Pyroprocess Experiments at ENEA Laboratories

Giorgio DE ANGELIS^{1*}, Elio BAICCHI², Mauro CAPONE³, Carlo FEDELI³, Massimo SEPIELLI³, Giuliano TIRANTI³, Mirko DA ROS⁴, Francesca GIACOBBO⁴, Marco GIOLA⁴, Elena MACERATA⁴, Mario MARIANI⁴

¹ Retired from Italian National Agency for New Technology, Energy and Sustainable Economic Development (ENEA),

Casaccia Research Center, Via Anguillarese 301, 00123 S. Maria di Galeria, Roma, Italy

² ENEA, Brasimone Research Center, Bacino del Brasimone, 40032 Camugnano (Bologna), Italy

³ ENEA, Casaccia Research Center, Via Anguillarese 301, 00123 S. Maria di Galeria, Roma, Italy

⁴ Politecnico di Milano, Department of Energy, Nuclear Engineering Division, Piazza L. da Vinci 32, 20133 Milano, Italy

(Received April 21, 2015 / Revised August 26, 2015 / Approved October 21, 2015)

A new facility, known as Pyrel III, has been installed at ENEA laboratories for pyrochemical process studies under inactive conditions. It is a pilot plant which allows electrorefining and electroreduction experiments to be conducted on simulated fuel. The main component of the plant is a zirconia crucible. The crucible is heated by a furnace which is supported in an externally water-cooled well under the floor of a steel glove-box, where an argon atmosphere is maintained by a continual purge of about 10 L·min⁻¹. The vessel is loaded with LiCl-KCl eutectic salt (59-41 mol%) and is currently operated at 460 °C. Several improvements on Pyrel II (the previous operating plant) have been introduced into Pyrel III. They are described in detail, together with the results from the first experimental campaign which used lanthanum metal.

Moreover, studies about the treatment of chloride salt wastes from pyroprocesses have been conducted in parallel. They follow two main routes: on one hand, a matrix termed sodalite, a naturally occurring mineral containing chlorine, has been synthesized from a mix of nepheline, simulated exhausted salts and glass frit; on the other hand, a novel method proposed by Korea Atomic Energy Research Institute (KAERI) is under assessment. The final waste forms have been fully characterized with the support of the Politechnique of Milan, by means of density measurements, thermal analysis, and stereomicroscopy observations, FTIR, XRD, and RAMAN spectra, as well as leach tests under static conditions.

Keywords: Pyroprocesses, Molten Salts, Chloride wastes, Sodalite, SAP matrix, Leach tests

*Corresponding author.

Giorgio DE ANGELIS, Retired from Italian National Agency for New Technology, Energy and Sustainable Economic Development (ENEA), E-mail: deatro@alice.it, Tel: +39-063-973-4219

ORCID

Giorgio DE ANGELIS Massimo SEPIELLI http://orcid.org/0000-0001-6894-9288 http://orcid.org/0000-0001-7059-0236 Carlo FEDELI Giuliano TIRANTI http://orcid.org/0000-0002-9765-1226 http://orcid.org/0000-0003-3996-650X

This is an Open-Access article distributed under the terms of the Creative Commons Attribution Non-Commercial License [http://creativecommons.org/licenses/ by-nc/3.0] which permits unrestricted non-commercial use, distribution, and reproduction in any medium, provided the original work is properly cited

1. Introduction

Pyrochemistry is a term referred to process reactions at high temperatures. At present it appears as a promising alternative for the separation of actinides fission products from lanthanides, which is ordinarily carried out in eutectic molten chloride salts, such as LiCl-KCl. This method is expected to be very suitable for advanced nuclear engineering due to its many advantages compared with the hydrometallurgy process currently used for the extraction of lanthanides from metal fuel. They include: radiation stability of molten salts (allowing processing of spent fuels of high radioactivity); absence of neutron moderator such as water; low waste production; non-proliferation requirements.

Since many years ENEA is involved in research programmes concerning separation of actinides from lanthanides and treatment of related wastes. The facilities recently installed for carrying out experiments in this field are described herein, together with data related to some experiments.

