *in situ* measurement position, soil samples were obtained from four locations: immediately below the ISOCS position and at three equidistant positions 2 meters out, see Figure 4-10. Each component sample consisted of ½ liter of soil from the surface layer (0-15 cm) which was sieved to remove organic material, rocks and items larger than 6 mm (1/4 in). The four components were combined, thoroughly mixed together and placed into a 1 liter container for shipment.

Analysis of samples by the analytical laboratory was performed in a low-background shielded cavity, with a HPGe detector attached to a computer-based, multi-channel gamma spectroscopy system. Prior to analysis, samples were dried, tumbled with steel shot to homogenize the matrix, and a 150-gram aliquot was sealed in an aluminum/tin can to isolate contaminants and facilitate sample handling during analysis. Results were reported as pCi/g.

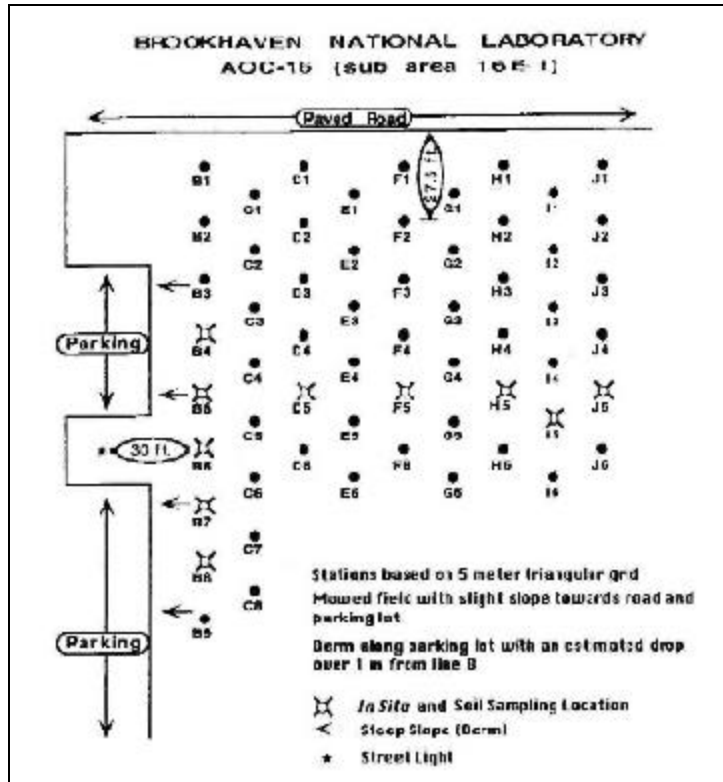


Figure 4-9. Analysis positions for *in situ* vs laboratory comparability

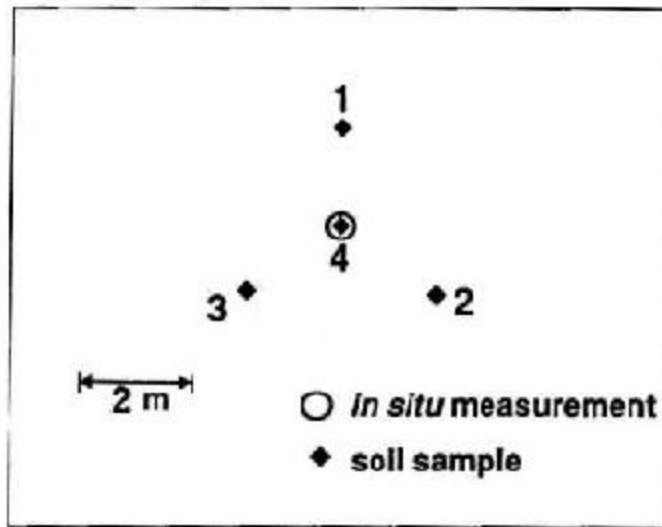


Figure 4-10. Locations of samples for composite at each analysis position

4.2.2.4 Results. A plot of the ISOCS results versus the laboratory results is provided in Figure 4-11. The only contaminant observed was Cs-137.

4.2.2.5 Discussion. *In situ* measurements with the ISOCS instrument were generally lower than the results reported by the laboratory. Indicated on the graph of Figure 4-11 is the least-squares linear regression line with slope 0.70 and correlation coefficient of 0.98. The regression indicates that the ISOCS result is 70% of the laboratory reported result, with a very strong correlation ($R^2 = 0.98$. 1.0) reflecting little non-random error. The low response bias of the ISOCS instrument can be partially explained by considering the following points that apply to any *in situ* measurement:

a. The *in situ* result is for as found soil, which contains a non-uniform moisture content, while the laboratory sample is dried before analysis. The uncontaminated moisture adds mass to the *in situ* soil that decreases the reported activity per gram. For a typical range of soil moisture of 5-15 %, the *in situ* result would be 5-18 % lower than the concentration reported by the laboratory.

b. The *in situ* result is for as found soil, which contains a non-uniform distribution of uncontaminated rocks and organic material. This material is removed from the sample before laboratory analysis. The uncontaminated material adds mass to the *in situ* soil that decreases the reported activity per gram. For a typical range of non-soil components of 10-20 %, the *in situ* result would be 11-25 % lower than the concentration reported by the laboratory.

4.2.2.6 Conclusions. The ISOCS *in situ* analysis provides an analytical process that yields interpreted results rapidly to support remediation decisions. The *in situ* analysis is correlated directly to sampling and remote laboratory analysis. The *in situ* results are biased low, due to inclusion of moisture, rocks, and biomass that is removed prior to laboratory analysis. The linear regression correlation provides a numerical coefficient that allows a prediction of laboratory results from the *in situ* results. This coefficient can be used to adjust *in situ* field data to be comparable to data that would result from conventional laboratory analysis. While the *in situ* measurement more accurately reflects activity concentrations as they exist in the field, prior reliance on and acceptance of the laboratory value may predicate adjustment of *in situ* data.

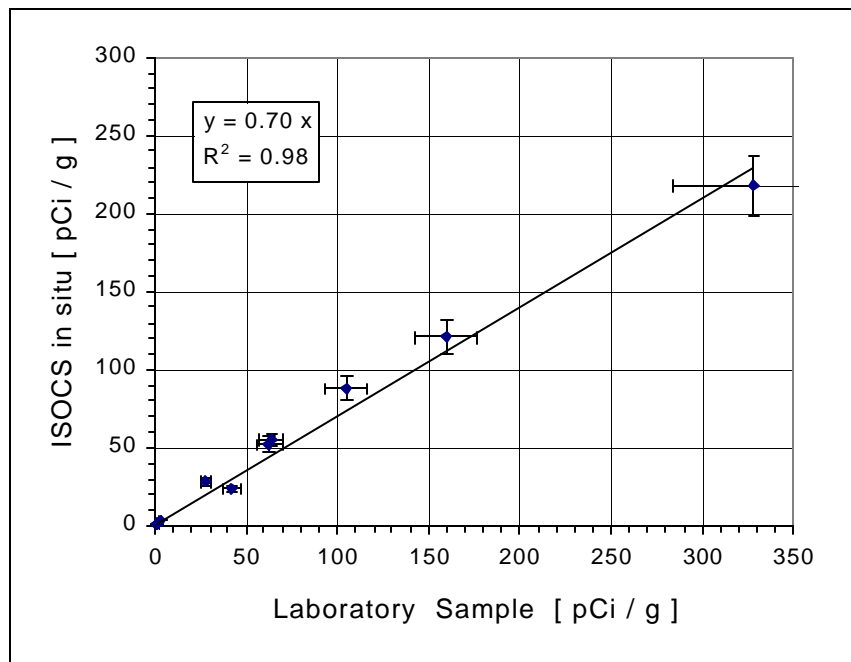


Figure 4-11. Correlation between ISOCS *in situ* and laboratory sample analysis

5.0 ANALYSIS OF DISTINCTIVE OBJECTS NOT EASILY EVALUATED BY CONVENTIONAL METHODS

One of the strengths of the ISOCS mathematical calibration and geometry templates is the capability to quantify objects of all shapes and sizes, without the expense of designing, fabricating and disposing calibration standards in the same geometry and matrix. This capability is demonstrated in the following discussion of analyses performed on distinctive objects during the D&D of the BGRR.

5.1 BGRR Fan House Fans

A typical ISOCS application can be illustrated by reviewing the characterization of core-cooling exhaust fans, prior to their removal, volume reduction, and shipment from the site. Each fan is a massive squirrel-cage type blower, nominally 8 ft x 10 ft x 12 ft, and 14,000 lbs. The fans became internally contaminated, likely as a result of fuel element failure, but the identity, extent, and quantity of radioactive material in the fan internals were unknown. External surveys revealed non-uniform internal deposition with highest readings in the vicinity of the fan volutes, where entrained dust particles would have had a higher probability of settling out due to eddies and dead spaces in air flow currents. Three of the five fans had been upgraded/replaced during the operating life of the reactor. Thus, physical configurations, dimensions, and radionuclide quantities were different from those in the other two fans.

The ISOCS was mobilized to the Fan House containing the five contaminated fans and *in situ* gamma spectra were acquired from Fan #5 (representative of Fans #5 and 4) and Fan #3 (representative of Fans #3, 2, and 1). Figure 5-1 is a photograph of the ISOCS deployed at the BGRR Fan House. Each fan housing was scanned using 44 mm annular shields and 90E field of view collimators to reduce interference as much as possible from adjacent contaminated structures. Because of the equipment layout, there was no position where gamma spectra could be acquired without structural components (concrete supports and carbon steel struts) shielding a portion of the field of view.

The ISOCS cart was positioned so that the detector was oriented diagonally downwards at the fan housing volute bottom, where surveys indicated an accumulation of radioactivity. Spectra were accumulated for 15 minutes each from two symmetric positions: NE of the housing facing SW and NW of the housing facing SE. Equipment setup, spectrum acquisition and equipment-breakdown

required less than two hours, with only minimal health and safety oversight and without breaching of contamination containment barriers.

Radioactivity in the fan housing was modeled as a layer of surface dust, uniformly covering the interior of the carbon steel fan volute (horizontal or diagonal rectangular plane). Due to the complex geometry with intervening structural members, several alternative geometry models were defined. The intervening structural members were adjusted in the models until the results from the symmetric scans were similar. The modeling and analysis of both fans required about six hours.



Figure 5-1. *In situ* measurement of Fan No. 3

Table 5-1. Results of *In Situ* Measurements at BGRR Fan House

Fan Unit	Measured Activity, μCi			
	Cs-137		Co-60	Am-241
	low estimate	high estimate		
Fan No 5	75 ± 3	600 ± 20	# 0.1	# 180
Fan No 3	114 ± 10	330 ± 30	4.8 ± 0.5	# 800

Note: The uncertainties in the table represent $\pm 2s$ counting error; values expressed as “#” represent an estimate of the bound on the activity and indicate that the radionuclide was not detected.

The results of the analysis are provided in Table 5-1. The range in the value is representative of the uncertainty in the analysis, and is primarily associated with assumptions on unobserved inner structures of the fan.

The results in Table 5-1 demonstrates that even when using a BEGe detector with enhanced low-energy response, the detection level can still be high when the source is shielded by a highly attenuating medium such as this example, inside a carbon steel fan housing. With the use of the ISOCS modeling software, a quantitative estimate of the activity in the fan was provided in approximately eight hours, without fabricating a physical radioisotope calibration standard, without breaching contaminated barriers, and without handling and transporting contaminated samples.

5.2 Graphite Pile Internals

A complex *in situ* analysis application involved the characterization of the BGRR graphite pile internals to support the determination of pile disposition. The characterization data was also needed for planning radioactive, hazardous, and mixed waste disposal, assessing potential health and safety hazards during stabilization D&D work, determining ALARA controls, and accurately scheduling the work. The analysis of experimental port penetrations illustrates the use of the ISOCS instrument in this survey, as well as some of the strengths and limitations of the application.

The pile is a 25-ft cube of refined graphite surrounded on all sides by a five-foot-thick, high-density concrete

biological shield. Between the shield and pile are air gaps to allow the flow of cooling air. The interior of the biological shield is faced with steel plates, 6 inches thick, providing structural support and thermal shielding, protecting the concrete from excessive dehydration due to the heat from the core and air. The graphite pile was de-fueled in 1968, with the final fuel shipment being made in 1972.

Access to the pile internals for collecting characterization data was through the penetration openings that exist on each face of the pile biological shield wall. The east and west faces are penetrated horizontally at 30 positions by 4.5-inch square ports for experimental access to the pile interior. Figure 5-2 is a view of the West face of the reactor, the location of several experimental ports visible by the dark, round port covers.

In situ experimental port measurements were performed by aiming the detector, aligned with and centered on the centerline of the experimental port, at each port opening located on the West face of the pile. The steel port covers and plugs were removed prior to each measurement, to minimize structural absorption of gamma rays along the analysis pathway. The resulting exposure condition was analogous to a 4.5-inch diameter borehole through the concrete shielding into the pile interior. Figure 5-3 is a photograph of an Experimental Port on the West Face (No. W-54) with the ISOCS instrument in place for spectrum acquisition.



Figure 5-2. West face of the BGRR showing access to experimental port locations

Physical restrictions near the pile face precluded the optimal geometry of a single, reproducible distance from the face at all ports. The *in situ* measurements were performed in conjunction with physical sampling and radiological surveys of the ports, so that radiological controls were in force during the spectrum acquisition. Instrument surveys along the experimental ports indicated maximum exposure rates at the position of the steel plate on the inside face of the biological shield.

The complex geometry of the spectrum acquisition necessitated several simplifying assumptions of the components scanned. The ISOCS geometry template method allowed the assumptions to be logically defined and consistently applied.

Assumptions concerning the geometry model included:

- The use of the thick, dense steel source and the intervening dense concrete absorber in the model causes an individual ISOCS efficiency calculation to take an extended amount of computer processing time, up to 200 minutes or more. Although the physical source resembles a slice of Swiss cheese (a 25 ft x 25 ft x 6 inch slab with 30, 4.5 inch holes), the source “seen” by the BEGe

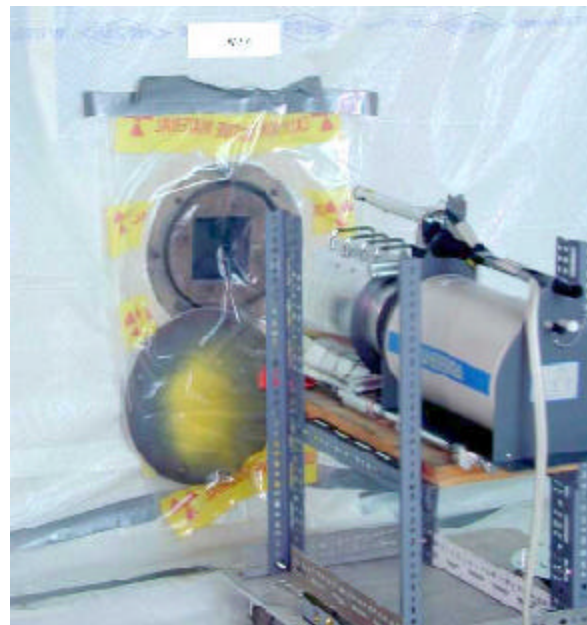


Figure 5-3. *In situ* measurement of pile internals at Experimental Port No W-54.

detector in a scan through an individual port is much less. As discussed in sections 4.1.1 and 4.1.2 above, there are “effective” source dimensions, based on source parameters and FOV collimators; greater dimensions contribute to the “exact” mathematical calculation, but contribute little to the effective activity measurement. Several alternative models of the geometry were performed to establish the “effective” source dimensions in order to reduce the analysis time for individual evaluations.

- The source activity was modeled as an activated, annular, carbon steel ring or pipe, with “effective” dimensions of an inside diameter of 4.5 inches (11.5 cm), a wall thickness of 2 inches (5.08 cm), and a length into the pile of 6 inches (15.24 cm). The steel ring is positioned at the end of the experimental port, 6 feet (183 cm) into the shield and flush to the concrete shielding around the port.
- The model assumes a homogenous distribution of activity throughout the steel source. Activity was calculated as concentration (pCi/g) in the steel to

enable extrapolation to the total activity in the entire steel liner.

- The intervening concrete biological shielding around the port was modeled as an external field of view collimator, 4.5 inches inner diameter, 9 inches (23 cm) outer diameter, and 6 feet (183 cm) in length. The ISOCS software allows the definition of external collimators, as well as two external absorbing layers.

The use of the “effective” dimensions for the steel source and the concrete collimator/absorber in the model reduced the individual ISOCS efficiency calculation times to between 20 and 40 minutes. Different acquisition distances necessitated individual efficiency calculations for each port evaluation. The initial evaluation of several alternative models of the geometry to establish the “effective” dimensions required 10-12 hours – 10-15 minutes of set-up time and 1.0-1.5 hours of computer run time per alternative. The modeling and analysis for 12 experimental ports averaged approximately 3 hours per port. Typical results of the analysis are illustrated in Table 5-2.

Table 5-2. *In situ* Analysis at West Face of BGRR Graphite Pile
Gamma Spectrum Analysis with the ISOCS Instrument

Experimental Port	W-51	W-54	W-56	W-30
ASTD Sample No	IG01231	IG01233	IG01229	IG01220
Radionuclide	Activity Concentration (pCi/g) <i>note 1</i>			
Am-241	ND [185,000]	ND [222,000]	ND [189,000]	243,000 ± 226,000 [370,000] <i>note 2</i>
Cs-137	ND [6,800]	ND [7,200]	ND [7,200]	1,200,000 ± 104,000 [20,500] <i>note 2</i>
Co-60	306,000 ± 12,500 [3,650]	413,000 ± 16,700 [4,300]	346,000 ± 14,200 [4,200]	960,000 ± 38,000 [7,650]
Eu-152	261,000 ± 10,400 [8,400]	316,000 ± 12,600 [9,300]	221,000 ± 10,200 [7,020]	153,000 ± 11,500 [11,100]
Eu-154	68,100 ± 4,700 [8,800]	108,600 ± 6,600 [12,800]	69,400 ± 5,000 [8,400]	30,300 ± 17,500 [39,200]

Notes: ND = Not Detected [] = Minimum Detectable Concentration in pCi/g

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 50 % and should be added to the results shown.
2. Am-241 and Cs-137 activity on Port W-30 is more probably surface contamination than volume activation, as indicated in the table.

The results in Table 5-2 provide the measured activity for Ports W-51, W-54 and W-56, experimental ports that are vertically 8 feet (250 cm) above the pile centerline, and that run West to East from 8 feet South (No W-51) to 8 Feet North (No W-56) of the centerline. The relative locations of these experimental ports are illustrated in Figure 5-4. The radionuclides identified in the measurements are those expected in activated steel. Comparing the activities for Port W-51 and W-56, it is seen that the concentrations are symmetrical about the centerline. Note that the high detector response, due to the presence of large amounts of Co-60, has elevated the minimum detectable concentration of the low-energy radionuclide Am-241. This is due to scattered Co-60 gammas increasing the Compton continuum in the energy range of the 59.5 keV Am-241 gamma.

For Port W-30 which passes through the pile center, the results identify additional radionuclides and different activity ratios among the radionuclides. Historical operations and survey data indicated the presence of

radioactive contamination in the Port infiltrated from failed fuel events in adjacent fuel channels. The analysis modeled the Cs-137 and Am-241 in the gamma spectrum as a volume activated source. To interpret the observed spectrum as surface activity would require that a different model be analyzed, with judgement applied to determine which portion of the spectrum belongs to which of the interpreted geometries. Since alternative methods were available to directly measure the removable contamination through a surface wipe, this effort was not considered necessary.

In situ analysis enabled the rapid evaluation of portions of the graphite pile and its internal components, without the expense and effort of core boring or dismantling the contaminated and activated pile. The modeling computation routine developed quantified estimates of internal activation. There were some ambiguities in the results, due to inconsistencies between the assumptions of the distribution of radionuclides and their physical location on or in the pile internal components.



Figure 5-4. Relative location of Experimental Ports 51-55, west face of BGRR

6.0 COST COMPARISON OF ISOCS AND BASELINE CHARACTERIZATION TECHNOLOGIES

This section provides a basis for comparing relative costs of the *In Situ* Object Counting System (ISOCS) deployed for characterizing gamma emitting radionuclides at the BNL BGRR with the conventional baseline approach of taking discrete physical samples for off-site analyses.

6.1 Categorization for Cost Comparison

Since the ISOCS system was deployed in numerous configurations and varying locations at BNL under this ASTD project, the cost analysis has been broken down into several broad categories to facilitate comparison.

- The first distinction for examining the cost breakdown is *in situ* vs. *ex situ*. The ISOCS system is well suited to conduct both *in situ* measurements of objects and areas requiring characterization and *ex situ* measurements of discrete samples when configured as an analytical field laboratory. *Ex situ* samples were further categorized as either soil or debris, sludge, and miscellaneous materials.
- The next distinction relates to the surface area being analyzed. *In situ* measurements were conducted on large areas (e.g., walls, floors, soil) and smaller unique samples (e.g., concrete cores, reactor components).
- The third and final category is related to the radiological conditions in which the samples were taken, i.e., whether the workers would be required to enter a radiologically controlled area in order to obtain the samples for analysis. One of the major advantages of the ISOCS system is the ability to conduct scans remotely, thus often avoiding the need to enter radiologically controlled areas. This reduces radiological exposure to workers, reduces cost, and reduces the time required to gather the analytical data. However, ISOCS analyses that would have required entry into radiologically controlled areas are identified to facilitate cost comparison with the baseline approach.

The ten general categories used for this cost comparison are summarized in Table 6-1.

All 920 of the *ex situ* samples analyzed for this deployment are summarized in Table A-1 of Appendix A. Of the total, 815 *ex situ* samples analyzed were soil and 105 were sludge, debris, or other materials. A total of 352 *in situ* measurements were conducted and are

Table 6-1. General Categories for BGRR ISOCS Characterization

	Description of Characterization Category
1	<i>Ex Situ</i> Sample Analysis of Soil
2	<i>Ex Situ</i> Sample Analysis of Debris, Sludge, and Miscellaneous Samples
3	<i>In Situ</i> Analysis of Large Uniform Areas (entry into Radiologically Controlled Area required for conventional sampling)
4	<i>In Situ</i> Analysis of Large Uniform Areas (no entry into Radiologically Controlled Area required for conventional sampling)
5	<i>In Situ</i> Analysis of Small Uniform Areas (entry into Radiologically Controlled Area required for conventional sampling)
6	<i>In Situ</i> Analysis of Small Uniform Areas (no entry into Radiologically Controlled Area required for conventional sampling)
7	<i>In Situ</i> Analysis of Large Heterogeneous Solid (entry into Radiologically Controlled Area required for conventional sampling)
8	<i>In Situ</i> Analysis of Large Heterogeneous Solid (no entry into Radiologically Controlled Area required for conventional sampling)
9	<i>In Situ</i> Analysis of Small Heterogeneous Solid (entry into Radiologically Controlled Area required for conventional sampling)
10	<i>In Situ</i> Analysis of Small Heterogeneous Solid (no entry into Radiologically Controlled Area required for conventional sampling)

summarized in Table A-2 of Appendix A, where they are further identified in terms of the categories described above. A total of 215 separate *in situ* scans were taken in which entry into radiologically controlled areas would have been required using the conventional baseline approach, and a total of 137 scans were taken in areas that would not require radiological controls for removal of characterization samples.

When comparing ISOCS with conventional analyses it is important to note that a single ISOCS measurement is not

necessarily equivalent to just one conventional analysis, so that a simple one-to-one comparison of ISOCS *in situ* measurements with conventional baseline samples is overly conservative. For example, a single *in situ* ISOCS measurement can potentially evaluate an area up to several square meters. In the case of heterogeneously contaminated solids, one ISOCS scan can provide an accurate report of average concentration values, whereas, numerous samples would be needed to derive a similar average value using the baseline approach. Thus, the overall savings associated with ISOCS is based on fewer samples to gather, prepare, ship, analyze, and evaluate.

In order to bracket a range of potential cost savings for this comparison, *in situ* measurements were categorized in terms of the approximate number of equivalent baseline analyses they potentially represent. *In situ* scans of large areas and objects were assumed to be equivalent to ten conventional samples and smaller areas and objects were assumed to be equivalent to five conventional samples.

Using this approach, 1760 baseline samples would be necessary to provide equivalent characterization data for samples in radiologically controlled areas and 1360 baseline samples in non-radiologically controlled areas. These represent the maximum estimated cost savings resulting from *in situ* characterization. Minimum cost savings are determined based on a 1:1 ratio for comparing baseline and *in situ* characterization analyses, i.e., 215 samples for radiologically controlled areas and 137 samples for non-radiologically controlled areas.

6.2 Methodology

Cost estimates were developed by preparing work breakdown structures (WBS) for *in situ* measurements and equivalent baseline measurements in a manner similar to the cost analysis prepared following the technology demonstration of the ISOCS at the Chicago Pile 5 Research Reactor [ref 15]. *In situ* sampling categories were grouped according to whether or not the work required sampling within radiologically controlled areas. The resulting WBS data sheets for the scenarios evaluated are identified in Table 6-2. The actual data tables are provided in Appendix C as Tables C-1 through C-10.

Estimates for the times required to conduct ISOCS measurements were based on actual deployment experience at the BNL BGRR. Times required to conduct the baseline activities were based on engineering estimates and information in the literature. Certain costs (e.g., capital equipment for innovative or baseline technologies, institutional overhead costs, training,

Table 6-2. Identification of Cost Comparison Scenario Data Tables

Work Breakdown Structure Categories for Cost Comparison	Table Number
Baseline <i>Ex Situ</i> Sampling and Analyses Soil Samples	C-1
ISOCS <i>Ex Situ</i> Sampling and Analyses Soil Samples	C-2
Baseline <i>Ex Situ</i> Sampling and Analyses Debris, Sludge, Misc. Samples	C-3
ISOCS <i>Ex Situ</i> Sampling and Analyses Debris, Sludge, and Misc. Samples	C-4
Baseline Sampling and Analyses in place of <i>In Situ</i> ISOCS for All Radiologically Controlled Areas (Maximum Cost Savings Assumptions)	C-5
Baseline Sampling and Analyses in place of <i>In Situ</i> ISOCS for All Radiologically Controlled Areas (Minimum Cost Savings Assumptions)	C-6
<i>In Situ</i> ISOCS Sampling and Analyses of All Radiologically Controlled Areas	C-7
Baseline Sampling and Analyses in place of <i>In Situ</i> ISOCS for all Non-Radiologically Controlled Areas (Maximum Cost Savings Assumptions)	C-8
Baseline Sampling and Analyses in place of <i>In Situ</i> ISOCS for all Non-Radiologically Controlled Areas (Minimum Cost Savings Assumptions)	C-9
<i>In Situ</i> ISOCS Sampling and Analyses of All Non-Radiologically Controlled Areas	C-10

Note: Individual Cost data tables are provided in Appendix C of this report.

project management) were not included to simplify the comparison and to facilitate comparison at other sites. It was assumed that these types of costs would be incurred regardless of the approach used and would be quickly amortized through future deployments.

Work conducted in radiologically controlled areas requires the use of personal protective equipment (PPE) with associated loss in productivity which is estimated by the Productivity Loss Factor (PLF). This factor is an

historically based estimate of the non-productive portion of the work day due to PPE changes, work rules based on As Low As Reasonably Achievable (ALARA) considerations, additional work breaks, etc. According to methodology developed by the Atomic Industrial Forum, tasks that are conducted within radiologically controlled areas are adjusted by a loss factor of 1.27, the product of factors of 1.15 to account for PPE and 1.10 to account for adjusted work-rest cycles [ref 16]. The additional costs associated with losses in productivity are calculated as the product of the amount of time required in radiologically controlled areas and the PLF. Other specific assumptions for each cost evaluation are provided as footnotes to the data tables.

6.3 Actual ASTD Project Costs

In order to evaluate the cost effectiveness of this deployment, project costs for completion of this two year ASTD project are presented in the following sections and summarized in Table 6-3. Many of the costs are one-time expenses, and others are unrelated to the cost of deploying ISOCS and thus are not considered in this analysis. A more rigorous Life Cycle Analysis to determine amortization of these expenses was not conducted. The costs presented here are actual costs, including all overhead burdens.

6.3.1 Capital and Other One-Time Costs. The ISOCS unit deployed at BNL was purchased for a cost of \$ 148,800. Other one-time costs included training and consulting for initial unique characterizations. On-site training of BNL staff and specialized consulting provided by Canberra for implementation of the system cost \$27,000. Maintenance/service contract support cost of \$10,694 is a recurring expense and thus is not discounted.

6.3.2 Project Costs Unrelated to ISOCS Deployment. This ASTD project included support for other activities related to *in situ* characterization, but not directly related to deployment of the ISOCS technology at BNL. These included support to project collaborators including \$150,000 to DOE EML for participation on MARSSIM implementation and \$55,000 to Bechtel Hanford to assist in secondary deployment activities. Support for deployment of BetaScint, a fiber optic sensor for near real-time analysis of Sr-90 was also part of the ASTD budget. Purchase of the BetaScint hardware cost \$87,500 with an additional \$30,200 for training and on-site demonstration. An estimated \$120,000 in BNL labor and support was devoted to the BetaScint

activities. Travel costs were discounted since they were programmatic in nature. The cost of travel to support presentations at meetings, symposiums, mid-year review, etc. totaled \$64,000 for the life of the two year project. Likewise, project management costs are not technology-specific and are required for either innovative or baseline characterization. Thus, the total available for the ISOCS deployment was \$806,300. When one-time capital equipment costs are subtracted out, a total of \$631,300 remains available for routine characterization of the facility, so that a more direct comparison with the baseline analytical approach is possible.

6.4 Results and Conclusions

This cost comparison quantifies relative costs for ISOCS and baseline sampling/analysis at BNL's BGRR Decommissioning Project and related activities. Actual costs for baseline analyses are highly dependent on site-specific conditions and the types of analyses that are required. In order to make reasonable comparisons with the conventional baseline approach and establish a range of potential cost savings, assumptions were made about the number of baseline analyses that would be displaced. While there is considerable uncertainty in projecting the number of baseline samples equivalent to *in situ* characterization scans, this approach serves to bracket potential cost savings in terms of minimum and maximum levels. A summary of the cost comparison is presented in Table 6-3 and graphically in Figures 6-1 and 6-2. Based on the assumptions described above, the relative cost of ISOCS characterization at BNL was \$81,769. Corresponding relative costs for baseline sampling/analysis ranged from a minimum of \$292,065 to a maximum of \$1,074,976. Discounting ASTD project investment costs, the resulting net cost savings of \$210,296 to \$993,207 represents savings ranging from 72% to 92% over the cost of the baseline technology. If the total ASTD project costs outlined in Table 6-3 are included, the cost comparisons range from -\$339,235 (loss) to +\$443,676 (savings). Average cost per sample for ISOCS characterization was \$76 compared with \$252 for baseline characterization.

While both the cost per sample and overall costs are significantly lower using ISOCS *in situ* and field laboratory characterization, perhaps the greatest cost savings associated with this innovative technology are "hidden" savings that are more difficult to quantify. These include savings associated with project schedule acceleration, the ability to characterize non-standard systems, and improved health and safety for D&D

workers. Many engineering decisions during D&D operations are dependent on radiological characterization of facilities and waste generated. For example, excavation of contaminated soils below the Pile Fan Sump and contaminated BNL Landscape Soils were greatly accelerated by the availability of near real-time analyses. Rapid analytical turn-around enabled project engineers to quickly determine when targeted clean-up levels were reached so that equipment and personnel could be efficiently staged. Net cost savings resulted from accelerated completion of the activities and minimization of non-productive use of resources. In addition, non-standard equipment and facilities were readily characterized using ISOCS, where no comparable technique was available using the baseline approach. For example, the large fan motors and plenums which contained inaccessible areas of contamination were evaluated through external *in situ* ISOCS scans and related modeling. Finally, numerous characterization activities of areas with significant levels of contamination were successfully completed without the necessity of extracting manual sub-samples, thereby avoiding radiological exposure to workers.

Both the documented cost savings and anticipated "hidden" cost savings demonstrated during this ASTD project were implemented over the course of about 1.5 years. Based on this success, use of ISOCS characterization has been integrated into the on-going D&D and environmental restoration activities at BNL and will continue to generate additional cost savings as these projects progress towards completion. For example, BNL's Environmental Restoration Division has successfully deployed ISOCS for final status surveys of the excavation of contaminated Landscape Soils. BGRR continues to support the use of ISOCS for numerous on-going characterization activities required during facility decommissioning. Investment in this technology has also been leveraged for a second ASTD initiative at BGRR beginning in FY01 in which contamination in subsurface soils will be characterized and modeled. If this approach can demonstrate that subsurface contamination is localized, some or all of the large below grade facilities (e.g., duct work) can be decontaminated and left in place, resulting in cost savings estimated to total over \$3.4 million. Additional deployments planned at Hanford (Canyon Disposition Initiative) and Nevada Test Site (D&D of former nuclear Rocket Test Facility) promise to further increase return on investment for cleanup of DOE sites.

