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Results for Case A for Task Order 041-001

K. N. Schwinkendorf Fluor Daniel Northwest, Inc., Richland, WA 99352 U.S. Department of Energy Contract DE-AC06-96RL13200

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Key Words: Spent Nuclear Fuel, MCO, MCNP

Abstract: A design change in the Multicanister Overpack (MCO) container scrap basket has required a reevaluation for criticality safety. The bounding case, the drop accident case for MarkIV-loaded MCOs, was reanalyzed with less mass in the baseplate of the upper scrap basket. The reduction in mass was from 50 lb to 271b. No statistically significant difference between the two cases was found. The original analysis is therefore considered valid and no additional analysis is required.

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CHECKLIST FOR INDEPENDENT REVIEW

Document Reviewe	d: Results for CASEA for Task Order 041-001
Author: <u>Yes No N/A</u>	K. N. Schwinkendorf
Щ	Problem completely defined.
Жпп	Necessary assumptions explicitly stated and supported.
AUD	Computer codes and data files documented.
<u>(א</u> ונו נו	Data checked for consistency with original source information as applicable.
[] [] 🕅	Mathematical derivations checked including dimensional consistency of results.
מו נו 🕅	Models appropriate and used within range of validity or use outside range of established validity justified.
[] [] [X	Hand calculations checked for errors.
ស្តាល	Code run streams correct and consistent with analysis documentation.
Хіпп	Code output consistent with input and with results reported in analysis documentation.
[][]]][]	Acceptability limits on analytical results applicable and supported. Limits checked against sources.
[1]	Safety margins consistent with good engineering practices.
)X[[] []	Conclusions consistent with analytical results and applicable limits.
X [] []	Results and conclusions address all points required in the problem statement.
	Have all reasonable accidents been considered?
[][]])	Has low density water (steam) been evaluated as a moderator?
Ж́ [] []	Is the fuel and other hardware composition correct?
[][][][X	Are the cases considered conservative? Too conservative?
[][]]))	Do the computer models adequately reflect the actual geometry? Have cross sectional cuts of the geometry been made and do they show the desired geometry?
ШЦЖ	Has the analysis been reviewed by Safety? This may not be required in a preliminary design.
[][])[)	Has the reviewer completed the Criticality Safety Course for Managers and Engineers?
Reviewed by:	Victor Roelman Date 6/18/97

NOTE: Any hand calculations, notes, or summaries generated as part of this review should be signed, dated, and attached to this checklist. Materials should be labeled and recorded so that it is intelligible to a technically-qualified third party.

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RESULTS FOR CASE A FOR TASK ORDER 041-001

Methodology

The Monte Carlo N-Particle (MCNP) (Breismeister 1993) computer code was used to perform the calculation documented in this report. MCNP was used to perform a Monte Carlo simulation of the transport of neutrons through a model of the Multicanister Overpack (MCO) container to obtain a statistical estimate of the effective neutron multiplication factor, $k_{\rm eff}$.

Assumptions

MCNP calculation CASE4 (Schwinkendorf 1997) was based on an assumption of 3/8-inch stainless steel baseplates forming the bottom of the scrap baskets. These baseplates are assumed in the MCNP input model to be solid (the holes are not explicitly modeled). Given the diameter of the baseplate, this model includes nearly 50 lbs of steel. The current scrap basket design calls for 27.13 lb instead of the 50 lbs assumed in the reference. This calculation was therefore modified by reducing the material density of the scrap basket baseplate so that the weight is reduced to 27.13 lb.

CASE4 is the Mark IV-loaded MCO after the drop accident, where all the intact fuel in the central three fuel baskets are assumed to fragment, and pack together into an optimal configuration with a packing fraction of 0.4 (the scrap baskets in the top and bottom locations remain loaded with optimal 0.95 wt% enriched uranium metal scrap).

Input Data

The input file for this calculation may be found in filename case4_nu.inp, located on the workstation server disk under the directory: /home_area/h56712/mcnp This data file is also appended to this calculation note.

Calculations

This calculation was performed using 50 neutron generations of 1,000 neutron histories per generation, and 10 generations were skipped before statistical information was included in the result. The calculation was performed on the sgi1 Silicon Graphics workstation.