2. Pyrel III Electrolyzer for Process Studies

Electrorefining is the key step in the pyroprocess for recovery of actinides (uranium and transuranic elements) by using electrochemical difference among elements in molten LiCl-KCl salt and liquid bismuth under high-purity argon atmosphere at 773 K [1,2]. Chopped spent fuel segments are placed in a perforated steel anode basket and lowered into the electrolyte salt. Noble metals are collected by the liquid bismuth in metal form, while the other elements are dissolved into the salt in the form of their chlorides.

Actinides are recovered at two different cathodes: a solid steel cathode, at which almost pure uranium is collected; a liquid-bismuth cathode, at which plutonium chloride is reduced and collected with uranium, other transuranic elements, and a small amount of rare-earth fission products. Chemically more active elements (alkali-metal, alkalineearth, and rare-earth elements) remain in the salt (Figure 1). A pilot plant termed Pyrel (Pyrometallurgical electrorefiner) III has been installed at ENEA laboratories for pyrochemical process studies under inactive conditions (Figure 2) [3]. This plant is an improvement with respect to previous Pyrel II [4]. The crucible of Pyrel III consists of a zirconia container (125 mm ID x 130 mm high), inserted in a steel vessel (Figure 3) externally heated by a furnace (Figure 4) supported in an externally water-cooled well under the floor of a steel glove-box (about 0.95 m wide, 0.55 m deep x 0.85 m high), where an argon atmosphere is



MA : Minor Actinides RE : Rare Earths NM : Noble Metals AM : Alkaline Metals AEM : Alkaline Earth Metals

Fig. 1. Electrorefining step of pyrometallurgical process.



Fig. 2. General view of Pyrel III plant.



Fig. 3. Crucible of the plant (left) and anode basket (right).

maintained by a continual purge of about 10 L·min⁻¹. The crucible is loaded with 1.39 kg LiCl-KCl eutectic (59-41 mol%) and is currently operated at 460°C, in order to keep the vapour pressure of the salts down to acceptable values. Five perforated disks of a special steel are welded just below the main flange, in order to refrain the heat from reaching the floor of the glove-box (Figure 3). The height of the liquid salt inside the crucible reaches 7.0 cm. The insulated top plate of the electrorefiner has six slots for the stirrer, the thermocouple tube, two reference electrodes, the fuel dissolution basket, and the cathode.

The oxygen content of the glove-box atmosphere is maintained below 20 ppm, and the moisture below 10 ppm during all the operations.

The glove-box is fitted with over- and under- pressure tips. It is also equipped with a transfer box (38 cm long, 19 cm OD) to limit the oxygen entrance during posting.

The chromel-alumel thermocouples allow to record the temperatures corresponding to the salt bath, the gas zone immediately over the salts, the gas zone over the crucible, the internal and external floor of the glove-box, within \pm 0.5°C during the experiments.

The electrodes are connected to an Amel Instruments galvanostat-potentiostat (Model 7050) with data logging via PC. The cell voltage is recorded together with the anode and cathode potentials during each experiment.

Steel basket anodes (Figure 3) and bar cathodes (Figure 5) are removed from the bath at the end of the experiments



Fig. 4. External vessel, heating system, and thermal insulation.



Fig. 5. Steel cathodes of increasing diameter.



Fig. 6. Steel crucible before *(top)* and after *(bottom)* internal coating with alumina and silica by means of plasma spraying and plasma enhanced chemical vapour deposition.

and analyzed. The cathode deposit normally contains up to 85-90wt% of salts, together with deposited metal.

The need for using a zirconia crucible comes from previous experiences with steel crucibles. In spite of the internal coating (Figure 6), made by applying an alumina



Fig. 7. Interior of a steel crucible at the end of an experimental campaign showing the salt layer *(left)*, and the bismuth layer *(right)* after the partial removal of the salt with hot water.



Fig. 8. Gas trapping system.



Fig. 9. Venturi tube.

layer by powder injection plasma spraying process (PIP-SP), followed by a layer of silica applied by plasma enhanced chemical vapour deposition (PECVD), the protection proved uneffective after a forthnight of experimental campaign (Figure 7).

A further device has been added in order to improve the performance of the plant: the gas trapping system, GTS (Figure 8), which allows to "capture" and remove the gases generated during electrorefining and/or electroreduction experiments, like chlorine, nitrogen, oxygen, carbon oxide and carbon anidride. The GTS consists of a shroud made of an alumina tube connected to a stainless steel tube joined to a Venturi tube in the upper part (Figure 9) [5].

