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**Dose Attenuation in Innovative Shielding Materials for Radiation Protection in Space:
Measurements and Simulations**

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(Article begins on next page)

Abstract

Galactic cosmic rays (GCR) are the number one hindrance to manned space exploration. Currently, the most realistic way to reduce the dangers caused by GCR to acceptable levels is passive shielding. Light materials guarantee the strongest dose attenuation per unit mass. High-density polyethylene is considered the gold standard for radiation protection in space. Nevertheless, accelerator-based experimental campaigns already showed the advantages of more hydrogen-rich innovative shielding materials such as lithium hydride. The experimental campaigns of this work focused on the absorbed dose attenuation properties of lithium-based hydrides chemically stabilised with a paraffin matrix. Such materials were compared to pure lithium-based hydrides, polyethylene, structural materials such as spacecraft aluminium alloys and lithium batteries, and *in situ* shielding materials such as Moon regolith and its main components silicon and silicon dioxide. The experimental results were compared to simulations performed with PHITS, FLUKA, and Geant4, which are among the most commonly used Monte Carlo codes for radiation protection in space. The simulations showed systematic differences and highlighted the pressing need for reliable nuclear cross-section models.

Keywords:

Shielding in space, absorbed dose measurements, lithium-based hydrides, Moon regolith, Monte Carlo simulations

1 Introduction

The dangers due to galactic cosmic rays (GCR) (Simpson, 1983) are the biggest hindrance to manned long-term deep-space exploration missions (Durante & Cucinotta, 2011; Cucinotta et al., 2013; Chancellor et al., 2014; Kennedy, 2014). Currently, the most promising radiation protection strategy is passive shielding (Council et al., 2008; Durante & Cucinotta, 2011; Washburn et al., 2015). However, the severe mass constraints of long-term space missions combined with the high penetration power of GCR, make it practically impossible to stop them within the shielding material. Therefore, the most efficient strategy to mitigate the effect of GCR is to exploit nuclear fragmentation processes and break heavy ions into lighter and lower-LET (linear energy transfer) particles, which are less dangerous because of their lower relative biological effectiveness (Zeitlin & La Tessa, 2016). The quantity to be maximised with the choice of shielding materials is the nuclear fragmentation cross-section per unit mass of material. This quantity scales as $A_s^{-1/3}$, A_s being the mass number of the shield. Therefore, light materials are considered to be the best option for passive shielding purposes (Miller et al., 2003; Shavers et al., 2004; Zeitlin et al., 2006; Guetersloh et al., 2006; Durante & Cucinotta, 2011).

The ROSSINI3 project aimed at testing stable and manageable high-performance shielding materials. The project was funded by the European Space Agency and is a collaboration of Thales Alenia Space Italia, GSI Helmholtzzentrum für Schwerionenforschung, and the University of Torino.

The main GCR contributors to dose equivalent behind thin shields are iron ions (Durante & Cucinotta, 2011) and their energy spectrum peaks around 1 GeV/u. Therefore, several accelerator-based experimental campaigns making use of high-energy (0.4 to 1 GeV/u) ^{56}Fe beams, were conducted in the past years (La Tessa et al., 2005; Zeitlin et al., 2006, 2008; Lobascio et al., 2008; Castellanos et al., 2017; Boscolo et al., 2020). In the framework of this work, dose attenuation curves for 1 GeV/u ^{56}Fe beams have been measured with the aim of directly comparing the dose-attenuation properties of lithium-based hydrides and their composites to other structural and potential shielding materials.

One could argue that, the comparison of the shielding effectiveness of different materials by using absorbed dose measurements performed with a single-energy and single-ion beam, is not significant *per se*. The galactic (and intergalactic) cosmic ray spectrum composition is in fact, rich both in ions, spanning from H to Ni, and in energies, from a few MeV up to the ZeV region. In addition, the most significant quantity to take care of is the endpoint-dependent relative biological effectiveness of the mixed radiation field composed of primary GCR and the secondary particles produced in the shield (Borak et al., 2014). Nevertheless, it is crucial to perform such “simple” accelerator-based experiments to compare the experimental results to the outcome of Monte Carlo (MC) simulations. Thanks to this comparison, it is in fact possible to understand more about the validity of the basic physics models underlying the MC codes. The improvement of the models leads also to improvements in the risk model calculations, on which radiation protection of astronauts fully relies. In addition, ^{56}Fe is the most relevant ion species of the GCR spectrum in free space (Durante & Cucinotta, 2011). For these reasons, this study and the corresponding MC benchmark provide valuable data. The thicker the shielding becomes, the higher the contribution of light ions to the dose equivalent is (Norbury et al., 2020). Therefore, experimental campaigns with protons and helium ions are ongoing at the GSI Helmholtz center for Heavy Ion research GmbH and the Heidelberg Ion-Beam Therapy Center.

The materials chosen for the ROSSINI3 experimental campaigns are listed and described in Section 2. Innovative shielding materials such as LiH, LiBH_4 , and the same two hydrides stabilised with paraffin were tested and compared to the gold-standard for radiation protection in space: polyethylene (Durante & Cucinotta, 2011; Shavers et al., 2004). The results of the ROSSINI2 program (Giraud et al., 2018; Schuy et al., 2018) suggested in fact, that the dose attenuation tests and simulations in the follow-up ROSSINI3 cam-

87 paign should focus on lithium-based hydride materials. In particular, LiH showed promising
 88 results in dose attenuation of ^{56}Fe beams within the ROSSINI2 experimental campaigns.
 89 Such lithium-based materials have also been the focus of attention of other promising sim-
 90 ulation campaigns (Naito et al., 2020). Nevertheless, since pure lithium-based hydrides are
 91 chemically reacting with moisture and difficult to handle, new composite materials were
 92 created in the framework of the ROSSINI3 project and tested alongside pure samples. A
 93 certain amount of paraffin was mixed to the hydrides to make them more stable. The
 94 longevity of such materials is discussed in Section 3.1. The production and characterisation
 95 of the composite materials was carried out by the chemistry department of the University
 96 of Torino. Pure paraffin was tested for comparison. Additionally, structural and *in situ*
 97 shielding materials were tested as well. Three different aluminium alloys commonly used
 98 in space, i.e. Al2024, Al2219 and Al2195 were compared to each other, as well as lithium
 99 batteries. Simulant of polar highland Moon regolith itself was tested as potential *in situ*
 100 shielding material, alongside Si and SiO_2 , which are the main components (SiO_2 ca. 50%,
 101 see Table 1) of Moon regolith. Some multi-layer configurations involving candidate shielding
 102 materials, simulant of Moon sand and concrete, and aluminium were also tested.

103 The results of the experimental campaigns were compared to the predictions of FLUKA
 104 (Ferrari et al., 2005; Böhlen et al., 2014; Aricò et al., 2019), PHITS (Iwase et al., 2002), and
 105 Geant4 (Agostinelli et al., 2003), which are among the most commonly used MC codes for
 106 radiation protection in space applications.

107 2 Materials and Methods

108 2.1 Target materials

109 LiH was produced by the Alfa Aesar company with a purity of 97+% (CAS number: 7580-
 110 67-8), LiBH_4 by Acros Organics with a purity of 95% (CAS: 16949-15-8) and paraffin by
 111 Sigma-Aldrich (Paraplast[®], CAS: 145686-99-3). Paraffin wax was selected to be mixed
 112 with hydrides, due to its high hydrogen content and relatively easy manufacturing processes
 113 (see Appendix A). The composite samples were produced using the highest possible hydride
 114 content that still guarantees sufficient mechanical and chemical stability of the samples. In
 115 particular 50% hydride content in weight was used for LiH-paraffin, and 40% for LiBH_4 -
 116 paraffin. All the pure and composite lithium-based hydrides were embedded into vacuum
 117 seal plastic bags (areal density of 0.009 g cm^{-2}) to keep them out of contact with air.

118 Pure paraffin samples were also prepared for comparison. A simple melting-cooling pro-
 119 cedure was performed in air, leading to paraffin samples having a similar size of composites.
 120 The procedure was repeated and additional material was added to obtain flat and parallel
 121 surfaces. Details about the preparation of the pure and composite lithium-based hydride
 122 and pure paraffin samples can be found in Appendix A, alongside the characterisation of
 123 their purity and homogeneity.

124 Three aluminium alloys generally used for structural components of space vehicles, were
 125 selected and tested in the ROSSINI3 experimental campaigns: aluminium 2024, 2219 and
 126 2195. All the selected samples belong to the AA 2000 series (alloyed with copper), which is
 127 the most extensively used alloy family in aerospace structures, due to its good mechanical
 128 properties.

129 Another material that could potentially become part of the spacecraft structure and
 130 that was tested within the ROSSINI3 experimental campaigns, is Li-poly batteries (stacked
 131 flat cellphone built-in Lithium polymer batteries without housing, relatively homogenous
 132 areal density).

133 High-density polyethylene (HDPE), golden standard for radiation protection in space,
 134 was also used in the experimental campaigns for comparison. Two HDPE wedges have been
 135 shifted relative to each other to obtain different material thicknesses.

