



**BeWS-15
2022**
15th International Workshop
On Beryllium Technology
Karlsruhe, Germany
14-15 September 2022

SPECIAL
JOINT
EVENT



**BeYOND-IX
2022**
9th Industrial Forum on
Beryllium Opportunities &
New Developments
Karlsruhe, Germany
16 September 2022

BeWS-15 is organized under the auspices of the IEA Technology Collaboration Program on Fusion Materials (FM TCP)

Current design of the EU DEMO Helium Cooled Pebble Bed breeding blanket

Dr. Guangming Zhou (KIT)
Lead Engineer of HCPB Breeding Blanket

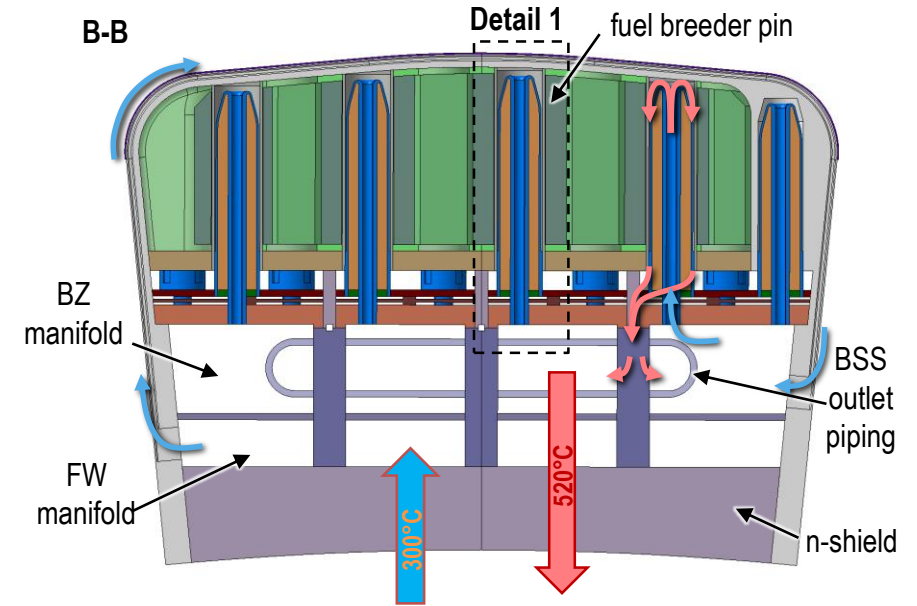
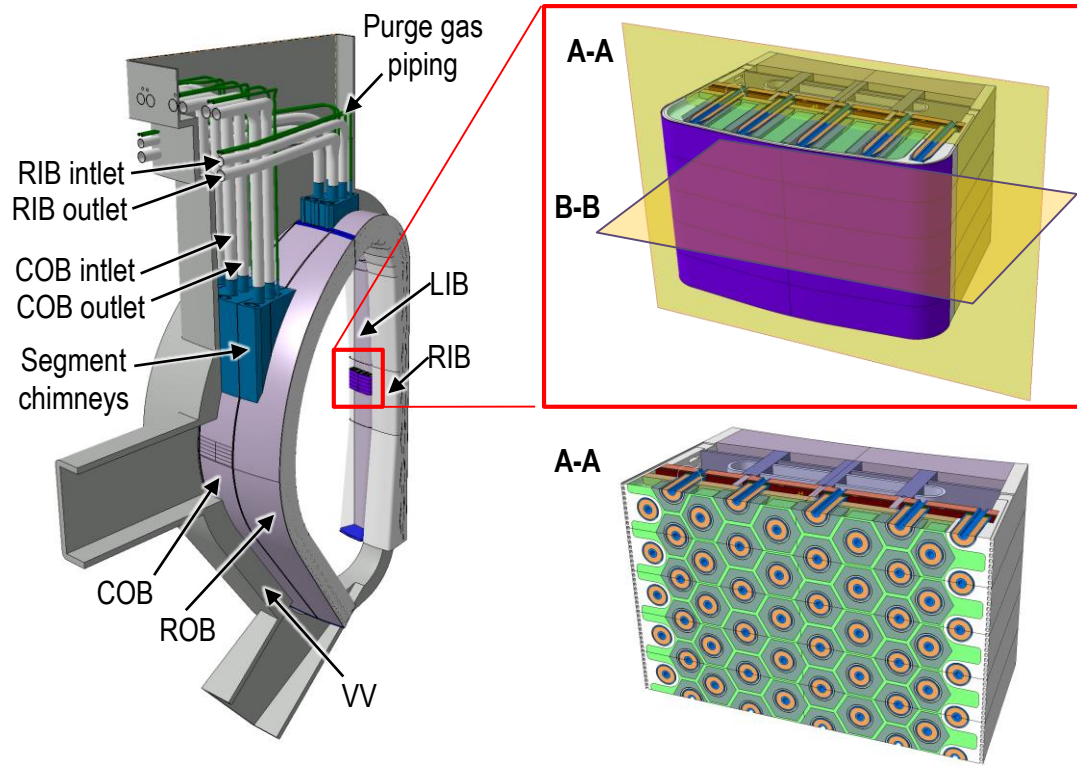