As the gas removal system will entrain salt vapors,



Fig. 10. Trap *(left)* inside the steel shroud as seen from the top *(middle)* and from the bottom *(right)*.



Fig. 11. Detection of chlorine and carbon anidride *(top)*, and nitrogen *(bottom)* generated by pyro experiments and transferred outside the glove-box through GTS.

the latter must be removed to prevent plugging and protect downstream equipment. To this end, the stainless steel tube of the shroud is internally equipped with a trap (Figure 10) to condense salt vapors on the cooled surface in a controlled process. The surface is actively cooled by a flux of argon gas which flows inside the trap and then reaches the Venturi tube. So, on one hand, it promotes condensation of the salt vapors, and, on the other hand, it allows to remove gases from the electrorefiner by creating the Venturi effect.

The gas brought outside the glove-box can be monitored or simply revealed by chemical reactions (Figure 11). In particular, chlorine and carbon anidride pass through two Drechsel bottles and react giving rise to silver chloride and calcium carbonate. Nitrogen gas, on its own, reacts with lithium beads and forms lithium nitride.

2.1 Experimental Tests

An experimental campaign has been conducted under inactive conditions by transferring Lanthanum metal from



Fig. 12. Schematic of the direct transportation experiment of La ingots from FDB to SSC, also showing the guiding-tube for thermocouples, the two reference electrodes(Ag/AgCl), and the stirrer.

the fuel dissolution basket, FDB, to the solid steel cathode, SSC, according to Figure 12. To this purpose, FDB loaded with La ingots has been immersed into the bath containing lanthanum trichloride, then a direct current has been applied between the electrodes.

The experiments had a duration of about 23 hours and were made under galvanostatic conditions. Currents from 50 to 100 mA have been applied, with current densities in the range 5.53 - 18.44 mA/cm²/wt% La. The theoretical yield varied from 1.99 to 3.82% and the Faradic yield from 5.07 to 17.59%. Cell potentials (E_{cell}) were recorded together with anode (E_a) and cathode (E_c) potential. A typical trend is shown in Figure 13. The weight of Lanthanum in the cathode deposit was between 38 and 382 mg, corresponding to a wt% of Lanthanum of 3.81 and 10.42 respectively. Therefore, most of weight deposit is given by deposition of salt (Figure 14). Furthermore, the deposits were characterized by high porosity (see Figure 14). Cathodes of different diameter (8, 12, and 20 mm) were used. The bath of molten salt was normally stirred at 70 revolutions per minute, RPM, during the experiments.

3. Conditioning of Chloride Salt Wastes

3.1 Conditioning with Sodalite

Experimental activities on conditioning of chloride salt



Fig. 13. Typical trend of the potentials recorded during an experiment of direct transportation of Lanthanum from FDB to SSC (E_{cell}: cell potential; E_a: anode potential; E_c: cathode potential).



Fig. 14. Solid cathode with deposit at the end of an experiment (*left*); detail at the stereomicroscopy of the cathode deposit, showing high porosity (*right*).



Fig. 15. Synthesis of sodalite from nepheline.

wastes coming from pyroprocesses have been finalized to demonstrate the feasibility of sodalite synthesis through a process (Pressureless Consolidation) recently proposed by Idaho National Laboratory in USA [6].

Experiments have been made at a laboratory scale, starting from a homogeneous powder of nepheline, chloride salts and glass frit (Figure 15) [7].



Fig. 16. Sodalite blended with glass frit after heating at 925°C for 7 hours.



Fig. 17. FITR spectrum of Sodalite blended with glass frit after heating at 925°C for 7 hours.

Such a mix was put into an alumina crucible and slightly pressed with another alumina crucible of a smaller diameter, inside which a stainless steel bar of about 280 g had been inserted. The entire assembly was introduced in a furnace inside an argon-atmosphere glove-box. The furnace temperature was then raised at 10°C/min to 500°C and kept for about one hour, allowing the evaporation of any residual moisture. The temperature was then raised to 925°C for 7 hours. The final waste form (Figure 16) shows a density of 2.382 g/cm³, satisfactory if compared with that reported for pure sodalite (2.1-2.3 g/cm³).