Table 1: Highland Moon regolith mass percentage composition. LOI stays for Low-Order Impurities.

Element	Mass contribution (%)	Element	Mass contribution (%)
SiO ₂	47.89	Al ₂ O ₃	27.06
TiO ₂	0.52	FeO	3.68
MnO	0.06	MgO	2.84
CaO	14.19	Na ₂ O	2.43
K ₂ O	0.25	P ₂ O ₅	0.2
LOI	0.88		

136 Other materials used in the experimental campaigns are simulant of highland Moon
 137 regolith, pure Si and SiO₂. Moon regolith is in fact a potential *in situ* material, and Si
 138 and SiO₂ are among its main components. Cylindrical Si and SiO₂ targets are characterised
 139 by a purity of 99 and 99.99%, respectively. The Off Planet Research OPRH2N Near-Side
 140 Highland Lunar regolith simulant was used for this experimental campaign. Its composition
 141 is reported in Table 1, and it consists of sand with grain diameter between 250 and 500 μm .
 142 The Moon simulant was irradiated into a PMMA container with a later area of 5 x 7 cm²
 143 and a length of 4 cm. The walls of the box are 0.5 mm thick. A 1 mm-thick plexiglas wall
 144 was used to confine the Moon sand. Its relative position was changed at each irradiation so
 145 that thicker and thicker Moon layers could be placed.

146 A complete list of the irradiated materials can be found in Table 2. Error bars for the
 147 areal densities of the irradiated targets were estimated through error propagation of the
 148 measured weight, thickness and area of the targets, and summed up to obtain the error
 149 associated to the areal density of the total irradiated thickness.

150 2.2 Experimental setup

151 Irradiations of the described materials were performed with 1 GeV/u ⁵⁶Fe beams in GSI
 152 Cave A. Depending on the experimental campaign, the beam intensity ranged between
 153 1 and 3×10^7 ions per spill and the beam full-width-half-maximum (FWHM) between
 154 5 and 10 mm. The setup consisted of variable target thicknesses placed in between two
 155 large area parallel-plate ionisation chambers (ICs): the closest to the beam exit window
 156 is called IC1 and the other IC2. The active detector thickness is 2 x 10 mm for the IC2
 157 and 2 x 5 mm for the IC1, with an active area of 26 x 26 cm². It was filled with a gas
 158 mixture of 80% argon and 20% CO₂. The electrodes are represented with black lines. They
 159 are 6.7 mg cm⁻² thick and made of a nickel coated polyester mesh (43% and 57% mass
 160 percentages, respectively). The outer foils are 25 μm thick and made of BoPET (mylar).
 161 The ICs were kept in the same position and the different targets were exchanged between
 162 subsequent irradiations. IC1 operated as reference monitor for normalisation. Therefore,
 163 the results are not affected by unavoidable fluctuations in the beam intensity. The charge
 164 readout of the ionisation chambers was realised with high-precision electrometers (model
 165 K6517, KEITHLEY), which guaranteed accuracy in the charge measurement to be below
 166 1‰. The large area of the parallel-plate ionisation chambers, laterally integrates almost the
 167 complete signal from the beam. Therefore, the sensitivity of the results to beam width and
 168 scattering effects is minimised. The same setup has been largely used in the past (Giraud
 169 et al., 2018; Schuy et al., 2018; Pfuhl et al., 2018). A schematic of the experimental setup
 170 is reported in Figure 1. For the fine steps around the peak of the paraffin Bragg curve, a
 171 so-called range shifter (RS) (Simeonov et al., 2017) was used. It is composed of ten PE
 172 foils and plates with well-defined areal densities, each roughly doubling in size. Since it is
 173 remotely controlled, it allows a quick change of the amount of material in the beamline,

Table 2: List of all single materials used in this study alongside their mass densities (ρ), areas perpendicular to the beamline (A) and irradiated thicknesses (t), with associated uncertainties. The thicknesses are given in areal densities. LiH_p and LiBH_{4,p} stay for LiH and LiBH₄ stabilised with paraffin. The numbers in parentheses refer to the uncertainties of the values, and apply to the least significant digits.

Material	ρ (g cm ⁻³)	A (cm ²)	t (g cm ⁻²) ^a
LiH	0.5279(48)	33.20(51)	0.615(13), 1.231(16), 1.863(19), 2.481(26), 3.096(29), 3.710(31), 4.315(34), 4.937(37), 5.543(39), 6.149(41), 6.756(42), 7.359(44), 7.959(46), 8.559(47)
LiBH ₄	0.5255(59)	33.20(51)	0.633(11), 1.264(18), 1.897(22), 2.525(27), 3.149(32), 3.758(34), 4.383(37), 5.003(39), 5.637(42), 6.261(45)
LiH _p	0.7844(50)	77.73(31)	0.733(13), 1.477(16), 2.227(21), 2.969(23), 3.718(25), 4.456(28), 5.216(30), 5.965(32), 6.731(34), 7.486(35), 8.232(37), 9.018(39), 9.721(40), 10.504(42), 11.267(45)
LiBH _{4,p}	0.7523(38)	28.23(19)	0.692(12), 1.277(17), 1.943(19), 2.627(24), 3.326(26), 4.003(28), 4.673(30), 5.353(31), 6.011(32)
Paraffin	0.9025(56)	93.48(55)	1.845(18), ... 34.06(17) ^b
HDPE	0.9270(93)	^c	3.587(37), 4.811(49), 6.016(61), 6.962(70)
Al2024	2.690(55)	402(1)	2.261(18), 3.392(27)
Al2219	2.835(10)	72.00300(11)	2.026(10), 6.079(29), 8.105(38)
Al2195	2.705(28)	129.59(37)	3.646(31), 6.382(50)
Si	2.404(37)	79(2)	0.962(16), 1.923(30), 2.885(45), 3.846(59), 4.808(74), 5.769(88), 6.73(10)
SiO ₂	2.213(32)	79(2)	0.885(13), 1.770(25), 2.656(38), 3.541(51), 4.426(63), 5.311(76), 6.196(89)
Moon regolith	1.345(91)	35.00(86)	0.243(28), 0.65(14), 1.32(15), 1.99(18), 2.66(22), 4.01(31), 5.62(41)
Li-poly batteries	2.270(43)	^d	0.6883(29), 1.3899(64), 2.091(10), 2.789(14), 4.221(21), 6.545(31), 8.896(41), 11.933(53)

^a The thicknesses of the pure and composite lithium-based hydrides include the thin plastic bags they were embedded in during the irradiations, as well as the thicknesses of the Moon regolith simulant include the plexiglas foil and PMMA walls of the box it was irradiated in.

^b The whole Bragg curve was measured for paraffin in steps of 1.8 g cm⁻². At the Bragg peak, the steps are smaller and realised with the HDPE foils of the range shifter.

^c The HDPE target consists of two wedges shifted relatively to each other. Thus, the area of the target changes according to the overlap of the wedges. The wedges have an area of 246 cm² (minimum area) and an angle of 16°.

^d The area of the Li-ion batteries varies from 39 to 59 cm², depending on the battery.

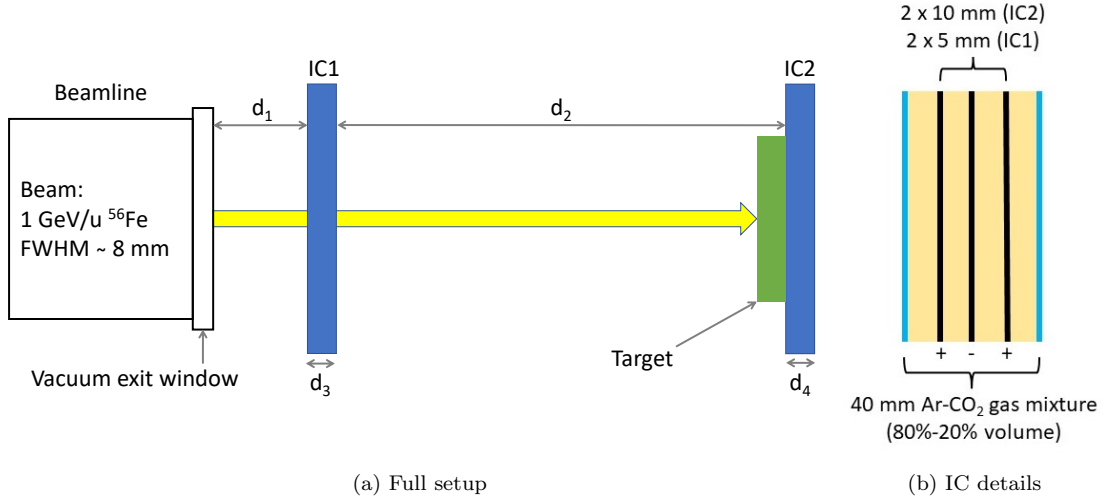


Figure 1: Panel (a): experimental setup. $d_3 = d_4 = 4$ cm. The distances d_1 and d_2 are beamtime dependent, d_1 being about 15 cm and d_2 about 1 m. The distance between the target and IC2 is also beamtime dependent and varies between 0 and 2 cm. The MC simulations have been performed accordingly. Panel (b): details about the ICs.