This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.

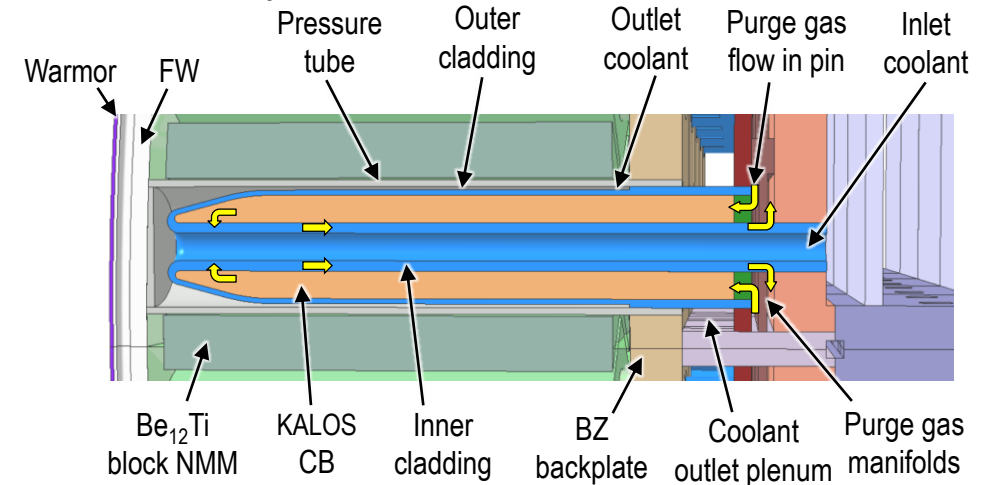


- Status at the end of Pre-Concept Design Phase (2014-2020)
- Identified risks
- Design activities to address the risks
- Outlook

Status at the end of Pre-Concept Design Phase (2014-2020)



Detail 1: Fuel-breeder pin

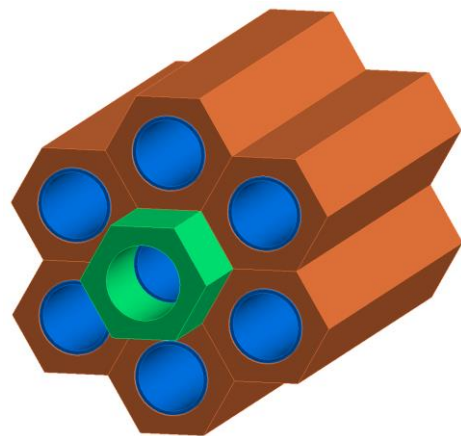


- Coolant: He @80 bar, 300-520°C
- Fuel-breeder pins containing advanced ceramic breeder (ACB) pebble bed
- Pins inserted into blocks of Be₁₂Ti neutron multiplier
- Structural steel: Eurofer97
- Purge gas: He + 0.1vol% H₂ @2 bar
- Easier manufacturing, easier filling of pebbles
- NA, TH & TM; TBR = 1.20; Ppump per blower < 6 MW; satisfying shielding