SEM analyses and stereomicroscopy observations show the absence of both microcracks and macroporosity. As shown in Figure 17, FTIR spectrum of the final sample shows the typical peaks of sodalite at 465 cm⁻¹ [O-Al(Si)-O bending vibrations], 668, 712 and 735 cm⁻¹ [Al(Si)-O-Al(Si) symmetric stretching], and 978 cm⁻¹ [Al(Si)-O-

INL
0.59
0.39
0.045
0.047
0.226

Table 1. Normalized releases (g/m-²) of sodalite samples after 7 days leaching at $90 \pm 2^{\circ}C$

Al(Si) asymmetric stretching]. Coherently, XRD analysis highlighted the presence of sodalite as the main phase and of nepheline in traces.

Leaching tests under static conditions were performed according to the ASTM C1285-02 procedure [8]. The concentrations of the elements in the leachates, measured by ICP-MS, are reported in Table 1. The leaching behavior of sodalite blended with glass frit is comparable to that of similar glass-based confining matrices reported in the literature. To this purpose Table 1 compares the normalized release with the data by Idaho National Laboratory [9].

3.2 Conditioning with SAP Matrix

A novel method proposed by Korea Atomic Energy Research Institute (KAERI) is also applied to the treatment of chloride salt wastes [10,11]. It is based on a matrix, SAP (xSiO₂-yAl₂O₃-zP₂O₅), synthesized by a conventional solgel process (Figure 18), able to stabilize the volatile salt wastes owing to the formation of metalaluminosilicates, metalaluminophosphates and metalphosphates.



Fig. 18. Outline of the SAP process.

The addition of a glass powder as a chemical binder and a treatment at around 1150°C gives the final waste form. With this method a higher disposal efficiency and a lower waste volume can be obtained.

LiCl-KCl melt containing chlorides of alkaline, alkaline-earth metals, and lanthanides has been used to simulate the waste salt. The composite SAP has been prepared by a sol-gel process, using tetraethyl ortosilicate (TEOS), aluminum chloride (AlCl₃.6H₂O) and phosphoric acid (H₃PO₄) as sources of Si, Al, and P, respectively. All reagents were dissolved in EtOH/H₂O and the mixture, tightly sealed, was placed in an electric oven at 70°C. After a gelling/ageing for 3 days, the transparent hydrogels were dried at 110°C for 2 days and then thermally treated at 600°C for 2 hours. The so-obtained SAP has a density of 2.291 g/cm³. SEM investigations highlighted a morphology similar to that observed for SAP samples reported in the literature [9], as shown in Figure 19 [12].

A series of final products (SAPs) were reacted with metal chlorides at 950°C for 30 hours inside an Ar-atmosphere glove-box, after mixing them at a SAP/metal chloride mixing ratio of 2.

XRD (Figure 20), FTIR and Raman spectra of samples at increasing temperatures indicated, for temperatures



Fig. 19. Comparison of SEM pictures of SAP samples prepared by KAERI (a) and by ENEA (b).



Fig. 20. XRD spectra of SAP sample (*bottom*) and of the final product (*top*).



Fig. 21. Salts/SAP samples obtained after blending with glass frit and heating at 1150°C for 4 hours.

Table 2. Normalized releases (g/m²) of SAP samples after 7 days leaching at $90\pm2^\circ C$

Element	Normalized release
Li	0.307
Na	0.328
Al	0.080
Si	0.015
K	0.127
Rb	0.095
Sr	0.004
Cs	0.039
Ва	0.002
La	0.0004
Nd	0.0003

above 850°C, the presence of peaks which can be assigned to metal-aluminosilicate, metal phosphate and metal-aluminophosphate.

The reaction products were mixed with glass powder at a mixing ratio of 70 (gel product) : 30 (glass) by weight and heat treated at 1150°C for 4 hours without pressing (Figure 21).

As expected, XRD analysis indicated an amorphous matrix with a few peaks which can be attributed to aluminum phosphate, while FTIR spectra highlighted the presence of three basic bands characteristic of the silicatephosphate glasses.

Static leaching tests [8] on the Salts/SAP samples revealed the interesting behavior of such matrix: after 7 days leaching the concentrations of salt elements in the leachates, reported in Table 2, were lower than those obtained with sodalite.