174 varying from 62 μ m of PE up to 64 mm, in steps of ca. 60 μ m. Since PE has almost the
 175 same stoichiometric composition as paraffin, it serves as a good supplement for the paraffin
 176 targets.

177 2.3 Data analysis

178 The ratio of the charge signal from the two ionisation chambers (Q_2/Q_1) normalised to the
 179 same ratio obtained without any target ($Q_{2,\text{no target}}/Q_{1,\text{no target}}$), provides the value for dose
 180 reduction due to the presence of the target:

$$181 \frac{D_2}{D_1} = \frac{Q_2/Q_1}{Q_{2,\text{no target}}/Q_{1,\text{no target}}}. \quad (1)$$

182 For each material thickness (or configuration of materials) at least two measurement points
 183 were taken. The final data were computed as the arithmetic average of these values, and
 184 the associated statistical fluctuations as their standard deviation. The systematic com-
 185 ponent of the errors was evaluated as the standard deviation of the fluctuation (mea-
 186 sured at different times of the experimental campaign) of the data obtained without target
 187 ($Q_{2,\text{no target}}/Q_{1,\text{no target}}$). This accounts for the uncertainty related to the non-linearities of
 188 the readout electronics, e.g. slight offsets.

189 2.4 Monte Carlo simulations

190 Simulations reproducing the ROSSINI3 experimental campaigns were carried out using the
 191 Monte Carlo particle transport codes PHITS (version 3.20), Geant4 (version 10.6 patch-02)
 192 and FLUKA (version 2020.0.3, and flair version 2.3-0 (Vlachoudis et al., 2009)). The main
 193 goal was to keep the simulations as similar as possible to focus on the differences among the
 194 physics models.

195 The simulation geometry reproduced the experimental setup. The dose ratios obtained
 196 for each target thickness simulated, were divided by the dose ratio obtained for the no target

197 case, as it was done with the experimental data. The doses were calculated in the active
 198 regions of the two ICs. The simulations also included the plastic bags and the box into
 199 which the lithium-based hydrides and Moon were respectively irradiated. The beam was
 200 modeled with a Gaussian lateral profile of 0.8 cm FWHM (no angular divergence), the ICs as
 201 two mylar walls filled with gas and no detailed internal electrode structures. Details about
 202 the ICs are in fact not crucial, since the dose ratios are normalised to the no target case.
 203 For the simulations, the number of generated primary ions has been selected high enough
 204 to decrease the statistical errors to less than 3% (10^4 to 10^5 ions).

205 **2.4.1 Geant4**

206 In Geant4, the dose in the detector was calculated by summing up the energy deposition in
 207 the active region of the detector and dividing it by the mass of the region itself. The default
 208 electron range cut of 0.1 mm was used for the pure and composite lithium-based hydrides,
 209 and for HDPE. It corresponds to an energy cut of 0.08 MeV in HDPE, and of 0.057 MeV
 210 in LiH. A 0.01 mm cut was used for electrons in the aluminium alloys, Si, SiO₂ and Moon
 211 regolith. It corresponds to an electron energy of 0.0325 MeV in aluminium, and of 0.03 MeV
 212 in Si. The reference physics lists QGSP_INCLXX, FTFP_BERT, and QBBC_EMY Geant4
 213 physics lists were used for the simulations. QGSP_INCLXX is an experimental physics
 214 list that uses the Quark Gluon String model for high-energy hadronic interactions, and
 215 the Leige Intranuclear Cascade model for proton, neutron and pion induced reaction with
 216 low energies, instead of the Binary or Bertini Cascade models. It is recommended to be
 217 used for shielding applications (“Geant4 Physics Reference manual, Release 10.6”, 2017).
 218 FTFP_BERT makes use of the FRITIOF String model for high-energy hadron interactions,
 219 and Bertini for low energies. It is recommended to be used for high-energy applications
 220 (“Geant4 Physics Reference manual, Release 10.6”, 2017). QBBC_EMY is a list created
 221 *ad hoc* for space, radiation biology, and radiation protection applications. It includes combina-
 222 tions of Binary, Bertini, Quark Gluon String, FRITIOF String and other models to reach
 223 high precision in the simulation of many hadron-ion and ion-ion interactions in a wide en-
 224 ergy range (Ivantchenko et al., 2012). This last list was chosen for the simulations because
 225 of the good agreement with the data presented in (Schuy et al., 2018) and more generally in
 226 the energy range between 100 MeV u⁻¹ and 1.5 GeV u⁻¹ (Ivantchenko et al., 2012). Also the
 227 reference physics lists QGSP_BERT, QGSP_BERT_EMV, and FTFP_INCLXX, were used
 228 to simulate the full paraffin Bragg curve for a deeper study of the differences among the lists
 229 themselves and the combination of the different models for different particles and energy
 230 ranges.

231 **2.4.2 PHITS**

232 In PHITS, the T-Deposit tally was used to calculate the dose deposition. A 1.0×10^{-3} MeV
 233 energy cut was used for electrons, gammas, and positrons, while the production threshold
 234 for delta rays was set to 0.1 MeV. The transport of electrons, positrons, and photons was
 235 based on the EGS5 algorithm, while gamma decay residual nuclei transport was based on
 236 the EBITEM model. Landau Vavilov energy straggling option was selected for charged
 237 particles and nuclei. The Lynch formula based on the Moliere theory was used for Coulomb
 238 diffusion, while JQMD-2.0 and SMM for nuclear reactions.

239 **2.4.3 FLUKA**

240 In FLUKA, transport cuts for e[±] and γ were set to 0.1 MeV, like the delta ray production cut.
 241 Projectile and target electromagnetic-dissociation and coalescence processes were activated,
 242 and the FLUKA evaporation model used was “New evaporation with heavy frag”.

3 Results and discussion

3.1 Stability of the lithium-based hydrides

As stated above, pure hydrides were mixed with paraffin in homogeneous composite pellets, with the aim of improving the hydrides' stability against moisture. Longevity tests were carried out to confirm the protection of paraffin against the reaction of the hydrides with moisture. The air stability tests are reported in Appendix B. The results show that the addition of the paraffin acts as an effective protective barrier to the hydride phase as the degradation kinetics due to the exposure to air is greatly reduced. Once properly embedded, the composite pellets do not change their chemical composition due to reaction with radiation, i.e. no radiolysis takes place. Such analysis was carried out after irradiation and is presented in greater detail in Appendix C. This can also be seen as a long-time longevity test for the developed materials in practical space applications.

3.2 Dose attenuation results

3.2.1 Experimental data

The measured dose ratio decreases with the increase of the target thickness. This is caused by the attenuation of the primary iron beam due to the fragmentation processes the ^{56}Fe ions undergo while traversing the target material. The produced fragments have a linear energy transfer (LET) lower than ^{56}Fe as $\text{LET} \propto Z_f^2$, where Z_f stands for the atomic number of the fragment. For 1 GeV/u ^{56}Fe ions, the slowing down of the primary beam cannot compensate for the dose reduction due to the fragmentation. Therefore, the dose ratio D_2/D_1 decreases. The analysis of the dose attenuation results has been performed as a function of the areal density of the targets (g cm^{-2}). The more fragmentation per unit mass happens, the stronger dose attenuation is expected for the same areal density of different targets. Lighter materials such as lithium-based hydrides are expected to provide stronger dose attenuation (Durante & Cucinotta, 2011). Heavier materials such as aluminium are expected to be less effective to this purpose.

The dose ratios (see Equation 1) obtained for single materials are plotted in Figure 2. As expected, the dose reduction is stronger for materials with lower mass number. LiH performs the best. The dose reduction obtained with LiBH_4 and the chemically stable LiH_p is the same and the second best after LiH. This result is very interesting as it shows that a chemically stable material can be a better shield from 1 GeV/u ^{56}Fe ions than HDPE, which is the so-far considered gold-standard for radiation protection in space. Aluminium, the main structural material of spacecrafts, is the worst shield among the tested materials. SiO_2 shows a dose reduction stronger than Si because $A_{\text{O}} < A_{\text{Si}}$. The results obtained with Moon regolith and SiO_2 are very similar as SiO_2 is the Moon regolith main component (see Table 1).

In Figure 2, only Al2219 alloy results are reported. A direct comparison between the three aluminium alloys is shown in Figure 3. They show similar dose attenuation curves, but thanks to the high precision of the experimental setup, such small differences can be resolved. However, no big role is expected to be played by the kind of aluminium alloy for radiation protection purposes.

For all the investigated materials, a small dose enhancement can be observed for very small target thicknesses. This phenomenon is called dose build-up (Pfuhl et al., 2018; Carlsson & Carlsson, 1977). More details about it are given in Appendix D.

Dose attenuation measurements were also performed with Li-ion batteries. As it can be seen in Figure 4, they perform better than aluminium in reducing the dose.