Table 6-3. Actual Costs to Implement ISOCS for Characterization of BGRR

Category	FY 1999	FY 2000	Total
ASTD Budget	\$742,000	\$802,000	\$1,544,000
Non BNL-Collaborators:			
EML	\$100,000	\$50,000	\$150,000
Bechtel Hanford	\$30,000	\$25,000	\$55,000
BetaScint			
Capital Equipment		\$87,500	\$87,500
Training		\$30,200	\$30,200
Labor		\$120,000	\$120,000
Travel	\$44,000	\$20,000	\$64,000
Project Management	\$111,000	\$120,000	\$231,000
Subtotal Available for ISOCS Deployment	\$457,000	\$349,300	\$806,300
One-Time Expenses:			
Equipment	\$148,000		\$148,000
ISOCS Training	\$27,000		\$27,000
Total Available for ISOCS Deployment:			
	\$282,000	\$349,300	\$631,300

Table 6-4. Cost Comparison Summary

Minimum Savings over Baseline:

<u>Characterization Description</u>	<u>Baseline Sampling Analysis</u>		<u>ISOCS</u>		<u>Savings</u>	
	Total	Per Sample	Total	Per Sample	Total	Per Sample
Soil Samples, <i>Ex Situ</i>	\$164,710	\$202	\$45,040	\$55	\$119,670	\$146
Debris, Sludge, Misc. Samples, <i>Ex Situ</i>	\$26,759	\$255	\$10,456	\$100	\$16,303	\$155
Rad Controlled Areas (1:1 ratio of baseline: ISOCS)	\$69,647	\$324	\$15,497	\$72	\$54,150	\$252
Non Rad Controlled Areas (1:1 ratio of baseline: ISOCS)	\$30,949	\$226	\$10,776	\$79	\$20,173	\$147
Total	\$292,065		\$81,769		\$210,296	
Average Cost/sample		\$252		\$76		\$175

Maximum Savings over Baseline:

<u>Characterization Description</u>	<u>Baseline Sampling Analysis</u>		<u>ISOCS</u>		<u>Savings</u>	
	Total	Per Sample	Total	Per Sample	Total	Per Sample
Soil Samples, <i>Ex Situ</i>	\$164,710	\$202	\$45,040	\$55	\$119,670	\$146
Debris, Sludge, Misc. Samples, <i>Ex Situ</i>	\$26,759	\$255	\$10,456	\$100	\$16,303	\$155
Rad Controlled Areas (1:1 ratio of baseline: ISOCS)	\$576,444	\$328	\$15,497	\$72	\$560,947	\$256
Non Rad Controlled Areas (1:1 ratio of baseline: ISOCS)	\$307,063	\$226	\$10,776	\$79	\$296,287	\$147
Total	\$1,074,976		\$81,769		\$993,207	
Average Cost/sample		\$253		\$76		\$176

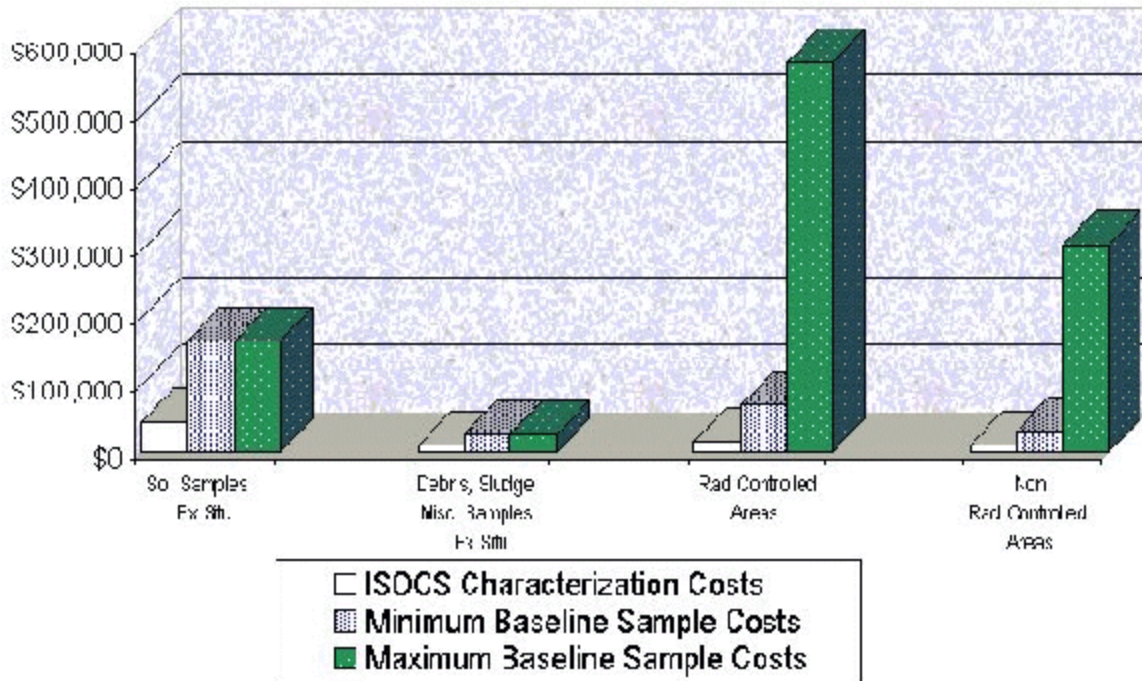


Figure 6-1. Comparison of ISOCS and baseline characterization costs by category

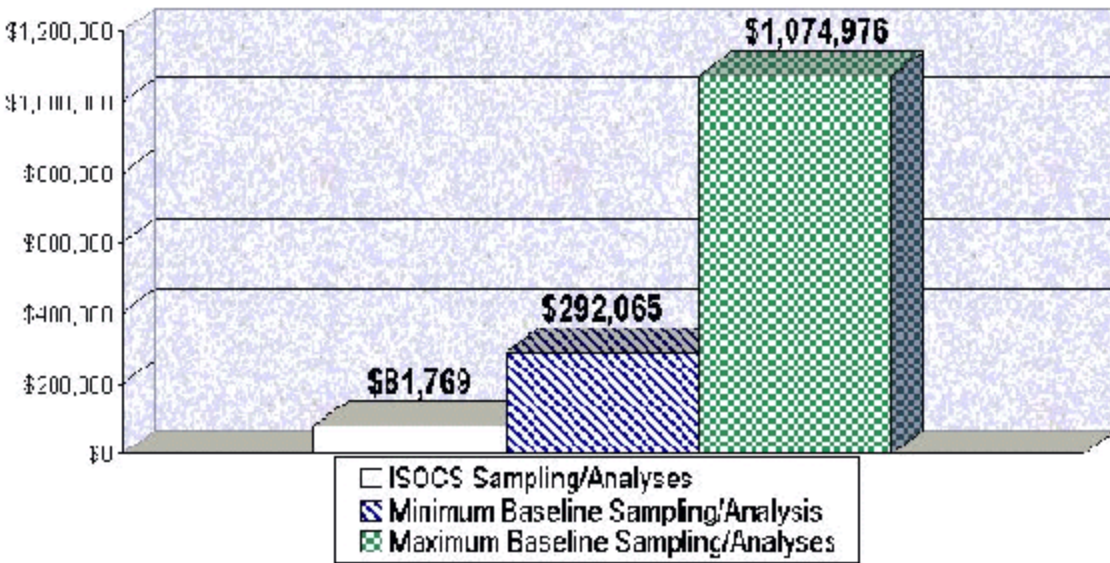


Figure 6-2. Comparison of total ISOCS and baseline characterization costs

7.0 REGULATORY/INSTITUTIONAL ISSUES

Decommissioning of the BGRR is being conducted as a CERCLA removal action and thus prior to initiating work incorporates a logical progression of establishing Data Quality Objectives (DQOs), Sampling and Analysis Plan (SAP), Engineering Evaluation/Cost Analysis (EE/CA), and period of public review and comment. At each stage, BGRR-DP engages the regulatory community (EPA Region II, NYS Department of Environmental Conservation (NYSDEC), and Suffolk County Department of Health) for review and comment, so that their input is incorporated prior to review/approval of the final Record of Decision (ROD). This dialogue allows for better insurance that the final ROD will be acceptable upon completion. The community is also kept informed as the project progresses so that their input can be considered prior to the final decision-making process.

This dialogue is accomplished in several ways. First, the BGRR-DP issues weekly status reports that are available to the regulators and the public via the BGRR web site (<http://www.bnl.gov/bgrr/Default.html>). Monthly status reports to the DOE EM program are also made available. Progress on the ASTD project is incorporated into both of these report formats. Regulators are then specifically

engaged as part of the DQO process by evaluating documentation and dialogue through face-to-face meetings. Several DQO meetings have been held on separate removal sub-actions (e.g., Pile Fan Sump). As part of this process, the ASTD project team prepared documentation on comparability of *in situ* characterization technologies for the regulators and were available to brief the regulators. *In Situ* characterization techniques were found to be acceptable and were then incorporated into the BGRR-DP Sampling and Analysis Plan(s). This form of regulatory “buy-in” is a major performance metric to demonstrate the success of the ASTD effort.

Community involvement has been incorporated through a formal process by BGRR-DP. A series of roundtable meetings open to the general public were held to discuss overall project goals and schedules and solicit input. A standing BGRR Working Group consisting of volunteer community members has been established. The BGRR Working Group meets monthly to be briefed in depth on current BGRR decommissioning status and provide input on issues of concern to the public. The ASTD team has briefed this group on specific activities associated with the project.

8.0 SUMMARY

The versatility of the ISOCS system has been demonstrated in numerous situations during initial characterization and decommissioning efforts at the BGRR. Guidance from the MARSSIM and the Data Quality Objectives process provided direction for survey planning and data quality assessment. Surface soil detection sensitivities of less than 1 pCi/g have been attained with count times as short as 10 minutes for common gamma emitters such as Cs-137. Final results have been reported the same day, following data review and validation. Lower activities or more difficult to measure objects, such as enclosed systems, buried sources, and low-level surface contamination, can take longer to measure and evaluate. However, large surface areas or volumes with heterogeneous material distributions can be assayed with a single *in situ* measurement, thus saving time over other, more manual, methods, such as sampling and remote laboratory analysis.

8.1 Comparability Assessment

This assessment of the *in situ* data quality using the DQIs

has demonstrated that the ISOCS data quality can be comparable to definitive level laboratory analysis when the field instrument is supported by an appropriate Quality Assurance Project Plan developed using the DQO process. Analytical results were used to calculate data quality indicators (DQI) were accuracy, precision, and bias.

a. The analytical accuracy of the BNL ISOCS instrument, expressed in terms of the percent difference (%D) was demonstrated in several modes:

- for a point source, the %D was -2.1%,
- for an extended source, the %D was -10.2%, and
- for an intercomparison to other *in situ* gamma spectrometers, the %D was -5.0%.

b. The periodic re-measurement of a single source demonstrated the precision of the instrument over an extended period. The relative standard deviation was only $\pm 0.7\%$, a variation in the activity measurement indicating that the ISOCS system response is very stable.

c. The analytical precision of the ISOCS instrument was illustrated by the results of repeat analyses performed on soil samples. For a series of paired original and repeat analyses on 25 samples, all values of the relative percent difference were within the criteria of the Project QAPP (RPD < 20%). The largest values of RPD occurred when the activity was small (5~6 pCi/g or less), otherwise the RPD was less than 10% for the sample set.

d. Very strong correlation was demonstrated between the ISOCS and laboratory results for *ex situ* ($R^2 = 0.99$) and *in situ* ($R^2 = 0.98$) analyses.

e. Analysis results for *in situ* surface soils were usually lower than the corresponding laboratory sample values. This low response “bias” is actually a demonstration that the instruments are analyzing different “samples”: the *in situ* measurement looks at a wide area of soil in its native condition, while the laboratory analyzes a small aliquot, that is perturbed by preparation for analysis (drying and sieving). The bias in the laboratory values is usually very linear and can be adjusted to be more directly comparable to the *in situ* analysis, or vice versa.

8.2 Benefits

The successful implementation of this device provides many advantages over the traditional methodology, which is sampling followed by laboratory analysis.

a. Results are available nearly instantaneously, which allows better decisions to be made. The results can be used to guide the selection of the next measurement for a more complete survey without the necessity to re-mobilize the sampling crew. Or the results can be used to guide the conduct of a decontamination activity to know when to stop. The quick and reliable results are also very useful to advise interested members of the public and/or regulatory bodies on the extent of contamination and the effectiveness of the cleanup operation.

b. Most situations of radiological contamination do not result in uniform deposition of the offending material. Consequently, the selection of a small sample to send to the laboratory is a difficult and imprecise task. One solution is to take very large samples. And this is just what *in situ* measurement generally does. This large sample averages the non-homogeneity of the sample deposition over the entire object or area. Where the contamination on or in an object is not homogeneous, the ISOCS total activity results are probably more accurate than

conventional samples, since a very large fraction of the sample is measured. *In situ* minimum detection limits are generally as low or lower than laboratory samples, since a very large sample size is used.

c. In many situations taking samples is difficult and/or presents health and safety hazards. Common examples are contaminated concrete, activated steel, radioactive liquids, corrosive or high temperature fluids, dusts and powders, sludge on the bottom of a tank, tightly adhered surface contamination, gaseous samples, etc. In these cases once samples are successfully collected, they must be properly packaged and transported to the laboratory, where additional handling is required. *In situ* measurements can avoid this in many cases.

d. Conventional sampling and analysis is expensive. The cost comparison discussed in section 6, above, describes estimates of the cost of taking a sample, processing it, and laboratory spectroscopy. The same analysis is also provided for *in situ* gamma spectroscopy. The sampling/laboratory analysis process is approximately three times more expensive than *in situ* gamma spectroscopy, on a sample-to-sample basis. And, because of the non-uniform nature of the contamination in most cases, more samples must be taken than for *in situ* measurements.

8.3 Limitations

There are limitations on the implementation and use of the ISOCS system

a. The 30E FOV collimator can not be effectively used with the BEGe. The 30E FOV collimator is designed for use with a standard “co-axial” HPGe detector, and the wide, squat shape of the BEGe results in the 30E FOV collimator shielding over 80% of the active BEGe detector region. This limitation could be rectified by the manufacturer producing a re-designed collimator for use with the BEGe detector

b. Analysis results for *in situ* surface soils are usually lower than the corresponding laboratory sample values. The bias in the laboratory values is usually very linear and can be adjusted to be more directly comparable to the *in situ* analysis, or vice versa. Samples should be weighed before and after drying so that an adjustment for soil moisture can be made. In addition, the weight of any material, such as rocks or biomass, that are removed from the sample should also be recorded so that a similar

adjustment can be made. Without such adjustments, laboratory measurements are biased towards higher concentrations than actually exist at the sampling site. It is after all, the *in situ* concentration that is needed as input to a dose or risk model. The adjustment of laboratory results to be in line with physical site conditions is often ignored in an effort to be “conservative” in estimating risks. However, this practice should not be allowed to prejudice the comparison of *in situ* results to soil samples. Data Quality Objectives dictate that the measurement which is most closely related to risk should be preferred.

c. The use and application of the ISOCS geometry templates to generate instrument efficiency from the parameters of the spectrum acquisition can be daunting, especially for complex source objects. The system can be programmed to rapidly analyze recurring geometries, such as in the final status survey of remediated open fields. But when characterizing singular components, such as during a reactor decommissioning, the time to model individual pieces can add up quickly. In such cases it is the DQO process that can help to decrease the analysis setup time, by identifying less stringent precision or accuracy needs, allowing previous geometry models that are “close enough” to be used, without the necessity to refine the model to achieve unnecessary precision.

8.4 Conclusions

The ISOCS is an effective instrument for decontamination surveys, environmental measurements, operational radiation protection surveys, and waste assay measurements. This study demonstrated that *in situ* measurements can meet the QC acceptance criteria established by the project QAPP. Definitive level data may be generated by field instruments, as long as the project DQOs and QA/QC requirements are satisfied. The assessment of the total data quality in the study, and not just the instrument used in the analysis, will establish the confidence in the analysis and will determine the quality level of the data.

The success of this ASTD project is measured on several levels. First, the deployment of this innovative approach using *in situ* characterization, portable field laboratory measurements, and implementation of MARSSIM was successfully established for all three phases of D&D characterization, i.e., pre-job scoping, on-going disposition of waste, and final status surveys upon completion of the activity. Unlike traditional D&D projects, since the BGRR-DP is operating on an accelerated schedule, much of the work is being carried out simultaneously. Rather than complete a full characterization of the facility before D&D work begins,

specific removal actions require characterization as the activity progresses. Thus, the need for rapid and cost-effective techniques for characterization is heightened.

Secondly, since the approach used for this ASTD project was not thoroughly proven prior to deployment, a large effort was devoted to demonstrating technical comparability to project managers, regulators and stakeholders. During the initial phases, large numbers of replicate samples were taken and analyzed by conventional baseline techniques to ensure that BGRR-DP quality assurance standards were met. ASTD project staff prepared comparisons of data gathered using ISOCS and BetaScint with traditional laboratory methods and presented this information to BGRR-DP staff and regulators from EPA Region II, NYS Department of Environmental Conservation, and the Suffolk County Board of Health. As the results of comparability evaluations became available, approval for these methods was received and the techniques associated with *in situ* characterization, portable field laboratory measurements, and implementation of MARSSIM were gradually integrated into BGRR-DP procedures.

Clearly, a significant metric for success is cost effectiveness. Integration of this approach and displacement of traditional baseline characterization at BGRR-DP resulted in far fewer samples that needed to be taken, packaged for transport to an off-site laboratory and analyzed. In addition, because samples are analyzed rapidly on-site, cost per sample is drastically reduced. For example, cost per sample for ISOCS measurements was calculated to be \$76, compared with \$252 per sample for conventional measurement, a savings of \$175 per sample. Discounting programmatic costs, the relative cost of deploying ISOCS for BGRR characterization in FY 99 and FY 00 was \$82,000. The cost for equivalent conventional baseline characterization was estimated to range from a minimum of \$292,000 to a maximum of \$1,075,000 depending on the number of analyses displaced. This represents net cost savings ranging from \$210,000 to \$993,000. If the total ASTD project cost associated with ISOCS deployment is included, the maximum cost savings range from -\$339,000 (loss) to \$443,000 (gain). While both the cost per sample and overall costs are significantly lower using ISOCS *in situ* and field laboratory characterization, perhaps the greatest cost savings associated with this innovative technology are “hidden” savings that are more difficult to quantify. These include savings associated with project schedule acceleration, the ability to characterize non-standard systems, and improved health and safety for D&D workers.

Finally, the most significant criterion for success of this ASTD effort is associated with continued deployment as part of the BGRR-DP and other projects at BNL as well as additional deployments at sites throughout the DOE complex. Investment in the initial ASTD project will continue to see dividends as the inherent cost savings continue to be realized. For example, based on proven success at BGRR (both technical and reduced cost), the BNL Environmental Restoration Division (BNL ERD) deployed ISOCS for cleanup of a large volume of contaminated landscape soils. ASTD team members provided both guidance and technical support for this deployment. In FY 01, BNL is engaged in another ASTD

project to characterize subsurface soils beneath structures at the BGRR. Both ISOCS and BetaScint have been incorporated into the project plan for that activity, facilitated through cost savings associated with this project. In addition, ISOCS and MARSSIM are currently being deployed for characterization of the Canyon Disposition Initiative at Hanford, based partly on support from this activity. A new effort to deploy ISOCS and MARSSIM is underway at the Nevada Test Site, as part of a large effort to D&D their former nuclear rocket test facilities. Based on expertise developed through this ASTD project, BNL is providing technical support and technology transfer assistance to DOE NV.

9.0 ACKNOWLEDGMENTS

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- 17 U. S. EPA, *Data Quality Objectives for Remedial Response Activities -- Development Process*, EPA/540/G-87/003, U. S. Environmental Protection Agency, Washington, DC. March 1987.

APPENDICES

APPENDIX A. Summary List of Project Measurements and Scans

Table A-1. ISOCS *Ex Situ* Sample Measurements

Sample Description	No. of Samples	Sample Description	No. of Samples
AGD Concrete dust from cutting down comers	3	AGD Concrete, Sludge, and Debris	22
AGD Cutting residuals	5	AGD North Duct Joint	6
AGD South Duct Joint	6	AGD Water from cutting down comers	3
Animal Tunnel East Debris	1	Background/Calibration/Source check	13
BGD and Cooler Drain Sludges	6	BGD Cooler Coils	3
BGD Filter Media	3	Canal Core Borehole Soil	78
Canal Debris	4	Canal House drill shavings	1
Canal Joint	1	Canal Sump	4
Canal Walk-way East	1	Canal Walk-way South	9
ERD Landscape Soil (pre-excavation surveys)	10	ERD Landscape Soil (post-excavation surveys)	51
ERD Soil from Sewage Treatment Plant	8	Fan House 1 Soil	1
Fuel Channel Extraction Tool	1	Graphite Plug stud section	1
PFS Asphalt (pre-excavation)	6	PFS Drainline Soil	7
PFS Drainline Soil and asphalt (post-excavation)	12	PFS Drainline Soil and asphalt (pre-excavation)	4
PFS Excavation Soil (during and post excavation)	606	PFS Soil (pre-excavation)	5
Soil for BetaScint Demo	30	Water Treatment House Debris	8
Water Treatment House Debris (west)	3		
TOTAL	922	NON-SOIL TOTAL	105
		SOIL TOTAL	817

Acronyms:

AGD = Above Ground Ducts

PFS = Pile Fan Sump

BGD = Below Ground Ducts

WMD = Waste Management Division

ERD = Environmental Restoration Division

Table A-2. ISOCS In Situ Scan Measurements

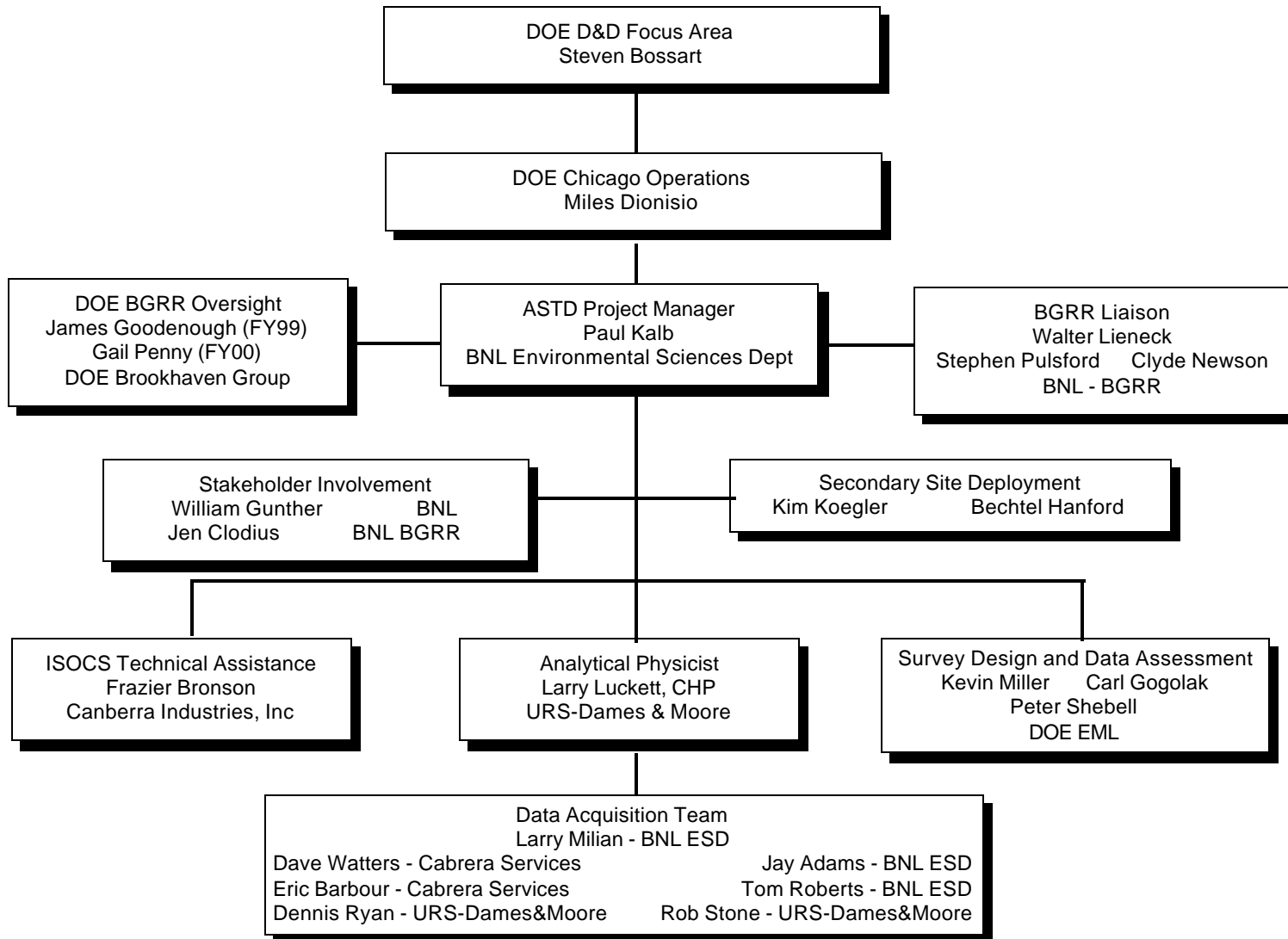
Scan Location	No. of Scans	Category	Sample Location	No. of Scans	Category
AGD (N&S hatch)	6	a-1	AGD Concrete Debris (from demob)	2	b-1
AGD (roof hatch)	3	a-1	Animal Tunnel East Debris	1	b-1
Canal House	1	a-1	Canal Sump Debris	3	b-1
ERD Waste Container	5	a-1	Canal walkway sludge	2	b-1
Experimental Port E23 (w cover)	2	a-1	ERD Chem Holes lead	1	b-1
Experimental Port W15 (w cover)	2	a-1	ERD C-magnets	11	b-1
Experimental Port E23	3	a-1	PFS cover closed	3	b-1
Experimental Port E30	2	a-1	PFS cover open	3	b-1
Experimental Port E24	3	a-1	Reactor sump cover	1	b-1
Experimental Port E26	3	a-1	TOTAL Category b-1	27	
Experimental Port N5	2	a-1	Canal Sump Debris (south bottom gate)	1	c-1
Experimental Port N6	2	a-1	Control Rod Guide Mechanism (SE)	2	c-1
Experimental Port N8	2	a-1	Control Rod Guide Mechanism (SW)	2	c-1
Experimental Port W12	1	a-1	Experimental Port W12 Graphite Debris	3	c-1
Experimental Port W12 (w cover)	1	a-1	Experimental Port W16 Debris	1	c-1
Experimental Port W15	1	a-1	Fan House 1	1	c-1
Experimental Port W16	1	a-1	Fan House 3	3	c-1
Experimental Port W30	1	a-1	Fan House 5	2	c-1
Experimental Port W31	2	a-1	Filter Bank media	4	c-1
Experimental Port W36	2	a-1	Glass block	1	c-1
Experimental Port W5	2	a-1	Graphite from Port 12	4	c-1
Experimental Port W51	2	a-1	Graphite from Port 30	5	c-1
Experimental Port W54	2	a-1	Graphite from Port 42	6	c-1
Experimental Port W56	3	a-1	Graphite from Ports 31 & 36	5	c-1
Northwest Side Scanner Slot	6	a-1	Rad Waste Bags (BLIP)	9	c-1
South Scanner Slot Samples (SE)	4	a-1	TOTAL Category c-1	49	
South Scanner Slot Samples (SW)	6	a-1	Excavated asphalt	11	c-2
Water Treatment House East	2	a-1	TOTAL Category c-2	11	
WMD Waste Box 1	4	a-1	AGD plug	1	d-1
WMD pig	9	a-1	AGD Sidewall Concrete Cores	4	d-1
WMD Waste Box 2 (bldg938)	3	a-1	Bioshield graphite plug	2	d-1
TOTAL Category a-1	88		Experimental Port W16 Graphite Powder	1	d-1
701 Parking lot	2	a-2	Fan House 3 dust	3	d-1
Agricultural Field Intercomparison Study	15	a-2	Fan House 5 dust	3	d-1
Bldg 703 floors	6	a-2	Instrument Tunnel East Debris	1	d-1
ERD Landscape soil	2	a-2	Interior Pile Debris (Port W32)	12	d-1
ERD lanscape soil (phyto)	8	a-2	Interior Pile Graphite Dust and Shavings	9	d-1
Fan House soils	27	a-2	North Scanner Slot Samples	14	d-1
PFS pipeline soil	59	a-2	PFS sludge	1	d-1
WMD Liquid Waste Tanker	5	a-2	TOTAL Category d-1	51	
TOTAL Category a-2	124		ERD Phyto plant	2	d-2
TOTAL Category b-2	0		TOTAL Category d-2	2	

ALL CATEGORIES TOTAL 352 RAD CONS TOTAL 137 NON RAD CON AREAS TOTAL 215 AREA

- a-1) large uniform area (homogeneity assumed). Controlled area entry for baseline sampling
- a-2) large uniform area (homogeneity assumed). No controlled area entry for baseline sampling
- b-1) small uniform area (homogeneity assumed). Controlled area entry for baseline sampling
- b-2) small uniform area (homogeneity assumed). No controlled area entry for baseline sampling
- c-1) large heterogeneous solid. Controlled area entry for baseline sampling
- c-2) large heterogeneous solid. No controlled area entry for baseline sampling
- d-1) small heterogeneous solid. Controlled area entry for baseline sampling
- d-2) small heterogeneous solid. No controlled area entry for baseline sampling

APPENDIX B. Project Participants

Accelerated Site Technology Deployment (ASTD) Project Organization Chart



APPENDIX C. Cost Analysis Data Tables

Table C-1
WBS for Baseline *Ex Situ* Sampling and Analyses
Soil Samples

	Labor Hours	Rate	Materials Quantity	Rate	Total Unit Cost	Total Quantity	Unit of Measure	Total Cost	Comment
Mobilization									
Transport equipment to work area	0.5	\$45			\$22.50	30	work days	\$675	
Prepare equipment for use	0.5	\$45			\$22.50	30	work days	\$675	
Characterization									
Set up and move equipment (auger, geoprobe, etc)	0.1	\$45				\$4.50	817		\$3,677
Collect sample	0.1	\$45			\$4.50	817		\$3,677	
Decon equip for next sampling	0.2	\$45			\$9.00	817		\$7,353	
Package Sample	0.1	\$45			\$4.50	817		\$3,677	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	817		\$9,191	
Transport, Analyze, Dispose of samples	0	\$0			\$146	817		\$119,282	(a)
Review/Evaluate Data	0.1	\$75			\$7.50	817		\$6,128	(b)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	30	work days	\$1,125	
Containers	0	\$0	817	\$6			each	\$4,902	
PPE	0		2	per day	\$50	30	work days	\$3,000	
Productivity Loss Factor (PLF)	0								(c)
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	30	work days	\$675	
Transport equipment to storage area	0.5	\$45			\$22.50	30	work days	\$675	
								TOTAL:	\$164,710
								Cost/sample:	\$201.60

Notes:

- (a) Avg cost for off-site gamma analyses (14d turnaround) is \$121 + \$25 shipping/handling = \$146
- (b) One CHP evaluates 10 data sets/hr
- (c) Adjusts for changes, breaks, respiratory protection, and ALARA
 (PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)

Table C-2**WBS for ISOCS *Ex Situ* Sampling and Analyses****Soil Samples**

	Labor Hours	Rate	Materials Quantity	Rate	Total Unit Cost	Total Quantity	Unit of Measure	Total Cost	Comment
Mobilization									
Transport sampling equipment to work area	0.5	\$45			\$22.50	30	work days	\$675	(a)
Prepare sampling equipment for use	0.5	\$45			\$22.50	30	work days	\$675	
ISOCS Quality Assurance Procedures	0.5	\$45			\$22.50	30	work day	\$675	(b)
Equipment Maintenance	0.5	\$45			\$22.50	6	week	\$135	(c)
ISOCS Liquid Nitrogen				\$0.90		30	work day	\$27	
Characterization									
Set up and move equipment (auger, geoprobe, etc)	0.1	\$45				\$4.50	817	each	\$3,677
Collect sample	0.1	\$45			\$4.50	817	each	\$3,677	
Decon equip for next sampling	0.2	\$45			\$9.00	817	each	\$7,353	
Package Sample	0.1	\$45			\$4.50	817	each	\$3,677	(d)
Analyze samples at ISOCS field lab	0.13	\$75			\$9.75	817	each	\$7,966	
Archive Files/Print Data	0.05	\$75			\$3.75	817	each	\$3,064	(e)
Review/Evaluate Data	0.05	\$75			\$3.75	817	each	\$3,064	
Daily Project/Safety Briefing	0.5	\$75			\$37.50	30	work days	\$1,125	
Containers	0	\$0	817	\$6			each	\$4,902	
PPE	0		2	per day	\$50	30	work days	\$3,000	(f)
Productivity Loss Factor (PLF)									
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	30	work days	\$675	
Transport equipment to storage area	0.5	\$45			\$22.50	30	work days	\$675	
							TOTAL:	\$45,040	
							cost/sample:	\$55.13	

Notes:

- (a) Includes sampling costs - ISOCS assumed to be installed in field lab
- (b) Daily calibration source and background check
- (c) Fill cryostat with liquid nitrogen
- (d) Assumes 5 min count time + 3 min set up
- (e) One CHP evaluates 20 data sets/hr
- (f) Adjusts for changes, breaks, respiratory protection, and ALARA

PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)

Table C-3**WBS for Baseline *Ex Situ* Sampling and Analyses****Debris, Sludge, Misc. Samples**

	Labor Hours	Rate	Materials Quantity	Rate	Total Unit Cost	Total Quantity	Unit of Measure	Total Cost	Comment
Mobilization									
Prepare area for sampling	0.5	\$45			\$22.50	10	work days	\$225	(a)
Characterization									
Collect sample	0.25	\$45			\$11.25	105	each	\$1,181	
Decon equip for next sampling	0.2	\$45			\$9.00	105	each	\$945	
Package Sample	0.1	\$45			\$4.50	105	each	\$473	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	105	each	\$1,181	(b)
Transport, Analyze, Dispose of samples	0	\$0			\$146	105	each	\$15,330	(c)
Review/Evaluate Data	0.1	\$75			\$7.50	105	each	\$788	
Daily Project/Safety Briefing	0.5	\$75			\$37.50	10	work days	\$375	
Containers	0	\$0	105	\$6			each	\$630	
PPE	0		2	per day	\$50	10	work days	\$1,000	
Productivity Loss Factor (PLF)	0.885	\$45			\$39.83	105	each	\$4,182	(d)
Demobilization									
Survey and Decon equipment	1	\$45			\$45.00	10	work days	\$450	
								TOTAL:	\$26,759
								Cost/sample:	\$254.85

Notes:

- (a) Prepare buffer area and assemble sampling equipment; assume 10 samples/day rate
- (b) Avg cost for off-site gamma analyses (14d turnaround) is \$121 + \$25 shipping/handling = \$146
- (c) One CHP evaluates 10 data sets/hr
- (d) Adjusts for changes, breaks, respiratory protection, and ALARA
(PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)

Table C-4**WBS for ISOCS *Ex Situ* Sampling and Analyses****Debris, Sludge, and Misc. Samples**

	Labor Hours	Rate	Materials Quantity	Rate	Total Unit Cost	Total Quantity	Unit of Measure	Total Cost	Comment
Mobilization									
Prepare area for sampling	0.5	\$45			\$22.50	10	work days	\$225	(a)
ISOCS Quality Assurance Procedures	0.5	\$45			\$22.50	10	work day	\$225	(b)
Equipment Maintenance	0.5	\$45			\$22.50	2	week	\$45	(c)
ISOCS Liquid Nitrogen				\$0.90		10	work day	\$9	
Characterization									
Collect sample	0.25	\$45			\$11.25	105	each	\$1,181	
Package Sample	0.1	\$45			\$4.50	105	each	\$473	
Analyze samples at ISOCS field lab	0.13	\$75			\$9.75	105	each	\$1,024	(d)
Archive Files/Print Data	0.05	\$75			\$3.75	105	each	\$394	
Review/Evaluate Data	0.05	\$75			\$3.75	105	each	\$394	(e)
Daily Project/Safety Briefing	0.5	\$45			\$22.50	10	work day	\$225	
Containers	0	\$0	105	\$6			each	\$630	
PPE	0		2	per day	\$50	10	work days	\$1,000	
Productivity Loss Factor (PLF)	0.885	\$45			\$39.83	105	each	\$4,182	(f)
Demobilization									
Survey and Decon equipment	1	\$45			\$45.00	10	work days	\$450	
								TOTAL:	\$10,456
								cost/sample:	\$99.58

Notes:

- (a) Prepare buffer area and assemble sampling equipment; assume 10 samples/day rate
- (b) Daily calibration source and background check
- (c) Fill cryostat with liquid nitrogen
- (d) Assumes 5 min count time + 3 min set up
- (e) One CHP evaluates 20 data sets/hr
- (f) Adjusts for changes, breaks, respiratory protection, and ALARA
(PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)

Table C-5**WBS for Baseline Sampling and Analyses in place of *In Situ* ISOCS for All Radiologically Controlled Areas;
Maximum Cost Savings Assumptions**

(Assuming 10:1 ratio of Baseline Sampling to ISOCS for Large Areas and 5:1 Ratio for Small Areas)

	Labor Hours	Rate	Materials Quantity	Rate	Total Unit Cost	Total Quantity	Unit of Measure	Total Cost	Comment
Mobilization									
Transport equipment to work area	0.5	\$45			\$22.50	440	work days	\$9,900	(a)
Prepare equipment for use	0.5	\$45			\$22.50	440	work days	\$9,900	
Characterization									
Set up and move equipment (auger, geoprobe, etc)		0.25	\$45			\$11.25	1760	each	\$ 1 9 , 8 0 0
	(b)								
Collect sample	0.25	\$45			\$11.25	1760	each	\$19,800	
Decon equip for next sampling	0.2	\$45			\$9.00	1760	each	\$15,840	
Package Sample	0.25	\$45			\$11.25	1760	each	\$19,800	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	1760	each	\$19,800	
Transport, Analyze, Dispose of samples	0	\$0			\$146	1760	each	\$256,960	(c)
Review/Evaluate Data	0.1	\$75			\$7.50	1760	each	\$13,200	(d)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	440	work days	\$16,500	
Containers	0	\$0		\$6		1760	each	\$10,560	
PPE	0		2	per day	\$50	440	work days	\$44,000	
Productivity Loss Factor (PLF)	1.27	\$45			\$57.15	1760	each	\$100,584	(e)
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	440	work days	\$9,900	
Transport equipment to storage area	0.5	\$45			\$22.50	440	work days	\$9,900	
							Total:	\$576,444	
							Cost/sample:	\$328	

Notes:

- (a) With 4.5 hrs for set up, prep, PL factor, survey, decon & demob, Assume 4 samples/ day; 440 work days required
- (b) Assuming 10 conventional baseline samples required for each ISOCS scan of large homogeneous areas; 5 conventional baseline samples required for each ISOCS scan of small homogeneous areas
- (c) Avg cost for off-site gamma analyses (14d turnaround) is \$121 + \$25 shipping/handling = \$146
- (d) One CHP evaluates 10 data sets/hr
- (e) Adjusts for changes, breaks, respiratory protection, and ALARA
(PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)
[This analysis assumes no compositing of analytical samples]

Table C-6**WBS for Baseline Sampling and Analyses in place of *In Situ* ISOCS for All Radiologically Controlled Areas;****Minimum Cost Savings Assumptions**

(Assuming 1:1 ratio of Baseline Sampling to ISOCS for All Samples)