4. Conclusions

A new facility has successfully been installed at ENEA laboratories for pyrochemical process studies under inactive conditions. Pyrel III pilot plant allows to perform experiments about electrorefining and electroreduction of simulated fuel. Moreover, an argon-atmosphere glove-box is used for conditioning of chloride salt wastes with sodalite or SAP matrix.

The data obtained from the experimental activities will give a useful contribution to the comprehension of the phenomena involved and to the improvement of the project design of industrial plants.

Acknowledgements

Financial support from the Nuclear Fission Safety Program of the European Union (project ACSEPT, contract FP7-CP-2007-211 267; project SACSESS, contract FP7-CP-2012-323282), as well as from Italian Ministry for Economic Development (Accordo di Programma: Piano Annuale di Realizzazione 2010, 2011, 2012, and 2013) are gratefully acknowledged.

REFERENCES

- T. Nishimura, T. Koyama, M. Iizuka, and H. Tanaka, "Development of an environmentally benign reprocessing technology – Pyrometallurgical reprocessing technology", Progress in Nuclear Energy, 32(3/4), 381-387 (1998).
- [2] Jae-Hyung Yoo, Jung-Kuk Kim, Han-Soo Lee, In-Seok Seo, Eun-ka Kim, "Patent Analysis for Pyroprocessing of Spent Nuclear Fuel", Journal of Nuclear Fuel Cycle and Waste Technology, 9(4), 247-258 (2011).
- [3] G. De Angelis, C. Fedeli, G. Tiranti, and E. Baicchi, "Facilities for Pyrochemical Process Studies at ENEA", GLOBAL 2013, ID 7277, September 29 – October 3, 2013, Salt Lake City.
- [4] G. De Angelis and E. Baicchi, "A New Electrolizer for Pyrochemical process Studies", Proceedings of GLOB-AL 2005, Paper No. 048, Oct 9-13, 2005, Tsukuba.
- [5] C. Baker, T. M. Pfeiffer, S. D. Herrmann., S. Park, and

S.-H. Kim, "Development of a kg-Scale Oxide Reduction Module for Spent Light Water Reactor Fuel", 2012 International Pyroprocessing Research Conference, August 26-29, 2012, Fontana, Wisconsin.

- [6] T. J. Battisti, M. Goff., K. J. Bateman, M. F. Simpson, and J.P. Lind, "Ceramic waste form production at ANL-West", American Nuclear Society Fifth Topical Meeting, September 10-20 2002, Charleston, SC.
- [7] G. De Angelis, C. Fedeli, A. Manniello, M. Da Ros, F. Giacobbo, E. Macerata, and M. Mariani, "Conditioning of chloride salt wastes from pyroprocesses through the pressureless consolidation process", Procedia Chemistry,7, 746-753 (2012).
- [8] ASTM C1285-02 (Reapproved 2008). Standard Test Method for Static Leaching of Monolithic Waste Forms for Disposal of Radioactive Waste.
- [9] S. Priebe and K. Bateman, "The Ceramic Waste Form Process at Idaho National Laboratory", Nuclear Technology, 162, 199-207 (2008).
- [10] H.-S. Park, I.-T. Kim, Y.-Z. Cho, H.-C. Eun, and H. -S. Lee, "Stabilization/Solidification of Radioactive Salt Waste by Using xSiO₂-yAl₂O₃-zP₂O₅ (SAP) Material at Molten Salt State", Environ. Sci. Technol., 42, 9357-9362 (2008).
- [11] I. H. Cho, H.-S. Park, S. N. Ahn, I.-T. Kim, and Y.-Z. Cho, "Stabilization/Solidification of Radioactive Li-Cl-KCl Waste Salt by Using SiO₂-Al₂O₃-P₂O₅ (SAP) inorganic composite: Part 1. Dechlorination Behaviour of LiCl-KCl and Characteristics of Consolidation", Journal of Nuclear Fuel Cycle and Waste Technology, 10(1), 45-53 (2012).
- [12] G. De Angelis, M. Capone, C. Fedeli, G.A. Marzo, M. Mariani, E. Macerata, and M. Giola, "A new matrix for conditioning chloride salt wastes from the electrorefining of spent nuclear fuel", Advances in Science and Technology, 94, 97-102 (2014).