Some irradiations were performed merely by exchanging the order of the targets, without changing the material type or quantity. Pure LiH and LiBH_4 were studied for this

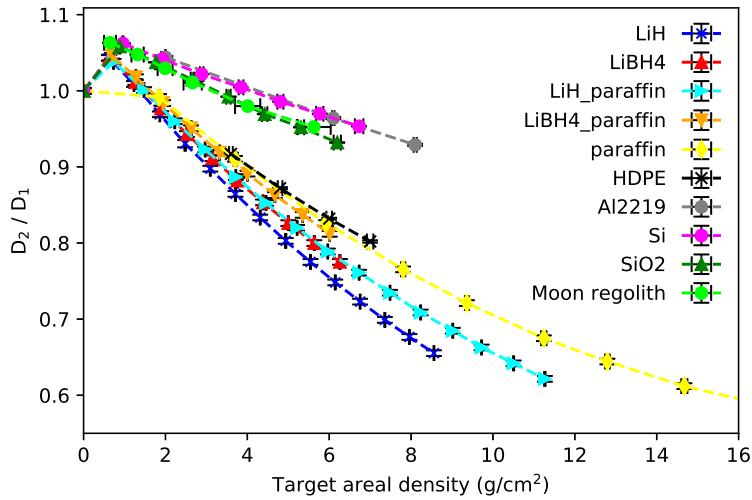


Figure 2: Dose attenuation results for all materials tested. The dashed lines simply connect the experimental points.

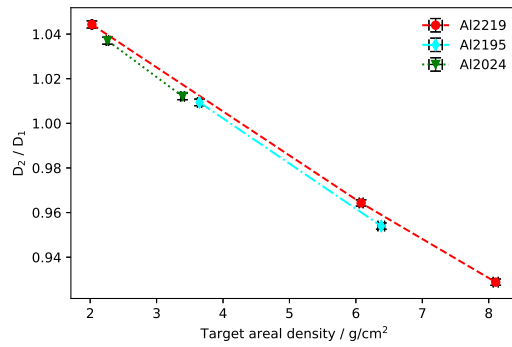


Figure 3: Dose attenuation results obtained with three different aluminium alloys generally used in space. The dashed lines simply connect the experimental points.

291 purpose, in combination with Al2219 and Al2195 alloys. The results of these exchange
 292 experiments are reported in Figure 5, and show that the absorbed dose attenuation is sys-
 293 tematically higher (1% effect ca.) if the shielding material (LiH or LiBH₄) is placed up-
 294 stream of the structural material (aluminium). The reason might be that the combination of energy
 295 loss and fragmentation is slightly more beneficial for the configuration in which the shielding
 296 material is placed upstream the aluminium. In a real case configuration, this would mean
 297 placing the shielding material inside the structural walls of the spacecraft. The consistency
 298 of such results is proved by the fact that the dose attenuation for shielding materials placed
 299 upstream Al is stronger independently of the choice of the Al alloy, the lithium-based hy-
 300 dride, and their thicknesses. It was already known that the order of the layers can have
 301 an impact on the shielding performance (Durante, 2008; Horst et al., 2022). Similar ex-
 302 periments were already performed by (Durante, 2008) with HDPE and aluminium, but did
 303 not show such systematic results. The reason is probably that the smaller area of the egg
 304 chamber used for those measurements cannot guarantee the same experimental precision
 305 of the data presented in this work. It is very interesting to notice that when looking at

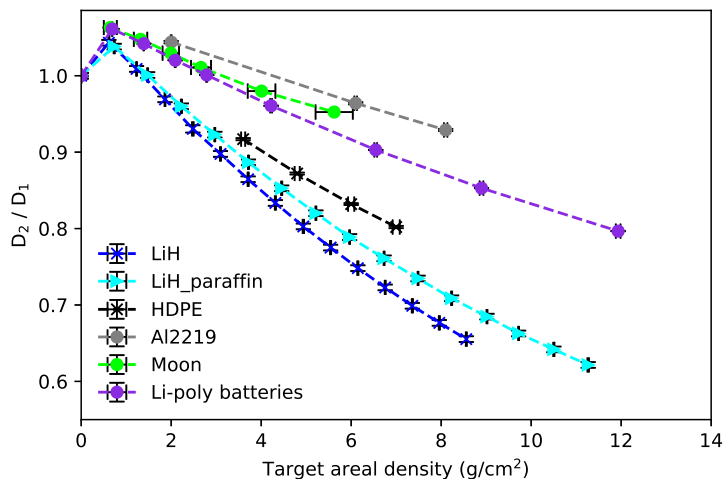


Figure 4: Dose attenuation results obtained with lithium batteries compared to aluminium, Moon regolith, PE, LiH_p and LiH.

306 dose equivalent the situation gets reversed, as placing the structural material (higher Z)
 307 outside and the shielding (lower Z) inside gives better dose equivalent attenuation. Such re-
 308 sults have been obtained by (Horst et al., 2022) with MC simulations for very thick shields
 309 taking into account also secondary neutrons. It was in fact proven that a stronger dose
 310 equivalent reduction is obtained by placing the hydron-rich material inside because of the
 311 better moderation of build-up neutrons. This highlights the need for studies involving the
 312 biological effectiveness of the shields.

313 Some realistic multi-layer configurations of spacecraft and *in situ* materials were also
 314 used for dose attenuation measurements. The results are presented in Appendix E.

315 3.2.2 Monte Carlo simulations

316 Monte Carlo predictions of the experimental results are reported in Figure 6 for the alu-
 317 minium alloys and HDPE, Figure 7 for the lithium-based hydrides and their composites, and
 318 Figure 8 for pure Si, SiO_2 , and Moon regolith. Error bars represent the statistical error due
 319 to the use of Monte Carlo approaches. PHITS reproduces well the results for aluminium
 320 and all the pure and composite lithium-based hydrides. On the other hand, PHITS over-
 321 estimates the results for HDPE, Si, SiO_2 and Moon regolith. FLUKA reproduces well the
 322 results for HDPE, all the pure and composite lithium-based hydrides and the Moon regolith.
 323 Nevertheless, it underestimates the results for the aluminium alloys, Si and SiO_2 . FLUKA
 324 tends therefore, to underestimate the results for the heaviest targets. All the Geant4 physics
 325 lists used can reproduce well and give consistent results for the aluminium alloys and Si,
 326 which are the heaviest tested targets. For all the other cases, QGSP_INCLXX is the list
 327 performing the best. QBBC_EMY and FTFP_BERT overestimate the results for HDPE,
 328 all the pure and composite lithium-based hydrides, SiO_2 and Moon regolith. These results
 329 confirm the recommendation of using QGSP_INCLXX for shielding applications. It should
 330 be also noted that QBBC_EMY, which is the reference physics lists recommended for med-
 331 ical and space application, is not able to reproduce well the experimental data for any of
 332 the studied materials, but the aluminium alloys.

333 The differences obtained between different MC codes can be large, reaching 15% for the
 334 largest amount of LiH_p . This is due to the different hadronic physics underlying FLUKA,

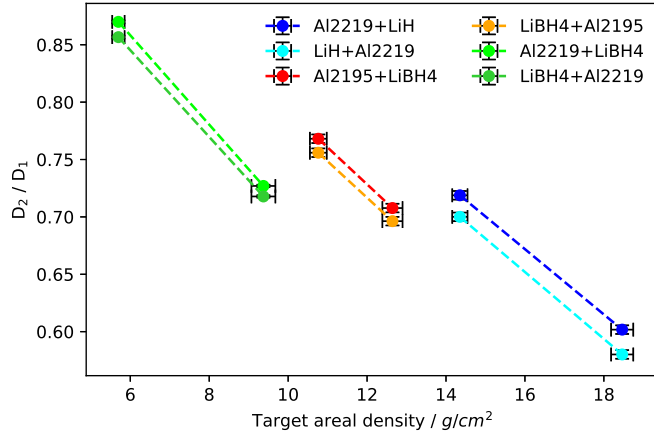


Figure 5: Dose attenuation results obtained merely exchanging the relative order of aluminium (spacecraft structural material) and lithium-based hydrides (shielding material). The dashed lines simply connect the experimental points.

PHITS and different Geant4 physics lists, and in particular, to the different nuclear cross-section parameterisations employed in the codes. These parameterisations are in fact, a crucial ingredient to Monte Carlo simulations (Luoni et al., 2021; Norbury et al., 2020, 2012; Townsend et al., 2002). If a simulation overestimates the dose reduction, the nuclear fragmentation cross-section of the primary ions in the target material is overestimated. The opposite is valid for Monte Carlo simulations underestimating the experimentally measured dose attenuation. Geant4 QBBC_EMY, for instance, underestimates the dose attenuation in HDPE and in all lithium-based hydrides. Therefore, we can deduce that the underlying physics might underestimate the nuclear fragmentation cross-section of 1 GeV/u ^{56}Fe on ^1H target nuclei. Since nuclear cross-section parameterisations are semi-empirical, they need cross-section measurement data to be validated. In Ref. (Luoni et al., 2021) it is shown that no mass-changing cross-sections are present in literature for the systems: $^{56}\text{Fe} + ^{27}\text{Al}$, $^{56}\text{Fe} + ^{28}\text{Si}$ and $^{56}\text{Fe} + ^{16}\text{O}$. This prevents the model validation for such systems and causes more uncertainty in the Monte Carlo simulation results.