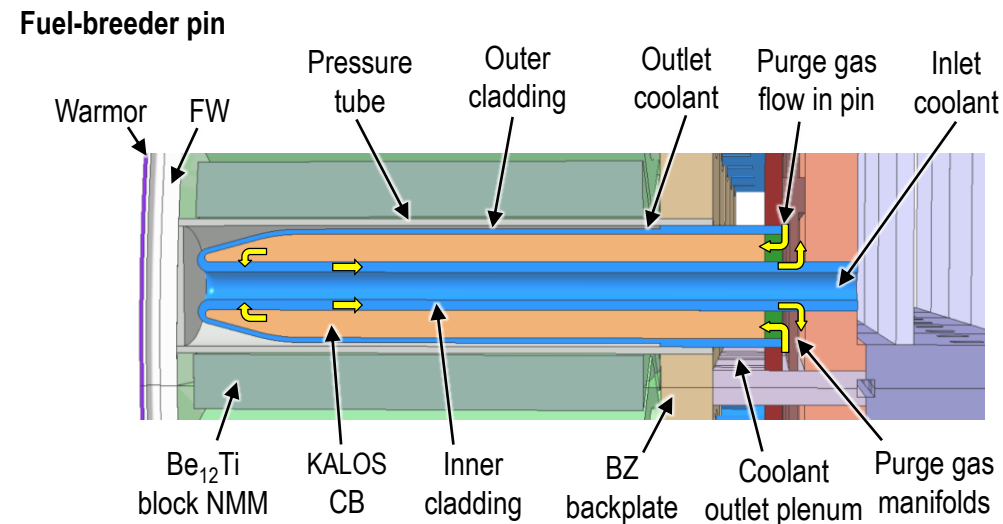
Identified risks related to HCPB BB



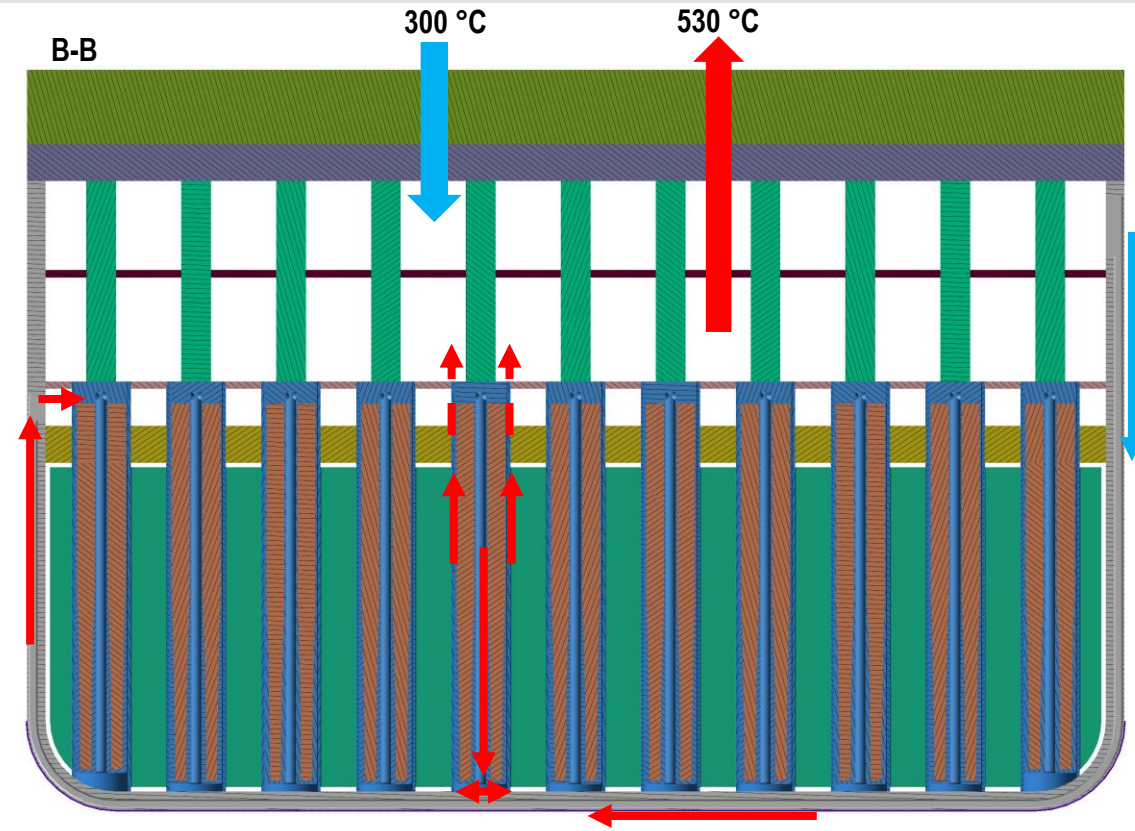
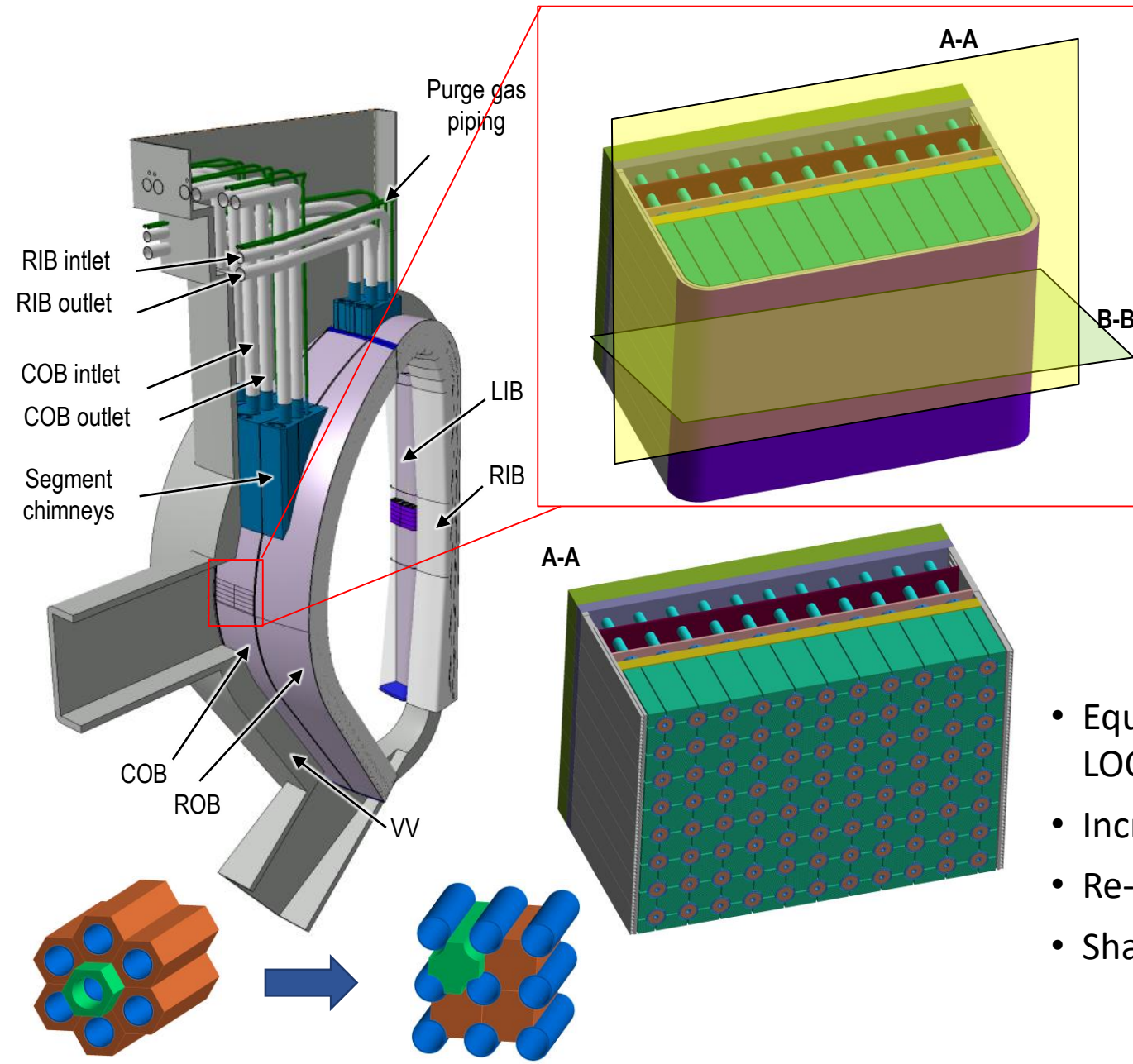
1. Low reliability of BB system under DEMO conditions [due to welds failure]
2. Loss of structural integrity of beryllide blocks → See Talk of S. Udartsev
3. High pressure drops in coolant loop contributing to total high pumping power
4. Large tritium permeation rates at the interface of breeder-coolant loop
5. Low BB shielding capability
6. Degradation of Eurofer at contact with pebbles in purge gas environment
7. Reduction of structural integrity of blanket during shutdown due to Eurofer irradiation embrittlement