	Labor		Materials		Total	Total	Unit of	Total	Comment
	Hours	Rate	Quantity	Rate	Unit Cost	Quantity	Measure	Cost	
Mobilization									
Transport equipment to work area	0.5	\$45			\$22.50	54	work days	\$1,215	(a)
Prepare equipment for use	0.5	\$45			\$22.50	54	work days	\$1,215	
Characterization									
Set up and move equipment (auger, geoprobe, etc)	0.25	\$45				\$11.25	215	each	\$2,419
Collect sample	0.25	\$45			\$11.25	215	each	\$2,419	
Decon equip for next sampling	0.2	\$45			\$9.00	215	each	\$1,935	
Package Sample	0.25	\$45			\$11.25	215	each	\$2,419	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	215	each	\$2,419	
Transport, Analyze, Dispose of samples	0	\$0			\$146	215	each	\$31,390	(b)
Review/Evaluate Data	0.1	\$75			\$7.50	215	each	\$1,613	(c)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	54	work days	\$2,025	
Containers	0	\$0	77	\$6		215	each	\$462	
PPE	0		2	per day	\$50	54	work days	\$5,400	
Productivity Loss Factor (PLF)	1.27	\$45			\$57.15	215	each	\$12,287	(d)
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	54	work days	\$1,215	
Transport equipment to storage area	0.5	\$45			\$22.50	54	work days	\$1,215	
							Total:	\$69,647	
							Cost/sample:	\$324	

Notes:

- (a) With 4.5 hrs for set up, prep, PL factor, survey, decon & demob, Assume 4 samples/ day; 54 work days required
- (b) Avg cost for off-site gamma analyses (14d turnaround) is \$121 + \$25 shipping/handling = \$146
- (c) One CHP evaluates 10 data sets/hr
- (d) Adjusts for changes, breaks, respiratory protection, and ALARA
(PLF is calculated by multiplying time spent applied to work within controlled areas by the adjustment factor of 27%)

Table C-7**WBS for *In Situ* ISOCS Sampling and Analyses of All Radiologically Controlled Areas**

	Labor		Materials		Total	Total	Unit of	Total	Comment
	Hours	Rate	Quantity	Rate	Unit Cost	Quantity	Measure	Cost	
Mobilization									
Transport ISOCS equipment to work area	0.5	\$45			\$22.50	27	work day	\$608	(a)
Prepare equipment for use	0.5	\$45			\$22.50	27	work day	\$608	
Quality Assurance Procedures	0.5	\$45			\$22.50	27	work day	\$608	(b)
Equipment Maintenance	0.5	\$45			\$22.50	6	week	\$135	(c)
Liquid Nitrogen				\$0.90		27	work day	\$24	
Characterization									
Set up and move equipment (ISOCS)	0.1	\$75			\$7.50	215	each	\$1,613	
Acquire data	0.25	\$75			\$18.75	215	each	\$4,031	(d)
Model Data	0.25	\$75			\$18.75	215	each	\$4,031	
Archive Files/Print Data	0.05	\$75			\$3.75	215	each	\$806	
Review/Evaluate Data	0.05	\$75			\$3.75	215	each	\$806	(e)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	27	work day	\$1,013	
PPE	0		0	per day	\$50	0		\$0	
Productivity Loss Factor (PLF)	0	\$45			\$0	0		\$0	(f)
Demobilization									
Equipment Disassembly	0.5	\$45			\$22.50	27	work day	\$608	
Transport equipment to storage area	0.5	\$45			\$22.50	27	work day	\$608	
								Total:	\$15,497
								Cost/sample:	\$72

Notes:

- (a) With 2.5 hrs for set up, prep, & demob, Assume 8 samples/ day; 27 work days required
- (b) Daily calibration source and background check
- (c) Fill cryostat with liquid nitrogen
- (d) Assumes 15 min count time
- (e) One CHP evaluates 20 data sets/hr
- (f) Adjusts for changes, breaks, respiratory protection, and ALARA
Assumes ISOCS data acquisition is conducted from outside controlled area

Table C-8**WBS for Baseline Sampling and Analyses in place of *In Situ* ISOCS for all Non-Radiologically Controlled Areas****Maximum Cost Savings Assumptions**

(Assuming 10:1 ratio of Baseline Sampling to ISOCS for Large Areas and 5:1 Ratio for Small Areas)

	Labor		Materials		Total	Total	Unit of	Total	Comment
	Hours	Rate	Quantity	Rate	Unit Cost	Quantity	Measure	Cost	
Mobilization									
Transport equipment to work area	0.5	\$45			\$22.50	227	work days	\$5,108	(a)
Prepare equipment for use	0.5	\$45			\$22.50	227	work days	\$5,108	
Characterization									
Set up and move equipment (auger, geoprobe, etc)		0.25	\$45			\$11.25	1360	each	\$ 1 5 , 3 0 0
	(b)								
Collect sample	0.25	\$45			\$11.25	1360	each	\$15,300	
Package Sample	0.25	\$45			\$11.25	1360	each	\$15,300	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	1360	each	\$15,300	
Transport, Analyze, Dispose of samples	0	\$0			\$146	1360	each	\$198,560	
Review/Evaluate Data	0.1	\$75			\$7.50	1360	each	\$10,200	(c)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	227	work days	\$8,513	
Containers	0	\$0		\$6		1360	each	\$8,160	
PPE	0		0	per day	\$50	0		\$0	
Productivity Loss Factor (PLF)	0	\$45			\$0	0		\$0	(d)
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	227	work days	\$5,108	
Transport equipment to storage area	0.5	\$45			\$22.50	227	work days	\$5,108	
							Total:	\$307,063	
							Cost/sample:	\$226	

Notes:

- (a) With 1.5 hrs for set up, prep, decon & demob, Assume 6 samples/ day; 227 work days required
- (b) Assuming 10 conventional baseline samples required for each ISOCS scan of large homogeneous areas; 5 conventional baseline samples required for each ISOCS scan of small homogeneous areas
- (c) One CHP evaluates 10 data sets/hr
- (d) No Productivity Loss Factor for work in Non-Radiologically Controlled areas

Table C-9**WBS for Baseline Sampling and Analyses in place of *In Situ* ISOCS for all Non-Radiologically Controlled Areas****Minimum Cost Savings Assumptions**

(Assuming 1:1 ratio of Baseline Sampling to ISOCS for All Samples)

	Labor		Materials		Total	Total	Unit of	Total	Comment
	Hours	Rate	Quantity	Rate	Unit Cost	Quantity	Measure	Cost	
Mobilization									
Transport equipment to work area	0.5	\$45			\$22.50	23	work days	\$518	(a)
Prepare equipment for use	0.5	\$45			\$22.50	23	work days	\$518	
Characterization									
Set up and move equipment (auger, geoprobe, etc)	0.25	\$45				\$11.25	137	each	\$1,541
Collect sample	0.25	\$45			\$11.25	137	each	\$1,541	
Package Sample	0.25	\$45			\$11.25	137	each	\$1,541	
Prepare shipment for off-site analyses	0.25	\$45			\$11.25	137	each	\$1,541	
Transport, Analyze, Dispose of samples	0	\$0			\$146	137	each	\$20,002	
Review/Evaluate Data	0.1	\$75			\$7.50	137	each	\$1,028	(b)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	23	work days	\$863	
Containers	0	\$0		\$6		137	each	\$822	
PPE	0		0	per day	\$50	0		\$0	
Productivity Loss Factor (PLF)	0	\$45			\$0	0		\$0	(c)
Demobilization									
Survey and Decon equipment	0.5	\$45			\$22.50	23	work days	\$518	
Transport equipment to storage area	0.5	\$45			\$22.50	23	work days	\$518	
							Total:	\$30,949	
							Cost/sample:	\$226	

Notes:

- (a) With 1.5 hrs for set up, prep, decon & demob, Assume 6 samples/ day; 23 work days required
- (b) One CHP evaluates 10 data sets/hr
- (c) No Productivity Loss Factor for work in Non-Radiologically Controlled areas

Table C-10**WBS for *In Situ* ISOCS Sampling and Analyses of All Non-Radiologically Controlled Areas**

	Labor		Materials		Total	Total	Unit of	Total	Comment
	Hours	Rate	Quantity	Rate	Unit Cost	Quantity	Measure	Cost	
Mobilization									
Transport ISOCS equipment to work area	0.5	\$45			\$22.50	23	work days	\$518	(a)
Prepare equipment for use	0.5	\$45			\$22.50	23	work days	\$518	
Quality Assurance Procedures	0.5	\$45			\$22.50	23	work days	\$518	(b)
Equipment Maintenance	0.5	\$45			\$22.50	5	work week	\$113	(c)
Liquid Nitrogen				\$0.90		23	work days	\$21	
Characterization									
Set up and move equipment (ISOCS)	0.1	\$75			\$7.50	137	each	\$1,028	
Acquire data	0.25	\$75			\$18.75	137	each	\$2,569	(d)
Model Data	0.25	\$75			\$18.75	137	each	\$2,569	
Archive Files/Print Data	0.05	\$75			\$3.75	137	each	\$514	
Review/Evaluate Data	0.05	\$75			\$3.75	137	each	\$514	(e)
Daily Project/Safety Briefing	0.5	\$75			\$37.50	23	work days	\$863	
PPE	0		0	per day	\$50	0		\$0	
Productivity Loss Factor (PLF)	0	\$45			\$0	0		\$0	(f)
Demobilization									
Equipment Disassembly	0.5	\$45			\$22.50	23	work days	\$518	
Transport equipment to storage area	0.5	\$45			\$22.50	23	work days	\$518	
								Total:	\$10,776
								Cost/sample:	\$79

Notes:

- a) With 3 hrs for set up, meeting, prep, & demob, Assume 6 samples/ day; 23 work days required
- b) Daily calibration source and background check
- c) Fill cryostat with liquid nitrogen
- d) Assumes 15 min count time
- e) One CHP evaluates 20 data sets/hr
- f) Adjusts for changes, breaks, respiratory protection, and ALARA
Assumes ISOCS data acquisition is conducted from outside controlled area

APPENDIX D.

Glossary

GLOSSARY

accuracy is a measure of the closeness of an individual measurement or the average of a number of measurements to the true value. Accuracy includes a combination of random error (precision) and systematic error (bias) components that result from sampling and analytical operations.

bias - the systematic or persistent distortion of a measurement process that causes errors in one direction (i.e., the expected sample measurement is different from the sample's true value).

boundaries - the spatial and temporal conditions and practical constraints under which environmental data are collected. Boundaries specify the area of volume (spatial boundary) and the time period (temporal boundary) to which a decision will apply.

comparability is the qualitative term that expresses the confidence that two data sets can contribute to a common analysis and interpolation. Comparability must be carefully evaluated to establish whether two data sets can be considered equivalent in regard to the measurement of a specific variable or groups of variables.

completeness is a measure of the amount of valid data obtained from a measurement system, expressed as a percentage of the number of valid measurements that should have been collected (i.e., measurements that were planned to be collected).

data quality assessment (DQA) - a statistical and scientific evaluation of the data set to determine the validity and performance of the data collection design and statistical test, and to determine the adequacy of the data set for its intended use.

data quality objectives (DQOs) - qualitative and quantitative statements derived from the DQO Process that clarify study objectives, define the appropriate type of data, and specify tolerable levels of potential decision errors that will be used as the basis for establishing the quality and quantity of data needed to support decisions.

data quality objectives process is a systematic planning tool to facilitate the planning of environmental data collection activities. Data quality objectives are the qualitative and quantitative outputs from the DQO Process.

error is the difference between the true value and the measured value of a quantity or parameter.

false acceptance decision error - the error that occurs when a decision maker accepts the baseline condition when it is actually false. Statisticians usually refer to the limit on the

possibility of a false acceptance decision error as beta (β) and it is related to the power of the statistical test used in decision making. An alternative name is false negative decision error.

false negative decision error - see false acceptance decision error.

false positive decision error - see false rejection decision error.

false rejection decision error - the error that occurs when a decision maker rejects the baseline condition (null hypothesis) when it actually is true. Statisticians usually refer to the limit on the possibility of a false rejection decision error as alpha, (α), the level of significance, or the size of the critical region, and it is expressed numerically as a probability. An alternative name is false positive decision error.

matrix is the predominant material of which the sample to be analyzed is composed. Matrix is not synonymous with phase (liquid or solid).

percent difference (%D) is used to compare two values; the percent difference indicates both the direction and the magnitude of the comparison, i.e., the percent difference may be either negative, positive, or zero. (In contrast, see relative percent difference.)

performance-based measurement system - a process in which the data quality needs or limitations of a program or project are specified and serve as a criterion for selecting appropriate analytical methods. Under the PBMS framework, the performance of the method employed is emphasized rather than the specific technique or procedure used in the analysis.

precision - a measure of mutual agreement among individual measurements of the same property, usually under prescribed similar conditions expressed generally in terms of the standard deviation. It may also be expressed as a percentage of the mean of the measurements, such as relative range (RR) (for duplicates) or relative standard deviation (RSD).

productivity loss factor (PLF) is an historically based estimate of the non-productive portion of the work day due to PPE changes, work rules based on As Low As Reasonably Achievable considerations, additional work breaks, etc, when working in an area of radioactive contamination.

quality assurance (QA) - an integrated system of management activities involving planning, implementation, documentation, assessment, reporting, and quality improvement to ensure that a process, item, or service is of the type and quality needed and expected by the customer.

QA Project Plan (QAPP) - a document describing in comprehensive detail the necessary quality assurance, quality control, and other technical activities that should be implemented to ensure that the results of the work performed will satisfy the stated performance criteria.

quality control (QC) - the overall system of technical activities that measure the attributes and performance of a process, item, or service against defined standards to verify that they meet the stated requirements established by the customer; operational techniques and activities that are used to fulfill requirements for quality.

random errors vary in a non-reproducible way around the limiting mean. These errors can be treated statistically by use of the laws of probability.

relative percent difference (RPD) - used to compare two values, the relative percent difference is based on the mean of the two values, and is reported as an absolute value, i.e., always expressed as a positive number or zero. In contrast, see percent difference.

representativeness is a measure of the degree to which data accurately and precisely represent a characteristic of a population parameter at a sampling point or for a process condition or environmental condition. Representativeness is

a qualitative term that should be evaluated to determine whether *in situ* and other measurements are made and physical samples collected in such a manner that the resulting data appropriately reflect the media and phenomenon measured or studied.

support - the support of a physical sample is the volume from which an individual sample is drawn. For a grab sample the physical support is exactly equal to the size of the physical sample. Arises when assessing the representativeness of results for a heterogenous population or distribution.

systematic errors are errors that are reproducible and tend to bias a result in one direction. Their causes can be assigned, at least in principle, and they can have both constant and variable components. Generally, these errors cannot be treated statistically.

type I error - the statistical term for false rejection decision error.

type II error - the statistical term for false acceptance decision error.

uncertainty is the range of values within which the true value is estimated to lie. It is a best estimate of possible inaccuracy due to both random and systematic errors.

Section 4

COMPARABILITY OF BETASCINT™ INSTRUMENT IN Sr-90 CHARACTERIZATION AT BROOKHAVEN NATIONAL LABORATORY*

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Abstract

This report describes a DOE Accelerated Site Technology Deployment project being conducted at Brookhaven National Laboratory to deploy innovative, radiological, *in situ* analytical techniques. The technologies are being deployed in support of efforts to characterize the Brookhaven Graphite Research Reactor (BGRR) facility, which is currently undergoing decontamination and decommissioning.

This report focuses on the deployment of the BetaScint Industries Strontium-90 (Sr-90) Spectrometer and assesses its data comparability to baseline methods of sampling and laboratory analysis. The BetaScint™ system consists of a multi-layer beta scintillation detector array, with a beta radiation entrance window measuring 30-cm by 60-cm. Soil samples are prepared, transferred to large area counting trays, and positioned beneath the system entrance window for analysis. Beta particles that pass through the entrance window will excite electrons in the scintillating ribbons resulting in the emission of light pulses, which are counted by photomultiplier tubes. Concentration of Sr-90 in the soil (pCi/g) can be determined in minutes using the field instrument, providing rapid, reliable analysis results and enabling expedited decisions on the remediation status.

The Project used the *Multi-Agency Radiation Survey and Site Investigation Manual* guidance and the Data Quality Objectives process to provide direction for survey planning and data quality assessment. Analytical results have been used to calculate data quality indicators (DQI) for the BetaScint measurements. Among the DQIs assessed in the report are sensitivity, accuracy, precision, bias, and influence of soil moisture. The assessment of the data quality using the DQIs demonstrates that the BetaScint instrument data quality can be comparable to definitive level laboratory analysis when the field instrument is supported by an appropriate Quality Assurance Project Plan.

*This work was performed under the auspices of the U.S. Department of Energy.

**COMPARABILITY OF THE BETASCINT™ INSTRUMENT
IN SR-90 CHARACTERIZATION
AT BROOKHAVEN NATIONAL LABORATORY**

Current Status

Substantial soil excavation efforts will be required to complete the D&D of many BGRR systems. Based on previous evaluations, the primary potential radionuclide contaminants in these soils are cesium-137 (Cs-137) and strontium-90 (Sr-90). In some areas these contaminants are surficially deposited, while in other locations, such as soils beneath failed underground piping, contamination is expected in subsurface soils covered by clean soil overburden. The relative distribution of Cs-137 and Sr-90 in subsurface soils has been found to vary considerably; Sr-90 contamination has been identified in soils deeper than Cs-137 in many locations. This is due to the high mobility of strontium in soil relative to cesium, which often results in an increase in the ratio of Sr-90 to Cs-137 contamination levels with depth, that is it makes it difficult to establish a ratio to use in quantifying Sr-90 by measuring Cs-137.

Soil contaminated above site-specific derived concentration guideline levels (DCGLs) will be removed during D&D of contaminated BGRR underground piping and systems. During such remediation actions, rapid and accurate quantification of Cs-137 and Sr-90 is essential. Results of these measurements are used to confirm that overburden is not contaminated, to determine if soil removal is necessary, and to establish the lower boundary of contamination in excess of site closure criteria (i.e., to determine when to stop digging.) Lack of timely data regarding radionuclide contaminants in the soil results in operational delays which substantially increase costs associated with heavy excavation equipment and remediation personnel.

Rapid quantification of Cs-137 activity concentrations in soil samples has been accomplished using the ISOCS system described previously, with turnaround times on the order of 20 minutes. Although this data can be used to determine whether Cs-137 concentrations are in excess of site closure criteria, Sr-90 activity concentrations cannot be inferred from the results due to the variable relative distribution of these contaminants. Sr-90 is a pure beta emitter with no direct photon emissions, and thus cannot be quantified using the ISOCS gamma spectroscopy system.

Quantification of Sr-90 using conventional EPA laboratory methods typically takes a minimum of two weeks (accelerated turnaround) or a month (standard turnaround). As a potential solution to this time delay, use of the BetaScint™ fiber-optic sensor has been applied and tested at the BGRR as a technique to rapidly quantify Sr-90 in soil samples. The preliminary results of this evaluation, summarized herein, indicate that the BetaScint™ system produces accurate and precise results with a quick turnaround time (approximately 20-30 minutes) with a detection sensitivity of approximately 1 pCi/gram.



Figure 1. Illustration of BetaScint™ Spectrometer and Soil Sample in Tray

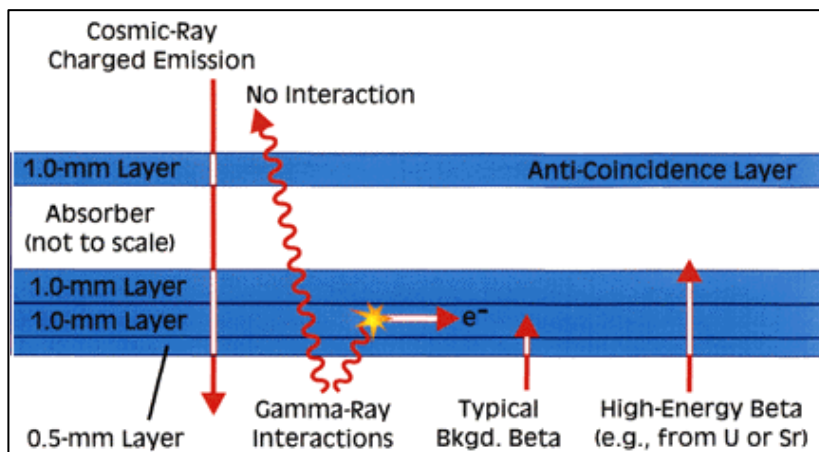


Figure 2 – Schematic Cross-section of BetaScint[®] System

Technology Description

The BetaScint™ system consists of a multi-layer beta scintillation detector array, with a beta radiation entrance window measuring 30-cm by 60-cm. Scintillating fibers are fashioned into ribbons, which are stacked vertically as shown in Figure 2. Soil samples are prepared, transferred to large area counting trays, and positioned beneath the system entrance window for analysis. Beta particles that pass through the entrance window will excite electrons in the scintillating ribbons resulting in the emission of light pulses, which are counted by photomultiplier tubes.

The BetaScint™ system relies on the relatively high energy of the beta particle emitted from Y-90 ($E_{\max} = 2.3$ MeV) a product of Sr-90 decay to quantify Sr-90 in the presence of other beta-emitting radionuclides and ambient background radiation, whose beta energies are lower. It should be noted that the BetaScint™ would also respond to uranium-238 contamination, due to the decay of Pa-234m ($E_{\max} = 2.29$ MeV); however, uranium-238 is not a significant radionuclide of concern at the BGRR and is not addressed in this paper.

The basic principles of the BetaScint™ technology to measure Sr-90 are as follows:

- Beta particles that interact and simultaneously produce light pulses in all three lower scintillation ribbons (i.e., one 0.5-mm thick ribbon and two 1.0-mm thick ribbons), but not in the upper anti-coincidence layer, are registered as detection events.
- Any other combinations of simultaneous interactions in any of the scintillation ribbons, including single interactions, are rejected as background (non-sample) events.

This configuration eliminates the system's response to low energy beta particles from naturally occurring background radioactivity and substantially reduces the system's response to cosmic rays and ambient gamma rays. Low energy betas are eliminated because they do not have sufficient energy to penetrate both the 0.5-mm thick layer and the first 1.0-mm layer; thus it is impossible for such beta particles to simultaneously produce light pulses in all three lower scintillation ribbons because they cannot reach the second 1.0-mm layer. Cosmic ray interference is reduced because cosmic events typically result in simultaneous interactions in all four scintillation ribbons, including the upper anti-coincidence layer. Ambient gamma ray interference is reduced because the probability that an incident gamma interacts in the three lower scintillation ribbons is much smaller than the probability of an interaction by a beta particle of comparable energy.

System Operation

1. For calibration and operation, a two-pound soil sample is typically dried, sieved to remove organic matter and rocks over 0.25 inches in size, and spread evenly over a large area (18" x 30") counting tray. The tray is then positioned beneath (within ¼" of) the radiation entrance window of the detector and counted for five minutes. Following the analysis, the system reports the number of coincident events occurring in the three scintillating layers during the counting period. This count is converted to Sr-90 activity

concentration in the soil, based on the detection efficiency correlation established using the spiked site soils.

2. Initially, the Sr-90 detection efficiency of the BetaScint™ system is established by measuring its response to site-specific calibration standards. These calibration standards are prepared by spiking clean (non-contaminated) site soils with known quantities of Sr-90. The net system response is directly proportional to Sr-90 activity concentration because it is almost entirely due to Sr-90 beta interactions.
3. Routine daily operations of the BetaScint™ system include the performance of daily quality control checks and background measurements.
 - Quality control checks consist of analysis of a calibration standard of known activity. The results of the quality control checks are compared against established acceptance criteria to determine whether the instrument is functioning properly.
 - Background checks consist of counting the sample tray without a sample in place. These measurements, which are performed daily at a minimum, are subtracted from gross sample counts to establish net detector response.

Implementation at BNL

The BetaScint™ system was initially deployed at Brookhaven for a two week demonstration during the period of December 6 - 17, 1999. Soil samples from various environmental restoration areas at the Brookhaven site were collected and analyzed using the system.

1. System Calibration

Four calibration standards were prepared from an NIST-traceable Sr-90 solution and site soils collected from an area that has not been impacted by site operations. Prior to spiking the standards, the soils were analyzed using the ISOCS gamma spectroscopy system to assist in determining whether the soils were impacted by site operations. The results of these analyses did not indicate the presence of gamma emitting radionuclide contaminants in excess of background levels. Following preparation, the calibration standards were analyzed by the BetaScint™ system and a correlated Sr-90 detection efficiency was established. Results are indicated in Figure 3.

2. Analytical Stability

Daily, or more frequent, quality control checks were performed (a total of 25 QC checks) by analyzing the 22.5 pCi/gram calibration standard that was prepared. The average result of these analyses was 22.2 pCi/gram (see Figure 4). All results were within ± 2 sigma of the average value, with the exception of one result that was slightly less than 2 sigma, indicating that the system response is stable over time and exhibits acceptable levels of precision. Duplicate analyses were also performed on most samples to evaluate the precision of the system. In general, the results of duplicate analyses were within acceptable statistical bounds.

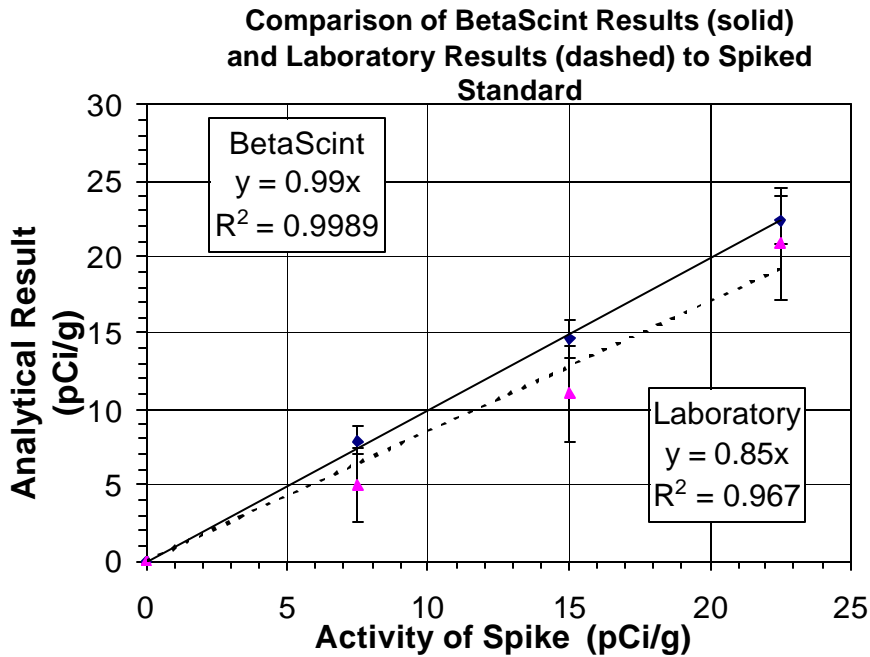


Figure 3. BetaScint and Laboratory Analysis of Spiked Standards

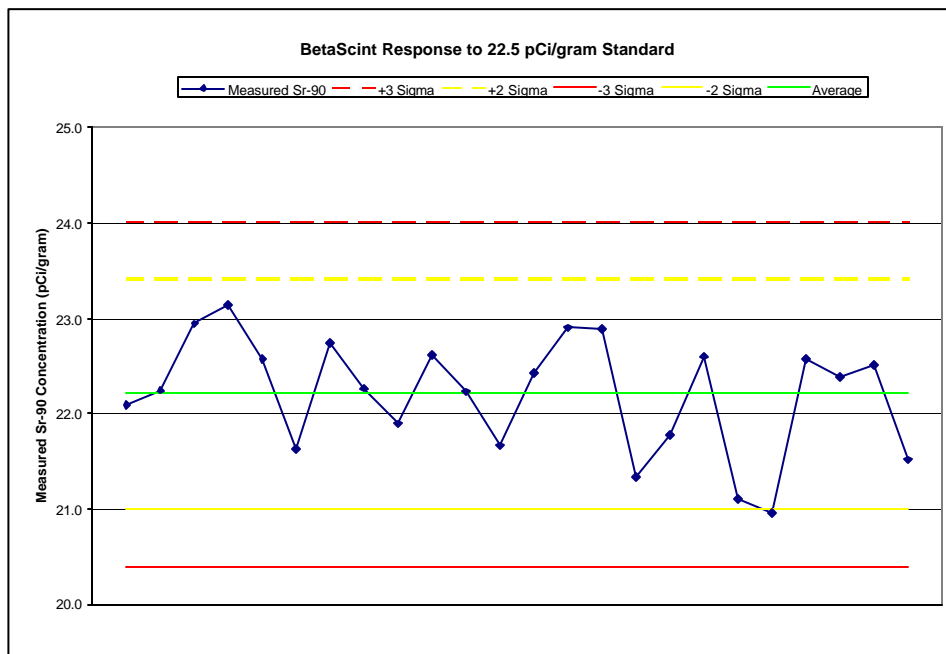


Figure 4. – Results of Quality Control Measurements

3. Analysis Variability

In order to evaluate the accuracy of the BetaScint™ analyses, the calibration standards were re-analyzed on the BetaScint™ following system calibration. The results of these analyses are summarized in Table 1. As can be seen, the BetaScint™ results were within $\pm 8\%$ of the calibration standard activity concentrations, indicating that the system exhibits acceptable levels of accuracy.

Table 1
Results of Calibration Standard Measurements

Spiked Sr-90 Concentration (pCi/g)	Measured Sr-90 Concentration (pCi/g)	% Difference
7.5	7.9 \pm 1	+ 5.4
15	16 \pm 2	+ 7.7
22.5	22 \pm 2	- 0.30
610	562 \pm 40	- 7.9

* Errors reported at the 95% confidence level

4. Analytical Intercomparison

To support the evaluation of system accuracy, 7 soil samples and aliquots from the 4 calibration standards were sent off-site for conventional baseline Sr-90 analysis. The results, shown in Figure 3, indicate a slight over response of the BetaScint instrument (slope = 1.11) and an acceptable correlation coefficient ($R^2 = 0.81$). It should be noted, however, that off-site Sr-90 analyses are based on a very small sample size (#1 g), whereas BetaScint evaluates 2 kg samples of soil. Thus, if contamination is not 100% homogeneously distributed, BetaScint may provide a more representative characterization.

5. Influence of Soil Moisture

Samples were prepared for analysis in accordance with BetaScint™ guidance. This involved sifting soil using a 0.25-inch mesh sieve, spreading the soil onto counting trays, and the drying the soil using heat lamps. During performance of this process, it was determined that the sample drying time was excessive and could adversely affect the objective of quick sample result turnaround. In order to determine whether sample drying was necessary, a cursory evaluation of the effects of soil moisture content on the system was performed. De-ionized water was added to the 610 pCi/gram calibration standard, the standard was thoroughly mixed, its moisture content was measured, and it was analyzed using the system. This process was repeated for moisture contents of 0, 4, 9.5, and 14.3%. At 14.3% moisture content the soil became saturated, as the site soil is sandy and does not absorb much water. The results of this evaluation are plotted in Figure 6. As can be seen, the system response was reduced by less than 20% for saturated soil compared to dried soil. Based on these results, a more rigorous analysis of the effects of moisture content could be used to develop a correction curve for un-dried samples. This would significantly increase the throughput of the system, eliminating the need for drying and only requiring a brief moisture analysis be performed. Although this correction would adversely

affect the detection sensitivity of the system, based on these results it is unlikely that it would be greater than 2 pCi/gram for a 5-minute count of saturated soil.

6. Sample Analysis

During the two-week demonstration, a total of 145 evaluations were performed on 35 samples. The analytical count time for each analysis was 5-minutes. Based on the BetaScint™ results, Sr-90 activity concentrations in these samples ranged from not detectable to approximately 70 pCi/gram. The 5-minute count time yielded a minimum detectable activity concentration of approximately 1.2 pCi/gram at the 95% confidence level, which is considerably less than the BGRR DGCL of 15 pCi/gram.

Conclusions

The BetaScint™ system was initially deployed on the Brookhaven site for a two-week field demonstration prior to integration into on-going characterization activities. Evaluation of the system indicates that it can provide accurate and repeatable analyses of Sr-90 in soil with an acceptable detection sensitivity and a throughput of approximately 20-30 minutes per sample. This throughput could be adversely affected by the need to dry samples prior to analysis. However, preliminary data indicates that soil moisture correction factors can be applied (if necessary) to eliminate the need to dry samples.

References

U. S. EPA, *EPA Guidance for the Data Quality Objectives Process (EPA QA/G-4)*, EPA/600/R-96/055, U. S. Environmental Protection Agency, Washington, DC. August 2000.

U. S. EPA, *EPA Guidance for Quality Assurance Project Plans (EPA QA/G-5)*, EPA/600/R-98/018, U. S. Environmental Protection Agency, Washington, DC. February 1998.

U. S. EPA, *Guidance for Data Quality Assessment (EPA QA/G-9)*, EPA/600/R-96/084, U. S. Environmental Protection Agency, Washington, DC. January 1998.

U. S. EPA, U. S. NRC, U. S. DOE, and U. S. Department of Defense, *Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM)*, EPA 402-R-97-016. Washington, DC. December 1997; revised August 2000. [also issued by U. S. Nuclear Regulatory Commission as NUREG-1575 and by U. S. Department of Energy as DOE/EH-0624].

U. S. DOE, *Innovative Technology Summary Report: BetaScint™ Fiber-Optic Sensor for Detecting Strontium-90 and Uranium-238 in Soil*, U. S. Department of Energy, Office of Science and Technology, December 1998.

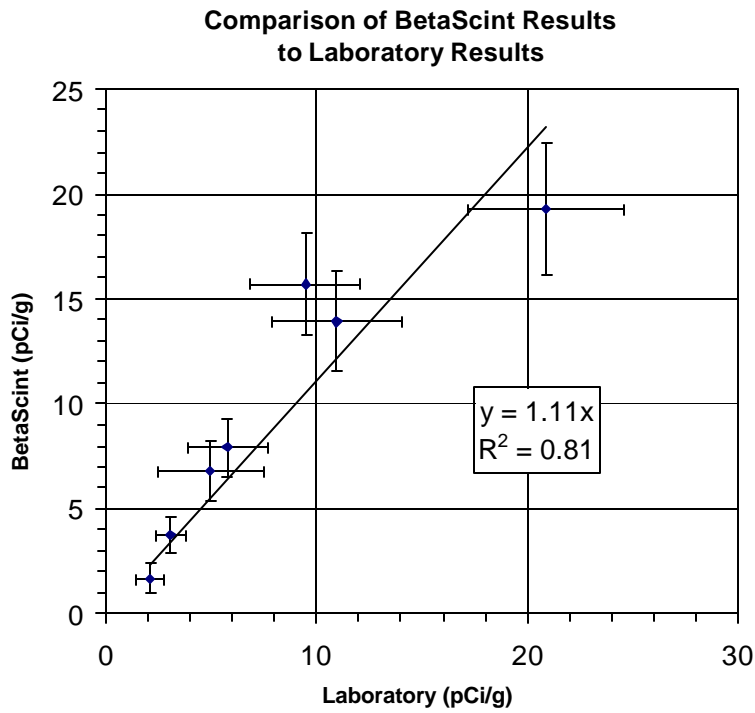


Figure 5. Comparison of field samples on two analysis systems.

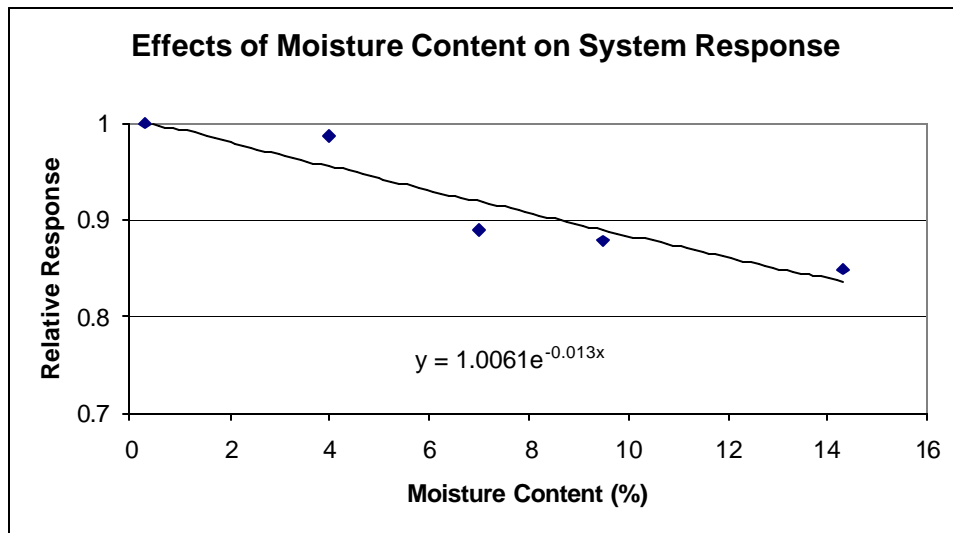


Figure 6. - Effects of Moisture Content on System Response

Section 5



Standard Operating Procedure

Analysis of Gamma Spectrum Files Using Canberra ISOCS System [software ver 3.0]

SOP - DAT2

for the
Brookhaven National Laboratory
Accelerated Site Technology Deployment Project

DEPLOYMENT OF INNOVATIVE CHARACTERIZATION TECHNOLOGIES AND IMPLEMENTATION OF THE MARSSIM PROCESS AT RADIOLOGICALLY CONTAMINATED SITES

Version 3

February 23, 2000

[replaces version 1, dated August 27, 1999]

APPROVED

Larry W. Lockett, CHP
Project Analytical Physicist, BNL-DAT

SOP-DAT2 Analysis of Gamma Spectrum Files Using Canberra ISOCS System [software ver 3.0]

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SOP-DAT2 Analysis of Gamma Spectrum Files Using Canberra ISOCS System [software ver 3.0]

1.0 PURPOSE

This procedure describes the steps necessary for quantitative gamma spectrum analysis and reporting using the Canberra In Situ Object Counting System (ISOCS). Instructions are provided for software environment setup, routine spectrum analysis, software modifications for specific acquisition and analysis requirements, and spectrum file management.