A full Bragg curve measurement has been carried out with paraffin. The first part of the curve can be found in Figure 2. The full curve and the Monte Carlo simulation results are reported in Figure 9. In panel (b) of Figure 9, the results obtained with different Geant4 physics lists are reported. Several lists have been used to study the differences in the results introduced by changing the hadronic and electromagnetic models implemented in Geant4. Since QGSP_INCLXX is the list performing the best, the results obtained with it are reported in panel (a) of the same figure, alongside the PHITS and FLUKA results. It can be observed that FLUKA, PHITS and Geant4 QGSP_INCLXX reproduce the fragmentation and tail regions well. PHITS however, underestimates the range of the primaries, and Geant4 QGSP_INCLXX overestimates the height of the Bragg peak. This overestimation is a sign of an underestimation of the nuclear fragmentation processes undergone by the primary Fe ions. In panel (b), it can be observed that the results strongly depend on the reference physics list used. QGSP_BERT_EMV, QBBC_EMY and QGSP_BERT underestimate the Fe nuclear fragmentation processes much stronger than QGSP_INCLXX and FTFP_INCLXX. The latter on the other hand, reproduce well the trend of the data before the Bragg peak and in the tail regions. QGSP_BERT_EMV and QGSP_BERT make use of the Bertini Cascade model at low energies, and QBBC_EMY for some interactions as well. On the other hand, QGSP_INCLXX and FTFP_INCLXX use the Leige Intranuclear Cascade model for proton,

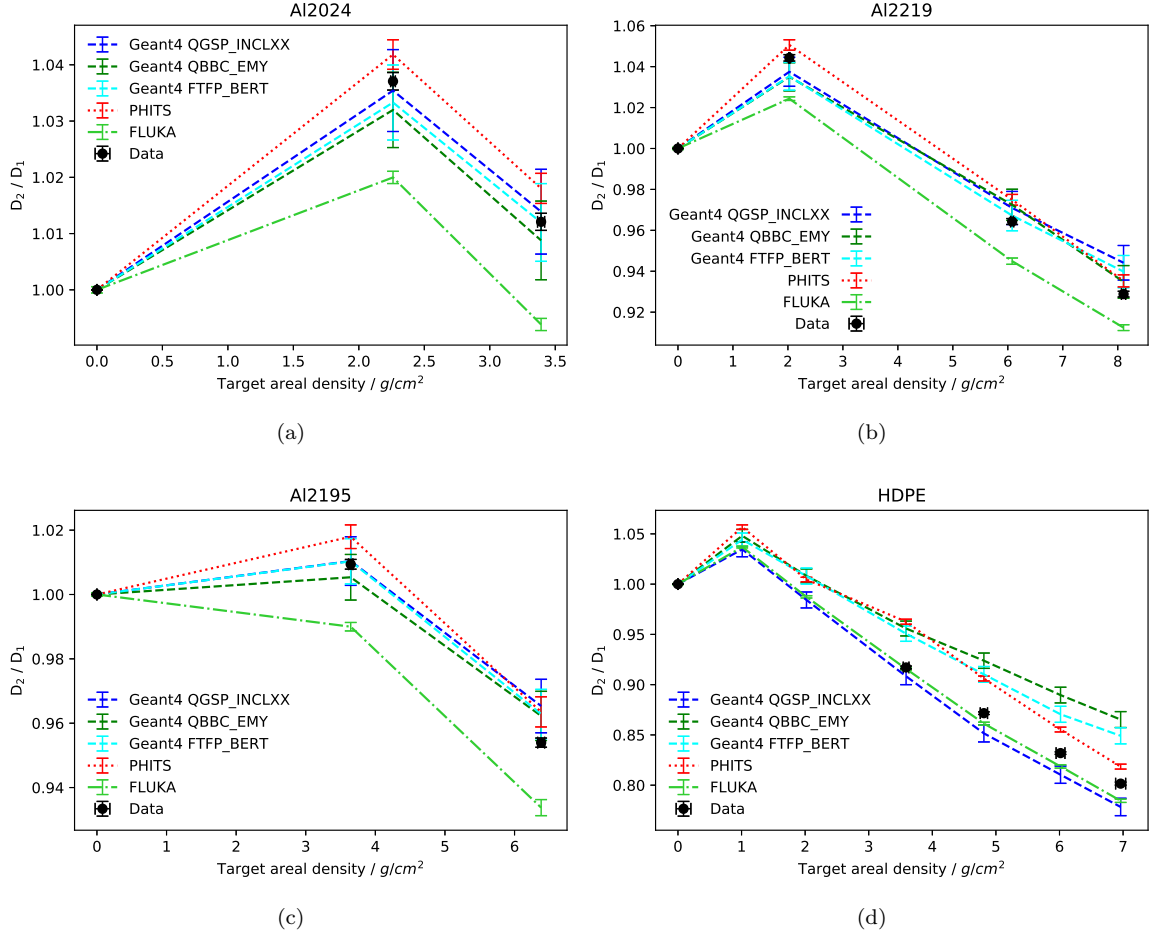


Figure 6: Comparison between experimental data and MC simulation results for the three tested aluminium alloys and for HDPE.

367 neutron and pion induced reactions at low energies. The differences among the lists look
 368 therefore mainly due to low-energy interactions.

369 4 Conclusions

370 Absorbed dose attenuation measurements of $1\text{ GeV/u } ^{56}\text{Fe}$ ions were performed with well-
 371 established, innovative and potential *in situ* shielding materials for long-term deep-space
 372 exploration. Valuable and precise data were obtained. The addition of paraffin to innovative
 373 lithium-based hydrides was proven to be effective to make mechanically and chemically
 374 stable promising composite shielding materials. In particular, LiH-paraffin seems to combine
 375 the excellent dose attenuation properties of LiH with a manageable chemical stability.

376 The experimental data were then compared with commonly used MC codes for space
 377 exploration simulations, i.e. FLUKA, PHITS, and Geant4. FLUKA resulted to be the best
 378 fit with the experimental data for hydrogen-rich materials (HDPE and lithium-based hy-
 379 drides), while the worst fit for heavier materials such as the aluminium alloys, Si and SiO_2 .
 380 In particular, FLUKA overestimates the fragmentation and therefore, the dose attenuation
 381 of high-Z materials. For what concerns the paraffin Bragg curve, FLUKA can reproduce well

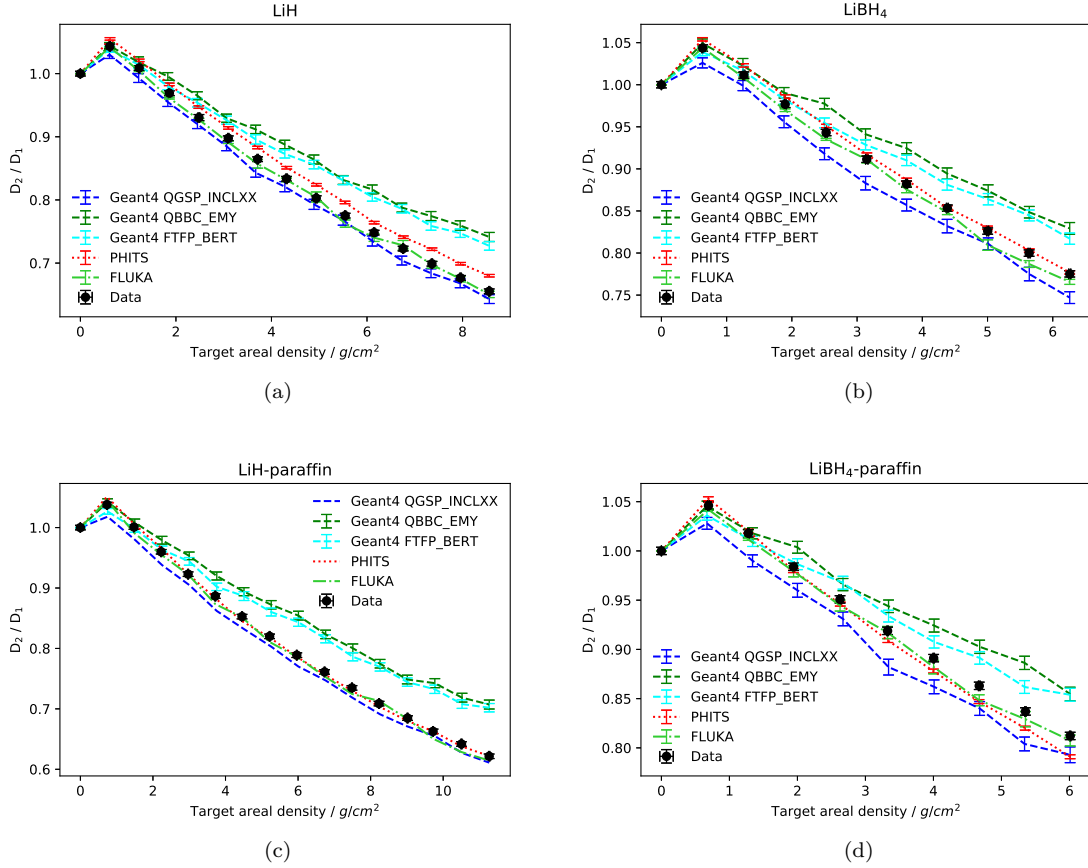


Figure 7: Comparison between experimental data and MC simulation results for the pure and composite lithium-based hydrides.