Be₁₂Ti block NMM



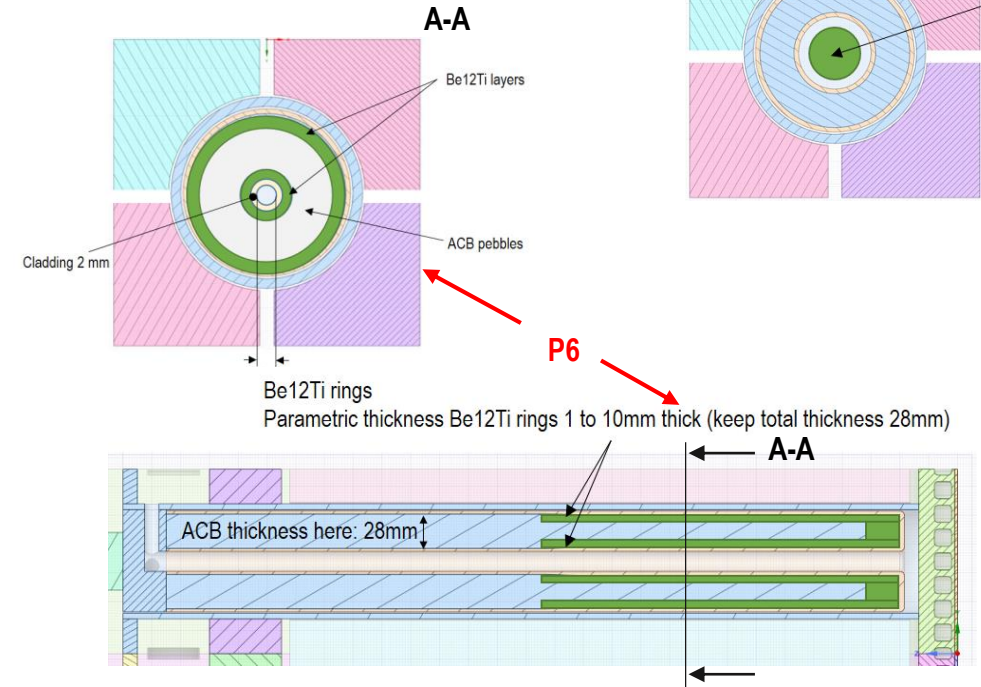
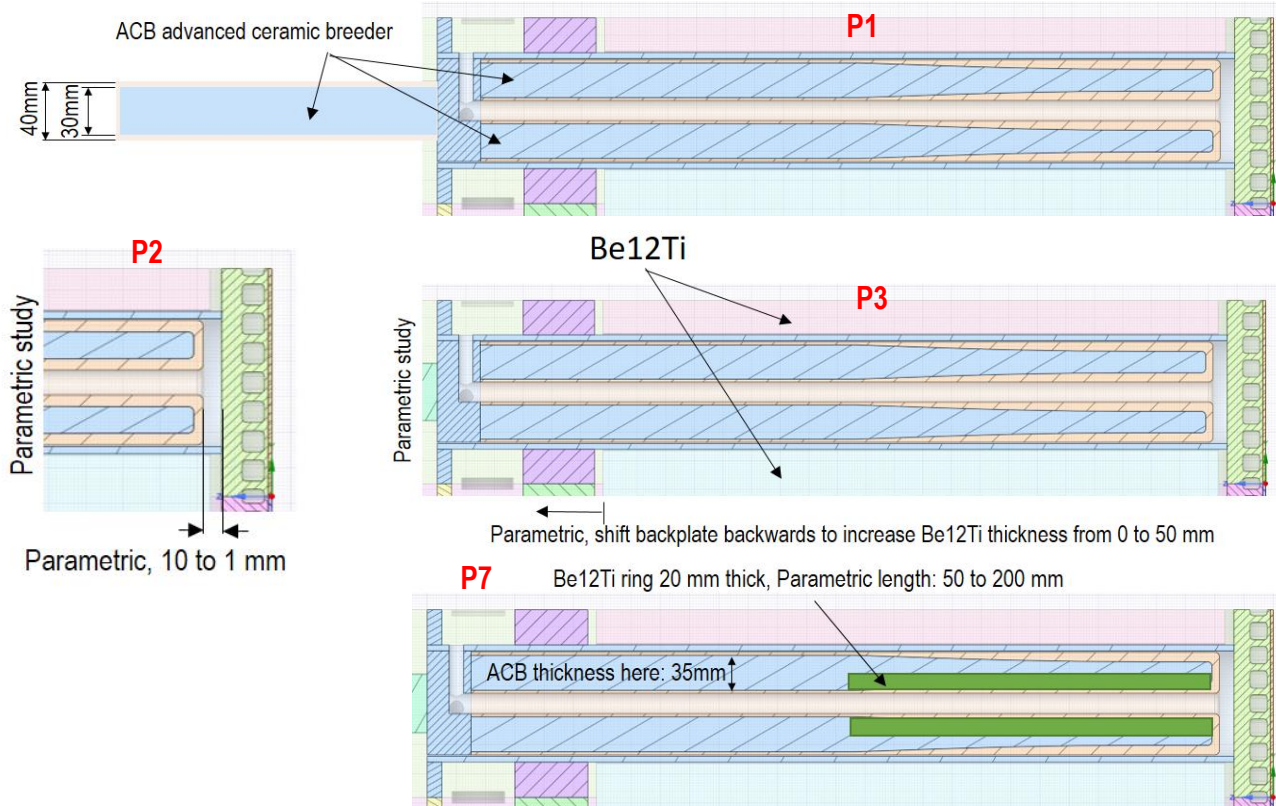
Proposed design changes for improvements



