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Thermal Neutron Flux Contours from Criticality Event

L. L. Carter Westinghouse Hanford Company, Richland, WA 99352 U.S. Department of Energy Contract DE-AC06-87RL10930

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Key Words: Monte Carlo, neutron flux, criticality

Abstract: The generation of thermal neutron flux contours from a criticality event is demonstrated for an idealized building with a criticality event in one of the rooms. The MCNP Monte Carlo computer code is used to calculate the thermal neutron flux.

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THERMAL NEUTRON FLUX CONTOURS FROM CRITICALITY EVENT Revision 0

WHC-SD-SQA-ANAL-502

Prepared by:

L. L. Carter

August 1996

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nan way.			• • • • •		•••	••••	4
Figure 2. Hallway .	Ihermal	Neutron Flux	Contours	(neutrons/cm ⁻ -s)	for	Generic	Room and 5

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1.0 SUMMARY

An idealized building was utilized to demonstrate the generation of thermal neutron flux contours by using the MCNP Monte Carlo code' to calculate the thermal fluxes. Such contours can be used in the determination of optimal placement of criticality alarms. All of the MCNP comparisons used the Plutonium Nitrate Solution Criticality neutron source of Reference 2 (Case 3) with source strength 6.237×10^{13} neutrons/second. The idealized building contains a room where the criticality event is postulated to occur and a corridor with one bend. The walls of the corridor and the building are all ordinary concrete 30 cm thick.

Shown in Figure 1 are the thermal neutron flux contours within the building, where the outside perimeter of Figure 1 is the 30 cm thick outer walls of the building. Figure 2 is an expanded view showing the room where the criticality event occurs and its immediate vicinity. These contours clearly show the flux decrease from attenuation through the walls and the increased flux down the corridor. The MCNP calculations required 37.5 hours of cpu time on an HP-735 so are quite number crunching intensive. The one standard deviation statistical uncertainties in the MCNP calculation are typically less than one percent for the room and vicinity shown in Figure 2 and less than ten percent within most of the building of Figure 1.

The following section describes the MCNP calculations in some detail including the optimization considerations. We have demonstrated with this generic problem that it is possible to optimize an MCNP calculation sufficiently to obtain the flux information to generate flux contours with computer times of a few days.

My thanks to Karl Hillesland for making the contour plots with the PV-Wave^a visualization package on a UNIX^b platform.

2.0 DETAILS REGARDING THE MCNP CALCULATIONS

2.1 GEOMETRY MODEL

The plan view of Figure 1 that shows the contours is also used to describe the geometry. The Plutonium Nitrate Solution Criticality neutron source of Reference 2 (Case 3) with source strength 6.237×10^{13} neutrons/second is in the center of the 10^8 thermal flux contour of Figure 1. The walls of this room, of the corridor, and of the building (perimeter of Figure 1) are all ordinary concrete 30 cm thick. The height of the corridor and all rooms

"PV-Wave is a trademark of Precision Visuals.

^bUNIX is a trademark of AT&T.

is assumed to be 240 cm, and the width of the corridor is also assumed to be 240 cm in this idealized calculation.

A spatial grid of 42 mesh intervals in x (abscissa in Figure 1), 39 mesh intervals in y, and one mesh interval in z along the 240 cm height between floor and ceiling were used by MCNP to calculate the thermal neutron fluxes for input into PV-Wave visualization package to obtain the contour plots. The MCNP calculations were actually made with the Hanford version of MCNP since this allows for automatically tallying on such a fine mesh. However, with some additional tallies the same data could be generated with the standard MCNP from LANL.

MCNP importances¹ were used to improve the calculational efficiency. The air and walls along each straight section of the corridor were divided into two sets of cells of approximately equal length for setting importances. The importance was increased a factor of four between each of these sets of cells along the corridor and by a factor of sixteen between the room containing the source and the entrance from this room into the corridor.

The MCNP input file for this generic calculation to demonstrate the generation of thermal neutron flux contours is listed in Appendix A.

3.0 REFERENCES

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- Miles, R.E., "DEVELOPMENT OF LOCATION CRITERIA FOR THE ROCKY FLATS PLANT CRITICALITY ALARM DETECTORS," Nuclear Safety Engineering Technical Report No. NSTR-001-91, EG&G Rocky Flats, August 7, 1991.



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APPENDIX A

LISTING OF MCNP INPUT FILE FOR CALCULATION OF THERMAL NEUTRON FLUXES

Prepared by:

Leland L. Carter Westinghouse Hanford Company August 1996

APPENDIX A

LISTING OF MCNP INPUT FILE FOR CALCULATION OF THERMAL NEUTRON FLUXES

generic building to demonstrate criticality detector contours, ilee С room with source 2 -23 2 -.0010556 19 99 -42 41 1 -3 3 -2.53 -4 2 -24 (3:-19:23) (-22:23) -42 41 2 18 3 3 -2.53 -5 2 -25 17 (4:-18:24) (-21:24) -42 41 corridor next to source C 2 -.0010556 4 -6 3 -23 22 -42 41 5 3 - 2.53-6 4 -24 23 -42 41 6 3 -2.53 -6 5 -25 24 -42 41 7 3 - 2.53-6 4 -22 21 -42 41 8 3 -2.53 -6 5 -21 20 -42 41 9 2 -.0010556 -10 6 -23 22 -42 41 10 3 -2.53 -8 -24 23 6 -42 41 -7 11 3 -2.53 6 -25 24 -42 41 12 3 -2.53 -11 6 -23 21 (10:-22)-42 41 13 3 -2.53 -13 6 -21 20 -42 41 14 3 -2.53 -13 11 -23 21 (-12:-22)-42 41 С corridor beyond bend 15 2 -.0010556 -10 9 -26 23 -42 41 16 3 -2.53 -9 23 8 -26 -42 41 17 3 -2.53 -8 7 -26 24 -42 41 23 18 3 -2.53 -11 10 -26 -42 41 19 3 -2.53 -12 11 -26 23 -42 41 20 2 -.0010556 ~10 9 -27 26 -42 41 21 3 -2.53 -9 8 -27 26 -42 41 22 3 -2.53 -8 7 -27 26 -42 41 23 3 -2.53 -11 10 -27 26 -42 41 24 3 -2.53 -12 11 26 -27 -42 41 с other rooms 25 -7 2 -27 25 2 -.0010556 -42 41 26 2 -.0010556 -13 12 -27 22 -42 41 27 2 -.0010556 -13 2 -20 16 (5:-17)-42 41 outside concrete walls с 28 3 -2.53 -2 1 -25 17 -42 41 29 3 -2.53 -14 1 -20 15 (13:(-2 - 17):-16)-42 41 30 3 -2.53 -14 10 -28 20 (13:27) -42 41 31 2 -.0010556 -10 9 -28 27 -42 41 32 3 -2.53 ~9 1 -28 25 (-2:27)-42 41 с ceiling and floor с above/below cells 1 and 28 41 3 -2.53 -5 1 -25 -44 42 17 42 3 - 2.53-5 -25 -46 1 17 44 43 3 -2.53 -5 -25 43 1 17 -41 44 3 - 2.53-5 1 -25 17 -43 45 с above/below cell 4

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