2.0 SCOPE

This procedure addresses operation of the Canberra ISOCS software system for quantitation of gamma emitting radionuclides *in situ* and in samples collected during remediation activities at the Brookhaven National Laboratory.

This procedure addresses operation of the ISOCS software, ver 3.0, distributed in September, 1999. The previous version of this procedure (ver 1, August 1999) addressed the operation of ISOCS software, version 1.2E.

Gamma radiation spectra will be accumulated in documented measurement configurations (geometries), using a high purity germanium (HPGe) detector. Procedures for spectrum acquisition are addressed in a separate procedure (SOP-DAT1, ref 3.6). Analysis activities and results will be documented in the Project files.

This procedure addresses the analysis of gamma spectra using the ISOCS system. The actual surfaces, items, and/or samples to be assessed are identified in a Project Specific Survey Plan (PSSP) that should be available to the instrument operator. The PSSP can vary in complexity from a verbal briefing by the Project Manager to a formally approved document specifying items, location/sample identifiers, locations, instrument positioning, and set-up parameters. At appropriate steps, this procedure identifies when reference to the PSSP is indicated.

3.0 REFERENCES

- 3.1 *Model S503 PROcount-2000: User's Manual*, No. 9231025B. Canberra Industries, Meriden, CT. June 1999. ^
- 3.2 *Model S500 GENIE-2000 Spectroscopy System: Operations*, No. 9230846E. Canberra Industries, Meriden, CT. June 1999. ^

- 3.3 *Model S505 Genie-2000 QA Software: User's Manual*, No. 9230871C Canberra Industries, Meriden, CT. June 1999. ^
- 3.4 *Model S573 ISOCS Calibration Software for Genie-2000: User's Manual*, No. 9231013B, Canberra Industries, Meriden, CT. August 1999.
- 3.5 *ISOCS Efficiency Calibration, Validation, and Internal Consistency Document*, No. 9231205C. Canberra Industries, Meriden, CT. September 1999.
- 3.6 BNL SOP-DAT1. *Gamma Spectrum Acquisition Using Canberra ISOCS System*. Department of Advanced Technology, BNL, Upton, NY. August 19, 1999.
- 3.7 BNL SOP-DAT3. *Quality Control Measurements When Using Canberra ISOCS System*. Department of Advanced Technology, BNL, Upton, NY. Under Development.

NOTE: References marked (^) are available as electronic files on the Documentation portion of the Genie-2000 software distribution CD-ROM and should be available for on-line review on the workstation computer.

4.0 DEFINITIONS.

ISOCS	In situ object counting system, the Canberra Industries name for the quantitative gamma spectroscopy analysis system
<i>in situ</i>	Performed in the natural or normal place; at the site of origin without invasive procedures or disruption of the media
PSSP	Project specific survey plan
System Manager	an individual with knowledge, training, or experience capable of setting up and modifying the software operating environment of the Canberra ISOCS.

5.0 RESPONSIBILITIES

5.1 The System Manager is responsible for

- maintaining the ISOCS instrument and accessories referred to in this procedure, and
- ensuring that the instrument is operational and calibrated for routine and non-routine geometries, as needed.

5.2 Personnel qualified on this procedure are responsible for

- reviewing the PSSP to ensure knowledge of the survey objective prior to performing the analysis,
- ① analyzing gamma radiation spectra in the several routine geometries specified this procedure, and
- ① transferring electronic data files upon completion of the analysis.

6.0 OPERATIONAL PRECAUTIONS

This procedure addresses the processing of electronic files on a computer work station. The operational hazards are those associated with routine computer use in an office location.

Processing of electronic files may also be performed on the laptop computer connected directly to the Canberra Industries ISOCS System. The system is a mobile instrument and can be taken to and used in locations with a variety of physical, environmental, health, and safety hazards. When performing in the field, the instrument operator should coordinate with BNL safety personnel to become familiar with any hazards and controls associated with the location being surveyed.

7.0 EQUIPMENT AND TECHNICAL REQUIREMENTS

7.1 ISOCS System Hardware

The Canberra Industries ISOCS hardware for gamma spectrum acquisition is identified in SOP-DAT1, ref 3.6. The only portion of the hardware required for this analysis procedure is the laptop computer or a separate computer workstation with the ISOCS software system loaded. Access to a printer from the computer workstation is necessary if hard copy reports are required.

7.2 Canberra Software

Required software programs and the setup of the operational environment are discussed in Appendix 1 of this procedure. These setup actions are only required when re-installing the software, or upon establishing the software on a new computer workstation. The need to perform or change any of these system components should be discussed with the System Manager.

8.0 PREREQUISITE KNOWLEDGE AND ACTIONS

8.1 Correspondence between Sample Number and Spectrum Data File

During spectrum acquisition, the Canberra operating system assigns a file name to the spectrum data file that is separate and independent of any sample number or identification assigned by the operator.

The name of the file will be of the form : *STCnnnnn.CNF*
where *STC* is the Sample Type Code,
nnnnn is the Sample Sequence Number, and
CNF is the file identifier for any spectrum data file.

8.1.1 To perform the analysis, one must know the name of the spectrum data file that corresponds to the sample or object description of interest. The spectrum data file name should have been recorded in the field logbook at the time of acquisition, and may be found in that notebook.

8.1.2 Alternatively, the lines of descriptive text entered in a file during spectrum acquisition can be observed using Genie2000 Gamma Acquisition and Analysis (GAA) to verify that the spectrum file corresponds to the sample or object of interest.

- a. Start the GAA program by double clicking on the Gamma Acquisition & Analysis icon or by choosing **Programs, Genie-2000, and Gamma Acquisition & Analysis** from the Start Menu. The main spectrum display screen is displayed.
- b. Click on **File** and then **Open Datasource** to obtain the file directory dialog box.
- c. Change the directory to the location of the *.cnf file to be reviewed.
- d. Highlight the file to be reviewed and press <OK> to display that file.
- e. Click **Edit** and then **Sample Info** to display the Sample information dialog box. The sample ID number is listed in a field on the upper right corner of the dialog box. The sample description is displayed in the left center of the dialog box.
- f. Click **Cancel** to return to the main spectrum display screen.
At this point additional spectrum data files may be reviewed until the sample of interest is found.
- g. To exit GAA, click **File** and **Exit**.

8.2 Locating Spectrum Data Files on the Workstation Computer.

8.2.1 Ensure that the spectrum file you wish to analyze is in the appropriate directory:

Spectrum data files acquired under PROcount are stored in a sample-type sub-directory within the directory:

C:\pcnt2k\camfiles\

The name of the file will be of the form as shown in §8.1, above.

8.2.2 If the spectrum was acquired directly in Genie2000 GAA, rather than using PROcount, the spectrum file will be stored in the directory:
C:\genie2k\camfiles\

Copy/move the file to the appropriate sub-directory within the directory
C:\pcnt2k\camfiles\ for use in this procedure.

8.3 Analysis Sequence File

The name of the appropriate pre-defined analysis sequence must be known prior to performing the analysis. This information will be needed in Step 9.2.10, below. For the BNL ASTD project, the typical analysis sequence is entitled: "ASTD Default Analysis with MDA", or a specific analysis may be specified in the PSSP.

The Canberra operating system uses an analysis sequence file (*.asf) to define the steps in analyzing and in reporting the results of an analysis. The files are found in the directory
C:\pcnt2k\asfiles\

The Analysis Sequence File Editor is used to define and edit an analysis sequence file. This operation defines such aspects as report format, reporting steps, algorithms for calculating error and minimum detectable activity, units and scale factors/multipliers, and the energy/decay library for radionuclide identification. The use of the Analysis Sequence File Editor is described in the Genie-2000 Users Manual, ref 3.2.

8.4 Canberra Software Inter-Communication

To enable communication and data transfer between the various Canberra programs, the Genie2000 Virtual Data Manager must be running in background on the analytical computer. If it is not already running, start the Virtual Data Manager by choosing **Programs, Genie-2000, and Virtual Data Manager** from the Start Menu. A button indicating "Genie 2000 VDM" should appear on the Windows Application Bar at the bottom of the screen.

8.5 Geometry Template Worksheet Forms

Use of this procedure is dependent on a familiarity with ISOCS methods and with the analysis geometries defined in the ISOCS models. The availability of ISOCS Geometry Template Worksheet Forms eases the visualization of the model geometry. The appropriate ISOCS Geometry Template Worksheet form should be filled-in with the chosen parameter values during the analysis and filed with the final data analysis document package upon completion.

NOTE: Additional copies of the Geometry Template Worksheets can be printed from the MS Word document file C:\isocs\template.lib\templates.doc

8.6 Physical Dimensions and Materials

Accurate quantitative evaluation relies on knowledge of the constituents and physical dimensions of the object in the detector field of view at the time of spectrum acquisition. Access to the field notebook and/or photographs and notes recorded during spectrum acquisition are necessary to performing the analysis. Additional information on material composition and dimensions may be provided in the PSSP.

9.0 QUANTITATIVE OBJECT ANALYSIS PROCEDURES

9.1 Creating the ISOCS Efficiency Calibration File

Prior to performing this procedure, see Section 8.0, prerequisite knowledge and actions.

9.1.1 Start the ISOCS Program by double clicking on the ISOCS icon or choosing **Programs, Genie-2000** and **ISOCS** from the Start Menu. The ISOCS Main Menu screen is displayed with a Header at the top and a Status Bar at the bottom.

NOTE: The mouse is not operable under ISOCS;
use arrow keys to maneuver around menu and to highlight your choice.
use < Enter > to accept a selection
use < Esc > to exit to previous level

9.1.2 To identify the Detector and Collimator used in the spectrum acquisition:

- a. Highlight **Equipment** on the Main Menu Header and Press <Enter>
 - b. Highlight **Detector** on the drop-down menu and Press <Enter>
 - c. Highlight the specific detector used in the analysis and Press <Enter>
 - d. Highlight **Circular Collimator** on the drop-down menu and Press <Enter>
 - e. Highlight the specific collimator on the drop-down menu and Press <Enter>
- You can now use the arrow to go to another Header on the Main Menu

9.1.3 To choose the Geometry Template for the analysis:

- a. Highlight **In Situ Template** on the Main Menu Header and Press <Enter>
- b. Highlight the specific geometry template on the drop-down menu and Press <Enter>

NOTE: At this point, to retrieve a previously saved ISOCS parameter file for the specific combination of detector-collimator-geometry, highlight **File** on the Main Menu Header, press <Enter>, highlight **Load** from the drop down menu, press <Enter>. A list of previously saved calibrations is displayed. If the list is too long, <Page Up> or <Page Down> is used to scroll to

additional file names. Highlight the specific file and press <Enter>. The previous physical parameter data is loaded and may be modified by the following steps.

9.1.4 To Enter Physical Data for the analysis:

NOTE: The current units for the various dimensions are displayed on the status bar at the bottom of the screen. When entering data, to change the units highlight **Parameter input** on the Main Menu Header, press <Enter>, highlight **Dimension Units on the drop down menu**, press <Enter>, highlight the unit of choice in the dimension category, and press <Enter>.

- a. Highlight **Parameter input** on the main Menu Header and Press <Enter>
- b. Highlight **Source Dimensions** on the drop-down menu and Press <Enter>

NOTE: Use the ISOCS Geometry Template Worksheet Form for the chosen geometry model as a guide for which data positions require values.

- c. Use arrow keys to move to the data entry position for the parameter of interest and type in a value. Note that the units for the various dimensions are displayed on the status bar at the bottom of the screen.
- d. Use the arrow keys to move to the next data entry position and type in the value.
- e. With the highlight in the **Material** column, press <Enter> to display a list of previously defined materials. You must select one from this list: use the arrow keys to highlight the specific choice and press <Enter>.

NOTE: To edit the material list and define a new material, navigate to **Parameter Input** on the Main menu header and select **Library Materials Edit** from the drop down menu. [Defining materials is discussed in Section 4.4.5, of the ISOCS User's Manual, ref 3.4]. If using a new definition, write the material definition on the Worksheet form.

- f. When all data have been entered on the screen form, write the values and units on the Worksheet form.
- g. Press <Esc> to return to the Main Menu Screen.

9.1.5 To perform the detector Efficiency Calculation for the geometry chosen:

- a. Highlight **File** on the Main Menu Header and Press <Enter>
- b. Highlight **Calc+Save** on the drop-down menu and Press <Enter>

NOTE: The highlight should re-appear in a file save dialog box directly above the status bar at the bottom of the screen.

- c. Use arrow keys to move to the **FILENAME** data entry position; type in a name limited to eight alphanumeric characters (do not use "." or a blank space; use underscore for space). The **FILENAME** may be the sample number identified in the PSSP.

NOTE: This **FILENAME** will be used to save the template dimensions in a geometry definition file and later is the default name for the efficiency calculation file. This is the Filename used to recall the geometry data for re-analysis, as discussed in the note at §9.1.3, above. This filename will also be used in the quantitative analysis, at §9.2.8, below.

media or objects, such as soil or waste in a container — the analysis will be performed to generate a report that quantifies the concentration of activity detected in the material defined in the efficiency file and reports it in units of activity/mass [$\mu\text{Ci/g}$].

Press < **Efficiency*Area** > to choose units of areal activity concentration [$\mu\text{Ci/m}^2$] in a surface such as open ground, walls or floors — the analysis will be performed to generate a report that quantifies the concentration of radioactivity per unit area detected in the geometry surface defined in the efficiency file and reports it in units of activity/area [$\mu\text{Ci/m}^2$].

Press < **Efficiency*Length** > to choose units of linear activity concentration [$\mu\text{Ci/m}$] in a cylindrical object, such as a pipe, rod or tank — the analysis will be performed to generate a report that quantifies the concentration of radioactivity per unit length detected in the object defined in the efficiency file and reports it in units of activity/length [$\mu\text{Ci/m}$].

NOTE: While this choice will select the units to be used in the calculation, the actual units will not appear in the final printed report. Failure to choose the appropriate units or failure to record the choice on the Worksheet form will result in mis-interpreting the analytical report.

Write the efficiency mode on the Geometry Template Worksheet Form.

After pressing one of the buttons, the system displays a message:

“Building Efficiency Results Table. Please Wait...”

- b. The next dialog box displays the *ISOCS: Efficiency Results* tabulated at the pre-determined energy values.
- to choose a different efficiency option Press < **Reselect** >
The system returns to the previous dialog box.
 - to accept the efficiency results Press < **Ok** >
The system displays a message

“Creating CAM file. Please Wait...”

- c. The next dialog box is the *Select Efficiency Function* that allows several options for reviewing or changing the results of the efficiency calculation
- Press < **Exit** > to quit the program without saving the calibration file.
- Select “*View Efficiency Plot*” and Press < **Ok** > to display the *Efficiency Calibration Curves* screen with several options for modifying or accepting efficiency curve coefficients. To accept the displayed efficiency curve data Press < **Ok** >

- Select “*Save Efficiency File*” and Press < **Ok** > to accept the results of the efficiency calculation.
- d. The next dialog box is the *Comment Edit* screen which displays the **COMMENTS** field entered in §9.1.5.e, above, with the text modified to reflect the units of the output chosen in §9.1.6.a, above.
- If the initial text in the comment field is “ISOCS”, then the units in the analysis reports will be in activity.
 - If the initial text in the comment field is “ISOCS UNITS=ACT/G”, then the units in the analysis reports will be in activity concentration.
 - If the initial text in the comment field is “ISOCS UNITS=ACT/M2”, then the units in the analysis reports will be in areal activity concentration.
 - If the initial text in the comment field is “ISOCS UNITS=ACT/M”, then the units in the analysis reports will be in linear activity concentration .
- Write the units text on the Geometry Template Worksheet Form.
To accept the displayed information Press < **Ok** > .
- e. The next dialog box is the *ISOCS File Menu* screen, which displays the eight character **FILENAME** assigned in §9.1.5.c, above, and allows assignment of a new name.
- NOTE: At this point, the name of the efficiency calibration file may be edited to contain in excess of eight characters, so that a more descriptive title may be assigned. However, the name of the geometry definition file, saved at §9.1.5.c, above, will not be changed by editing the filename here.
- Write any new **FILENAME** on the Geometry Template Worksheet Form.
To accept the displayed information Press < **Ok** > .

Displays a message “Processing...Please Wait” and then exits to Windows. The calibration file is saved as c:\genie2k\calfiles\FILENAME.cal

9.2 Gamma Spectrum Analysis using the ISOCS Efficiency File.

Prior to performing this procedure, see Section 8.0, prerequisite knowledge and actions. This analysis can only be performed following the creation of the ISOCS calibration file using the procedure in §9.1, above.

- 9.2.1 Start PROcount software package by double clicking on the PROcount 2000 icon or by choosing **Programs**, **Genie-2000**, and **PROcount 2000** from the Start Menu.
- 9.2.2 LOGON to the system by typing your Username and Password, and Press < **ENTER** > or < **Ok** > .
Displays Genie PROcount Main Selection Screen, showing selections “**A-H**”.

- 9.2.3 Select < **G. System Maintenance** > and Press < **Ok** > .
Displays the System Maintenance Menu Screen, showing selections “**A-F**”.
- 9.2.4 Select < **A. Analyze Sample Data File** > and Press < **Ok** > .
Displays the Analyze Sample Data File Screen
- 9.2.5 Highlight/select the sample type < **ISOCS Generic** > and
Press < **Choose by Sample Type** > .
Displays a dialog box/file list of sample files with Sample ID, Description and
Acquisition date/time displayed
- 9.2.6 Use the arrow and/or page down key to highlight the sample to be analyzed and
Press < **Select** > .
Displays the Re-Analyze a PROcount Sample Menu, showing selections “**A-G**”.
NOTE: The spectrum data file name (*stcnnnnn.cnf*) is displayed in the header
line of the menu screen. This is the name the system will use to save
the report file when it is generated. [See §9.2.11, below].
- 9.2.7 Highlight < **B. Select Alternate External Efficiency Calibration** > and
Press < **Ok** > .
Displays a dialog box/file list of saved geometry efficiency calibrations. In the
column “*Filename*” are available geometries that may be recalled for use in this
analysis.
- 9.2.8 Use the arrow and/or PgDn (page down) key to scroll through the list and reveal
additional geometry efficiency filenames. One of the filenames in the list should
be the eight character alphanumeric name that was entered as the filename in
§9.1.5.c, above. Highlight the desired efficiency filename and
Press < **Ok** > .
Re-displays the Re-Analyze a PROcount Sample Menu.
- 9.2.9 Highlight < **C. Select Alternate Analysis Sequence File** > and
Press < **Ok** > .
Displays a dialog box identifying the detector and geometry and a drop-down list
of alternate analysis sequence files.
- 9.2.10 Highlight on the drop-down menu the desired analysis to be performed. For the
BNL ASTD project, the typical analysis sequence is entitled: “ASTD Default
Analysis with MDA”. Refer to the PSSP for any specific analysis sequence.
Press < **Ok** > .
Re-displays Re-Analyze a PROcount Sample Menu, showing selections “**A-G**”.
- 9.2.11 Highlight < **F. Perform Analysis, Show Report** > and Press < **Ok** > .
After a few moments, the Report Window appears in the upper half of the
display. Use the arrow keys and the scroll bar to review the report.
NOTE: A copy of the analysis report is saved as an ASCII text file under:
C:\pcnt2k\repfiles\stcnnnnn.rpt
where *stcnnnnn* is the spectrum data file name of the spectrum in
analysis. Any re-analysis of the spectrum file *stcnnnnn.cnf* will over-
write this report file, so if the report is wanted, it should be renamed

or copied to another disk location prior to performing another analysis on the same spectrum data file.

- 9.2.12 When finished reviewing the report, Press < **Ok** > .
Closes the Report Window and Re-displays the Re-Analyze a PROcount Sample Menu, showing selections "**A-G**".
- 9.2.13 Press < **Cancel** > .
Re-Displays the System Maintenance Menu Screen, showing selections "**A-F**".
- 9.2.14 Press < **Return to Main** >
Re-displays Genie PROcount Main Selection Screen, showing selections "**A-H**".
- 9.2.15 Press < **Exit** > . System returns to Windows.

10.0 DATA FILE BACKUP AND TRANSFER

Daily, or at the end of each spectrum analysis session, the Data Acquisition Technical Team member will assemble data and files for transfer to the Analytical Physicist. The data transfer and records retention activities include:

- 10.1 Fill out the Data Acquisition and File Control Form (Form DAT-02) for the scans analyzed; this is the chain-of-custody for the following electronic files:
 - a. Geometry Definition files created by the system during §9.1.6.c are stored in
c:\isocs\geometry\filename.n Where "n" is a digit from 1 to 7
 - c Copy each geometry definition file (*filename.n*) to a labeled diskette.
 - b. Efficiency Calibration files, created by the system during §9.1.7.d, are stored in
c:\genie2k\calfiles\filename.cal
 - c Copy each efficiency calibration file (*filename.cal*) to a labeled diskette.
 - c. Analyzed Data Report, created by the system during §9.2.11, are stored in:
C:\pcnt2k\repfiles\stcnnnnn.rpt
 - c Copy each analyzed data report file (*STCnnnnn.rpt*) to a labeled diskette.
- 10.2 Assemble the annotated hard copy of the ISOCS Geometry Template Worksheet for each analysis and the hard copy of reports, if printed.
- 10.3 Transfer to the Project Analytical Physicist.

11.0 MAINTENANCE

NOTE: Any maintenance performed on the ISOCS System shall be recorded in the ISOCS Laboratory/Field Notebook.

- 11.1 Routine maintenance is not required on the Canberra Software operating environment.
- 11.2 Non-routine maintenance on the electronics should be performed by the BNL Cognizant Environmental Scientist or designated alternate.
- 11.3 The Canberra Industries customer service department should be consulted should problems arise which cannot be handled by site personnel.

APPENDIX: Installing the ISOCS Software and Establishing the ISOCS Operating Environment

APPENDIX : Installing ISOCS Software and Establishing the ISOCS Operating Environment

Prior to the analysis of a gamma spectrum, the Canberra PROcount software program environment must be created on the system computer. These functions are explained, in detail, in Chapter 2 and Chapter 4 of the PROcount-2000 Users Manual (ref 3.1). The following system functions must be performed by the System Manager prior to routine spectrum acquisition and analysis.

1.0 ISOCS System Software. The following are components of the software programs supplied by Canberra Industries for the ISOCS Inspector multichannel analysis system. Each software program is supplied on a single 3.5 inch/1.44 Mb disk, except as indicated.

a.	S504	Genie-2000 Basic Spectroscopy (CD ROM)	Model G2KV13	v 1.3
b.	S501	Genie-2000 Gamma Analysis	Model 501J-6	v 1.3A
c.	S503	PROcount-2000	Model 503J-3	v 1.1
d.	S505	Genie-2000 Quality Assurance	Model 505J-5	v 1.3
e.	S506	Interactive Peak Fit	Model 506J-4	v 1.2
f.	S561	Batch Programming Support	Model 561J-4	v 1.2
g.	S573	ISOCS Calibration Software (2 disks)	Model 573J-5	v 3.0
h.		Detector-Specific Characterization Disk		

2.0 Initial Software Installation.

2.1 Procedures for loading the software to the workstation computer are provided in the appropriate Canberra manuals for the individual programs. In general:

2.1.1 **Insert** the distribution disk (CD ROM or 3.5 inch) into the drive.

2.1.2 **Select** the "RUN" command.

2.1.3 **Type** "A:\setup.exe" in the dialog box.

2.1.4 **Select** "Run" and follow the screen commands to complete the setup.

2.2 The Detector Specific Characterization File is loaded to the workstation computer by using the **COPY** command, rather than a Setup routine. Instructions for the files transfer are provided in a file *README.txt* found on the distribution disk.

2.3 After installing the Canberra ISOCS Software, the program may not run ISOCS from the ISOCS icon nor from the Start Menu, due to a mismatch with the Entrex batch mode processor. The path for the target file in the ISOCS Icon Shortcut must be revised.

- 2.3.1 In the Windows Explorer, **open the following folder:**
C:\WINDOWS\Start Menu\Programs\Genie-2000.
There should be a shortcut icon for ISOCS in this folder.
- 2.3.2 With your mouse **Right click** on the ISOCS icon
- 2.3.3 **Select** "Properties" from the pop-up menu.
- 2.3.4 **Select** the "Shortcut" tab
- 2.3.5 In the Target entry line **type** in the following :
C:\entrex\rexx.exe C:\isocs\runisocs.rex INSI
- 2.3.6 **Select** "OK".

You should now be able to execute ISOCS from the icon or from the Start Menu.

- 3.0 ISOCS System Management Components. Several system functions are rarely, if ever, changed following the initial set-up actions.

<u>System Component or Function</u>	<u>Reference</u>
Setting up the Calibration Function	§4.8, PROcount Users Manual
Setting up the Quality Control Function	§4.9, PROcount Users Manual
Organizing the Software Security System	§4.2, PROcount Users Manual

- 4.0 ISOCS System Analysis Components. Several system functions may be modified more frequently, based on requirements of the Project Specific Survey Plan.

<u>System Component or Function</u>	<u>Reference</u>
Setting Operating Defaults	§4.7, PROcount Users Manual
Defining Acquisition Geometries	§4.4, PROcount Users Manual
Defining the Sample Types	§4.6, PROcount Users Manual
Developing Analysis Sequence Files	§7, Genie-2000 Users Manual
Creating and maintaining radionuclide libraries	§5, Genie-2000 Users Manual

Section 6

ERD Operations Procedures Manual

**4.3 PROCEDURE FOR ASTD SAMPLE PROCESSING
TO SUPPORT BGRR DECOMMISSIONING OPERATIONS**

Text Pages 1 through 5
No Attachments

Approved: /s/ Stephen K. Pulsford Date: July 12, 2000
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ERD-OPM-4.3, Revision 0

PROCEDURE FOR ASTD SAMPLE PROCESSING TO SUPPORT BGRR DECOMMISSIONING OPERATIONS

1.0 PURPOSE AND SCOPE

This procedure provides guidance and describes the sample handling sequence for Accelerated Site Technology Deployment (ASTD) Project personnel performing sample analysis in support of BGRR. The procedure discusses the field analysis of samples using two identified instruments: (1) the Canberra Industries, Inc. ISOCS gamma radiation spectrometer, and (2) the BetaScint, Inc. beta radiation spectrometer.

2.0 RESPONSIBILITIES

- 2.1 ASTD Project provides trained and experienced personnel to operate a field analysis laboratory to perform rapid analysis of samples during BGRR decommissioning operations. Project personnel may include contractor/vendor instrument operators during initial field trials or instrument demonstrations.
- 2.2 BGRR Project obtains the physical samples during decommissioning operations and provides health and safety and radiological control support for the field laboratory operation.

3.0 PREREQUISITES

- 3.1 BGRR Project will evaluate anticipated sample contamination levels during the project planning and use that information to determine the need for and to develop a radiation work permit (RWP) for the field sample analysis laboratory.
- 3.2 Special Training and Qualifications for ASTD operators include manufacturer's training on a specific instrument and/or prior experience in operating the instrument in the field. Personnel will have appropriate, current qualifications to meet requirements identified in the RWP.
- 3.3 ASTD Operators will perform instrument checks and calibrations at the frequency identified in manufacturer's literature or operating procedures to assure confidence in instrument operations. Quality control criteria for the Canberra ISOCS instrument are addressed in Section 4.0 of the ASTD Project Quality Assurance Project Plan (QAPP), Reference 7.1.

- 3.4 BGRR Project will designate a location for the ASTD field laboratory, convenient to the operations to provide timely support but not where it would unduly interfere with the anticipated operations. The routine operational location is in the former physics shop area on the 110' level of Building 701.

4.0 PRECAUTIONS AND LIMITATIONS

- 4.1 The samples obtained from decommissioning operations during the BGRR Decommissioning Project are contaminated, or potentially contaminated, with radioactive materials. Anticipated or previously identified levels of contamination will be discussed in project planning documents.
- 4.2 Work permits (hazardous or radiological) shall specify protective equipment and measures for personnel protection against injury or exposure, and may provide levels of contaminant that will trigger additional control measures. Adherence to requirements of the work permits is mandatory. Failure to follow requirements of the work permit may expose the individual unnecessarily to hazardous materials, could lead to personal contamination, and will result in suspension of the work.
- 4.3 The instruments being used and evaluated by the ASTD Project are extremely sensitive and respond to low levels of radioactivity. Instrument contamination leading to invalid analysis can result if contamination control procedures are not carefully followed.

5.0 PROCEDURE

- 5.1 Analysis of Gross Soils by ISOCS and BetaScint Instruments
- 5.1.1 Receive the sample and chain of custody (COC) from BGRR radiological control technician (RCT).
- 5.1.2 Record the sample description identification in the ASTD logbook.
- 5.1.3 Prepare labels for poly bottle (ISOCS) and mixing pan (BetaScint).
- 5.1.4 Position the ¼" sieve over a 5-gallon deep sided mixing pan with a clean plastic bag liner.
- 5.1.5 Transfer the sample to the sieve.
- 5.1.5.1 Shake the sieve slightly to mobilize the sample through the sieve into the plastic bag lining the mixing pan.
- 5.1.5.2 Return those sample components that do not pass through the sieve into the original sample container.

NOTE: *If ISOCS results are already available for this sample, or if the sample is only to be analyzed by the BetaScint instrument, jump to Step 5.1.15*

- 5.1.6 Extract approximately 750 ml of sample from the plastic bag liner, and place into the labeled 1 liter poly bottle.
- 5.1.7 Place the poly bottle into a clean plastic bag and seal with tape.
- 5.1.8 Weigh the bagged and bottled sample and record the weight in the logbook.
- 5.1.9 Transfer the bagged and bottled sample to the ISOCS instrument.

NOTE: *Procedure for acquiring a gamma spectrum analysis with the Canberra ISOCS instrument is provided in DAT – SOP1, reference 7.2*

- 5.1.10 Acquire a gamma spectrum analysis on the ISOCS instrument.
- 5.1.11 Observe the gamma analysis results and record in the logbook.
- 5.1.12 Compare the results to values in the RWP and notify the BGRR RCT if a hold point level has been triggered.
- 5.1.13 Retrieve the bagged and bottled sample from the ISOCS instrument and return the sample contents to the corresponding labeled, plastic bag lined mixing pan.
- 5.1.14 Discard the empty poly bottle and corresponding plastic bag into contaminated waste receptacle.
- 5.1.15 Close the plastic bag liner and seal with a “J seal”, place the sealed bag into a clean, secondary bag (heavy duty trash compactor style). Tie a label to the clean secondary bag, with the sample identification number clearly written on the label. Place the double lined plastic bag within a clean 5 gallon plastic bucket and install lid in preparation for sample transfer. Obtain HP smear on secondary plastic bag and transfer bucket, and obtain clearance to move the container from the preparation area to the BetaScint counting area. Complete chain of custody data sheet and signatures.
- 5.1.16 Dampen lightly a paper towel with distilled water. Wipe the container exterior, the mixing pan, the sieve and scoops with the dampened paper towel(s) until items are visually clean. Discard used paper towels into contaminated waste receptacle.
- 5.1.17 Deliver samples in transfer bucket to BetaScint counting area, obtain chain of custody signatures. Open transfer bucket and remove selected sample (confirm sample ID number if several samples are contained in the transfer bucket). Place the clean sample bag on a shallow aluminum or plastic “Baker’s tray, place the inner sample bag on top, cut open the inner sample bag and discard as radwaste. Spread the sample on the plastic lined Baker’s tray in preparation for BetaScint analysis.

NOTE: *The operation and interpretation of the beta radiation spectrometer will be performed according to the BetaScint instrument procedure.*

- 5.1.18 Obtain a beta radiation spectrum analysis on the BetaScint instrument.
- 5.1.19 Observe the beta analysis results and record in the logbook.
- 5.1.20 Compare the results to values in the RWP and notify the BGRR RCT if a hold point level has been triggered.

NOTE: *At this point, the preliminary analytical results may be verbally reported to the BGRR Engineer.*

- 5.1.21 Retrieve the sample from the BetaScint instrument by placing it back in the plastic bag liner. Note that this bag originally served as the secondary containment, but is now being used as the primary containment. Re-seal the bag with a “J seal”, and place in a new, clean, secondary containment bag. Replace the RAM tag, which clearly identifies the sample ID number on the secondary bag, replace the sample within the 5-gallon plastic bucket, and replace lid in preparation for storage.
- 5.1.22 Obtain HP smear on secondary plastic bag and transfer bucket, and obtain clearance to move the container from the BetaScint counting area. Complete chain of custody data sheet and signatures. Place the sample container into the post-analysis staging area for BGRR disposition.

5.2 ISOCS Analysis of Samples

- 5.2.1 Receive the bagged or bottled sample and COC from BGRR radiological control technician (RCT).
- 5.2.2 Record the sample description identification in the ASTD logbook.
- 5.2.3 Place the sample or poly bottle into a clean secondary plastic bag and seal with tape.
- 5.2.4 Weigh the bagged and/or bottled sample and record the weight in the logbook.
- 5.2.5 Transfer the bagged and/or bottled sample to the ISOCS instrument.

NOTE: *Procedure for acquiring a gamma spectrum analysis with the Canberra ISOCS instrument is provided in DAT – SOP1, reference 7.2*

- 5.2.6 Acquire a gamma spectrum analysis on the ISOCS instrument.
- 5.2.7 Observe the gamma analysis results and record in the logbook.
- 5.2.8 Compare the results to values in the RWP and notify the BGRR RCT if a hold point level has been triggered.

NOTE: *At this point, the preliminary analytical results may be verbally reported to the BGRR Engineer. Procedure for interpreting a gamma spectrum analysis with the Canberra ISOCS instrument is provided in DAT – SOP3, reference 7.3*

- 5.2.9 Retrieve the bagged and/or bottled sample from the ISOCS instrument.
- 5.2.10 Discard any empty plastic bag into contaminated waste receptacle.
- 5.2.11 Complete the chain of custody data sheet and signatures. Place the sample into the post-analysis staging area for BGRR disposition.

6.0 RECORDS

The ASTD Project operator shall record sample data, observations, and analytical results in the field logbook. The ASTD operator shall transfer electronic files from the computer/workstation to removable disk for archiving with Project records as discussed in Section 6.0 of the ASTD Project QAPP, Reference 7.1.

7.0 REFERENCES

- 7.1 *In Situ Analysis Quality Assurance Project Plan*, version 1, July 2, 1999.
- 7.2 SOP-DAT1, *Gamma Spectrum Acquisition Using Canberra ISOCS System*, version 1, August 19, 1999.
- 7.3 SOP-DAT2, *Analysis of Gamma Spectrum Files Using Canberra ISOCS System [software ver 3.0]*, version 3, February 23, 2000.

8.0 ATTACHMENTS

None.

9.0 DEFINITIONS

ASTD	Accelerated Site Technology Deployment
BGRR	Brookhaven Graphite Research Reactor
COC	chain of custody
ISOCS	<i>in-situ</i> object counting system
RCT	radiological control technician
RWP	radiation work permit
QAPP	Quality Assurance Project Plan

**PLEASE CLICK HERE TO COMPLETE READING
ACKNOWLEDGEMENT FORM**

Section 7

ASTD Project-Specific Survey Plan No. 2k-01

Use of ISOCS in Support of BGRR Fan House Drain Pipeline Excavation and Pile Fan Sump Removal

1.0 PURPOSE

Perform ISOCS scans of disturbed surface soils in the area between the BGRR Exhaust Stack, the Fan House, and the Pile Fan Sump to

- (i) monitor progress of excavation (have we excavated far enough to identify the limits of any contaminated pipe/soil?)
- (ii) assist with determination of soil disposition (which pile does this soil go into?)