382 the first part of it, but it does not match the Bragg peak position and the tail as precisely
 383 as Geant4 QGSP_INCLXX does. PHITS fits well the experimentally obtained dose ratios
 384 with the lithium-based hydride composites, pure LiBH_4 and paraffin, but tends to under-
 385 estimate the fragmentation and dose attenuation for all the other tested materials. The Geant4
 386 physics lists results are compatible with each other for heavy targets such as aluminium and
 387 Si. These are also the cases in which the Geant4 results fit the data the best. In all the
 388 other cases, QGSP_INCLXX is the best fit to the experimental data, while QBBC_EMY
 389 and FTFP_BERT underestimate the fragmentation. The more systematic comparison of
 390 the Geant4 lists performed for the paraffin Bragg curve, showed a better fit for the lists
 391 making use of the Leige Intranuclear Cascade model for low energies (QGSP_INCLXX and
 392 FTFP_INCLXX). Overall, the MC simulation results show significant and systematic dif-
 393 ferences among codes that are used for the same radiation protection purposes. The list
 394 performing the best is QGSP_INCLXX, confirming the recommendations of choosing it for
 395 shielding applications.

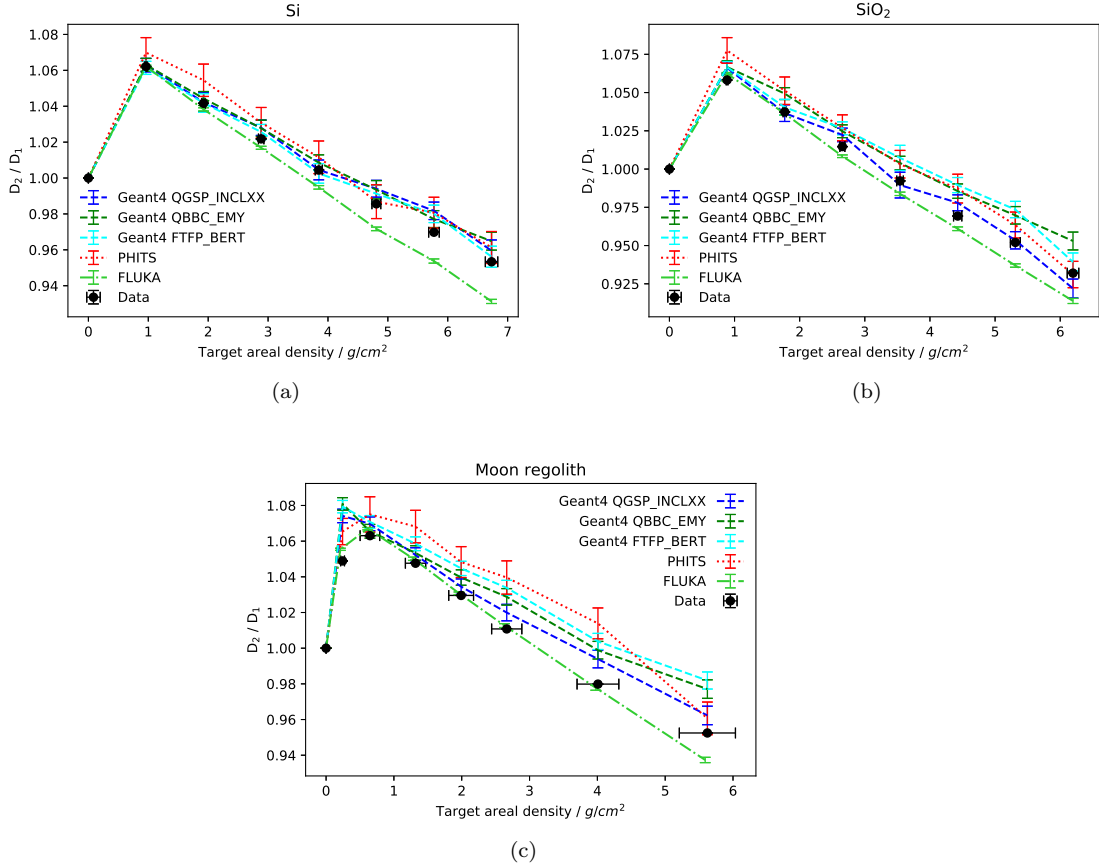


Figure 8: Comparison between experimental data and MC simulation results for Si, SiO₂ and highland Moon regolith.

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Appendix A Pure and composite hydrides: production, purity, and homogeneity

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The following procedure was selected to obtain compact pellets of pure hydrides. Inside a glove box (MBraun LABstar, working gas Ar 5.5), approximately 20 g of LiH or LiBH₄ were loaded into a mould (65 mm diameter) and a load was applied up to about 12 tons. This load was reached with a stepwise procedure; at each step the load was increased by 3 tons and left for 2 minutes. During holding time, it has been observed that the load of the press decreased because the compaction of the powder was occurring. Finally, the sample was left under the maximum pressure for about 5 minutes. Just after the extraction from the mould, the thickness of the pellet was measured with a caliber and then it was weighted. Finally, the pellet was packaged in a sealed plastic bag, (bi-layered film: PE-nylon) in order to prevent its contact with moisture.

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The composites were prepared by directly mixing the hydrides with paraffin in the same glove-box mentioned above. The two systems have been produced using silicon moulds with two different diameter size. For LiH-paraffin samples, a silicon mould (100 mm diameter) was filled with about 30 g of hydride and 30 g of paraffin. The mould and a magnetic stirrer (about 40 mm long) were closed in a PE-nylon bag. The plastic bag was removed from the glove box and the PE bag containing the mould loaded with paraffin+hydrides was inserted in an oven preheated at 85 °C (temperature at which the paraffin is liquid and the plastic

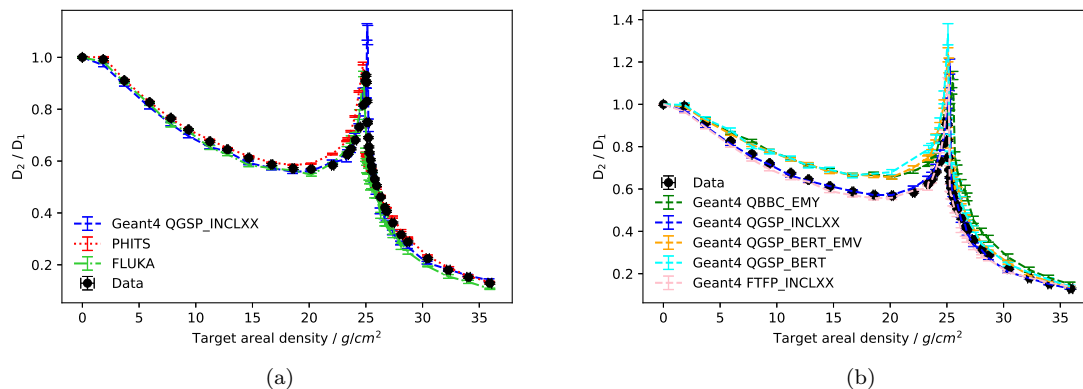


Figure 9: Comparison between experimental data and simulation results for paraffin. In panel (a) simulation results obtained with FLUKA, PHITS and Geant4 physics list QGSP_INCLXX are reported. In panel (b) results obtained with several Geant4 physics lists are shown.

415 bag still preserves its mechanical and chemical behaviour), holding it for 2 hours at fixed
 416 temperature. When the paraffin was completely molten, the magnetic stirrer was inserted
 417 in the liquid mixture. The mould was then moved from the oven to a plate pre-heated at
 418 70 °C and stirred for 3 minutes, in order to remove the gas from the liquid. After stirring,
 419 the mould was inserted again in the oven and maintained at 85 °C for 20 minutes. This
 420 operation has been repeated five times, until bubbles no longer appeared on the surface of
 421 the liquid mixture. After cooling down to room temperature, the plastic bag was inserted
 422 again inside the glove box, the cooled pellet was removed from the PE/nylon bag and
 423 the solid pellet was extracted from the mould. The exceeding composite was removed by
 424 abrading the surface with a sandpaper and the pellet was enveloped in a PE bag and closed
 425 under vacuum for storage. The same method has been applied for preparing LiBH₄-paraffin
 426 composites, mixing the two components with a 12g:8g ratio in a silicon mould of 60 mm
 427 diameter.

428 The crystalline phases present in the prepared samples have been determined by X-
 429 ray diffraction using a MalvernPanalytical Xpert PRO MPD diffractometer with Cu anode
 430 and Bragg-Brentano geometry. The results are reported in Figure A1. It is clear that the
 431 preparation of samples does not introduce any contamination or microstructural changes.