- Equalize purge gas and coolant pressure to eliminate in-box LOCA welds to improve reliability
- Increase ΔT (300°C-530°C) to further reduce pressure drop
- Re-arrange flow scheme to cool key structure with fresh coolant
- Shape of Be12Ti block to square

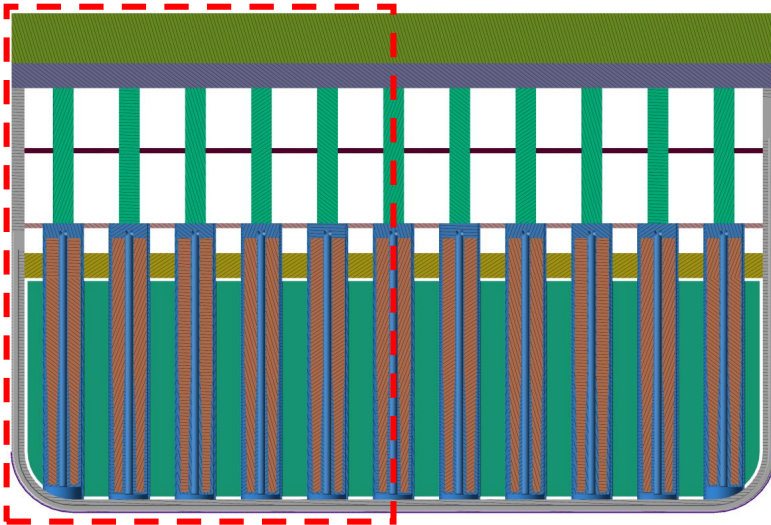
Tritium breeding ratio (TBR) optimization

- P1. Study influence of ACB in back side of the pin (whole length of back side of pin)
- P2. Study reduction of the front pin cladding distance to FW
- P3. Study influence of Be12Ti radial length
- P4. Study influence of Be12Ti block gaps
- P5. Introduction of a Be12Ti rod in the inner tube
- P6. Introduce Be12Ti in ACB pebbles (on both sides)
- P7. Like P6, Introduce Be12Ti in ACB pebbles (only on inner side)
- P8. Combined the positive effects