NOTE: The use of the BetaScint instrument to support this BGRR excavation by the field analysis of samples for Sr-90 activity is addressed in a separate ASTD Project Specific Survey Plan (PSSP No. 2K-02, in press).

2.0 DATA QUALITY OBJECTIVES

2.1 State the Problem

Using the Canberra ISOCS instrument, in contrast to baseline technology, provides the opportunity to obtain data on radionuclide identification and quantification quickly, without having a delay for transportation and preparation of the sample. An area of interest for this application is the BGRR Fan House Pipeline (FHP), a drain pipe that runs from the BGRR Exhaust Stack, to the Fan House, Bldg 704, to the Pile Fan Sump, and into Bldg 801 as well as the Pile Fan Sump and surrounding soils. Data are needed to determine, with the combination of activity and shielding present,

- (a) the radionuclide content of the surface and subsurface soils during excavation;
- (b) whether the residual radioactivity in the pipeline is such that it can be detected or bounded by *in situ* measurements;

2.2 Identify the decision.

The decision to be made is essentially a simple detection limit decision. What radionuclides does the ISOCS instrument detect in the soil and from the pipeline? If the pipeline scans are not different from the non-pipeline scans, can an upper bound on the activity present be quantified?

2.3 Identify inputs.

Information is available on the geometry and constituents of the Fan House Pipeline and the Pile Fan Sump from construction drawings. Plant Engineering has attempted to mark the pipe location using sounding techniques with some success near the ends, but not in the center of the run. Results of prior analysis of water and sludge samples from the Pile Fan Sump are available for review. Prior analysis has detected different radionuclides in the water and sludge phases (see Annex A)

The DCGL (derived concentration guideline level) for radionuclides of interest have been identified by BGRR (see Annex B).

Additional direct surface readings and soil samples will be obtained by BGRR staff, and the analytical results will be available at a later date to use in interpreting the ISOCS analysis.

2.4 Define the boundaries.

The pipeline from BGRR Exhaust Stack to the east end of the Fan House (Bldg 704, on the hill SE of reactor), and from the Fan House to the Pile Fan Sump (in the parking lot drive E of the reactor and S of Bldg 801) is 4-in diameter, cast iron of approximately 40 meters (130 feet) in length. The surface over the pipeline is approximately 60% earth and grass and 40% asphalted. The pipeline from the Sump to Building 801 is 2-in diameter, stainless steel of approximately 15 m (50 ft) in length, all asphalt covered. Approximately 40 m of pipeline below Bldg 704 will not be removed at this time.

The area of interest on the sub-surface is a corridor of overburden soil along the pipeline, the pipe itself, and any contaminated soil that may be delineated under the pipe. The pipeline, where it exits from Bldg 704 running northward, is at approximately 8 ft depth, so that a significant amount of overburden will be removed to expose the pipe and to slope the sides of the excavation. From this location, the pipeline does not drop as rapidly as the surface contour, so that the pipe enters the sump at only approximately 2 ft depth and the trench width can be narrowed as the excavation progresses towards the sump. The pipeline between the stack and Bldg 704, and between the sump and Bldg 801 is only 2-4 ft below grade so that extensive excavation and sloping will not be required.

2.5 Develop decision rules. If there is an identifiable gamma peak in the ISOCS spectrum that can be attributed to a source location in the ISOCS field of view, the activity will be quantified using an planar distribution geometric model representing the soil surface. Otherwise, it will be concluded that the combination of source activity and shielding is such that it is below the detection capability of the ISOCS.

2.6 Specify the tolerable limits on decision errors.

The decision error of most concern would be to conclude that the instruments cannot detect the activity, when perhaps with a longer counting time, it might have been able to. The consequence of counting longer than necessary would primarily be the eventual loss in resolution caused by electronic instrument drift, or the delay of excavation progress due to overly-conservative counting times. To provide an indication of instrument analysis variability, performance stability, and drift, duplicate and replicate analysis of samples will be performed:

- Field Replicate - 1 in 20 (5%) samples will be counted twice
- Field Duplicate/Split - 1 in 20 (5%) samples will be collected as double volume, composited, split, and counted separately
- Field Consistency - each day of sampling, one sample should be recounted after a period of two hours has elapsed since the initial count

2.7 Optimization.

2.7.1 *in situ* considerations. The use of the ISOCS instrument to directly scan soils and pipes uncovered *in situ* during in the excavation is complicated by two aspects:

- the large field of view of the detector without collimators does not allow the distinction between contaminated materials still inside the pipe from those in the soil surrounding the pipe, and
- when collimators are added to restrict the field of view, the weight of the instrument and cart makes it difficult to maneuver over uneven excavation surfaces.

2.7.2 Sample scanning. The primary analysis mode used in support of the excavation will be the field screening/analysis of soil samples in a reproducible, shielded geometry. The Canberra annular collimators will be arranged to construct a shielded counting cavity around the detector that accepts a 1 liter poly bottle. Soil sample bottle must be filled to the upper crown to meet the description of the geometry model.

2.7.3 Detection sensitivity. The primary information need for the control of excavation is the identity of radionuclides present and whether they are present at levels near or above the DCGL. The gamma spectrum will be accumulated for a time that assures that the minimum detectable concentration is less than 50% of the DCGL for the gamma emitters identified in the BGRR list (Annex B).

- for soil sample counted in the shielded cavity, the acquisition time will be 5 minutes (300 seconds)
- for analysis of finite objects such as sections of pipe, acquisition time will be 10 minutes (600 seconds)
- for analysis of trench wall/bottom the acquisition time will be 16.6 minutes (1000 seconds) to maintain consistency with previous surface soil evaluations.

2.7.4 Sample Sequencing. For this project, samples will be analyzed by both the ISOCS and Betascint instruments. Because the BetaScint analysis requires manipulation of the potentially contaminated soil in an un-contained geometry, special sample handling procedures and a radiological material handling area have been specified. ISOCS operators will be familiar with these procedures and comply with requirements.

3.0 Equipment

ISOCS Hardware

- a. the InSpector portable gamma spectroscopy interface,
- b. a broad energy germanium (BEGe) detector with preamp,
- c. Multi-Attitude Cryostat with extended detector to allow use of a back-shield,
- d. notebook computer with Canberra Genie, PROcount-2000 and ISOCS software,
- e. assorted connecting cables, power cords, batteries and battery chargers, and
- f. a cart for relocating and positioning the detector and shield assembly, and
- i. wheel chocks.

Annular shields and field of view collimators

- a. Set of 2-inch thick annular shields (3 pieces),
- b. 2-inch thick back shield (2 pieces),
- c. 1-inch annular shields (3 pieces),
- d. 1-inch thick end cap, and
- e. Collimator frame extensions

Additional Equipment and Instruments

- a. Calibrated microR or microRem meter
- b. Digital camera or Polaroid camera w/ film
- c. Tape measure, 100 foot or 50 m
- d. Field notebook
- e. 3.5 inch disk for file transfer
- f. DAT vehicle
- g. 50 ft extension cords

4.0 COORDINATION

4.1 Health and Safety BGRR Health and Safety POC : Reggie Suga x-8248

- (i) Samples may contain radioactive material contamination. There is a Radiation Work Permit (RWP) for the handling of contaminated soil samples during the performance of this analysis. Operators must comply with the requirements of the RWP.
- (ii) Sample handling procedures are specified in a separate soil handling/processing procedure. Personnel should read and be familiar with that procedure prior to performing soil analysis.
- (iii) When working in parking lots and driveways be aware of traffic, and use vehicle flashers or cones to warn drivers and increase your visibility.
- (iv) Some analysis positions are on a sloped hillside. Choose your footing carefully, and brace the instrument carefully to avoid falls or tip-overs.
- (v) Be aware of heat/cold stress concerns when working in open uncovered areas. A light-weight, pop-up awning for shade and cover is available and should be used. Fluids should be consumed regularly to replace body losses.

4.2 Facility Access

Primary: Steve Masciulli x-8247

Alternate: Manni Lilimpakis x-7628

5.0 SPECIFIC SCANS AND PARAMETERS

Specific areas of interest are identified in the BGRR survey plan (Ref 3). Individual targets or scans within the areas may be added at the discretion of the operator when in the area. A project sample identification scheme is described.

5.1.1 ASTD Project Sample Numbers.

Targets for gamma spectrum acquisition will be identified with a sample number of the form:

aaaxnnnz

where aaa a three character code identifying the portion of the excavation

(FPE) for Fan Pipe Excavation Sample

(FSE) for Fan Sump Excavation Sample

- x a single character code identifying the scan target
 - S** For a sample counted within the field sample analysis cavity
 - P** For a section of pipeline
 - T** For a section of the excavated trench
 - C** For a section of the concrete sump
- nnn is a three digit number incrementing in sequence indicating a sample sequence number for the project
- z is a single letter indicating a subsequent scan of the same sample, such as at a different target center, or from a different angle or position

For example: **FPES012b** is the second scan (**b**) of sample (**S**) number 12 (**012**) obtained during the fan house pipeline excavation (**FPE**). The code **S** for sample indicates it was counted in the field analysis cavity.

The ASTD Project Sample Number will be entered into the scan description by the instrument operator during scan acquisition, to tie the sample number to the scan record file. The BGRR sample number will also be entered into the scan description field and the field notebook. The ASTD Sample Number will also be cross-referenced with the default scan filename on the File Custody Record Form.

5.1.2 Scan Acquisition

- a. Scans should be accumulated to detect the radioactivity and to investigate the impact of ISOCS configuration on the analytical results.
- b. Initial scan time of 300 seconds; longer if necessary to see activity peaks especially in highly shielded objects such as through the pipe wall
- c. ISOCS codes for sample type and geometry

Sample Type **ISOCS Generic** Geometry **ISOCS**

Table 5-1. Studies of the Fan Pipeline Excavation, BGRR

Sample No	Annular Shield	Field of View	Initial Count Time	Sample Size	Orientation	Note
FPESnnna	2 inch around detector	180° inside with cavity capped	300 sec	1 liter	Vertical sample count configuration	Fan Pipe Excavation Sample
FSESnnna	1 inch around sample		300 sec	1 liter		Fan Sump Excavation Sample
FPETnnna	2 inch around detector	90° field of view	600 sec	1 sq meter field of view	Vertical down towards trench bottom or perpendicular to surface	

6.0 REFERENCES

- 6.1 BNL-DAT, *In Situ* Analysis Quality Assurance Project Plan, Version 1, July 7, 1999.
- 6.2 BNL-DAT SOP-DAT1, *In Situ* Gamma Spectroscopy Using the Canberra ISOCS System, version 1, August 19, 1999.
- 6.3 BGRR Sampling and Analysis Plan for Pile Fan Sump Excavation (in press).
- 6.4 BGRR Procedure for Handling Contaminated Soil Samples (in press).

ANNEX A Previous Analytical Results

- 1 **Surface Activity Scans.** A summary of the results of the *in situ* analysis of surface soils is provided in Table 1.1. Detailed listings of each sample analysis are provided in the Appendix.

Table 1.1. Average Radioactivity in Surface Soils (in pCi/g) in the Vicinity of the Pipeline Between the BGRR Fan House (Bldg 704) and the Hot Laboratory (Bldg 801)

Radio-nuclide	Ground Surface Facing the Detector		
	Grass (N=11)	Asphalt near Bldg 704 and Diagonal Drive (N=11)	Asphalt near Bldg 801 (N=9)
K-40	4.6 ± 1.3 (10)	9.7 ± 1.8 (10)	11.7 ± 2.6 (9)
Cs-137	0.43 ± 0.26 (9)	0.9 ± 0.8 (11)	0.5 ± 0.6 (8)
Pb-212	0.56 ± 0.27 (10)	1.0 ± 0.7 (11)	1.2 ± 0.8 (9)
Pb-214	0.39 ± 0.15 (11)	0.8 ± 0.2 (11)	1.0 ± 0.3 (8)
Ac-228	0.5 ± 0.5 (5)	0.6 ± 0.3 (2)	0.9 ± 0.6 (6)
Am-241	mda: # 0.8	mda: # 8	mda: # 8
Co-60	mda: # 0.005	mda: # 0.08	mda: # 0.08

NOTE: (*) is the number of samples in the ground surface category that reported detection of the radionuclide
Errors are reported as ± 2s.

The analysis consistently identified naturally occurring radionuclides (K-40, Uranium series, and Thorium series) and Cs-137 in the gamma spectra. The activity concentration of Cs-137 in the grass area (the area not covered by asphalt) is consistent with global fall-out levels. The minimum detectable activities of other radionuclides are indicated.

- 2 **Pipeline.** Possible contaminants in the pipeline, or in the soil if the pipeline had leaked, can be inferred from the analysis of the sludge and water in the Pile Fan Sump, where pipeline constituents accumulated. Results of prior analysis of water and sludge samples from the Pile Fan Sump are available for review. Prior analysis has detected different radionuclides in the water and sludge phases.
 - 2.1 Sampling and analysis results of the Pile Fan Sump from March 1997 are summarized in Table 2.1. The sampling identified elevated concentrations of Tritium, Sr-90, Cs-137, and Ra-226 in sump water; analyses of sump sludge identified elevated concentrations of Cs-137, Ra-226, Am-241, and Co-60.

Table 2.1. Contaminants Observed in the Pile Fan Sump Contents in 1997 Sampling

Radionuclide	Concentration in water (pCi/liter)	Concentration in sludge (pCi/liter)
Tritium (H-3)	260,000	NR
Gross beta	6,600	NR
Cs-137	2,000	15,700
Ra-226	2,300	365
Am-241	NR	71
Co-60	NR	3.1
Sr-90	2,270	NR

NR = Not Reported

Ref: Root Cause Analysis Summary Form, Occurrence CH-BH-BNL-1997-0012, Apr 1997.

- 2.2 The PFS was opened on August 12, 1999 following a significant rainfall on August 11. Results of analysis of a water sample obtained from the volume in the sump (sample 9908527-02) are provided in Table 2.2.

Table 2.2. Contaminants Observed in Pile Fan Sump Contents in Aug 1999 sampling

Radionuclide	Concentration in water (pCi/liter)
U-238/234	23.0 ± 3.4
Pu-239/240	13.5 ± 2.8
Cs-137	102,000 ± 9,500
Am-241	3.0 ± 1.0
Na-22	19.3 ± 5.5
Sr-90	76,500 ± 40

- 2.3 While the PFS was open on August 12, 1999, an *in situ* gamma spectrum was obtained using the ISOCS instrument (ASTD Sample PFS2123b). The analysis identified Cs-137 in the gamma spectrum. The analysis did not detect any other gamma-emitting radionuclides observed in the water (Ra-226) or in the sludge (Am-241, Co-60 and Ra-226). Estimated activity concentrations and minimum detectable concentrations are provided in Table 2.3 for each ISOCS geometry model that was evaluated.

Table 2.3. Activity Concentrations in PFS Determined by ISOCS Analysis-Open Sump Cover

Medium	Assumed Thickness	Activity Cs-137			Total Activity in the sump (μCi)		
		(dpm / 100 cm ²)	(μCi / 100 cm ²)	(μCi / g)	Cs-137	Am-241	Co-60
Residue on Sump Floor	0.05 in evenly over surface	4.9 E+06 ± 0.6 E+06	2.2 ± 0.3	0.11 ± 0.014	378 ± 42	< 4.72	< 0.7
Sludge pool	10 in deep; 1.5 ft ³	---	---	0.019 ± 0.001	1,280 ± 180	ND	< 1.92
Residue on Sump Floor	0.05 in evenly over surface	1.6 E+06 ± 0.2 E+06	0.73 ± 0.10	0.035 ± 0.005			
Residue on Upper S Wall	0.05 in evenly over surface	7600 ± 4900	0.0034 ± 0.0022	170 ± 110 pCi/g	---	< 0.004 μCi / 100 cm ²	< 0.003

ND = not determined due to absorption attenuation in thick constituents.
 --- = not calculated for the assumed geometry
 Errors are reported as ± 2s.

2.4 The PFS was opened again on August 23, 1999 and samples of the sludge and smears of removable contamination of the sump interior were obtained. Analysis of the sludge sample (No. 99082309-01) had Cs-137 reported as 0.104 μCi/g with 1-sigma error of 8.5% and Am-241 was not detected. Five smears (No 990823-01 to -05) were reported as detecting Cs-137 (0.6-1.3E-03 μCi/smear) and Am-241 (8.5E-05 μCi/smear). Am-241 was detected on 3 of 5 smears, and Am-241 was not detected when the Cs-137 activity was below 7E-04 μCi/smear.

Section 8

Analysis of Surface Soil Radioactivity BGRR Stack to Fan House Drain Pipeline Excavation Area

1. Purpose

In situ gamma spectroscopy measurements with the ISOCS instrument were performed on undisturbed surface soils and asphalt in the area between the BGRR Fan House, Building 704, and the Exhaust Stack near Building 802. The measurements were performed to characterize surface soils above the Stack Drain Pipeline prior to excavation, and to establish baseline conditions of adjacent surface areas prior to laying down soil excavated during the pipeline removal. This report addresses measurements in areas that had exposed grass or soil at the surface; areas covered by asphalt pavement will be addressed in a subsequent report.

2. Background.

The Fan House Pipeline is an underground pipe that runs approximately 40 meters (130 feet) from the BGRR Fan House, Bldg 704 in an easterly direction, under Building 715 to a point between Building 802 and the Stack. (See figure 1). The pipeline collects rainwater from the exhaust stack (actively shared with the HFBR), carries it to the Fan House drain system and then to the Pile Fan Sump. The BGRR is presently planning an excavation of the pipeline (and the sump), as part of the decommissioning program at the BGRR.

3. Known Contaminants

3.1 **Surface soils.** Analytical results from previous sampling of the ground surface in this specific location were not provided. BGRR will perform sampling of the soils as part of the baseline characterization to support the excavation planning.

3.1.1 In a cooperative effort between BNL and the Suffolk County Department of Health Services, soil samples from farms in the vicinity of the Laboratory are collected annually. In data for 1996 provided in a recent *BNL Site Environmental Report*, the analysis of five surface soil samples for three off-site farms confirmed the presence of natural radionuclides such as K-40 (5.9 - 8.0 pCi/g) and Pb-212 (0.7 - 1.4 pCi/g), as well as the globally-distributed Cs-137 (non detect to 0.4 pCi/g).

3.1.2 Surface soils in the area north of Building 704 were evaluated *in situ* by the ASTD Project in September 1999. In that evaluation the analysis consistently identified naturally occurring radionuclides (K-40, Uranium series, and Thorium series) and Cs-137 in the gamma spectra. The activity concentration of Cs-137 in the grass area (the area not covered by asphalt) was consistent with global fall-out levels. The results reported are summarized in Table 1.

Table 1. Average Radioactivity in Surface Soils (in pCi/g) in the Vicinity of the Pipeline Between the BGRR Fan House (Bldg 704) and the Hot Laboratory (Bldg 801)

Radio-nuclide	Ground Surface Facing the Detector		
	Grass (N=11)	Asphalt near Bldg 704 and Diagonal Drive (N=11)	Asphalt near Bldg 801 (N=9)
K-40	4.6 ± 1.3 (10)	9.7 ± 1.8 (10)	11.7 ± 2.6 (9)
Cs-137	0.43 ± 0.26 (9)	0.9 ± 0.8 (11)	0.5 ± 0.6 (8)
Pb-212	0.56 ± 0.27 (10)	1.0 ± 0.7 (11)	1.2 ± 0.8 (9)
Pb-214	0.39 ± 0.15 (11)	0.8 ± 0.2 (11)	1.0 ± 0.3 (8)
Ac-228	0.5 ± 0.5 (5)	0.6 ± 0.3 (2)	0.9 ± 0.6 (6)
Am-241	mda: # 0.8	mda: # 8	mda: # 8
Co-60	mda: # 0.005	mda: # 0.08	mda: # 0.08

NOTE: (*) is the number of samples in the ground surface category that reported detection of the radionuclide
Errors are reported as ± 2s.

3.2 **Pipeline.** Possible contaminants in the pipeline, or in the soil if the pipeline had leaked, can be inferred from the analysis of the sludge and water in the Pile Fan Sump, where pipeline constituents accumulated.

4. Pipeline Geometry

Information is available on the geometry and constituents of the Fan House Pipeline from construction drawings.

- 4.1 Plant Engineering has attempted to mark the pipe location using sounding techniques, with some success near the ends where the pipe exits Bldg 704 and enters the PFS. In the center of the run, the pipe location was lost amid the noise of crossing pipes and interference.
- 4.2 Between the Fan House, Building 704, and the sump, the pipeline is a 4-inch diameter cast iron pipe at approximately 2-4 ft depth. An asphalt apron extends from the Fan House for approximately 30 feet. Midway to the sump, an asphalt access drive crosses over the pipeline on a diagonal. The area where the pipe enters the sump and the area over the sump itself are also asphalt. The remainder of the surface is rough grass over surface soil with steep local grades.
- 4.3 Between the PFS and Building 801 the line is a 2-inch diameter, stainless steel line, running adjacent to the 14-inch acid off-gas line. The surface area over this pipeline is asphalted.

5. ISOCS Scan Acquisition and Analysis

- 5.1 **Personnel** Gamma scan acquisition was performed by Larry Milian, and Tom Roberts, while spectrum analysis and ISOCS geometry modeling were performed by Dave Watters. The report generation and data interpretation were performed by Dave Watters and Larry Luckett. All individuals are members of the ASTD Project Data Acquisition Team under the direction of the Project Analytical Physicist, Larry Luckett, CHP.
- 5.2 **Surface Activity Scans**. Scans were acquired on a 5-m equilateral triangular grid at 24 locations (12 asphalt and 11 grass and 1 horizontal pipe) along and either side of the apparent pipeline run between Bldg 704 and the Exhaust stack Base near Bldg 802. A drawing showing the general location of the survey area is provided as Figure 1. The ISOCS measurements were performed using the 180° field of view collimator with the 1.75 in (4.4 cm) annular lead side shields. The detector was positioned perpendicular to the local ground surface and followed the ground surface as terrain slopes varied. The distance from the face of the detector to the ground surface was 39.4 inches (100 cm). The spectrum at each location was accumulated for 1000 seconds (16.7 minutes).
- 5.3 **Spectrum Analysis**. Analysis was performed assuming that detected radionuclides were distributed uniformly in a 6-inch (15-cm) thick layer of soil with a density of 1.6 g/cm³.

6.0 Results

- 6.1 **Surface Activity Scans**. A summary of the results of the *in situ* analysis of surface soils is provided in Table 2. Detailed listings of each sample analysis are provided in the Appendix.
- 6.2 **Anomalous Radionuclides**. The analysis did not identify any locations where the local spectrum deviated from the nominal set of naturally occurring radionuclides seen throughout the area.

7.0 Conclusions

The ISOCS was capable of detecting and quantifying gamma-emitting radionuclides in the surface soils at sensitive levels. Small but detectable levels of Cs-137, consistent with global fall-out, were observed, along with naturally occurring radionuclides. Several locations of possible interference from sources in adjacent structures were identified.

8.0 References

- 8.1 ASTD Project-Specific Survey Plan No. 99-04, *Surface Trace of BGRR Fan House Drain Pipeline*, Version 1, August 10, 1999.
- 8.2 BNL-DAT, *In Situ* Analysis Quality Assurance Project Plan, Version 1, July 7, 1999.

- 8.3 BNL-DAT SOP-DAT1, *In Situ* Gamma Spectroscopy Using the Canberra ISOCS System, version 1 (Draft) of August 19, 1999.
- 8.4 *BNL Site Environmental Report for Calendar Year 1996*, Report BNL-52543, January 1998.
- 8.5 ASTD Project Data Report No 99-04-01, Analysis of Surface Soil Radioactivity -- BGRR Fan House Drain Pipeline Excavation Area, September 27, 1999

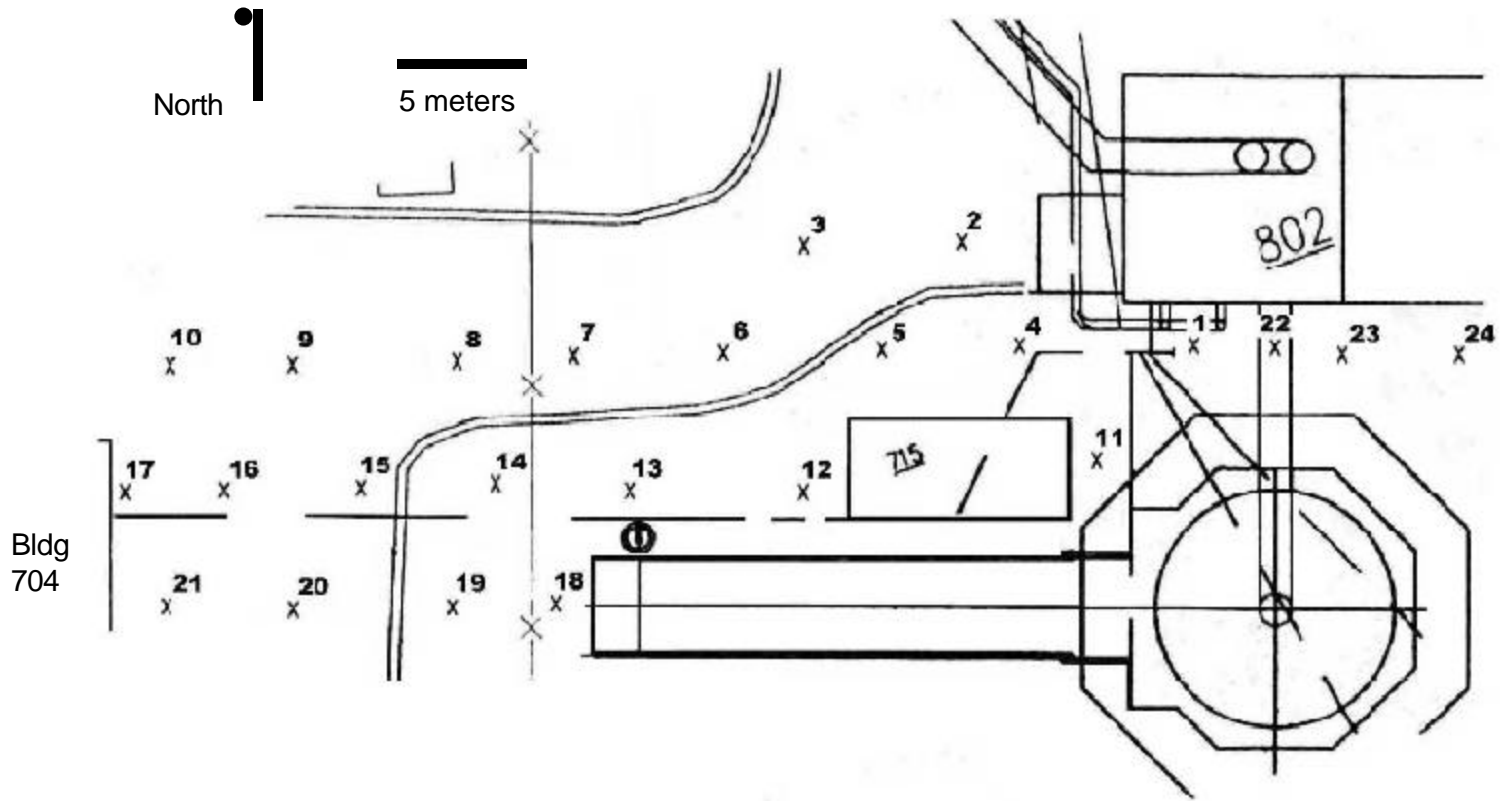


Figure 1. Locations of in situ gamma spectrum acquisition in the area between Bldg 704 and the Exhaust Stack.

Appendix 1. Prior Analysis of Water and Sludge Samples from the Pile Fan Sump

- A1. Sampling and analysis results of the Pile Fan Sump from March 1997 are summarized in Table A1. The sampling identified elevated concentrations of Tritium, Sr-90, Cs-137, and Ra-226 in sump water; analyses of sump sludge identified elevated concentrations of Cs-137, Ra-226, Am-241, and Co-60.

Table A1. Contaminants Observed in the Pile Fan Sump Contents in 1997 Sampling

Radionuclide	Concentration in water (pCi/liter)	Concentration in sludge (pCi/liter)
Tritium (H-3)	260,000	NR
Gross beta	6,600	NR
Cs-137	2,000	15,700
Ra-226	2,300	365
Am-241	NR	71
Co-60	NR	3.1
Sr-90	2,270	NR

NR = Not Reported

Ref: Root Cause Analysis Summary Form, Occurrence CH-BH-BNL-1997-0012, Apr 1997.

- A.2 The PFS was opened on August 12, 1999 following a significant rainfall on August 11. Results of analysis of a water sample obtained from the volume in the sump (sample 9908527-02) are provided in Table A2.

Table A2. Contaminants Observed in Pile Fan Sump Contents in Aug 1999 sampling

Radionuclide	Concentration in water (pCi/liter)
U-238/234	23.0 ± 3.4
Pu-239/240	13.5 ± 2.8
Cs-137	102,000 ± 9,500
Am-241	3.0 ± 1.0
Na-22	19.3 ± 5.5
Sr-90	76,500 ± 40

- 3 While the PFS was open on August 12, 1999, an *in situ* gamma spectrum was obtained using the ISOCS instrument (ASTD Sample PFS2123b). The analysis identified Cs-137 in the gamma spectrum. The analysis did not detect any other gamma-emitting radionuclides observed in the water (Ra-226) or in the sludge (Am-241, Co-60 and Ra-226). Estimated activity concentrations and minimum detectable concentrations are provided in Table A3 for each ISOCS geometry model that was evaluated.

Table A3. Activity Concentrations in PFS Determined by ISOCS Analysis-Open Sump Cover

Medium	Assumed Thickness	Activity Cs-137			Total Activity in the sump (μCi)		
		(dpm / 100 cm ²)	(μCi / 100 cm ²)	(μCi / g)	Cs-137	Am-241	Co-60
Residue on Sump Floor	0.05 in evenly over surface	4.9 E+06 ± 0.6 E+06	2.2 ± 0.3	0.11 ± 0.014	378 ± 42	< 4.72	< 0.7
Sludge pool	10 in deep; 1.5 ft ³	---	---	0.019 ± 0.001	1,280 ± 180	ND	< 1.92
Residue on Sump Floor	0.05 in evenly over surface	1.6 E+06 ± 0.2 E+06	0.73 ± 0.10	0.035 ± 0.005			
Residue on Upper S Wall	0.05 in evenly over surface	7600 ± 4900	0.0034 ± 0.0022	170 ± 110 pCi/g	---	< 0.004 μCi / 100 cm ²	< 0.003

ND = not determined due to absorption attenuation in thick constituents.

--- = not calculated for the assumed geometry

Errors are reported as ± 2s.

- A.4 The PFS was opened again on August 23, 1999 and samples of the sludge and smears of removable contamination of the sump interior were obtained. Analysis of the sludge sample (No. 99082309-01) had Cs-137 reported as 0.104 μCi/g with 1-sigma error of 8.5% and Am-241 was not detected. Five smears (No 990823-01 to -05) were reported as detecting Cs-137 (0.6-1.3E-03 μCi/smear) and Am-241 (8.5E-05 μCi/smear). Am-241 was detected on 3 of 5 smears, and Am-241 was not detected when the Cs-137 activity was below 7E-04 μCi/smear.
- A.5 Additional direct surface readings and soil samples will be obtained by BGRR staff. The analytical results will be available at a later date for use in interpreting the ISOCS analysis.

ACTION	INFO	FILE BNL ASTD Project
Connie Simiele - BGRR	Paul Kalb - ASTD Project Steve Masciulli - BGRR	Topical Data Reports

FROM Larry Luckett, Analytical Physicist, ASTD Project **DATE** December 1, 1999

SUBJECT: Analysis of Soil Samples, Fan House Pipeline Test-Dig

- On October 20, 1999 the BGRR transferred to the ASTD Project 12 samples obtained from the Fan House Pipeline test dig pit adjacent to Building 704. The samples, six labeled as soil and six labeled as asphalt, were in 1 liter poly bottles.

The samples were individually analyzed in Building 830 using the ISOCS gamma spectrometer. The lead shielding was arranged on the instrument cart for field sample analysis, providing a 1.75 inch annular shield around the detector and a 1 inch annular shield around and on top of the sample bottle. Each sample was weighed, a gamma spectrum accumulated for 900 seconds, corrected for ambient background in the shielded enclosure, modeled using the ISOCS software, and reported as activity per gram of sample. The results of the analysis of the soil samples is provided in the table.

Table 1. Radioactivity concentration in soil samples from test dig adjacent to the Fan House.

ASTD Sample No	BGRR Sample Description	Activity (pCi/g)		
		Cs-137	Am-241	K-40
FHS0120a	Soil at 1 ft depth	# 0.11	# 0.19	3.5 ± 1
FHS0220a	Soil at 2 ft depth	# 0.11	# 0.19	2.8 ± 1
FHS0320a	Soil under pipe joint	43 ± 4	# 0.48	2.1 ± 1
FHS0420a	Soil 6" below joint	38 ± 3	# 0.41	2.4 ± 1
FHS0520a	Soil 6"-12" below joint	60 ± 6	# 0.55	1.8 ± 1
FHS0620a	Soil 12"-18" below joint	57 ± 5	# 0.50	2.5 ± 1

- On October 21 1999 the ASTD Project used the ISOCS instrument to count soil and asphalt samples during a excavation adjacent to the Pile Fan Sump to uncover pipe joints for observation. Samples were provided in 1 liter poly bottles without a chain of custody.

The samples were individually analyzed in Building 701 using the ISOCS gamma spectrometer. The lead shielding was arranged on the instrument cart for field sample analysis, providing a 1.75 inch annular shield around the detector and a 1 inch annular shield around and on top of the sample bottle. Each sample was weighed, a gamma spectrum accumulated for 900 seconds, corrected for ambient background in the shielded enclosure, modeled using the ISOCS software,

and reported as activity per gram of sample. The results were reported to BGRR personnel as the results were calculated in the field. The results of the analysis of the soil samples is provided in Table 2:

Table 2. Radioactivity concentration in soil samples from test dig adjacent to the Pile Fan Sump

ASTD Sample No	BGRR Sample Description	Activity (pCi/g)		
		Cs-137	Am-241	K-40
FSS0100a	4 ft depth close to sump	< 0.15	< 0.26	3.9 ± 2
FSS0200a	Asphalt pieces, south side of PFS (A-7)	< 0.19	<0.24	< 3.7
FSS0300a	Asphalt pieces, north side of PFS (A-8)	< 0.24	< 0.33	1.8 ± 2
FSS0400a	soil at 2.5' below 14" pipe south side PFS	638	< 1.6	2.4 ± 1.5

- On November 2, 1999 the BGRR transferred to the ASTD Project 2 samples obtained from the Fan House Pipeline test dig pit adjacent to the Pile Fan Sump.

The samples were individually analyzed in Building 830 using the ISOCS gamma spectrometer. The lead shielding was arranged on the instrument cart for field sample analysis, providing a 1.75 inch annular shield around the detector and a 1 inch annular shield around and on top of the sample bottle. Each sample was weighed, a gamma spectrum accumulated for 900 seconds, corrected for ambient background in the shielded enclosure, modeled using the ISOCS software, and reported as activity per gram of sample. The results were reported to BGRR personnel as the results were calculated. The results of the analysis of the soil samples is provided in the table.

Table 3. Radioactivity concentration in additional soil samples from test dig adjacent to the Pile Fan Sump

ASTD Sample No	BGRR Sample Description	Activity (pCi/g)		
		Cs-137	Am-241	K-40
FSS0500a	10/22/99 @1310 PFS - 4" inlet line at joint before sump wall	< 0.13	< 0.24	3.71
FSS0600a [may be same as FSS0100a]	10/21/99 PFS, 4' next to sump	< 0.19	<0.36	6.2

4. On November 9, 1999 the BGRR transferred to the ASTD Project 2 samples obtained from the Fan House Pipeline test dig pit adjacent to the Pile Fan Sump.

The samples were individually analyzed in Building 830 using the ISOCS gamma spectrometer. The lead shielding was arranged on the instrument cart for field sample analysis, providing a 1.75 inch annular shield around the detector and a 1 inch annular shield around and on top of the sample bottle. Each sample was weighed, a gamma spectrum accumulated for 900 seconds, corrected for ambient background in the shielded enclosure, modeled using the ISOCS software, and reported as activity per gram of sample. The results were reported to BGRR personnel as the results were calculated. The results of the analysis of the soil samples is provided in the table.