Results

The results are as follows:

CASE4 from the Schwinkendorf reference: $k_{eff} = 0.9341 \pm 0.0028, 95\%$ upper bound estimate = 0.9398

Modified CASE4 with the reduced-density scrap basket baseplate: $k_{eff} = 0.9329 \pm 0.0027$, 95% upper bound estimate = 0.9382

Conclusions

The removal of neutron-absorbing material (reduced steel density in the scrap basket baseplate) from the MCNP model should have increased the k_{eff} of the system. However, this change in the model is so slight that MCNP was not able to discern a statistically-significant difference in the neutron multiplication constants between the two cases. The difference between the two k_{eff} values is only one-third of the 1-sigma statistical error in the two cases. The conclusion is that, within the statistical uncertainty in the Monte Carlo results, there is no statistically significant difference between how the scrap basket baseplates were modeled in the reference and the modified design. It is felt that no further analysis is required.

References

- Breismeister, J. F., Editor, 1993, MCNP A General Monte Carlo N-Particle Transport Code, Version 4A, LA-12625, Los Alamos National Laboratory, Los Alamos, New Mexico.
- Schwinkendorf, K. N., 1997, Criticality Safety Evaluation Report for Spent Nuclear Fuel Processing and Storage Facilities, HNF-SD-SNF-CSER-005 Revision 3, Fluor Daniel Northwest, Richland, Washington.

MCNP input file

The following is a listing of the input file used to perform this calculation:

message:

mu	lti	ple canist	er ove	er-pac	k - F	RUN 4 -	MKIV D	rop Ca	ase	
1	1	-18.82 -1		-				u=1	imp:n=1	\$ MKIV fuel scrap
2	2	-6.55 1	- 2					u=1	imp:n=1	\$ fuel clad
3	4	-1.00 2						u=1	imp:n=1	\$ lattice water
4	0	-56-9	8 - 7	10			lat=2	u=2	fil1=1	imp:n=1
5	1	-18.82 -3						u=3	imp:n=1	\$ MKIV fuel rubble
6	2	-6.55 3	-4					u=3	imp:n=1	\$ fuel clad
7	4	-1.00 4						u=3	imp:n=1	\$ lattice water
8	0	-11 12 -	15 14	4 -13	16	•	lat=2	u=4	fill=3	imp:n=1
9	4	-1.00 -2	0 21	-19					imp:n=1	\$ top water reflector
10	4	-1.00 -2	1 22	-17					imp:n=1	\$ water in gap #1
11	0	-17 -2	2 23				fill=2		imp:n=1	\$ scrap in basket #1
12	3	-4.717 -2	3 24	-17					imp:n=1	\$ ss plate #1
13	0	-17 -2	4 25				fill=4		imp:n=1	\$ MKIV rubble #2
14	3	-8.03 -2	5 26	-17					imp:n=1	\$ ss plate #2
15	0	-17 -2	6 27				fill=4		imp:n=1	\$ MKIV rubble #3
16	3	-8.03 -2	7 28	-17					imp:n=1	\$ ss plate #3
17	0	-17 -2	8 29				fill=4		imp:n=1	\$ MKIV rubble #4
18	3	-8.03 -2	9 30	-17					imp:n=1	\$ ss plate #4
19	0	-17 -3	0 31				fill=2		imp:n=1	\$ scrap in basket #5
20	3	-8.03 -3	1 32	-17					imp:n=1	\$ ss plate #5
21	3	-8.03 -3	2 33	-18					imp:n=1	\$ ss mco bottom end cap
22	3	-8.03 3	6 -18	-21	32				imp:n=1	\$ steel on side of mco
23	4	-1.00 -1	9 18	-21	33				imp:n=1	\$ water surrounding mco
24	4	-1.00 -3	3 34	-19					imp:n=1	\$ water below ss end cap
25	3	-8.03 1	7 -35	32	-21				imp:n=1	\$ mco liner
26	4	-1.00 3	5 -36	32	-21				<pre>imp:n=1</pre>	\$ water gap
27	0	20	: -34	: 19					imp:n=0	<pre>\$ outside world</pre>

1 cz 0.9

\$ optimum scrap radius

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2	сz	0.96444	\$ scrap token clad radius
3	cz	1.5	\$ optimum rubble radius
4	сz	1.60741	\$ rubble token clad radius
5	px	1.51418	\$ lattice hexagon planes for scrap