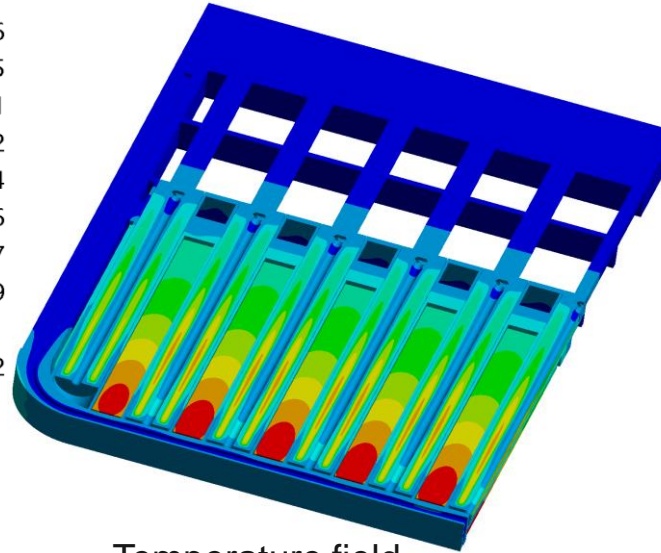
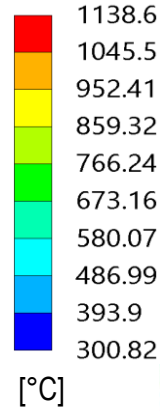


• Combined all positive effects, TBR = 1.17 > 1.15

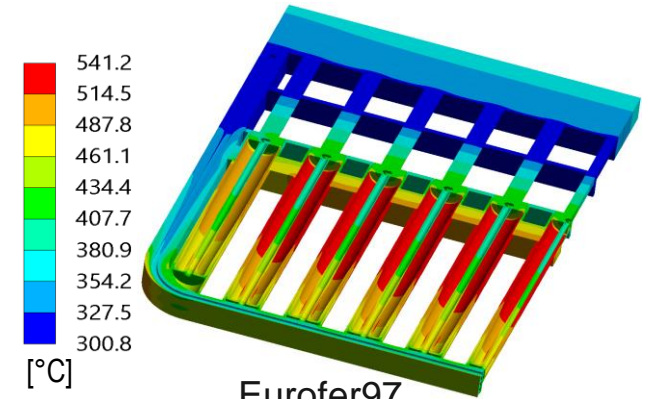
Thermal and structural analysis



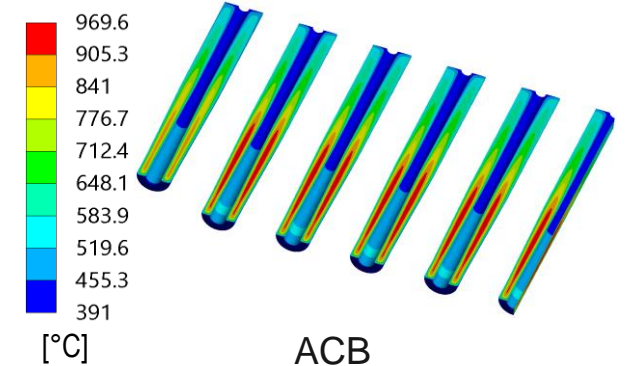
FEM model



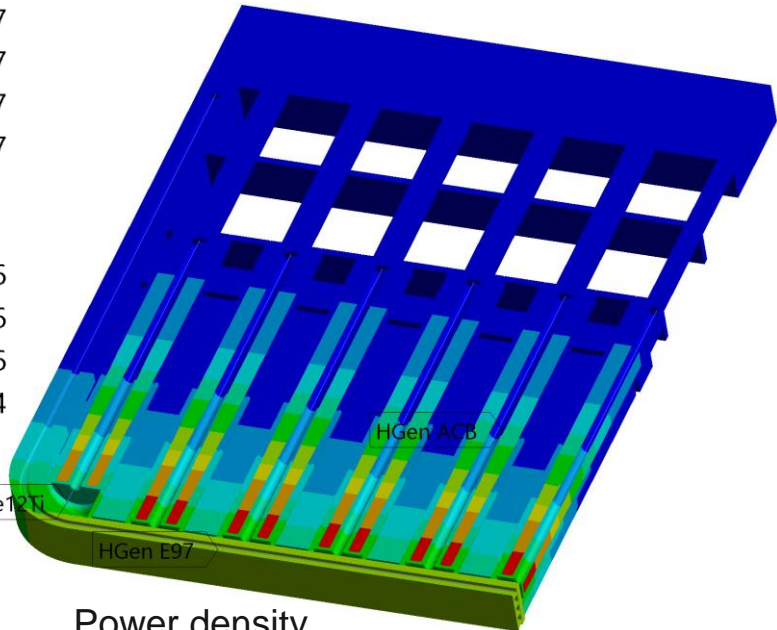
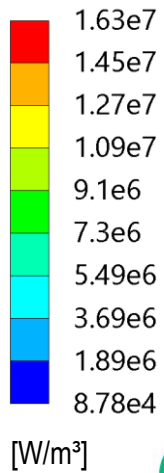
Temperature field