Table 4. Radioactivity concentration in further soil samples from test dig adjacent to the Pile Fan Sump

ASTD Sample No	BGRR Sample Description	Activity (pCi/g)			
		Cs-137	Co-60	Am-241	K-40
FSD0100a	PFS N. side under 18" pipe and at depth 0-2' soil	271	<0.22	< 1.34	9.28
	repeat analysis	264	<0.24	< 1.33	8.69
FSD0200a	PFS N. side under 18" pipe and at depth 5' soil	125	<0.27	<0.99	10.4



MEMO

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ACTION	INFO (w/o encl)	FILE
Matt Lebarge - BGRR	Paul Kalb, Proj Mgr Stephen Pulsford - BGRR Kevin Miller - DOE-EML	ASTD Project 30822-262-023

FROM Larry Luckett, ASTD Proj

DATE Mar 29, 2000

SUBJECT: Results of Soil Analysis during PFS Excavation

1. Enclosed is a tabulation of the data results from the ISOCS analysis of soil samples during the excavation of the Pile Fan Sump. This tabulation compiles and summarizes the analysis results that were available at the time of analysis in the ISOCS instrument logbook and transmitted verbally to BGRR staff as they needed the information.
2. Samples were provided by BGRR personnel in a 1-liter poly bottle and were counted on the ISOCS instrument in the sample analysis configuration using 1" and 2" lead annular side shields and 1" lead end cap. Samples were counted in the ISOCS support area established on the Main level (110' elevation) of Building 701. All samples were received from and have been transferred back to BGRR control under appropriate chains of custody record.
3. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern) and K-40 (a naturally-occurring radionuclide). Results are sorted in the table by the BGRR sample number identified on the chain of custody provided with the sample. Results for sample numbers with '*' or 'R*' indicate a recount of a sample in accordance with the provisions of the ASTD Project QAPP. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records.
4. The data set includes 581 primary samples and 25 quality recounts for a total of 606 samples analyzed.
 - All 581 samples were reported as Non Detected for Co-60, with a minimum detectable concentration typically 0.2 pCi/g.
 - All 581 samples were reported as Non Detected for Am-241, with a minimum detectable concentration typically 0.4 pCi/g.
 - The 581 samples exhibited a range of Cs-137: 414 samples reported as Non Detected with a minimum detectable concentration typically 0.4 pCi/g, 52 samples detected # 1.0 pCi/g, 73 samples detected between 1 and 23 pCi/g, 25 samples reported between 23 and 100 pCi/g, and 17 samples reported with activity concentration above 100 pCi/g.
 - All quality recounts of samples confirmed the initial analysis.
4. Should you have any questions, please contact me at x-2777.



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ACTION	INFO	FILE BNL ASTD Project
Connie Simiele - BGRR	Steve Masciulli - BGRR	30822-207
Steve Moss - BGRR (w/o encl)	Paul Kalb - ASTD Project	

FROM Larry Luckett, Analytical Physicist, ASTD Project **DATE** November 16, 1999

SUBJECT: Radiological Results for BGRR Fan House No. 3

1. Dust Sample Analysis. On September 16, 1999, BGRR provided two samples of dust, identified as extracted from the North face volute housing of Fan No 3. The samples were scanned in the field at Bldg 701, using 1.75" annular shields and 90° field of view collimators on the detector to reduce interference as much as possible. Results of the analysis are provided in Table 1.

2. *In Situ* Analysis at Fan No 3
 - a. At your request on November 9, Larry Milian and Tom Roberts, ASTD ISOCS Technical Team, and I used the ISOCS instrument to perform *in situ* analysis of the fan housing of Fan No 3, located in the BGRR Fan House, Building 704. Because of the dimensions of the housing, physical construction, and the layout of the fan room, there were no access points where the spectrum could be accumulated without interference from fan support structures. BGRR RCT support did allow us into the radiologically controlled area to access the most beneficial angle for spectrum acquisition of the North volute. We could not access nor evaluate the contents of the South volute, but from the fan design the contents should be similar to the North volute, prior to the dust extraction of September 16.
 - b. Discussion of the analysis: The fan housing was scanned *in situ* using 1.75" annular shields and 90° field of view collimators on the detector to reduce interference as much as possible. The detector was oriented horizontally at the fan housing volute, north face, where surveys indicated an accumulation of radioactivity. Spectra were accumulated from two symmetric positions: East of the housing facing West and West of the housing facing East. Activity was modeled as a layer of surface dust, uniformly covering the fan housing bottom (adjacent rectangular planes). The intervening structural members were adjusted in the model until the results from the symmetric scans were of similar magnitude.
 - c. Discussion of the results. Results of the *in situ* analysis are provided in Table 2. Given the uncertainty of internal fan geometry, the total activity remaining in the North volute of Fan No 3 after the dust extraction could be as much as twice the activity indicated in either of the individual *in situ* analyses. Due to the carbon steel wall thickness and low energy of the gamma photon, the ISOCS model could not realistically quantify a minimum detectable activity for the Am-241; however, the activity detected in the separate dust sample provides an indication of the relative magnitude of Am-241 that might be expected to be still in the fan.

Please contact me if you need further information.

Larry W. Luckett, CHP
Analytical Physicist, ASTD Project

TABLE 1. Analysis of dust samples extracted from Fan No 3

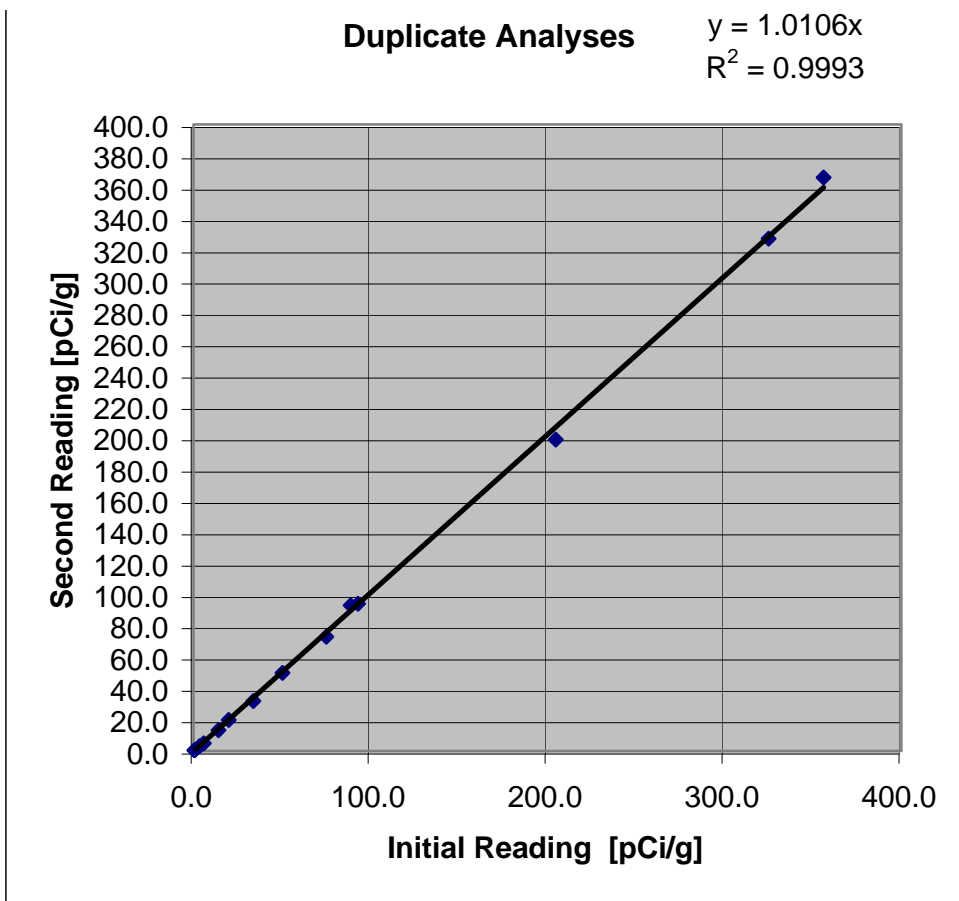
ASTD Sample No	Sample Description	Cs-137	Co-60	Eu-152	Eu-154	Am-241
FH3V129a	“Heavy”/ Gravity separated fraction 143 g in 0.75" layer, 1 liter poly bottle					
	Total Activity (µci)	2025 ± 140	30 ± 2	12 ± 2	4.4 ± 1.5	10.4 ± 6.3
	Activity Concentration (µCi/g)	14.2 ± 1.0	0.21 ± 0.01	0.085 ± 0.015	0.031 ± 0.010	0.073 ± 0.044
FH3V229a	“Light”/ Filter trapped fraction 49 g in 0.4" layer, 1 liter poly bottle					
	Total Activity (µci)	74 ± 5	1.2 ± 0.2	< 1.5	< 1.0	< 0.9
	Activity Concentration (µCi/g)	1.50 ± 0.11	0.024 ± 0.004	< 0.03	< 0.02	< 0.02

Notes: Uncertainties in the Table represent ±2s counting error;
 Values given as “<” represent the minimum detectable activity (MDA), and indicate the radionuclide was not detected

TABLE 2. *in situ* Analysis of Fan No 3 in Bldg 704, BGRR

ASTD Sample No	Sample Description	Cs-137	Co-60	Eu-152	Eu-154	Am-241
FH30329a	Fan No 3, North face volute “Left” side or from East facing West					
	Total Activity (µci)	114 ± 10	1.6 ± 0.1	< 2.5	< 1.7	ND/NQ
FH30429a	Fan No 3, North face volute “Right” side or from West facing East					
	Total Activity (µci)	165 ± 15	2.4 ± 0.2	< 4.2	< 2.9	ND/NQ

Notes: Uncertainties in the Table represent ±2s counting error;
 Values given as “<” represent the minimum detectable activity (MDA), and indicate the radionuclide was not detected.
 ND = Not Detected; a peak for the radionuclide was not observed in the spectrum
 NQ = Non-Quantifiable; the ISOCS model could not quantify a realistic MDA due to
 [1] physical constraints of the *in situ* spectrum acquisition (the fan wall thickness and carbon steel composition), and
 [2] the low energy of the radionuclide (Am-241) resulted in significant attenuation in the ISOCS model analysis



Cs-137

R.P.D.

0.5	0.5	0.00%
0.5	0.5	0.00%
3.5	3.1	12.90%
5.7	4.9	16.33%
14.1	13.3	6.02%
19.9	19.8	0.51%
33.9	31.8	6.60%
50.2	49.8	0.80%
75.1	73.0	2.88%
88.8	93.0	-4.52%
93.0	94.0	-1.06%
204.7	198.8	2.97%
325	327	-0.61%
356	366	-2.73%
2169	2164	0.23%

Minimum detectable concentrations (MDCs) at the 95% confidence level

ND = not detected; result was less than the 95% critical level

Uncertainty reported at the 95% confidence level and represents counting error and ISOCs-generated efficiency error.

Section 9

ASTD Project-Specific Survey Plan No. 2K-02

Use of BetaScint Instrument in Support of BGRR Fan House Drain Pipeline Excavation and Pile Fan Sump Removal

1.0 PURPOSE

Perform beta spectroscopy of soil samples from disturbed surface soils in the area between the BGRR Exhaust Stack, the Fan House, and the Pile Fan Sump to

- (i) monitor progress of excavation (have we excavated far enough to identify the limits of any contaminated pipe/soil?)
- (ii) assist with determination of soil disposition (which pile does this soil go into?)

NOTE: The use of the Canberra ISOCS instrument to support this BGRR excavation by the field analysis of samples for gamma activity is addressed in a separate ASTD Project Specific Survey Plan (PSSP No. 2K-01, Nov, 1999).

2.0 DATA QUALITY OBJECTIVES

2.1 State the Problem

BetaScint, Inc., has developed an instrument using fiber optics and scintillating phosphor fibers to discriminate high-energy beta particles, as from Sr-90 and U-238, from lower energy betas found in ambient background. Using the BetaScint instrument, in contrast to baseline technology, provides the opportunity to obtain data on radionuclide identification and quantification quickly, without having a delay for transportation and preparation of the sample. An area of interest for this application is the BGRR Fan House Pipeline (FHP), a drain pipe that runs from the BGRR Exhaust Stack, to the Fan House, Bldg 704, to the Pile Fan Sump, and into Bldg 801, as well as the Pile Fan Sump removal. Within the combination of activity present and self-absorption in the soil, data are needed to determine the radionuclide content of the surface and subsurface soils during excavation.

2.2 Identify the decision.

The decisions to be made are twofold:

- (i) a utility decision: Does the BetaScint instrument detect radionuclides in the soil with a sensitivity sufficient to guide the progress of excavation in a timely manner? and
- (ii) a comparability decision: Are the results of analysis by the BetaScint instrument comparable in accuracy and precision

2.3 Identify inputs.

Information is available on the geometry and constituents of the Fan House Pipeline and Pile Fan Sump from construction drawings. Plant Engineering has attempted to mark the pipe location using sounding techniques with some success near the ends, but not in the center of the run.

Results of prior analysis of water and sludge samples from the Pile Fan Sump are available for review. Prior analysis has detected different radionuclides in the water and sludge phases (see Annex A)

The DCGL (derived concentration guideline level) for Sr-90 has been identified by BGRR as 15 pCi/g.

Additional direct surface readings and soil samples will be obtained by BGRR staff, and the analytical results will be available at a later date to use in interpreting the BetaScint analysis.

2.4 Define the boundaries.

The pipeline from BGRR Exhaust Stack to the east end of the Fan House (Bldg 704, on the hill SE of reactor), and from the Fan House to the Pile Fan Sump (in the parking lot drive E of the reactor and S of Bldg 801) is 4-in diameter, cast iron of approximately 40 meters (130 feet) in length. The surface over the pipeline is approximately 60% earth and grass and 40% asphalted. The pipeline from the Sump to Building 801 is 2-in diameter, stainless steel of approximately 15 m (50 ft) in length, all asphalt covered. Approximately 40 m of pipeline below Bldg 704 will not be removed at this time.

The area of interest on the sub-surface is a corridor of overburden soil along the pipeline, the pipe itself, and any contaminated soil that may be delineated under the pipe. The pipeline, where it exits from Bldg 704 running northward, is at approximately 8 ft depth, so that a significant amount of overburden will be removed to expose the pipe and to slope the sides of the excavation. From this location, the pipeline does not drop as rapidly as the surface contour, so that the pipe enters the sump at only approximately 2 ft depth and the trench width can be narrowed as the excavation progresses towards the sump. The pipeline between the stack and Bldg 704, and between the sump and Bldg 801 is only 2-4 ft below grade so that extensive excavation and sloping will not be required.

2.5 Develop decision rules. If there is a significant instrument response that can be attributed to a source location in the detector field of view, the activity will be quantified using factors derived from correlating instrument response to spiked soil samples. Otherwise, it will be concluded that the combination of source activity and shielding is such that it is below the detection capability of the BetaScint instrument.

2.6 Specify the tolerable limits on decision errors.

The decision error of most concern would be to conclude that the instrument cannot detect the activity, when perhaps with a longer counting time or better sample preparation (uniform spread or dryness), it might have been able to. The consequence of longer preparation or counting than is necessary would primarily be the eventual loss in resolution caused by electronic instrument drift, or the delay of excavation progress. To provide an indication of instrument analysis variability, performance stability, and drift, duplicate and replicate analysis of samples will be performed:

- Instrument Replicate - 1 in 10 (10%) samples will be immediately re-counted, without disturbing the soil distribution in the sample tray.
- Field Duplicate/Split - 1 in 10 (10%) samples will be collected as double volume, composited, split, and counted separately.

- Instrument Consistency - each day of sampling, two samples should be recounted after a period of two hours has elapsed since the initial count. This involves re-distribution of the soil across the tray and repositioning the tray with respect to the detector.

2.7 Optimization.

2.7.1 Instrument considerations. The use of the BetaScint instrument to directly scan soil samples is complicated by two aspects:

- the large field of view of the detector without collimators requires a sample size of approximately 2 kg (5 lbs). The operator must ensure that sample soil is spread evenly at a uniform thickness across the sample tray.
- the range (energy) of the beta radiation exiting the soil surface is affected by moisture in the soil. The operator must insure that each individual soil sample is dried to the same soil moisture content as in the spiked soil samples used to establish the instrument response correlation.

2.7.2 Sample scanning. The primary analysis mode used in support of the excavation will be the screening/analysis of soil samples in a reproducible geometry. The instrument operator must ensure that adequate soil volume is obtained, and that consistent sample preparation is performed prior to analysis. Otherwise analysis results will be qualified to explain the limitation on use of the data.

2.7.3 Detection sensitivity. The primary information need for the control of excavation is the identity of radionuclides present and whether they are present at levels near or above the DCGL. The BetaScint analysis will be accumulated for a time that assures that the minimum detectable concentration is less than 50% of the DCGL for the Sr-90 identified by the BGRR (50% Of 15 pCi/g).

- for soil sample counted in the field laboratory, the acquisition time will be 5 minutes (300 seconds)
- other analysis times may be identified by the project Analytical Physicist in response to instrument stability observations.

3.0 Equipment

BetaScint Hardware

- a. the BetaScint beta radiation sensor/detector,
- b. DC power supply with A/C cord,
- c. cable for power supply to sensor,
- d. laptop computer with BetaScint software loaded, and
- e. null modem, female/female DB9 cable to connect sensor and computer

Additional Equipment and Instruments

- a. Calibrated microR or microRem meter,
- b. Digital camera or Polaroid camera w/ film
- c. Field notebook
- d. Sample counting trays and tray racks,
- e. heat lamps for drying samples,
- f. zip-lock bags, bottles or buckets for sample packaging,
- g. PPE as required by the RWP

4.0 COORDINATION

4.1 Health and Safety BGRR Health and Safety POC : Reggie Suga x-8248

- (i) Samples may contain radioactive material contamination. There is a Radiation Work Permit (RWP) for the handling of contaminated soil samples during the performance of the BetaScint analysis. Operators must comply with the requirements of the RWP.
- (ii) Sample handling procedures are specified in ERD-OPM-4.3. Personnel should read and be familiar with that procedure prior to performing soil analysis with the BetaScint instrument.
- (iii) When working in parking lots and driveways be aware of traffic, and use vehicle flashers or cones to warn drivers and increase your visibility.
- (iv) Be aware of heat/cold stress concerns when working in open uncovered areas. A light-weight, pop-up awning for shade and cover is available and should be used. Fluids should be consumed regularly to replace body losses.

4.2 BGRR Facility Access

Primary: Steve Masciulli x-8247

Alternate: Manni Lilimpakis x-7628

5.0 SPECIFIC SCANS AND PARAMETERS

Specific areas of interest are identified in the BGRR survey plan (Ref 3). Samples and or areas of samples will be determined by the BGRR Field Engineer. BGRR personnel will deliver samples to the ASTD field laboratory or ASTD personnel may be requested to assist with sampling to obtain the soil for analysis.

Samples will be counted on the ISOCS gamma spectroscopy system prior to performing BetaScint analysis. A project sample identification scheme is described in the PSSP for operation of the ISOCS instrument (PSSP No 2K-01). The ASTD Project Sample Number will be entered into the BetaScint analysis log by the instrument operator during analysis. The BGRR sample number will also be entered into the log and the field notebook.

6.0 REFERENCES

- 6.1 BNL-DAT, *In Situ* Analysis Quality Assurance Project Plan, Version 1, July 7, 1999.
- 6.2 BNL-DAT SOP-DAT1, *In Situ* Gamma Spectroscopy Using the Canberra ISOCS System, version 1, August 19, 1999.
- 6.3 BGRR Sampling and Analysis Plan for Pile Fan Sump Excavation (in press).
- 6.4 ERD-OPM-4.3, Procedure for ASTD Soil Sample Processing to Support BGRR Excavations, Rev 0 (in press).
- 6.5 Operational Procedures for Beta Sensor, BetaScint, Inc.

ANNEX A

Previous Analytical Results for Sr-90

1. **Pipeline.** Possible contaminants in the pipeline, or in the soil if the pipeline had leaked, can be inferred from the analysis of the sludge and water in the Pile Fan Sump, where pipeline constituents accumulated. Results of prior analysis of water and sludge samples from the Pile Fan Sump are available for review. Prior analysis has detected different radionuclides in the water and sludge phases.
 - 1.1 Sampling and analysis results of the Pile Fan Sump from March 1997 are summarized in Table A.1. The sampling identified elevated concentrations of Tritium, Sr-90, Cs-137, and Ra-226 in sump water; analyses of sump sludge identified elevated concentrations of Cs-137, Ra-226, Am-241, and Co-60.

Table A.1. Contaminants Observed in the Pile Fan Sump Contents in 1997 Sampling

Radionuclide	Concentration in water (pCi/liter)	Concentration in sludge (pCi/liter)
Tritium (H-3)	260,000	NR
Gross beta	6,600	NR
Cs-137	2,000	15,700
Ra-226	2,300	365
Am-241	NR	71
Co-60	NR	3.1
Sr-90	2,270	NR

NR = Not Reported

Ref: Root Cause Analysis Summary Form, Occurrence CH-BH-BNL-1997-0012, Apr 1997.

- 1.2 The PFS was opened on August 12, 1999 following a significant rainfall on August 11. Results of analysis of a water sample obtained from the volume in the sump (sample 9908527-02) are provided in Table A.2:

Table A.2. Contaminants Observed in Pile Fan Sump Contents in Aug 1999 sampling

Radionuclide	Concentration in water (pCi/liter)
U-238/234	23.0 ± 3.4
Pu-239/240	13.5 ± 2.8
Cs-137	102,000 ± 9,500
Am-241	3.0 ± 1.0
Na-22	19.3 ± 5.5
Sr-90	76,500 ± 40

- 2.4 The PFS was opened again on August 23, 1999 and samples of the sludge and smears of removable contamination of the sump interior were obtained. Analysis of the sludge sample (No. 99082309-01) had Cs-137 reported as 0.104 µCi/g with 1-sigma error of 8.5% and Am-

241 was not detected. Five smears (No 990823-01 to -05) were reported as detecting Cs-137 (0.6-1.3E-03 $\mu\text{Ci/smear}$) and Am-241 (8.5E-05 $\mu\text{Ci/smear}$). Am-241 was detected on 3 of 5 smears, and Am-241 was not detected when the Cs-137 activity was below 7E-04 $\mu\text{Ci/smear}$.

Section 10



MEMO

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ACTION	INFO	FILE
Memo for Matt Labarge, BGRR	Paul Kalb, DAT Steve Masciulli, BGRR	ASTD Project 30822-262
FROM Larry Lockett, CHP ASTD Project Analytical Physicist		DATE Apr 10, 2000
SUBJECT:	BetaScint Analysis of Composited PFS Soil Samples	

In his e-mail memo of March 30, 2000, Steve Masciulli identified 30 samples of soil from the Pile Fan Sump (PFS) excavation on which he requested BetaScint analysis. As he suggested, we combined the paired samples in his list to increase the sample volume for BetaScint requirements. We then sieved the soils to remove extraneous objects and organics, weighed each composite, and counted the samples on the BetaScint instrument. Results of the analysis are provided in the Table.

Even after compositing, most (10 of 15) samples were below the requisite mass for definitive BetaScint analysis. To provide you the most information possible, we analyzed each sample, even when the mass was sub-optimal. Results are reported in a "Response Range", indicating that while the actual concentration of Sr-90 could not be quantified, its relative magnitude could be estimated. The response ranges in this analysis are:

- Response Range 1: Sr-90 concentration less than 5 pCi/g.
- Response Range 2: Sr-90 concentration between 5 pCi/g and 15 pCi/g.
- Response Range 3: Sr-90 concentration greater than 15 pCi/g.

If you have any questions on the analysis or interpretation, please contact me or Larry Milian of the ASTD Project.

Composite PFS Soil Samples Counted on BetaScint Spectrometer				
BGRR Soil Sample I.D.	Previous ISOCS	BetaScint Sr-90 Analysis		Composite/Sieved Soil Weight, g
	Cs-137 pCi/g	Response Range (1)	Indicated pCi/g (2)	
PFS- 95	93.0	2	6.1	1396.5
PFS- 96	29.9			
PFS- 209	204.7	1	1.1	2250.4
PFS- 210	22.5			
PFS- 220A	44.5	1	0.3	2231.1
PFS- 222	31.3			
PFS- 235	2169.0	3	23	1816.5
PFS- 269	26.9			
PFS- 279	365.1	1	4.2	1433.0
PFS- 280	230.8			
PFS- 347	27.9	1	0.2	1728.8
PFS- 349	20.8			
PFS- 356	200.0	2	5.2	1186.2
PFS- 358	128.0			
PFS- 412	131.0	3	26	1867.3
PFS- 413	33.9			
PFS- 414	325.0	2	10	1708.4
PFS- 416	260.0			
PFS- 422	80.4	2	6.0	1583.8
PFS- 423	114.0			
PFS- 464	21.5	1	1.9	1816.8
PFS- 465	33.2			
PFS- 507A	83.2	2	6.8	2445.5
PFS- 508A	135.0			
PFS- 528	21.2	1	0.4	1675.0
PFS- 530	33.2			
PFS- 533	14.1	1	2.3	2270.6
PFS- 535	88.8			
PFS- 539	8.8	1	2.3	2530.2
PFS- 543W	13.1			

NOTES: (1) Response Range 1 : Sr-90 concentration less than 5 pCi/g
Response Range 2 : Sr-90 concentration between 5 pCi/g and 15 pCi/g
Response Range 3 : Sr-90 concentration greater than 15 pCi/g

(2) Due to sub-optimal volumes for soil samples, analytical response of the BetaScint instrument should be considered only semi-quantitative; response ranges are identified to provide interpretation of the instrument response.

Section 11



MEMO

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~~FILE~~ ~~ASTD Project~~

ACTION Steve Masciulli - BGRR **INFO** Paul Kalb, DAT **FILE** 30822-262
FROM Larry Lockett **DATE** Feb 22, 2000
SUBJECT: Analysis Results : Support Columns of the Above Grade Ducts

On February 17, 2000 the ASTD Project Technical Team used the ISOCS instrument to perform gamma spectrum analysis of cement columns of the BGRR Above Grade Duct on the rooftop of the Fanhouse, Building 704. These columns had been identified and marked by your surveyors as having surface contamination.

We located the ISOCS instrument with the detector positioned at 9 in (23 cm) from the marked surface of the concrete support column. The detector was shielded with 45 mm thick annular lead side-shields, and the field of view was restricted to 90 degrees. The detector was lifted on supports so that the geometry was optimized to have the field of view encompass the indicated active area. The observed field of view of the ISOCS in this geometry is 420 cm². Spectra were accumulated for 600 seconds. Additional spectra were accumulated at each location with the endcap closed, to account for activity in the duct interfering with the primary area of analysis.

The analysis of each spectrum indicated that the only radionuclide detected was Cs-137. The results are summarized in the table, below.

Surface Activity observed on concrete columns of Above Ground Ducts

Sample No Description	RDS-0229a Above Ground Support No 5, West		RDS03229a Above Ground Support No 6, West	
	Total µCi	Areal Concentration averaged over FOV µCi/100 cm ²	Total µCi	Areal Concentration averaged over FOV µCi/100 cm ²
Cs-137	4.6 ± 0.4	1.1 ± 0.1	0.18 ± 0.02	0.041 ± 0.004
Co-60	# 0.004	# 0.0009	# 0.003	# 0.0007
Eu-154	# 0.01	# 0.003	# 0.008	# 0.002
Am-241	# 0.015	# 0.004	# 0.007	# 0.002

- Notes:
1. Areal concentrations are averaged over the ISOCS 420 cm² field of view and are reported per 100 cm²; measurements with an instrument with a smaller detector, or considering only the actual contaminated surface area might be numerically different.
 2. Uncertainties (±) are reported as 1.96 sigma; MDA (#) is the 95% confidence level.

Larry W. Lockett, CHP
Senior Health Physicist



MEMO

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ACTION Memo for Matt Labarge, BGRR **INFO** Paul Kalb, DAT **FILE** ASTD Project
 30822-262

FROM Larry Lockett, CHP **DATE** Apr 5, 2000
 ASTD Project Analytical Physicist

SUBJECT: Preliminary Analysis Results : BGRR Above Grade Ducts

On April 5 the ASTD ISOCS Technical Team used the ISOCS instrument to perform gamma spectrum analysis of samples identified as having been obtained from the April 3 entry of the BGRR Above Grade Duct, on the rooftop over the fan house. The analysis of the spectra are summarized in the table below. Full data analysis reports are on file.

Activity Concentration on samples from Above Ground Ducts

Description	Sludge from Expansion Joint between Fans #1 and #2	Rust (4' to bottom) East side of Fan #1	Concrete dust from before Fan #2 donut	Concrete from N wall corner horizontal Fan #1	Concrete parts from scabbling
BGRR Sample No	AGD-1	AGD-2	AGD-3	AGD-4	AGD-5
ASTD Sample No	SAM00822	SAM00818	SAM00819	SAM00820	SAM00821
Radionuclide	Activity Concentration (pCi/g)				
Am-241	249,000 ± 246,000 [165,000]	ND [5]	136 ± 30	50 ± 11	23 ± 11
Cs-137	64E6 ± 4E6	2,850 ± 200	41,000 ± 3,000	7,400 ± 600	6,700 ± 500
Co-60	115,000 ± 8,500	7.5 ± 0.7	260 ± 10	37 ± 2	21 ± 2
Eu-152	ND [40,000]	. 3 ± 2	120 ± 10	20 ± 3	. 10 ± 5
Eu-154	ND [26,000]		50 ± 7	8 ± 1	

ND = Not Detected

[] = Minimum Detectable Concentration

Uncertainties are counting errors (1.96s);

Additional uncertainties due to volume and density variations are approximately ± 30 % and should be added to the tabulated results.

Section 12



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ACTION	INFO	FILE
Matt Labarge- BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Musolino-BGRR	Stephen Pulsford - BGRR	30822-262-023
Rob Stone-BGRR(URS)		

FROM Larry Luckett, CHP
ASTD Project Analytical Physicist

DATE August 3, 2000

SUBJECT: Results of Analysis for **Concrete Samples from Duct Core Bores, Bldg 704**

1. Enclosed is a tabulation of the results from the ISOCS analyses of four concrete samples collected by BGRR/URS during the evaluation of duct core boring operations, on the ground adjacent to Bldg 704.
2. Concrete samples were provided by BGRR/URS personnel in poly-wrapped bags (dry) and were counted on the ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. BGRR/URS RCT personnel assisted in identifying the "hot" side of the sample, so that we were assured of appropriate geometry. All samples were received from, and have been transferred back to, BGRR/URS control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Luckett, both members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Total Activity on Concrete Duct Surface from Duct Cores, Bldg 704
BGRR Duct Removal Project
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	CB-A-1-N	CB-9A	CB-5A	CB-5B
BGRR Sample Description	Concrete Chunk from Core Bore in Duct	Concrete Chunk from Core Bore in Duct Segment No 9	Concrete Chunk from Core Bore in Duct Segment No 5	Concrete Chunk from Core Bore in Duct Segment No 5
Sample Mass	315.7 g	178.4 g	132 g	275 g
Measured area of contaminated surface	33.2 cm ²	21.2 cm ²	19.6 cm ²	28.3 cm ²
ASTD Sample No	IG01317	IG01319	IG01320	IG01318
Radionuclide	Total Activity (pCi)			
Am-241	2.10 ± 0.68 [0.84]	ND [1.10]	ND [1.15]	1.02 ± 0.61 [0.96]
Cs-137	946 ± 84 [1.67]	642 ± 57 [1.52]	560 ± 50 [1.37.]	693 ± 61 [1.37]
Co-60	1.65 ± 0.29 [0.50]	ND [0.83]	ND [0.84]	1.02 ± 0.29 [0.59]
Eu-152	ND [1.90]	ND [1.54]	ND [1.62]	ND [1.47]
Eu-154	ND [1.33]	ND [1.08]	ND [1.13]	ND [1.05]

ND = Not Detected

[] = Minimum Detectable Activity in pCi

DATA QUALIFIERS:

1. Analysis assumes 1 mm thin, uniform, homogeneous distribution of contaminants on surface of concrete matrix
2. Note that results are provided in units of TOTAL ACTIVITY on the chunk, to assist in determination of package quantity for shipping purposes.



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ACTION	INFO	FILE
Matt Labarge- BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Musolino-BGRR	Stephen Pulsford - BGRR	30822-262-023
Rob Stone-BGRR(URS)		
FROM Larry Lockett, CHP		DATE July 19, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis for **Concrete Dust from Duct Cutting, Bldg 704**

1. Enclosed is a tabulation of the results from the ISOCS analyses of two concrete dust samples collected by BGRR during the evaluation of operations from the cutting area for duct section No 6 and 8, on the roof of Bldg 704.
2. Concrete samples were provided by BGRR personnel in 1000-ml HDPE bottles (wet) or in poly-wrapped bags (dry) and were counted on the ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Eric Barbour, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Activity Concentration in Concrete Dust from Duct Cutting, Bldg 704
BGRR Duct Removal Project
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	DC6	DC8
BGRR Sample Description	Dry concrete dust from Downcomer #6 in plastic bag	Wet concrete slurry ½ liquid - ½ solids from Downcomer #8
Sample Mass	1,372 g	1,320 g
ASTD Sample No	SAM00906	SAM00905
Radionuclide	Activity Concentration (pCi/g)	
Am-241	ND [0.37]	ND [0.38]
Cs-137	1.04 ± 0.26 [0.26]	3.13 ± 0.46 [0.26]
Co-60	ND [0.24]	ND [0.27]
Eu-152	ND [0.47]	ND [0.53]
Eu-154	ND [0.32]	ND [0.38]

ND = Not Detected

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix



MEMO

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ACTION	INFO	FILE
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Steve Musolino-BGRR	Stephen Pulsford - BGRR	30822-262-023
Rob Stone-BGRR(URS)		

FROM Larry Luckett, CHP
ASTD Project Analytical Physicist

DATE Aug 25, 2000

SUBJECT: Results of Analysis for **Concrete Sludge from Duct Cutting, Bldg 704**

1. Enclosed are tabulation of the results from the ISOCS analyses of five concrete sludge and water samples collected by BGRR during the evaluation of operations from the cutting area for duct section No9, on the ground adjacent to Bldg 704.
2. Sludge and water samples were provided by BGRR personnel and were counted on the ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Luckett, members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

NOTE: Sludge samples were provided by BGRR personnel in 160-ml HDPE containers which is a non-standard geometry for samples. Use of 1-liter wide mouth HDPE bottles would result in more accurate analytical results.