6	рx	~1.5141	.8		
7	р	~0.5773	5 1.0 0	.0 1.748	42
8	р	0.5773	5 1.0 0	.0 -1.748	42
9	Р	0.5773	5 1.0 0	.0 1.748	42
10	p	-0.5773	5 1.0 0	.0 -1.748	42
11	px	2,2586	1		\$ lattice hexagon planes for rubble
12	nx	-2 2586	-		t second sendon preside respire
12	p	-0 5773	E 1 0 0	0 2 606	10
1.5	P	0.5775	5 1.0 0	.0 2.000	2
14	p	0.5773	5 1.0 0	.0 -2.608	J2
15	₽	0.5//3	5 1.0 0	.0 2.608	J2
16	Р	-0.5773	5 1.0 0	.0 -2.608	02
17	сz	29.5275			\$ basket diameter 23,25"
18	сz	50.8			\$ outer steel radius 20"
19	СZ	81.28			\$ water outside mco
20	pz	209.4856	5		\$ top of water reflector
21	pz	179.0056	5		\$ top of water gap #1
22	pz	161.0368	5		\$ top of scrap #1
23	pz	93.7903	5		\$ top of ss plate #1
24	nz	92.8378	5		s top of MKIV rubble #2
25	nz	31 5809	5		\$ top of ss plate #2
26	00	20 6294	5		top of MKIN rubble #3
20	22	30.0204	5		¢ top of an plate #3
21	pz	~30.6284	5		S top of ss plate #3
28	pz	-31.5809	-5		S top of MKIV rubble #4
29	pz	-92.8378	15		S top of ss plate #4
30	pz	-93.7903	5		\$ top of scrap #5
31	pz	-161.0368	5		\$ top of ss plate #5
32	pz	-161.9893	5		\$ top of ss end cap
33	pz	-181.0393	5		\$ bottom of ss end cap
34	pz	-211.5193	5		\$ water below ss end cap
35	cz	30.7975			\$ mco liner
36	07	31.75			S water gap
mod kco ksr	le n de c	1000 1.0 12.11344 -12.11344 9.03444	10 50 0.0 0.0 0.0	122.89 122.89 59.19	155 155 17
		-9.03444	0.0	59.19	47
		9.03444	0.0	0.0	
		-9.03444	0.0	0.0	
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		-12.11344	0.0	-122.0:	
m1		92235.500	-0.009471	92238.50	C -0.990529 \$ MKIV IMMEIS
m2		40000.50c	-1.000		s zr clad
m3		6000.50c	-0.0004		\$ stainless steel 304
		25055.50c	-0.0200		\$ (8.03 g/cc)
		14000.50c	-0.0100		
		24000.50c	-0.1900		
		28000.50c	-0.0925		
		26000.55c	-0.6871		
m4		26000.55c	-0.6871 -0.1119	8016.50	c -0.8881 S water
m4 mt4		26000.55c 1001.50c lwtr.01t	-0.6871 -0.1119	8016.50	c ~0.8881 \$ water
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c	-0.6871 -0.1119 -0.000396	8016.50	c -0.8881 \$ water \$ borated stainless steel 300
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c	-0.6871 -0.1119 -0.000396	8016.50	c -0.8881 \$ water \$ borated stainless steel 30 \$ (8 03 g/cc)
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c 25055.50c	-0.6871 -0.1119 -0.000396 -0.0198	8016.50	c -0.8881 \$ water \$ borated stainless steel 304 \$ (8.03 g/cc)
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c 25055.50c 14000.50c	-0.6871 -0.1119 -0.000396 -0.0198 -0.0099	8016.50	c -0.8881 \$ water \$ borated stainless steel 304 \$ (8.03 g/cc)
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c 25055.50c 14000.50c 24000.50c	-0.6871 -0.1119 -0.000396 -0.0198 -0.0099 -0.1881	8016.50	c -0.8881 \$ water \$ borated stainless steel 304 \$ (8.03 g/cc)
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c 25055.50c 14000.50c 24000.50c 28000.50c	-0.6871 -0.1119 -0.000396 -0.0198 -0.0099 -0.1881 -0.091575	8016.50	c -0.8881 \$ water \$ borated stainless steel 304 \$ (8.03 g/cc)
m4 mt4 m5		26000.55c 1001.50c 1wtr.01t 6000.50c 25055.50c 14000.50c 24000.50c 24000.50c 26000.55c	-0.6871 -0.1119 -0.000396 -0.0198 -0.0099 -0.1881 -0.091575 -0.680229	8016.50	c -0.8881 \$ water \$ borated stainless steel 304 \$ (8.03 g/cc)
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