432 The homogeneity of the filler distribution in the paraffin matrix was determined by
 433 an Oxford INCA Energy-dispersive X-ray spectroscopy coupled with a Scanning Electron
 434 Microscope Zeiss EVO 50. By carrying out Energy dispersive X-ray spectroscopy mappings,
 435 and taking into account the instrumental limits of the elemental detection of such technique
 436 (detection of elements with $Z \geq 6$), the distribution of the hydride in the paraffin matrix has
 437 been indirectly verified looking at the oxygen distribution. Pure hydrides are very sensitive
 438 to the moisture, and they quickly degrade towards reaction products, forming hydroxide
 439 or hydrated species of the hydrides. As an example, in Figure A2, the oxigen mapping in
 440 LiH-paraffin sample (50% w/w) is shown. The red colour is uniformly distributed over the
 441 samples, confirming that the hydride is homogeneously dispersed in the composite. The
 442 homogeneity of the dispersion of the hydrides was also studied in the vertical section of the
 443 sample. To have an objective demonstration that there is no sedimentation of the charge
 444 due to gravity during solidification both parallel flat surfaces have been subjected to XRD.
 445 If there was no sedimentation the diffractograms should be comparable and the intensity of

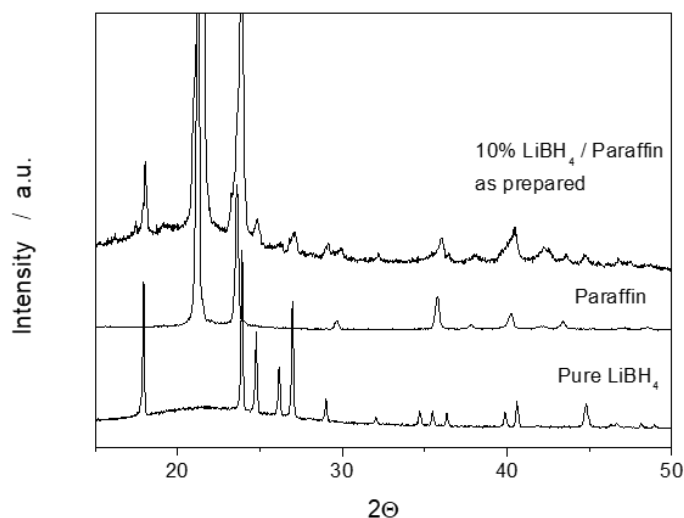


Figure A1: XRD patterns of LiBH_4 , paraffin and composite $\text{LiBH}_4(10\%)\text{-paraffin}$

446 the reflexes due to the charge should be about the same. The patterns collected on the two
 447 different faces (not shown) were comparable, confirming no sedimentation by the filler, i.e.
 448 a homogeneous dispersion of it.

449 **Appendix B Longevity of LiH and LiBH_4 -paraffin composites**

450 In Figure B1, are reported XRD patterns (B D, 1978) collected with 10% wt. of LiH (panel
 451 (a)) and 10 wt.% of LiBH_4 (panel (b)) in paraffin at different times after preparation. In
 452 panel (a), immediately after preparation (t_0), diffraction peaks of LiH and paraffin can be
 453 observed. Additionally, a rather small signal due to the LiOH phase can be seen around
 454 a diffraction angle of 33° , suggesting that the charge (LiH) undergoes a small hydration
 455 reaction during the synthesis of the composite. With increasing time of exposure to air, XRD
 456 signals related to the degradation products increase in intensity. Firstly, lithium hydroxide
 457 is generated and, subsequently, its hydrate form ($\text{LiOH}\cdot\text{H}_2\text{O}$) starts to appear. These results
 458 suggest that the kinetics of lithium hydride degradation with moisture within the composite
 459 is rather low, as the charge signals are still present even after 2 weeks of exposure of the
 460 sample to air. The main degradation products observed when the composite is exposed to
 461 air are LiOH and $\text{LiOH}\cdot\text{H}_2\text{O}$, suggesting that the paraffin presence hinders the formation
 462 of Li_2CO_3 , observed as degradation product of pristine LiH (Machin & Tompkins, 1966).
 463 In conclusion, the addition of paraffin acts as an effective protective barrier for the hydride
 464 phase, so that the degradation kinetics is greatly reduced. It is worth noting that, after 2
 465 weeks of exposure, LiH remains as the main filler phase in the composite, as shown by the
 466 presence of corresponding XRD peaks in panel (a) of Figure B1.

467 In panel (b), it can be observed that as soon as the LiBH_4 -paraffin composite was
 468 taken out of the glove box, it did not show evidence of degradation. This means that
 469 during the synthesis, the hydride has not undergone decomposition reactions. After 16
 470 hours of exposure, however, in addition to the LiBH_4 XRD peaks, which are still present,
 471 the composite begins to show new signals due to the appearance of compounds formed by
 472 hydration of the hydride. In particular, the XRD peaks of degradation $\text{LiB}(\text{OH})_4$ compound

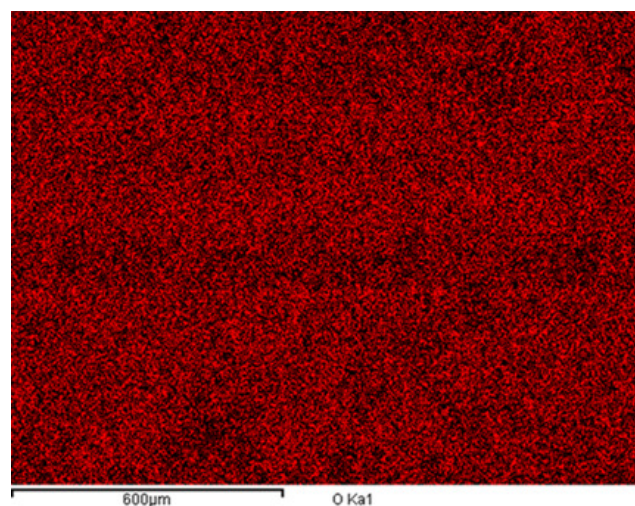
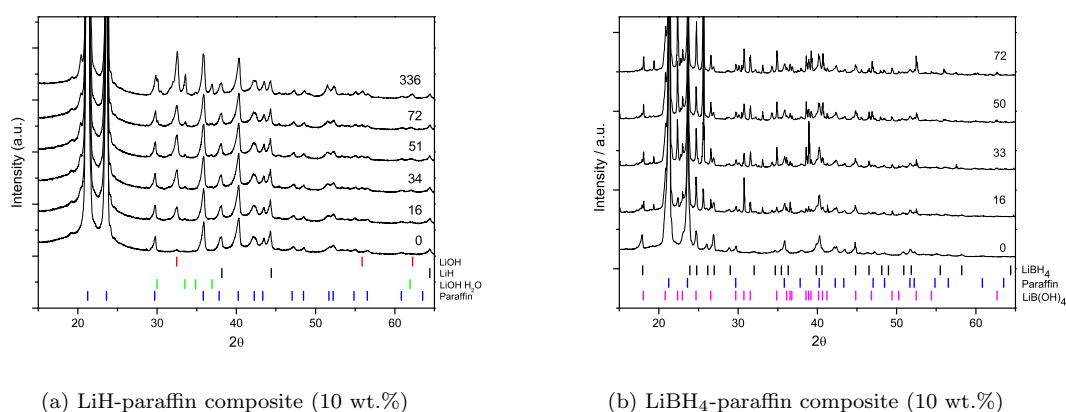


Figure A2: EDS mapping on LiH-Paraffin sample (50% w/w)



(a) LiH-paraffin composite (10 wt.%)

(b) LiBH₄-paraffin composite (10 wt.%)

Figure B1: XRD patterns of LiH and LiBH₄-paraffin composite (10 wt.%) collected at different times (in hours) after preparation. Vertical bars correspond to diffraction peak position for crystal phases reported on the right side.

473 increase in intensity over time, at the expense of the charge-related signals, which decrease
 474 until they finally disappear after around 72 hours. Also in this case the presence of paraffin
 475 greatly reduces the degradation kinetics of the hydride. In fact, such a degradation due
 476 to the reaction with the moisture to form LiB(OH)₄ is practically instantaneous for pure
 477 LiBH₄, while it needs approximately 3 days for the composite, as shown by the absence of
 478 XRD peaks related to LiBH₄ in panel (b) of Figure B1.