Activity Concentration in Concrete Dust from Duct Cutting, Bldg 704
BGRR Duct Removal Project
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No				DC6	DC8
BGRR Sample Description				Dry concrete dust from Downcomer #6 in plastic bag	Wet concrete slurry ½ liquid - ½ solids from Downcomer #8
Sample Mass				1,372 g	1,320 g
ASTD Sample No				SAM00906	SAM00905
Radionuclide	Activity Concentration (pCi/g)				
Am-241				ND [0.37]	ND [0.38]
Cs-137				1.04 ± 0.26 [0.26]	3.13 ± 0.46 [0.26]
Co-60				ND [0.24]	ND [0.27]
Eu-152				ND [0.47]	ND [0.53]
Eu-154				ND [0.32]	ND [0.38]

ND = Not Detected

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steven Masciulli - BGRR	Stephen Pulsford - BGRR	30822-262-023

FROM Larry Lockett, CHP
ASTD Project Analytical Physicist

DATE May 10, 2000

SUBJECT: Results of Analysis for **Cooler Coil Sections and Dusts**

1. Enclosed is a tabulation of the results from the ISOCS analyses of samples collected by BGRR during the duct entry and coil cutting of May 4 to view the air filters.
2. When appropriate to the sample matrix, samples were provided by BGRR personnel in a 1-liter poly bottle and were counted on the ISOCS instrument in the sample analysis configuration using 1" and 2" lead annular side shields and 1" lead end cap. Two bags of odd-sized or large pieces of coil were counted at a measured distance from the ISOCS instrument, and modeled appropriate to the acquisition geometry. All samples were received from, and have been transferred back to, BGRR control under appropriate chain of custody records.
3. Samples were counted in the ISOCS support area established on the Main level (110' elevation) of Building 701. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Dave Watters, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Results are sorted in the table by the BGRR sample number identified on the chain of custody provided with the sample. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

**Activity Concentration on samples of Cooler Coil and Dusts
from BGRR Below Ground Ducts**
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	AGD-024	AGD-025	AGD-026	AGD-027
Description	Rust and Fine Dust from South Duct Upstream of cooler coils	Section of cooler coil in 1 l bottle from South duct	Bag of 4 tubes from South Duct cooler coils	Bag of 7 tubes from North Duct cooler coils
Sample Mass	72.9 g in 1 l bottle 1/8 full	125 g 12 cm long by 3.5 cm dia	2,350 g	4,980 g
ASTD Sample No	SAM00870	SAM00871	IG01173	IG01174
Radionuclide	Activity Concentration (pCi/g)			
Am-241	60 ± 16 [17.6]	56 ± 28 [43]	29 ± 19 [30]	ND [12]
Cs-137	16,800 ± 1300 [21.7]	5,140 ± 400 [13.5]	6,600 ± 500 [28]	5,100 ± 400 [10.4]
Co-60	34 ± 3 [4.4]	18 ± 2.5 [3.7]	19 ± 5 [9.9]	20 ± 3 [1.7]
Eu-152	ND* [20]	ND [18.8]	ND [32]	16 ± 5 [14.2]
Eu-154	ND [14]	ND [13.2]	ND [23]	ND* [12.3]

ND = Not Detected

* = Indications of the presence of Eu-152 or Eu-154 at small, non-quantifiable levels

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steven Masciulli - BGRR	Stephen Pulsford - BGRR	30822-262-023

FROM Larry Luckett, ASTD Proj **DATE** May 04, 2000

SUBJECT: Results of Sample Analysis during Duct Entry/Cooler Characterization

1. Enclosed is a tabulation of the data results from the ISOCS analyses of samples collected during the duct entry to evaluate the air coolers. This tabulation compiles and summarizes the analysis results, which were available at the time of analysis in the ISOCS instrument logbook and transmitted verbally to BGRR staff as they needed the information.
2. When appropriate to the sample matrix, samples were provided by BGRR personnel in a 1-liter poly bottle and were counted on the ISOCS instrument in the sample analysis configuration using 1" and 2" lead annular side shields and 1" lead end cap. Some odd-sized or large pieces were counted at a measured distance from the ISOCS instrument, and modeled appropriate to the acquisition geometry. All samples were received from, and have been transferred back to, BGRR control under appropriate chain of custody records.
3. Samples were counted in the ISOCS support area established on the Main level (110' elevation) of Building 701. Spectrum acquisition was performed by members of the ASTD Project Team, Milian or Adams, and quantification modeling results were performed or reviewed by Watters or Luckett.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Results are sorted in the table by the BGRR sample number identified on the chain of custody provided with the sample. Results for sample numbers with 'R' indicate a recount of a sample. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

**Activity Concentration on samples from Below Ground Ducts
during Air Cooler Characterization**
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	DC-1	DC-1R	DC-2	DC-2R	SDCSD-001	NDCSD-001
Description	N Drain Cooler Sump Water from Pump Out Process	DC-1 shaken and re-counted	N Drain Cooler Sump Water with Graphite	DC-2 shaken and re-counted	S Drain Cooler Sump Debris Sample	N Drain Cooler Sump Debris - Orange Liquid
Sample Mass	1018 g	1018 g	941 g	941 g	578 g	682 g
Fractional Fullness of Liter bottle	4 / 4	4 / 4	4 / 4	4 / 4	2 / 4	2 / 4
ASTD Sample No	SAM00864	SAM00865	SAM00866	SAM00867	SAM00868	SAM00869
Radionuclide	Activity Concentration (homogenized pCi/g)					
Am-241	ND [1.1]	ND [0.9]	ND [1.2]	ND [0.9]	ND [230]	3.9 ± 2.9 [4.8]
Cs-137	205 ± 16 [1.1]	82.5 ± 6.6 [0.8]	190 ± 15 [0.9]	72.6 ± 5.9 [0.8]	178,000 ± 14,000 [186]	6,250 ± 480 [6.4]
Co-60	ND [0.3]	ND [0.3]	ND [0.4]	ND [0.3]	495 ± 27 [15]	46.7 ± 2.1 [1.7]
Eu-152	ND [1.3]	ND [1.0]	ND [1.3]	ND [1.0]	ND* [99]	8.1 ± 1.7 [2.9]
Eu-154	ND [0.9]	ND [0.7]	ND [0.9]	ND [0.7]	ND* [61]	ND* [2.3]

ND = Not Detected

* = Indications of the presence of Eu-152 or Eu-154 at small, non-quantifiable levels

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.

Analytical results are reported in the table assuming samples were homogeneously mixed, with solids re-suspended. As seen in the sample description, several samples were aqueous suspensions; sample activity in the settled solid phase may be significantly different from the activity in the aqueous phase.



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr Stephen Pulsford - BGRR	ASTD Project 30822-262-023

FROM Larry Lockett, CHP
ASTD Project Analytical Physicist

DATE June 08, 2000

SUBJECT: Results of Analysis for **Duct Filter Media and Dusts**

1. Enclosed is a tabulation of the results from the ISOCS analyses of samples collected by BGRR during the collection of samples from the air filters in the underground ducts.
2. When appropriate to the sample matrix, samples were provided by BGRR personnel in a 1-liter poly bottle and were counted on the ISOCS instrument in the sample analysis configuration using 1" and 2" lead annular side shields and 1" lead end cap. All samples were received from, and have been transferred back to, BGRR control under appropriate chain of custody records.
3. Samples were counted in the ISOCS support area established on the Main level (110' elevation) of Building 701. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Dave Watters, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Results are sorted in the table by the BGRR sample number identified on the chain of custody provided with the sample. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

**Activity Concentration on samples of Filter Media and Dusts
from BGRR Below Ground Ducts**
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	AGD-028	AGD-029	AGD-030	AGD-031	AGD-032	AGD-033
Description	North Filter Bank Corrosion, dust, dirt from insulation under shield block in 1L bottle	North Filter Bank debris from floor in 1L bottle	North Filter Bank Light weight filter media in 1L bottle	South Filter Bank Debris on floor in 1L bottle	South Filter Bank Filter media in 1L bottle	South Filter Bank light weight mesh in 1L bottle
Sample Mass	37.7 g	479.5 g	74.5 g	430.9 g	104.5 g	106.7 g
ASTD Sample No	SAM00873	SAM00874	SAM00875	IG01177	IG01178	IG01176
Radionuclide	Activity Concentration (pCi/g)					
Am-241	ND [4.9]	530 ± 160 [205]	3,950 ± 900 [722]	2,140 ± 1,640 [12,700]	ND [14,500]	ND [1,120]
Cs-137	890 ± 70 [4.6]	101,000 ± 8,900 [135]	1,500,000 ± 132,000 [1,230]	454,000 ± 25,000 [1,520]	5,900,000 ± 326,000 [11,800]	279,000 ± 15,500 [1,070]
Co-60	ND [2.7]	1,400 ± 64 [43]	2,700 ± 170 [121]	3,980 ± 390 [484]	3,100 ± 1,300 [2,550]	ND [620]
Eu-152	ND [5.9]	270 ± 42 [49]	2,280 ± 330 [404]	ND [2,700]	ND [17,800]	ND [1,750]
Eu-154	ND [4.2]	145 ± 32 [50]	ND* [542]	ND [1,600]	ND [9,900]	ND [1,200]

ND = Not Detected

* = Indications of the presence of Eu-154 at small, non-quantifiable levels

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.



MEMO

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ACTION	INFO	FILE
Memo for Matt Labarge, BGRR	Paul Kalb, DAT Steve Masciulli, BGRR	ASTD Project 30822-262

FROM Larry Luckett, CHP
ASTD Project Analytical Physicist

DATE Apr 12, 2000

SUBJECT: Preliminary Analysis Results : samples from BGRR Air Cooling Ducts [Part 2]

On April 7 and 10, the ASTD ISOCS Technical Team used the ISOCS instrument to perform gamma spectrum analysis of samples identified as having been obtained from the April 6 entry of the BGRR Above Grade Duct, on the diagonal slope down to the east face of the coolers. The analysis of the spectra are summarized in the table below. Full data analysis reports are on file.

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume and density estimates could be approximately $\pm 30\%$ and should be added to the tabulated results.
 2. Analytical results are reported in the table as if for homogeneously mixed samples with a density of 1.4 g/ml ("homogeneous" pCi/g). As seen in the sample description, the samples provided were of numerous materials, pieces, sizes, and shapes collected during the duct entry.
- For samples of sludge, dirt, rust, etc, (AGD-10, -11, -12, -13, -14, -15, -18, and -19), the results in the table are quantitative.
 - For inhomogeneous samples such as pieces of coolers and chips of concrete, (AGD-6, -7, -8, -9, -16, and -17), the results are indicative of the identity and relative magnitudes of radionuclides observed, but are not quantitatively comparable to the homogeneous samples. *Contact the ASTD Project if definitive quantification of specific samples should be needed.*

REVISED

Activity Concentration on samples from Above Ground Ducts

BGRR Sample No	AGD-6	AGD-6A	AGD-7	AGD-8	AGD-9	AGD-10	AGD-11	AGD-12 REVISED
Description	Cooler Pieces Front	Cooler Pieces Back	Concrete pcs, 2 S wall, S Duct	Concrete pcs corner S wall, S Duct	North Duct pcs of Cooler Mat.	Rust from Cooler I-beams	North Duct sand/water south wall	South Duct sludge from expansion joint
Sample Mass	2090.3	2090.3	863.9	275.1	115.0	1030.7	1253.8	678.1
Fractional Fullness of Liter bottle	n/a	n/a	1/2	1/4	4/4	3/4	3/4	1/2
ASTD Sample No	SAM00841	SAM00842	SAM00840	SAM00826	SAM00827	SAM00828	SAM00829	SAM00830r
Radionuclide	Activity Concentration (homogenized pCi/g)							
Am-241	ND [0.8]	ND [0.7]	ND [1.0]	53 ± 12	46 ± 16	6 ± 2	280 ± 50	354 ± 337
Cs-137	54 ± 4	47 ± 4	109 ± 9 [0.8]	11,300 ± 860	4,500 ± 340	1,900 ± 140	22,000 ± 1,600	61,000 ± 4,000
Co-60	0.5 ± 0.1	0.6 ± 0.1	ND [0.3]	14 ± 1	100 ± 7	17 ± 1	440 ± 20	75 ± 7.7 [5.8]
Eu-152	ND [0.8]	ND [0.9]	ND [1.1]	12 ± 2	81 ± 11	7.4 ± 1.3	250 ± 10	56 ± 22 [54]
Eu-154	ND [0.6]	ND [0.6]	ND [0.7]	4 ± 3	20 ± 8	1.9 ± 1.7	93 ± 8	25 ± 10 [32]

ND = Not Detected

[] = Minimum Detectable Concentration

Uncertainties are counting errors (1.96s); Additional uncertainties

due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.

Results are reported as if for homogeneously mixed samples with a density of 1.4 g/ml. Thus the results in the table are quantitative for samples of sludge, dirt, rust, etc. For inhomogeneous samples, such as pieces of coolers and chips of concrete, (AGD-6, -7, -8, -9, -16, and -17) the results are indicative of the identity and relative magnitudes of radionuclides observed.

REVISED

Activity Concentration on samples from Above Ground Ducts [continued]

BGRR Sample No	AGD-13	AGD-14	AGD-15	AGD-16	AGD-17	AGD-17R	AGD-18	AGD-18R	AGD-19
Description	North Duct Sludge from floor by cooler	South Duct Rust from South Venturi	South Duct sludge from in front of Cooler	South Duct South Wall concrete	Chipped concrete corner N. Duct	Re-count of AGD-17	S Duct S Venturi Dirt	Re-count of AGD-18	North Duct Sludge from Expansion joint
Sample Mass	1214.7	952.1	356.2	800.5	405.4	405.4	541.4	541.4	822.2
Fractional Fullness of Liter bottle	3/4	3/4	1/2	1/2	1/2	1/2	3/4	3/4	1/2
ASTD Sample No	SAM00831	SAM00832	SAM00830	SAM00834	SAM00835	SAM00838	SAM00836	SAM00839	SAM00837
Radionuclide	Activity Concentration (homogenized pCi/g)								
Am-241	22 ± 7	55 ± 12	165 ± 40	ND [1.4]	11 ± 4	13 ± 4	83 ± 22	94 ± 25	94 ± 21
Cs-137	9,600 ± 730	8,500 ± 650	52,000 ± 4,000	210 ± 16	1,020 ± 80	1,200 ± 90	17,800 ± 1,400	20,900 ± 1,600	24,900 ± 2000
Co-60	74 ± 3	20 ± 1	85 ± 4	ND [0.3]	5.7 ± 0.6	6.5 ± 0.7	31 ± 2	35 ± 2	88 ± 4
Eu-152	31 ± 3	16 ± 2	55 ± 9	ND [1.3]	5.5 ± 1.6	4.0 ± 2.5 [4.9]	23 ± 4 [5.1]	29 ± 4	59 ± 5
Eu-154	8 ± 2	8 ± 2	21 ± 6	ND [0.7]	2.5 ± 1.1 [2.5]	2.5 ± 1.1 [1.4]	12 ± 3 [6.0]	13 ± 4	21 ± 4

ND = Not Detected

[] = Minimum Detectable Concentration

Uncertainties are counting errors (1.96s); Additional uncertainties due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.

Results are reported as if for homogeneously mixed samples with a density of 1.4 g/ml. Thus the results in the table are quantitative for samples of sludge, dirt, rust, etc. For inhomogeneous samples, such as pieces of coolers and chips of concrete, (AGD-6, -7, -8, -9, -16, and -17) the results are indicative of the identity and relative magnitudes of radionuclides observed.



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ACTION	INFO	FILE
Steve Musolino - BGRR	Paul Kalb, Proj Mgr Stephen Pulsford - BGRR	ASTD Project 30822-262-023

FROM Larry Lockett, CHP
ASTD Project Analytical Physicist

DATE July 12, 2000

SUBJECT: Results of Analysis for **Water Samples, Bldg 704**

1. Enclosed is a tabulation of the results from the ISOCS analyses of water samples collected by BGRR during the evaluation of water leaking from the cutting area for duct section No 9, on the roof of Bldg 704. All samples were "ND" for non-detect.
2. Water samples were provided by BGRR personnel in 500-ml HDPE bottles in poly-wrapped bags and were counted on the ISOCS instrument in an ISOCS counting area established on the upper level(143' elevation) of Building 701 near the north face. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Eric Barbour, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Activity Concentration in Water samples from vicinity Duct Section No 9
BGRR Duct Removal Project
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample Description	Water Inside containment Section No 9	Water @ Roof drain Bldg 704	Water @ drain pipe East side Bldg 704
Sample Mass	534.8 g	521.2 g	523.9 g
ASTD Sample No	SAM00901	SAM00902	SAM00903
Radionuclide	Activity Concentration (pCi/g)		
Am-241	ND [0.25]	ND [0.32]	ND [0.30]
Cs-137	ND [0.44]	ND [0.37]	ND [0.39]
Co-60	ND [0.28]	ND [0.38]	ND [0.35]
Eu-152	ND [0.56]	ND [0.52]	ND [0.45]
Eu-154	ND [0.41]	ND [0.34]	ND [0.29]

ND = Not Detected

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix

Section 13

ASTD Project-Specific Survey Plan No. 2K-03

Use of ISOCS in Support of BGRR Reactor Pile Characterization

1.0 PURPOSE

The purpose of this Project Specific Survey Plan (PSSP) is to perform ISOCS scans in support of the BGRR need to obtain specific radiological characterization data on the Reactor Pile. The characterization will include but will not be limited to the graphite, fuel channels and penetrations in the graphite, biological shield wall, control rods, large equipment transfer area (Animal tunnel), small equipment transfer area (Instrument tunnel), and the primary air cooling system east and west inlet areas, which are known to have been contaminated during the operation of the BGRR

2.0 DATA QUALITY OBJECTIVES

2.1 State the Problem

Using the Canberra ISOCS instrument, in contrast to baseline technology, provides the opportunity to obtain data on radionuclide identification and quantification quickly, without having a delay for transportation and preparation of the sample. An area of interest for this application is the characterization of the reactor pile at the BNL Brookhaven Graphite Research Reactor.

Data are needed to determine, with the combination of activity and shielding present,

- (a) the radionuclide content of the surface contaminated and activated components of the pile; and
- (b) whether the residual radioactivity in the pile is such that it can be detected or bounded by *in situ* measurements.

Data will be used to support the BGRR determination of extent and magnitude of radioactivity activation and contamination in the pile and its components. Characterization data will be used to establish levels of effort and costs for alternative remediation endstates in disposition of the reactor pile and components.

2.2 Identify the decision.

The decision to be made is essentially a simple detection limit decision. What radionuclides does the ISOCS instrument detect in the pile and materials from the pile? Can an upper bound on the activity present be quantified?

2.3 Identify inputs.

Information is available on the geometry and constituents of the BGRR pile from construction drawings. BGRR Field Engineer will identify analytical targets and provide samples for analysis.

Results of a prior analysis of a fuel channel plug removed from the North Face of the pile are available for review.

Table 1.1. Radioactivity Observed on/in a Fuel Channel Plug Removed from the North Pile Face

Radionuclide	Total Activity (nanocuries)	
	Innermost End Surface Facing the Detector	Minimum Detectable
Cs-137	26.3 ± 0.8	0.07
Co-60	0.65 ± 0.08	0.06
Eu-152	3.82 ± 0.07	0.12
Am-241	0.13 ± 0.04	0.12

NOTE: Errors are reported as ± 2s.

Additional direct surface readings and samples will be obtained by BGRR staff, and the analytical results will be available at a later date to use in interpreting the ISOCS analysis.

2.4 Define the boundaries.

The graphite pile was de-fueled in 1968, with the final fuel shipment being made in 1972. The control rods have been inserted into the graphite pile as have the cadmium balls that were intended for emergency shutdown of the pile during operation. Access to the Pile for collecting characterization data will be through the penetration openings that exist on each face of the pile biological shield wall. The characterization data will be gathered at the faces of the biological shield wall access locations and other locations as appropriate. Extracted components, scrapings and wipes will be analyzed *ex situ* in a sample analysis area designated by BGRR.

Sample/Areas of interest identified in the BGRR Sampling Plan include:

Failed Fuel Channels	South face	17 of 17
Failed Fuel Channels	North face	10 of 10
Fuel Channels	South Face	4 (1 each quadrant)
Fuel Channels	North Face	4 (1 each quadrant)
Non-Fuel Channels	South Face	4 (1 each quadrant)
Non-Fuel Channels	North Face	4 (1 each quadrant)
Pneumatic Tubes	North face	4
Experiment/Instrument Ports	West Face	8 of 36
Experiment/Instrument Ports	East Face	8 of 36
Animal/Instrument Transfer Pits	West face	8 of 8
Animal/Instrument Transfer Pits	East face	8 of 8
Control Rods	West Face Corner	1 of 8
Control Rods	East Face Corner	1 of 8

2.5 Develop decision rules.

If there is an identifiable gamma peak in the ISOCS spectrum that can be attributed to a source location in the ISOCS field of view, the activity will be quantified using a geometric model representing the sample, surface or volume being analyzed. Otherwise, it will be concluded that the combination of source activity and shielding is such that it is below the detection capability of the ISOCS.

2.6 Specify the tolerable limits on decision errors.

The decision error of most concern would be to conclude that the instruments cannot detect the activity, when perhaps with a longer spectrum acquisition time or different orientation, it might have been able to. The consequence of counting longer than necessary would primarily be the eventual loss in resolution caused by electronic instrument drift, or the delay of project progress due to overly-conservative counting times. To provide an indication of instrument analysis variability, performance stability, and drift, duplicate and replicate analysis of samples will be performed:

- Field Replicate - 1 in 20 (5%) samples will be re-counted
- Field Duplicate/Split - 1 in 20 (5%) samples will be collected as double volume, composited, split, and counted separately
- Field Consistency - each day of sampling, one sample should be re-counted after a period of two hours has elapsed since the initial count

2.7 Optimization.

2.7.1 *in situ* considerations. The use of the ISOCS instrument to directly scan pile and components uncovered *in situ* during in the characterization is complicated by two aspects:

- the large field of view of the detector without collimators does not allow the distinction between contaminated materials still inside the pile from those in the specific area of interest, and
- when collimators are added to restrict the field of view, the weight of the instrument and cart makes it difficult to maneuver and align with specific fuel channels of interest on the north and south faces.

2.7.2 Sample scanning. The primary analysis mode used in support of the characterization will be the field screening/analysis of samples in a reproducible geometry. The Canberra annular collimators will be arranged to construct a shielded counting cavity around the detector that accepts a 1 liter poly bottle. Soil sample bottle must be filled to the upper crown to meet the description of the geometry model.

2.7.3 Detection sensitivity. The primary information need for the control of characterization is the identity of radionuclides present and whether they are present at levels near or above the DCGL. The gamma spectrum will be accumulated for a time that assures that the minimum

detectable concentration is less than 50% of the DCGL for the gamma emitters identified in the BGRR list (Annex B).

- for samples counted in the shielded cavity, the acquisition time will be 5 minutes (300 seconds);
- for analysis of finite objects such as fuel channel plugs, acquisition time will be 5 to 10 minutes (300 to 600 seconds), adjusted for configuration and distance. Often times the distance may need to be increased to diminish detector dead-time from highly active samples; and
- for analysis of transfer pits wall/bottom the acquisition time will be 10 minutes.

3.0 Equipment

ISOCS Hardware

- a. the InSpector portable gamma spectroscopy interface,
- b. a broad energy germanium (BEGe) detector with preamp,
- c. Multi-Attitude Cryostat with extended detector to allow use of a back-shield,
- d. notebook computer with Canberra Genie, PROcount-2000 and ISOCS software,
- e. assorted connecting cables, power cords, batteries and battery chargers, and
- f. a cart for relocating and positioning the detector and shield assembly,
- g. A locally-constructed instrument support stand, and
- h. wheel chocks.

Annular shields and field of view collimators

- a. Set of 2-inch thick annular shields (3 pieces),
- b. 2-inch thick back shield (2 pieces),
- c. 1-inch annular shields (3 pieces),
- d. 1-inch thick end cap, and
- e. Collimator frame extensions

Additional Equipment and Instruments

- a. Calibrated microR or microRem meter
- b. Digital camera or Polaroid camera w/ film
- c. Tape measure, 100 foot or 50 m
- d. Field notebook
- e. 3.5 inch disk for file transfer
- f. DAT vehicle
- g. 50 ft extension cords

4.0 COORDINATION

4.1 Health and Safety	BGRR Health and Safety POC :	Reggie Suga	x-8248
	BGRR Lead Radiological Control Tech	D. Mergen	x-2374

The internal surfaces of the pile are suspected of being contaminated with radionuclides such as Co-60, Cs-137, Am-241, Pu-239, Sr-90 and others. There is a radiation work permit (RWP) developed for the performance of the Pile characterization. The requirements of the RWP shall be STRICTLY adhered to in the performance of the work activities prescribed herein. A BGRR procedure (ERD-OPM-4.3) has been developed to address the handling of contaminated samples in the field analysis area set up in the BGRR. Personnel should read and be familiar with this procedure prior to performing sample analysis in the BGRR field laboratory area.

4.2 Facility Access

Primary: Steve Masciulli x-8247 Alternate: Manni Lilimpakis x-7628

5.0 SPECIFIC SCANS AND PARAMETERS

Specific areas of interest are identified in the BGRR survey plan (Ref 3). Individual targets or scans within the areas may be added at the discretion of the operator when in the area. A project sample identification scheme is described.

5.1.1 ASTD Project Sample Numbers.

Targets for gamma spectrum acquisition will be identified with a sample number of the form:

aaainnnz

- where aaa a three character code identifying the portion of the characterization
 (**W25**) for West Experimental Port No 25
 (**E53**) for East Experimental Port No 53
- i a single character code identifying the scan target
 S For a sample counted within the field sample analysis cavity
 C For a section of fuel channel plug
 E For a section of experiment Port plug
 T For a view of the transfer pit
- mmn is a three digit number incrementing in sequence indicating a sample
 sequence number for the project
- z is a single letter indicating a subsequent scan of the same sample, such as at
 a different target center, or from a different angle or position

For example: **W25S012b** is the second scan (**b**) of sample (**S**) number 12 (**012**) obtained from the West No 25 Experimental port (**W25**). The code **S** for sample indicates it was counted in the field analysis cavity.

The ASTD Project Sample Number will be entered into the scan description by the instrument operator during scan acquisition, to tie the sample number to the scan record file. The BGRR sample number will also be entered into the scan description field and the field notebook. The ASTD Sample Number will also be cross-referenced with the default scan filename on the File Custody Record Form.

5.1.2 Scan Acquisition

- a. Scans should be accumulated to detect the radioactivity and to investigate the impact of ISOCS configuration on the analytical results.
- b. Initial scan time of 300 seconds; longer if necessary to see activity peaks especially in highly shielded objects such as through the plugged channels
- c. ISOCS codes for sample type and geometry

Sample Type	ISOCS Generic	Geometry	ISOCS
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Table 5-1. Studies of the BGRR Pile Characterization

Sample No	Annular Shield	Field of View	Initial Count Time	Sample Size or FOV	Orientation	Note
Sample Analysis	2 inch around detector 1 inch around sample	180 ⁰ inside with cavity capped	300 sec	1 liter	Vertical sample count configuration	Sample/scrapings extracted from the Pile
Spectrum Acquisition from items and pieces	2 inch around detector	90 ⁰ field of view	600 sec	1 sq meter field of view at 57 cm	Vertical down towards piece perpendicular to surface	Channel Plugs Experimental Hole Plugs graphite pieces
Spectrum Acquisition through Bioshield	2 inch around detector	90 ⁰ field of view	600 sec (?)	12.5 sq meter FOV at 2 m	Horizontal aligned with open channel; distance appropriate for dead time	“Beam” from open channel detected outside contamination control boundary

6.0 REFERENCES

- 6.1 BNL-DAT, *In Situ* Analysis Quality Assurance Project Plan, Version 1, July 7, 1999.
- 6.2 BNL-DAT SOP-DAT1, *In Situ* Gamma Spectroscopy Using the Canberra ISOCS System, version 1, August 19, 1999.
- 6.3 BNL-DAT SOP-DAT2, Analysis of Gamma Spectrum Files Using Canberra ISOCS System [software ver 3.0], February 23, 2000
- 6.3 ERD-BGRR-TP-00-07, Characterization Sampling Analysis Plan for Pile, Associated Equipment and Areas (in press) [draft of Apr 25, 2000 used to develop this PSSP].
- 6.4 ERD-OPM-4.3, Procedure for ASTD Soil Sample Processing to Support BGRR Excavations, Rev 0, December 15, 1999.

Section 14



MEMO

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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023
FROM Larry Lockett, CHP		DATE Sep 19, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis, **Control Rods from Interior of BGRR Graphite Pile**

1. Enclosed is a tabulation of the results from the ISOCS analyses of control rods revealed during the pile characterization. Spectra from the control rods were obtained *in situ* as 24" portions of the rods were exposed as the rods moved in and out of unshielded areas of the control rod drives.
2. Spectrum acquisition was performed by Larry Milian, and quantification modeling results were performed by and results were reviewed by Larry Lockett, both members of the ASTD Project Team.
3. The table provides data on BGRR radionuclides of concern (Co-60, Cs-137 and Am-241). Full gamma spectroscopy analysis reports for each spectrum are on file with ASTD Project records and are available upon request.
4. All rod portions were evaluated assuming a homogenous distribution of activity throughout the rod matrix. Results are reported in units of activity per length of control rod ($\mu\text{Ci}/\text{ft}$).

Should you have any questions, please contact me at x-2777.

Activity Concentration in Pile Control Rods
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

Description	2' section of exposed control rod marked 25' and 26' South East Corner	2' section of exposed control rod marked 19' South East Corner	2' section of exposed control rod marked 25' and 26' South West Corner	2' section of exposed control rod marked 10' to 12' South West Corner
ASTD File No	IG01352	IG01354	IG01356	IG01358
Radionuclide	Activity per foot of exposed rod ($\mu\text{Ci}/\text{ft}$)			
Co-60	813 \pm 31.5 [2.23]	245 \pm 9.6 [0.95]	175 \pm 7.0 [1.1]	7.4 \pm 0.3 [0.12]
Cs-137	ND [4.3]	ND [1.8]	ND [2.3]	0.142 \pm 0.123 [0.20]
Eu-152	ND [2.1]	ND [0.74]	ND [1.2]	ND [0.16]
Eu-154	ND [4.4]	ND [2.0]	ND [2.5]	ND [0.30]
Eu-155	ND [28]	ND [12]	ND [18]	ND [1.5]
Am-241	ND [62]	ND [27]	ND [37]	ND [3.3]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA (one or more, but not all, gamma lines observed)

[] = Minimum Detectable Concentration in $\mu\text{Ci}/\text{ft}$

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately \pm 50 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix;



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023
FROM Larry Luckett, CHP		DATE July 25, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis, *in situ* West Face Experimental Ports, BGRR Graphite Pile

1. Enclosed is a tabulation of the results from the ISOCS *in situ* analyses of Experimental Ports during the graphite pile characterization through the penetrations into the West Face of the Biological Shield.
2. Spectrum acquisition was performed by Larry Milian, Tom Roberts, Jay Adams and Eric Barbour, quantification modeling results were performed by Eric Barbour, and results were reviewed by Larry Luckett, all members of the ASTD Project Team.
3. The table provides data on BGRR radionuclides of concern (Co-60, Cs-137 and Am-241). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.
4. All experimental port measurements were performed by aiming the detector, centered, at port openings located on the West side of the pile. The steel port covers were removed prior to each measurement. The following assumptions were made for the port ISOCS models:
 - The source of activity is the steel plate liner positioned at 6.5 feet (1.98 m) into the port and flush to the inside surface of the concrete shielding. The steel of the liner has a density of 7.86 g/cm³
 - The model assumes a homogenous distribution of activity throughout the steel source.

Should you have any questions, please contact me at x-2777.

Activity Concentration *in situ* Analysis West Face of Graphite Pile
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

	EXPERIMENTAL PORTS									INSTRUMENT PORT
Description	W-12	W15	W16	W30	W31	W-36	W51	W54	W56	I-W5
ASTD Sample No	IG01219	IG01214	IG01211	IG01220	IG01222	IG01224	IG01231	IG01233	IG01229	IG01270
Radionuclide	Activity Concentration (pCi/g)									
Am-241	ND [188,000]	ND [176,000]	ND [158,000]	243,000 ± 226,000 [370,000]	ND [207,000]	ND [189,000]	ND [185,000]	ND [222,000]	ND [189,000]	ND [99,300]
Cs-137	ND [9,600]	ND [7,600]	ND [4,600]	1,200,000 ± 104,000 [20,500]	90,000 ± 10,200 [9,900]	ND [9,900]	ND [6,800]	ND [7,200]	ND [7,200]	ND [3,000]
Co-60	774,000 ± 30,700 [6,600]	414,000 ± 16,900 [5,000]	26,200 ± 2,000 [2,000]	960,000 ± 38,000 [7,650]	1,450,000 ± 56,500 [7,2400]	99,300 ± 39,000 [7,080]	306,000 ± 12,500 [3,650]	413,000 ± 16,700 [4,300]	346,000 ± 14,200 [4,200]	15,000 ± 1,140 [986]
Eu-152	210,000 ± 12,100 [6,300]	155,000 ± 9,000 [6,700]	196,000 ± 8,300 [7,450]	153,000 ± 11,500 [11,100]	121,000 ± 8,300 [7,410]	89,700 ± 7,240 [7,200]	261,000 ± 10,400 [8,400]	316,000 ± 12,600 [9,300]	221,000 ± 10,200 [7,020]	95,300 ± 4,800 [4,600]
Eu-154	ND* [13,600]	58,500 ± 5,300 [10,000]	ND* [7,820]	30,300 ± 17,500 [39,200]	115,000 ± 7,100 [12,500]	142,000 ± 9,020 [14,200]	68,100 ± 4,700 [8,800]	108,600 ± 6,600 [12,800]	69,400 ± 5,000 [8,400]	ND [3,600]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

3. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 30 % and should be added to the tabulated results.
4. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix; surface depositions probably would have higher pCi/g but less total activity on each piece.



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023
FROM Larry Lockett, CHP		DATE Aug 9, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis, **Debris Samples from Interior of BGRR Graphite Pile and Tunnels underneath**

1. Enclosed is a tabulation of the results from the ISOCS analyses of debris samples obtained during the pile characterization, through a penetration into the West Face of the Biological Shield and from the tunnels underneath the reactor pile. Debris samples were provided by BGRR personnel in 1000-ml HDPE bottles or in poly-wrapped bags and were counted on the ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, BGRR control.
2. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Larry Milian, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
3. The table provides data on BGRR radionuclides of concern (Co-60, Cs-137 and Am-241). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.
4. All dusts were evaluated assuming a homogenous distribution of activity throughout the sample matrix.

Should you have any questions, please contact me at x-2777.

Activity Concentration Debris Samples
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

Description	Debris from Experimental Port W-32, 1 ft before removable core 32 mR/hr on contact	Debris from Animal Tunnel East	Debris from Instrument Tunnel East chunks of debris and ~ 25 cm wire
BGRR Sample No	W-32-D	ATE	ITE
Mass (g)	4.6	164.8	354.2
ASTD File No	IG01322	IG01324	IG01325
Radionuclide	Activity Concentration (pCi/g)		
Co-60	1.24E+08 ± 6.6E+06 [3.03E+05]	2,590 ± 153 [24.6]	584 ± 28 [12.9]
Cs-137	3.36E+06 ± 4.4E+05 [5.08E+05]	128 ± 39 [60.0]	46 ± 13 [18.4]
Eu-152	ND [4.8E+05]	581 ± 44 [42.5]	1,150 ± 50 [40.5]
Eu-154	ND [6.1E+05]	ND [44.2]	ND* [37.3]
Eu-155	ND [6.4E+05]	ND [23.2]	ND [21.5]
Am-241	ND [8.2E+05]	ND [31.8]	ND [30.2]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA (one or more, but not all, gamma lines observed)

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 50 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix;



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023
FROM Larry Lockett, CHP		DATE July 12, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis for **Samples from Plenum Access, BGRR Graphite Pile**

1. Enclosed is a tabulation of the results from the ISOCS analyses of samples collected by BGRR during the graphite pile characterization through the penetrations into the North and South Plenum.
2. Samples were provided by BGRR personnel in poly-wrapped bags or HDPE bottles and were counted on the ISOCS instrument in an ISOCS counting area established on the Main level (110' elevation) of Building 701 near the west face. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Eric Barbour, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Results are sorted in the table by the pile penetration from which they were extracted. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Activity Concentration on samples from Plenum Access
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Pile Sample Identification	P-1a	P-2	P-3	P-4	P-5	P-6	P-7
Description	Fine graphite powder in 500 ml bottle West CRD Top Plate	Plug Retainer Spring, West side South Plenum	Plug Retainer Spring Scanner Slot #4 North Plenum	Graphite Plug Scanner Slot #4 North Plenum	Fuel Anchor Portion (Hot end only, in 1L bottle)	Thermocouple wire, folded in 1L bottle scanner slot #4 North Plenum	Particulate debris from removable core plate, sample and rust
Sample Mass	- 1 g	136.5 g	71.6 g	316.5 g	45.1 g	31.6 g	25.9 g
ASTD Sample No	IG01253	IG01248	IG01251	IG01246	IG01247	IG01245	IG01254
Radionuclide	Activity Concentration (pCi/g)						
Am-241	ND [74,000]	ND [5,300]	ND [79,100]	ND [184]	ND [4,200]	ND [69,400]	2,900 ± 1,500 [2,380]
Cs-137	22,000,000 [note 3] [51,000]	5,200 ± 800 [1,050]	7,800 ± 7,100 [11,800]	9,000 ± 820 [123]	7,600 ± 1,700 [2,470]	ND [7,670]	202,000 ± 18,000 [2,190]
Co-60	578,000 ± 29,600 [21,000]	230,000 ± 9,000 [763]	3,000,000 ± 120,000 [11,000]	1,370 ± 98 [115]	732,000 ± 29,000 [2,110]	2,800,000 ± 100,000 [5,530]	225,000 ± 9,000 [1,350]
Eu-152	ND [42,000]	ND [574]	ND [9,000]	270 ± 88 [170]	ND [2,240]	ND [5,330]	ND* [2,000]
Eu-154	ND [34,000]	ND [1,260]	ND [19,200]	ND [132]	ND [3,830]	ND [8,750]	ND [2,120]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 30 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix; surface depositions probably would have higher pCi/g but less total activity on each piece.
3. The mass of several samples, especially P-1a, was so small that significant uncertainty is associated with the values reported.

REVISED
Activity Concentration Graphite Dust Samples
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

Description	Graphite Dust Sample							
BGRR Sample No	W-32-B-2	W-32-C-3	W-32-D-4	W-35-C-2	W-35-3-B(a)	W-35-1-D	W-32-A	W-35-A
Mass (g)	94.2	93.0	104.3	57.3	62.5	57.4	60.0	93.9
ASTD File No	IG01307	IG01308	IG01309	IG01310	IG01312	IG01313	IG01314	IG01315
Radionuclide	Activity Concentration (pCi/g)							
Co-60	40,800 ± 1,550 [155]	9,400 ± 380 [83]	23,800 ± 910 [122]	3,860 ± 180 [62]	3,310 ± 155 [85]	1,670 ± 94 [69]	14,000 ± 560 [120]	4,990 ± 210 [76]
Ba-133	480 ± 90	560 ± 90	680 ± 105	254 ± 62	634 ± 87	361 ± 68	2,510 ± 230	1,100 ± 120
Cs-137	2,700 ± 290 [250]	760 ± 120 [150]	1,200 ± 160 [190]	400 ± 77 [101]	1,620 ± 176 [128]	970 ± 135 [152]	4,520 ± 440 [236]	2,650 ± 260 [126]
Eu-152	ND [250]	ND [205]	670 ± 110 [165]	ND* [278]	ND [215]	705 ± 125 [244]	ND [221]	ND [165]
Eu-154	14,000 ± 560 [230]	21,300 ± 660 [144]	20,900 ± 590 [190]	13,600 ± 470 [167]	18,500 ± 605 [189]	25,400 ± 665 [200]	10,800 ± 426 [170]	13,000 ± 440 [136]
Eu-155	1,400 ± 200 [118]	2,100 ± 175 [119]	1,900 ± 200 [106]	1,600 ± 230 [193]	2,300 ± 230 [177]	2,550 ± 315 [193]	1,325 ± 165 [184]	1,375 ± 240 [118]
Am-241	234 ± 122 [190]	154 ± 94 [150]	198 ± 96 [150]	ND [95]	184 ± 101 [157]	ND [108]	ND [123]	ND [104]
Ag-108m	ND [105]	ND [105]	ND [105]	195 ± 112 [100]	ND [105]	ND [105]	ND [105]	119 ± 41 [89]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA (one or more, but not all, gamma lines observed)

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

- Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 50 % and should be added to the tabulated results.
- Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix;



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023

FROM Larry Lockett, CHP
ASTD Project Analytical Physicist

DATE July 24, 2000

SUBJECT: Results of Analysis for **Samples from South Face, BGRR Graphite Pile**

1. Enclosed is a tabulation of the results from the ISOCS analyses of samples collected by BGRR during the graphite pile characterization through the penetrations into the South Plenum and the South Face of the Biological Shield.
2. Samples were provided by BGRR personnel in poly-wrapped bags and were counted on the ISOCS instrument in an ISOCS counting area established on the Main level (110' elevation) of Building 701 near the south face. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Jay Adams and Eric Barbour, quantification modeling results were performed by Eric Barbour, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Activity Concentration on samples from South Face of Graphite Pile
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

				GRAPHITE				
Description	Fuel Anchor from South Plenum Floor [hot end removed] [see note 3]	2 nd Fuel Anchor from South Plenum Floor [whole]	Mock Fuel Element shell, Hollow aluminum vaned rod	Hollow Graphite plug from South Plenum	Graphite Plug from South face C-7-14	Graphite Plug from South face C-6-14	Graphite Plug from South face D-13-13	Graphite Plug from South face D-11-13
Sample Mass	200 g	330 g	680 g	310 g	1,435 g	1,443 g	1,534 g	1,441 g
ASTD Sample No	IG01278	IG01279	IG01275	IG01274	IG01273	IG01272	IG01271	IG01270
Radionuclide	Activity Concentration (pCi/g)							
Am-241	ND [844]	ND [354]	ND [300]	ND [340]	ND [325]	ND [437]	ND [302]	ND [424]
Cs-137	2,660 ± 630 [930]	3,200 ± 460 [541]	4,400 ± 500 [445]	9,300 ± 820 [266]	1,030 ± 110 [116]	11,300 ± 1,020 [255]	4,960 ± 460 [167]	2,500 ± 260 [174]
Co-60	265,000 ± 10,000 [760]	60,000 ± 2,400 [270]	59,200 ± 2,300 [197]	310 ± 70 [87]	440 ± 46 [54]	460 ± 55 [67]	270 ± 41 [62]	420 ± 50 [61]
Eu-152	2,000 ± 470 [520]	1,560 ± 250 [314]	5,200 ± 400 [294]	26,000 ± 950 [465]	33,000 ± 850 [220]	55,400 ± 1,400 [353]	28,000 ± 730 [264]	54,100 ± 1,300 [383]
Eu-154	ND* [900]	ND [422]	ND* [456]	ND* [560]	ND* [330]	2,350 ± 160 [266]	ND* [287]	2,100 ± 140 [268]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 30 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix; surface depositions probably would have higher pCi/g but less total activity on each piece.
3. The hot end of this fuel anchor had been separated and was analyzed independently as Sample P-5, reported to BGRR on July 12, 2000.



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr Stephen Pulsford - BGRR	ASTD Project 30822-262-023
FROM Larry Lockett, CHP ASTD Project Analytical Physicist		DATE June 21, 2000

SUBJECT: Results of Analysis for **West Face Samples, BGRR Graphite Pile**

1. Enclosed is a tabulation of the results from the ISOCS analyses of samples collected by BGRR during the graphite pile characterization through the penetrations on the west pile face.
2. Samples were provided by BGRR personnel in poly-wrapped bags and were counted on the ISOCS instrument in an ISOCS counting area established on the Main level (110' elevation) of Building 701 near the west face. All samples were received from, and have been transferred back to, BGRR control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian and Eric Barbour, and results were reviewed by Larry Lockett, all members of the ASTD Project Team.
4. The table provides data on Co-60, Cs-137 and Am-241 (BGRR radionuclides of concern). Results are sorted in the table by the pile penetration from which they were extracted. Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

Activity Concentration on samples from West face Penetrations
BGRR Graphite Pile Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Pile Penetration Identification	EXPW-31	EXPW-36	EXPW-42	EXPW-42	EXPW-12	EXPW-30	EXPW-30 B-11-1
Description	Solid graphite channel piece 4"x4"x24"	Hollow graphite channel piece 4"x4"x24" w/ 2"x2" hole	Conveyor Sample Cup #364	Conveyor Sample Cup #368	U-shape graphite channel piece 4"x4"x24" w/ 2"x2" trough	Graphite Piece 10cmX3cmX5cm irregular	Two tiny graphite pieces from the No 8 Fuel Channel
Sample Mass	9,672 g	7,662 g	147 g	142 g	6,188 g	150.8 g	2.1 g
ASTD Sample No	IG01186	IG01188	IG01191	IG01192	IG01196	IG01201	IG01203
Radionuclide	Activity Concentration (pCi/g)						
Am-241	160 ± 156 [256]	ND [214]	104 ± 100 [167]	98 ± 102 [169]	ND [300]	ND [481]	ND* [8,080]
Cs-137	1,500 ± 250 [350]	700 ± 100 [114]	1,800 ± 170 [101]	920 ± 140 [178]	ND [134]	300 ± 180 [290]	6,760 ± 6,750 [6,700]
Co-60	42,000 ± 1,600 [119]	1,600 ± 78 [52]	5,900 ± 245 [63]	2,500 ± 116 [60]	205 ± 30 [38]	7,800 ± 375 [185]	53,200 ± 5,300 [620]
Eu-152	ND [146]	14,000 ± 400 [220]	3,300 ± 171 [203]	6,800 ± 260 [198]	30,000 ± 750 [311]	33,000 ± 1,000 [530]	ND* [12,600]
Eu-154	475 ± 70 [159]	26,700 ± 600 [143]	31,300 ± 750 [222]	30,600 ± 700 [211]	28,200 ± 1000 [347]	32,700 ± 990 [354]	ND* [8,930]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume and density estimates could be approximately ± 30 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the graphite matrix; surface depositions probably would have higher pCi/g but less total activity on each piece.

Section 15



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ACTION	INFO	FILE
Matt Labarge - BGRR	Paul Kalb, Proj Mgr	ASTD Project
Steve Moss - BGRR	Stephen Pulsford - BGRR	30822-262-023
FROM Larry Lockett, CHP		DATE Sep 21, 2000
ASTD Project Analytical Physicist		

SUBJECT: Results of Analysis, **Canal Debris from Interior of BGRR Canal House**

1. Enclosed is a tabulation of the results from the ISOCS analyses of debris samples obtained during the canal/canal house characterization. Debris samples were provided by BGRR personnel and were counted on the ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, BGRR control.
2. Spectrum acquisition and quantification modeling results were performed by Larry Milian and results were reviewed by Larry Lockett, both members of the ASTD Project Team.
3. The table provides data on BGRR radionuclides of concern (Co-60, Cs-137 and Am-241). Full gamma spectroscopy analysis reports for each spectrum are on file with ASTD Project records and are available upon request.
4. All samples were evaluated assuming a homogenous distribution of activity throughout the sample matrix. Results are reported in units of activity per mass of sample (pCi/g).

Should you have any questions, please contact me at x-2777.

Activity Concentration in Canal Debris Samples
BGRR Canal and Water Treatment House Characterization
Gamma Spectrum Analysis with the ISOCS Instrument

BGRR Sample No	Canal-001	Canal-002 [see note 3]	Canal-003	Canal-004	Canal-005 [see note 3]
Description	Debris from Canal Sump No 3	Debris from Canal Sump No 1	Debris from Canal South Bottom Gate	Debris from Canal North Bottom Joint	Debris from Canal Sump No 2
Sample Mass (grams)	52.8	11.1	103.2	156.0	21.1
ASTD File No	IG01345	IG01346	IG01347	IG01349	IG01350
Radionuclide	Activity concentration (pCi/g)				
Co-60	207 ± 27.6 [32.2]	3,500 ± 246 [179]	62,400 ± 2,520 [505]	61.9 ± 10.5 [13.4]	1,450 ± 114 [93.2]
Cs-137	108,500 ± 9,500 [162]	79,500 ± 7,060 [405]	410,000 ± 36,100 [2,100]	17,600 ± 1,550 [54.0]	99,900 ± 8,800 [271]
Eu-152	ND [160]	ND* [457]	ND* [874]	ND [71.4]	ND [356]
Eu-154	ND [107]	ND [324]	ND* [1,020]	ND [42.5]	ND [247]
Eu-155	ND [156]	ND [293]	ND [20,000]	ND [54]	ND [246]
Am-241	ND [198]	891 ± 226 [360]	ND [252,000]	ND [81.4]	650 ± 230 [382]

ND = Not Detected

* = Indications of the presence of the radionuclide at non-quantifiable levels near the MDA (one or more, but not all, gamma lines observed)

[] = Minimum Detectable Concentration in pCi/g

DATA QUALIFIERS:

1. Uncertainties reported in the table are counting errors (1.96s); additional uncertainties due to volume, mass, and density estimates could be approximately ± 50 % and should be added to the tabulated results.
2. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix.
3. While there is good confidence in the identification of radionuclides and their relative magnitudes, there is considerable uncertainty in the absolute value of activity expressed in units of “pCi/g” due to small sample mass (Samples Canal-002 and Canal-005).

Soil Samples Collected in 1 Liter Bottles at the Canal and Water Treatment House Characterization											
BGRR Sample ID		ANALYSIS DATE	ASTD Sample ID	Estimated Activity Concentration (pCi/gram)							
				K-40		Co-60		Cs-137		Am-241	
Bore Number	Depth			Results	MDC	Results	MDC	Results	MDC	Results	MDC
CCB-011	0"-6"	9/22/2000	SAM01041	13.6 ± 3.0	0.4	0.3 ± 0.1	0.2	ND	0.4	ND	0.6
CCB-011	06"-12"	9/22/2000	SAM01042	12.7 ± 2.8	0.4	ND	0.4	ND	0.4	ND	0.4
CCB-011	12"-18"	9/22/2000	SAM01043	9.5 ± 2.6	2.3	0.3 ± 0.1	0.1	ND	0.3	ND	0.4
CCB-011	18"-24"	9/22/2000	SAM01044	12.3 ± 2.7	0.4	ND	0.3	ND	0.3	ND	0.4
CCB-012	0"-6"	9/22/2000	SAM01045	10.1 ± 3.7	4.9	ND	0.4	ND	0.4	ND	0.6
CCB-012	06"-12"	9/22/2000	SAM01046	9.1 ± 2.5	2.0	ND	0.3	ND	0.3	ND	0.4
CCB-012	12"-18"	9/22/2000	SAM01047	8.9 ± 2.7	2.6	ND	0.3	ND	0.3	ND	0.4
CCB-012	18"-24"	9/22/2000	SAM01048	11 ± 2.8	0.2	ND	0.3	ND	0.3	ND	0.3
CCB-019	0"-6"	9/20/2000	SAM00980	11.6 ± 2.7	1.6	ND	0.2	6.8 ± 0.8	0.3	ND	0.5
CCB-019	06"-12"	9/20/2000	SAM00981	8.9 ± 2.2	1.4	ND	0.2	ND	0.4	ND	0.4
CCB-019	12"-18"	9/20/2000	SAM00982	11.9 ± 2.9	1.7	ND	0.3	0.1 ± 0.1	0.2	ND	0.5
CCB-019	18"-24"	9/20/2000	SAM00983	8.5 ± 2.3	1.6	ND	0.2	ND	0.3	ND	0.3
CCB-019	24"-30"	9/20/2000	SAM00984	ND	5.8	ND	0.2	5.4 ± 0.6	0.3	ND	0.4
CCB-019	30"-36"	9/20/2000	SAM00985	10.4 ± 2.4	0.4	ND	0.2	0.6 ± 0.2	0.2	ND	0.4
CCB-020	0"-6"	9/21/2000	SAM01002	11.2 ± 2.8	2.4	ND	0.2	5.1 ± 0.6	0.2	ND	0.5
CCB-020	06"-12"	9/21/2000	SAM01003	7.9 ± 2.2	1.7	ND	0.2	ND	0.4	ND	0.3
CCB-020	12"-18"	9/21/2000	SAM01004	8.7 ± 2.3	1.5	ND	0.3	0.2 ± 0.1	0.2	ND	0.4
CCB-020	18"-24"	9/21/2000	SAM01005	7.7 ± 2.4	2.7	ND	0.2	0.6 ± 0.2	0.2	ND	0.4
CCB-020	24"-30"	9/21/2000	SAM01006	9.8 ± 2.5	2.2	ND	0.3	ND	0.3	ND	0.4
CCB-020	30"-36"	9/21/2000	SAM01007	9.1 ± 2.6	2.6	ND	0.3	ND	0.3	ND	0.4
CCB-021	0"-6"	9/21/2000	SAM01008	ND	5.7	ND	0.2	0.3 ± 0.1	0.2	ND	0.4
CCB-021	06"-12"	9/21/2000	SAM01009	10.4 ± 2.4	3.8	ND	0.3	ND	0.3	ND	0.4
CCB-021	12"-18"	9/21/2000	SAM01010	8.9 ± 2.3	1.8	ND	0.2	ND	0.3	ND	0.4
CCB-021	18"-24"	9/21/2000	SAM01011	10.5 ± 2.6	1.8	ND	0.4	ND	0.3	ND	0.4
CCB-021	24"-30"	9/21/2000	SAM01012	8.6 ± 2.6	2.5	ND	0.4	ND	0.3	ND	0.4
CCB-021	30"-36"	9/21/2000	SAM01013	10.6 ± 2.8	2.4	ND	0.4	ND	0.3	ND	0.4

Minimum detectable concentrations (MDCs) at the 95% confidence level

ND = not detected; result was less than the 95% critical level

Uncertainty reported at the 95% confidence level and represents counting error and ISOCS-generated efficiency error.

Soil Samples Collected in 1 Liter Bottles at the Canal and Water Treatment House Characterization											
BGRR Sample ID		ANALYSIS DATE	ASTD Sample ID	Estimated Activity Concentration (pCi/gram)							
Bore Number	Depth			K-40		Co-60		Cs-137		Am-241	
		Results	MDC	Results	MDC	Results	MDC	Results	MDC		
CCB-023	0"-6"	9/22/2000	SAM01023	11.1 ± 2.9	2.7	ND	0.4	0.6 ± 0.2	0.3	ND	0.6
CCB-023	06"-12"	9/22/2000	SAM01024	7.8 ± 2.4	2.5	ND	0.3	ND	0.3	ND	0.4
CCB-023	12"-18"	9/22/2000	SAM01025	10.6 ± 2.8	2.7	ND	0.3	0.2 ± 0.1	0.2	ND	0.4
CCB-023	18"-24"	9/22/2000	SAM01026	9.5 ± 2.5	2.0	ND	0.3	ND	0.3	ND	0.3
CCB-029	0"-6"	9/19/2000	SAM00957	12.3 ± 3.1	2.8	ND	0.3	3.4 ± 0.5	0.3	ND	0.5
CCB-029	06"-12"	9/19/2000	SAM00958	ND	5.6	ND	0.2	0.2 ± 0.2	0.3	ND	0.4
CCB-029	12"-18"	9/19/2000	SAM00959	11.3 ± 2.7	1.8	ND	0.3	ND	0.2	ND	0.4
CCB-029	18"-24"	9/19/2000	SAM00960	9.1 ± 2.3	1.5	ND	0.2	ND	0.2	ND	0.3
CCB-029	24"-30"	9/19/2000	SAM00961	10.7 ± 2.5	0.4	ND	0.2	ND	0.4	ND	0.4
CCB-029	30"-36"	9/19/2000	SAM00962	9 ± 2.3	1.6	ND	0.2	ND	0.3	ND	0.4
CCB-029B	0"-6"	9/19/2000	SAM00969	14 ± 3.0	0.4	ND	0.4	15.5 ± 1.5	0.4	ND	0.6
CCB-029B	06"-12"	9/19/2000	SAM00970	7.7 ± 2.2	1.5	ND	0.3	2.4 ± 0.4	0.2	ND	0.4
CCB-029B	12"-18"	9/19/2000	SAM00971	ND	5.5	ND	0.3	1.4 ± 0.3	0.3	ND	0.4
CCB-029B	18"-24"	9/19/2000	SAM00972	7.2 ± 2.2	2.0	ND	0.3	1.9 ± 0.3	0.3	ND	0.4
CCB-029B	24"-30"	9/19/2000	SAM00973	8.7 ± 2.5	2.5	ND	0.3	4.9 ± 0.6	0.3	ND	0.5
CCB-029B	30"-36"	9/19/2000	SAM00974	10 ± 2.6	2.2	ND	0.3	8.4 ± 0.9	0.3	ND	0.5
CCB-029B	36"-44"	9/21/2000	SAM00992	9.1 ± 2.6	2.6	ND	0.2	7.7 ± 0.8	0.2	ND	0.5
CCB-029B	44"-52"	9/21/2000	SAM00993	8.3 ± 2.5	2.5	ND	0.3	6.4 ± 0.7	0.3	ND	0.5
CCB-029B	52"-60"	9/21/2000	SAM00994	7.8 ± 2.0	3.3	ND	0.2	101 ± 8.0	0.5	ND	1
CCB-029B	60"-68"	9/21/2000	SAM00995	8.9 ± 2.2	3.7	ND	0.2	185 ± 14.3	0.9	ND	1.4
CCB-029B	68"-76"	9/21/2000	SAM00996	11.2 ± 2.7	1.8	ND	0.3	425 ± 32.6	1.4	ND	2.2
CCB-029B	68"-76"R	9/21/2000	SAM00998	9.7 ± 2.7	2.7	ND	0.3	417 ± 32.0	1.5	ND	2.1
CCB-029B	76"-84"	9/21/2000	SAM00997	11 ± 2.9	2.9	ND	0.3	24.4 ± 2.1	0.4	ND	0.7
CCB-029B	108" (09')	9/22/2000	SAM01035	8.8 ± 2.6	2.6	ND	0.4	7.3 ± 0.8	0.2	ND	0.4
CCB-029B	120" (10')	9/22/2000	SAM01036	8.4 ± 2.5	2.4	0.2 ± 0.1	0.1	102 ± 8.1	0.7	ND	1.1
CCB-029B	132" (11')	9/22/2000	SAM01037	ND	5.6	ND	0.3	51.9 ± 4.2	0.7	ND	0.9
CCB-029B	144" (12')	9/22/2000	SAM01038	8.2 ± 2.2	1.5	0.2 ± 0.1	0.2	40.0 ± 3.3	0.5	ND	0.7
CCB-029B	156" (13')	9/22/2000	SAM01039	7.5 ± 2.3	2.3	ND	0.3	23.7 ± 2.0	0.4	ND	0.6
CCB-029B	168" (14')	9/22/2000	SAM01040	10.5 ± 2.5	1.7	ND	0.4	25.8 ± 2.2	0.4	ND	0.7
CCB-035	24"-30"	9/19/2000	SAM00963	10 ± 2.6	1.7	ND	0.3	ND	0.4	ND	0.5
CCB-035	30"-36"	9/19/2000	SAM00964	10.5 ± 2.7	1.9	ND	0.2	ND	0.3	ND	0.4

Minimum detectable concentrations (MDCs) at the 95% confidence level

ND = not detected; result was less than the 95% critical level

Uncertainty reported at the 95% confidence level and represents counting error and ISOCS-generated efficiency error.

Soil Samples Collected in 1 Liter Bottles at the Canal and Water Treatment House Characterization											
BGRR Sample ID		ANALYSIS DATE	ASTD Sample ID	Estimated Activity Concentration (pCi/gram)							
				K-40		Co-60		Cs-137		Am-241	
Bore Number	Depth			Results	MDC	Results	MDC	Results	MDC	Results	MDC
CCB-039	0"-6"	9/20/2000	SAM00986	ND	5.9	ND	0.3	16.4 ± 1.5	0.3	ND	0.5
CCB-039	06"-12"	9/20/2000	SAM00986	14.4 ± 3.0	0.4	ND	0.3	0.3 ± 0.1	0.1	ND	0.5
CCB-039	12"-18"	9/20/2000	SAM00986	11.5 ± 1.8	1.8	ND	0.3	ND	0.4	ND	0.4
CCB-039	18"-24"	9/20/2000	SAM00986	12.6 ± 2.8	0.4	ND	0.3	1.7 ± 0.4	0.4	ND	0.4
CCB-039	24"-30"	9/20/2000	SAM00986	11.2 ± 2.9	2.4	ND	0.3	ND	0.3	ND	0.4
CCB-039	30"-36"	9/20/2000	SAM00986	10 ± 2.7	2.6	ND	0.2	0.8 ± 0.2	0.2	ND	0.4
CCB-039	4'-12'	9/22/2000	SAM01031	8.1 ± 2.2	1.5	ND	0.2	ND	0.3	ND	0.4
CCB-039	20'-30'	9/22/2000	SAM01032	8.6 ± 2.3	1.5	ND	0.4	ND	0.3	ND	0.3
CCB-039	30'-40'	9/22/2000	SAM01033	9.7 ± 2.5	2.0	ND	0.4	ND	0.3	ND	0.4
CCB-039	40'-46'	9/22/2000	SAM01034	2.1 ± 2.5	4.1	ND	0.1	ND	0.3	ND	0.4
CCB-040	0"-6"	9/22/2000	SAM01027	9.5 ± 2.5	2.0	ND	0.3	0.3 ± 0.2	0.2	ND	0.4
CCB-040	06"-12"	9/22/2000	SAM01028	8.5 ± 2.5	2.4	ND	0.3	ND	0.4	ND	0.4
CCB-040	12"-18"	9/22/2000	SAM01029	9.6 ± 2.4	1.9	ND	0.2	0.2 ± 0.2	0.2	ND	0.4
CCB-040	18"-24"	9/22/2000	SAM01030	9.2 ± 2.5	2.2	ND	0.3	0.2 ± 0.2	0.2	ND	0.3
CCB-042	0"-6"	9/19/2000	SAM00968	14.2 ± 3.0	0.4	ND	0.3	ND	0.3	ND	0.4
CCB-042	06"-12"	9/19/2000	SAM00966	12.1 ± 2.8	1.7	ND	0.3	ND	0.3	ND	0.4
CCB-042	12"-18"	9/19/2000	SAM00967	9.9 ± 2.6	2.3	ND	0.3	ND	0.3	ND	0.4
EEC	0"-6"	9/25/2000	SAM01049	3 ± 2.5	4.0	ND	0.3	1.5 ± 0.3	0.3	ND	0.4
EEC	06"-12"	9/1/1932	SAM01050	ND	6.0	ND	0.3	0.5 ± 0.2	0.2	ND	0.4
EEC	12"-18"	9/1/1932	SAM01051	ND	5.3	ND	0.3	0.6 ± 0.2	0.2	ND	0.4
EEC	18"-24"	9/1/1932	SAM01052	5.6 ± 2.9	4.3	ND	0.3	1.6 ± 0.3	0.2	ND	0.4
CJ	0"-6"	9/25/2000	SAM01054	ND	82.8	ND	5.4	4,400 ± 390	24.5	ND	37.9
CJ	06"-12"	9/25/2000	SAM01055	3.6 ± 2.8	4.5	0.2 ± 0.1	0.2	3,825 ± 291	5	ND	6.2
CJ	12"-18"	9/25/2000	SAM01056	3.7 ± 2.6	4.0	ND	0.3	662 ± 50.6	2	ND	2.9
CJ	12"-18"R	9/25/2000	SAM01057	2.1 ± 2.6	4.4	ND	0.3	662 ± 50.6	2	ND	2.9
CJ	18"-24"	9/25/2000	SAM01058	2.5 ± 2.7	4.5	ND	0.3	158 ± 12.3	0.9	ND	1.7
CS-3	0"-6"	9/25/2000	SAM01059	5.4 ± 3.9	6.4	ND	1.3	16,410 ± 1248	14.1	ND	12.7
CS-3	06"-12"	9/25/2000	SAM01060	ND	5.2	ND	0.4	5,545 ± 422	6	ND	7.0
WWS	0"-6"	9/25/2000	SAM01061	2.2 ± 2.3	3.8	ND	0.3	31.5 ± 2.6	0.4	ND	0.7
WWS	06"-12"	9/25/2000	SAM01062	3.1 ± 2.3	3.7	ND	0.2	8.9 ± 0.9	0.3	ND	0.5

Minimum detectable concentrations (MDCs) at the 95% confidence level

ND = not detected; result was less than the 95% critical level

Uncertainty reported at the 95% confidence level and represents counting error and ISOCS-generated efficiency error.

Section 16



MEMO

Bldg 830, Railroad Ave
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Upton, NY 11973
Voice 631-344-2777
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ACTION Jim Brower- ERD
INFO Paul Kalb, Proj Mgr
Glenn Van Sickle, ERD
FILE ASTD Project
30822-262-023
DATE Sep 20, 2000
FROM Larry Lockett, CHP
ASTD Project Analytical Physicist
SUBJECT: Results of Analysis for Soil Samples, AOC16G, Landscape Soil Remediation

- 1. Enclosed is tabulation of the results from the ISOCS analyses of five (5) soil samples collected by ERD during the evaluation of operations from the Landscape Soil Remediation Project. Soils were identified as from Brookhaven Center, AOC16G. Results reflect that CS-137 was the only gamma-emitting radionuclide detected, other than primordial radionuclides occurring naturally in soil. Preliminary results were previously provided verbally.
2. Soil samples were provided by ERD personnel and were counted on the ASTD ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, ERD control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Lockett, members of the ASTD Project Team.
4. The table provides data on Cs-137 (the radionuclide of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

NOTE 1: Results are provided "as reported" by ISOCS; they have not yet been corrected by any correlation factor as recommended by EPA.

Activity Concentration of CS-137 in Soil Samples
ERD Landscape Soil Remediation Project
Gamma Spectrum Analysis with the ISOCS Instrument

Table with 6 columns: ASTD Sample No, ERD Sample No, ERD Sample Description, Sample Mass (g), Cs-137 concentration (pCi/g) (Activity, MDA). Rows include samples SAM00973 through SAM00977 with their respective ERD sample numbers, descriptions (AOC16G, Center A-E), masses, and activity/MDA values.

ND = Not Detected

MDA = Minimum Detectable Concentration in pCi/g

Uncertainties are reported at the 95% confidence level and represent only counting error and ISOCS-related efficiency errors

DATA QUALIFIERS:

- 1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix



MEMO

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ACTION	INFO	FILE
Jim Brower- ERD	Paul Kalb, Proj Mgr Glenn Van Sickle, ERD	ASTD Project 30822-262-023

FROM Larry Lockett, CHP
ASTD Project Analytical Physicist

DATE Sep 11, 2000

SUBJECT: Results of Analysis for
Soil Samples, Bldg 494, AOC16G and ITD, Landscape Soil Remediation

1. Enclosed is tabulation of the results from the ISOCS analyses of ten (10) soil samples collected by ERD during the evaluation of operations from the Landscape Soil Remediation Project. Soils were identified as from various locations, including Bldg 494, AOC16G, and ITD. **Results reflect that CS-137 was the only gamma-emitting radionuclide detected, other than primordial radionuclides occurring naturally in soil.** Preliminary results were previously provided verbally.
2. Soil samples were provided by ERD personnel and were counted on the ASTD ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, ERD control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Lockett, members of the ASTD Project Team.
4. The table provides data on Cs-137 (the radionuclide of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

NOTE 1: Results are provided “as reported” by ISOCS; they have not yet been corrected by any correlation factor as recommended by EPA.

Activity Concentration of CS-137 in Soil Samples
ERD Landscape Soil Remediation Project
Gamma Spectrum Analysis with the ISOCS Instrument

ASTD Sample No	ERD Sample No	ERD Sample Description	Sample Mass (g)	Cs-137 concentration (pCi/g)	
				Activity	MDA
SAM00938	7363-001	Hot Soils Bldg 494, depth 4 ft, AOC16E-2 W001	1,608	8.8 ± 0.9	0.3
SAM00939	7362-001	AOC16S-F Final, 0-0.5 ft depth	1,315	7.1 ± 0.8	0.3
SAM00940	7362-002	Hot Area in AOC16S-W010, 0-0.16 ft depth	1,130	19.5 ± 1.8	0.5
SAM00942	7381-001	AOC16G, depth 0.16'	1,064	23.2 ± 2.0	0.4
SAM00943	7381-002	AOC16G, depth 0.16'	1,429	ND	0.3
SAM00944	7381-003	AOC16G, depth 0.16'	1,129	98.1 ± 7.7	0.8
SAM00945	7381-004	AOC16G, depth 0.16'	1,112	52.1 ± 4.2	0.6
SAM00946	7381-005	AOC16G, depth 0.16'	1,044	18.6 ± 0.4	0.4
SAM00947	7381-006	AOC16G, depth 0.16'	983	6.3 ± 0.8	0.4
SAM00948	7381-007	AOC16G, depth 0.16'	1,091	28.2 ± 2.4	0.3

ND = Not Detected

MDA = Minimum Detectable Concentration in pCi/g

Uncertainties are reported at the 95% confidence level and represent only counting error and ISOCS-related efficiency errors

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix



MEMO

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ACTION	INFO	FILE
Jim Brower- ERD	Paul Kalb, Proj Mgr Glenn Van Sickle, ERD	ASTD Project 30822-262-023

FROM Larry Luckett, CHP
ASTD Project Analytical Physicist

DATE Sep 11, 2000

SUBJECT: Results of Analysis for
Soil Samples, ORISE - NYSDEC Samples, Landscape Soil Remediation

1. Enclosed is a tabulation of the results from the ISOCS analyses of eight (8) soil samples provided by ERD during the evaluation of operations from the Landscape Soil Remediation Project. Soils were identified as samples between ORISE and NYSDEC from various locations. **Results reflect that CS-137 was the only gamma-emitting radionuclide detected, other than primordial radionuclides occurring naturally in soil.** Preliminary results were previously provided verbally.
2. Soil samples were provided by ERD personnel and were counted on the ASTD ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, ERD control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Luckett, members of the ASTD Project Team.
4. The table provides data on Cs-137 (the radionuclide of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

NOTE 1: Results are provided "as reported" by ISOCS; they have not yet been corrected by any correlation factor as recommended by EPA.

Activity Concentration of CS-137 in Soil Samples
ERD Landscape Soil Remediation Project
Gamma Spectrum Analysis with the ISOCS Instrument

ASTD Sample No	ERD Sample No	ERD Sample Description	Sample Mass (g)	Cs-137 concentration (pCi/g)	
				Activity	MDA
SAM00949	7394-001	Landscape Soil - ORISE_F1	1,510	79.2 ± 6.3	0.3
SAM00950	7394-002	Landscape Soil - DEC_F1	1,432	74.4 ± 5.9	0.7
SAM00951	7394-003	Landscape Soil - ORISE_F4	1,395	84.0 ± 6.7	0.7
SAM00952	7394-004	Landscape Soil - DEC_F3	1,499	14.3 ± 1.3	0.4
SAM00953	7394-005	Landscape Soil - ORISE_F2	1,530	12.0 ± 1.2	0.4
SAM00954	7394-006	Landscape Soil - ORISE_F3	1,588	55.8 ± 4.5	0.6
SAM00955	7394-007	Landscape Soil - DEC_F2	1,512	44.2 ± 3.6	0.6
SAM00956	7395-001	Landscape Soil - AOC16S6F1	1,454	20.8 ± 1.8	0.5

ND = Not Detected

MDA = Minimum Detectable Concentration in pCi/g

Uncertainties are reported at the 95% confidence level and represent only counting error and ISOCS-related efficiency errors

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix

ACTION

Jim Brower- ERD

INFO

Paul Kalb, Proj Mgr
Glenn Van Sickle, ERD

FILE ASTD Project

30822-262-023

FROM Larry Luckett, CHP
ASTD Project Analytical Physicist

DATE Aug 31, 2000

SUBJECT: Results of Analysis for
Soil Samples, Various Locations, Landscape Soil Remediation

1. Enclosed are tabulation of the results from the ISOCS analyses of ten (10) soil samples collected by ERD during the evaluation of operations from the Landscape Soil Remediation Project. Soils were identified as from various locations, including AOC16E2, AOC16F, Trucks, and ITD. **Results reflect that CS-137 was the only gamma-emitting radionuclide detected, other than primordial radionuclides occurring naturally in soil.** Preliminary results were previously provided verbally.
2. Soil samples were provided by ERD personnel and were counted on the ASTD ISOCS instrument in an ISOCS counting area established on the ground level (110' elevation) of Building 701. All samples were received from, and have been transferred back to, ERD control.
3. Spectrum acquisition was performed by Larry Milian, quantification modeling results were performed by Milian, and results were reviewed by Larry Luckett, members of the ASTD Project Team.
4. The table provides data on Cs-137 (the radionuclide of concern). Full gamma spectroscopy analysis reports for each sample are on file with ASTD Project records and are available upon request.

Should you have any questions, please contact me at x-2777.

NOTE 1: Results are provided "as reported" by ISOCS; they have not yet been corrected by any correlation factor as recommended by EPA.

NOTE 2: Soil samples were provided by ERD personnel in 500-ml HDPE containers which is a non-standard geometry for samples. Use of 1-liter wide mouth HDPE bottles would result in more timely, efficient, and accurate analytical results.

Activity Concentration of CS-137 in Soil Samples
ERD Landscape Soil Remediation Project
Gamma Spectrum Analysis with the ISOCS Instrument

ASTD Sample No	ERD Sample No	ERD Sample Description	Sample Mass (g)	Cs-137 concentration (pCi/g)	
				Activity	MDA
SAM00927	AOC16F-W001	Soil sample 3.5 - 4.0 ft depth	1,585	12.2 ± 1.2	0.4
SAM00928	AOC16F-W002	Soil sample 3.0 - 3.5 ft depth	1,597	13.2 ± 1.2	0.3
SAM00929	7328-001	Hot soil from Pit at 16 F	1,401	43.8 ± 3.6	0.5
SAM00930	7278-001	Landscape soil - Truck 1-A	855	11.7 ± 1.1	0.4
SAM00931	7278-002	Landscape soil - Truck 1-B	919	11.7 ± 1.1	0.4
SAM00932	7278-003	Landscape soil - Truck 2-A	1,012	7.2 ± 0.7	0.3
SAM00933	7278-004	Landscape soil - Truck 2-B	979	9.1 ± 0.9	0.2
SAM00934	7330-001	Landscape soil - ITDF	1,070	17.8 ± 1.6	0.4
SAM00935	7330-001R	Recount of 7330-001	1,070	18.3 ± 1.7	0.3
SAM00936	7331-001	Landscape soil - 0 - .16'	859	17.5 ± 1.7	0.4
SAM00937	7332-001	AOC16E2-CS15; depth 0-6"	1,555	ND	0.3

ND = Not Detected

MDA = Minimum Detectable Concentration in pCi/g

Uncertainties are reported at the 95% confidence level and represent only counting error and ISOCS-related efficiency errors

DATA QUALIFIERS:

1. Analysis assumes uniform, homogeneous distribution of contaminants throughout the sample matrix