479 Appendix C Post irradiation test characterisation

480 As reported in Appendix A, pure hydrides and composites pellets were protected by a by-
 481 layered Nylon/PE bag. X-ray diffraction analysis was performed on composite pellets after
 482 the irradiation experimental campaigns with the aim of checking the chemical degradation

483 of the samples caused by potential packaging defects or failure, or by the irradiation itself
 484 (radiolysis). Results are reported in Figure C1, for LiH (pattern a) and LiBH₄ (pattern b),
 together with an XRD pattern of the Nylon/PE bag (pattern c). In Figure C1 (pattern a),

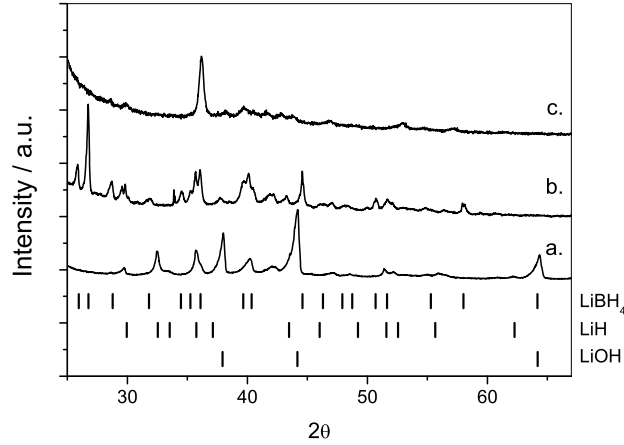


Figure C1: XRD patterns after irradiation experiments of LiH-paraffin composite (50 wt.%) (pattern a) and LiBH₄-paraffin composite (40 wt.%) (pattern b) embedded in Nylon/PE bag. The pattern of the Nylon/PE bag is also reported for comparison (pattern c).

485 together with XRD peaks related to the paraffin matrix and the Nylon/PE bag, those due
 486 to the LiH phase can be observed. In addition, a weak XRD peak due to the LiOH phase
 487 is also present at about 33° 2θ, suggesting that the occurrence of a reaction with moisture still
 488 occurred. As suggested by the longevity test of section Appendix B, Figure C1 (pattern b)
 489 confirms that LiBH₄ composite pellets has a strong stability against hydration processes,
 490 since no extra diffraction peaks appear in the pattern.
 491

492 Appendix D Dose build-up

493 A small dose build-up can be seen for the very first g cm⁻² of material. It should be
 494 noted that it is not observed for paraffin since the thinnest target irradiated is already
 495 much thicker than the maximum build-up depth. It is due to additional particles generated
 496 within the target material, which are mainly forward directed. The depth of the dose
 497 build-up effect corresponds to the range of such particles in the target material. After the
 498 build-up effect reaches equilibrium at a certain depth, the dose attenuation due to projectile
 499 fragmentation takes over and the dose ratio starts to decrease. This effect is much stronger
 500 for lighter ions (Giraud et al., 2018). Additional MC simulations have been performed with
 501 FLUKA to deepen the phenomenon. The results obtained for LiH and Al2219 are reported
 502 in Figure D1. They show that the build-up observed in the experimental data is merely due
 503 to delta electrons. The target fragments generated through nuclear interactions between
 504 the ⁵⁶Fe ions and the target nuclei do not contribute to it. This is very different than what
 505 happens with light-ion beams. The curve in magenta reports the simulation results obtained
 506 when no delta electrons are generated. The electronic production threshold was set to
 507 57 GeV, which is higher than the kinetic energy of the primary ions. On the other hand,
 508 the light green curve was generated through setting the electronic production threshold to
 509 0.1 MeV. Some additional points are reported with respect to panel (a) of Figure 7 with the

510 aim of showing the actual depth of the build-up effect, which is smaller than the first LiH
thickness used.

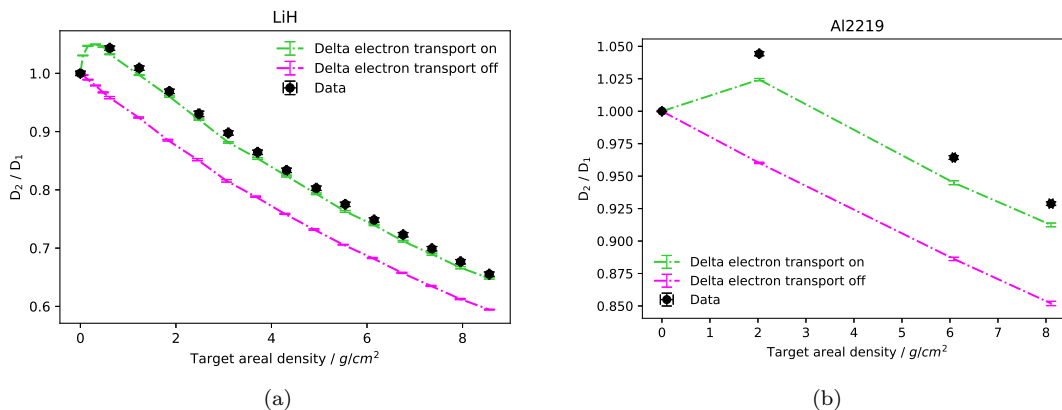


Figure D1: Comparison between the result of MC simulations performed with FLUKA for pure-LiH target (panel (a)) and Al2219 (panel (b)), including (light green) and excluding (magenta) the dose contribution of delta electrons.

511

512 Appendix E Multi-layer configurations

513 Some realistic multi-layer configurations have been used for dose attenuation measurements.
514 Similar measurements were already performed in the past ROSSINI2 campaign (Giraud et
515 al., 2018). The materials used for these tests are: one of the aluminium alloys described in
516 Section 2.1, LiBH₄ and simulants of Moon regolith and concrete. The Moon regolith used
517 for the multi-layer measurements is not the same as for the single material irradiations. It
518 is JSC-1 Moon regolith in dust form (not pressed) and enclosed inside three T75 Falcon
519 Tissue Culture Treated Flasks. The Moon concrete is produced by mixing this regolith
520 with a chemical binder. More details about the Moon regolith and concrete used for the
521 multi-layer configurations can be found in reference (Giraud et al., 2018). The results
522 are reported in Figure E1 alongside simulation results obtained with PHITS and Geant4
523 (QGSP_INCLXX physics list). ML1 stays for “Multi-Layer 1” and it is made of Moon
524 regolith + Al2219 + LiBH₄. ML2 is made of Moon concrete + Al2219 + LiBH₄, and ML3
525 of Al2195 + LiBH₄. ML1 and ML2 are supposed to reproduce realistic *in situ* material
526 configurations, one of which made out of pure Moon regolith and another of Moon concrete.
527 ML3 on the other hand, reproduces a spacecraft material configuration. The lower the ratio
528 is, the better the configuration for shielding from 1 GeV/u ⁵⁶Fe is. The agreement between
529 experimental data and simulation results is generally good, even if the simulations tend
530 to underestimate the experimental results. A disagreement of about 10% is found for ML1
531 (28.4 g cm⁻²). The total areal density of such configuration is 28.4 g cm⁻². This thickness
532 lays close to the Bragg peak region, where by definition, the absorbed dose varies greatly
533 with the target thickness. Therefore, even small thickness uncertainties can result in a large
534 error in the simulated dose.

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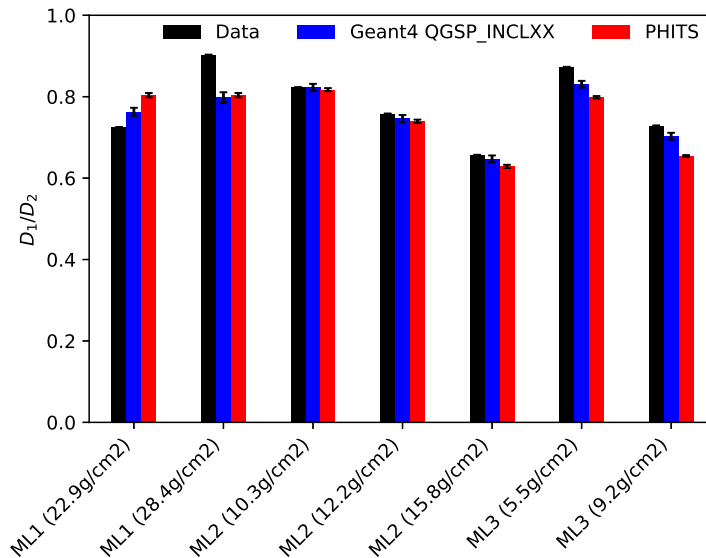


Figure E1: Experimental and simulation results for multi-layer configurations. In abscissa, the multi-layer acronyms are reported, alongside their areal densities. ML1 stays for “Multi-Layer 1” and it is made of Moon regolith + Al₂SiO₅ + LiBH₄. ML2 is made of Moon concrete + Al₂SiO₅ + LiBH₄, and ML3 of Al₂SiO₅ + LiBH₄. For the same configurations with different areal densities, only the LiBH₄ content is different.

538 measurements were performed in GSI Helmholtzzentrum für Schwerionenforschung in Darm-
 539 stadt (Germany) Cave A, in the frame of FAIR Phase-0.

540 Contributions

541 FL, UW, DB, FH, CAR, CS and LB performed the experiments. UW and FL wrote the parts
 542 of the manuscript related to the experimental work. GF, AB and MB made the samples
 543 and wrote the parts related to them. FL performed the FLUKA simulations and wrote
 544 about them. LB performed the Geant4 simulations and wrote about them. MG and CC
 545 performed the PHITS simulations and wrote about them. FL coordinated the manuscript
 546 writing. UW, MB, MG, GS and MD coordinated the work. In particular, UW coordinated
 547 the experimental work, MB the sample generation work, GS and MG the ROSSINI3 project,
 548 together with the help of MD. All authors contributed to the manuscript writing.

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