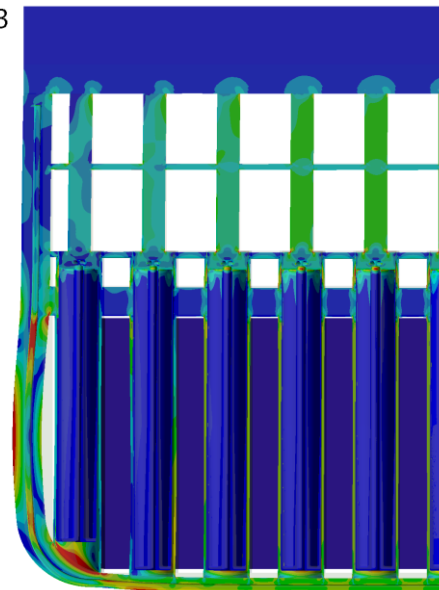
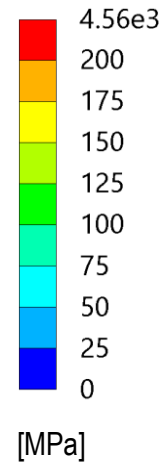
Eurofer97



ACB



Power density



Stress field (P)

$T_{in} / T_{out} = 300 / 530 \text{ °C}$

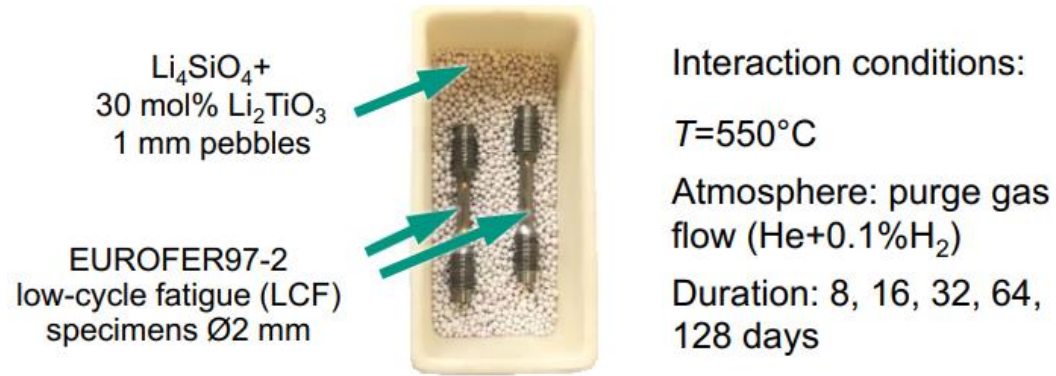
Temp. within design limits

Stresses of steel are within allowables of code

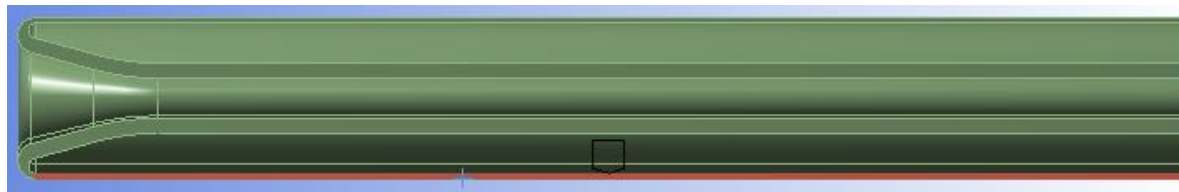
Assessment of pebble-Eurofer interaction



- Acc. to [1], the fatigue lifetime reduced due to interaction between pebbles and Eurofer97

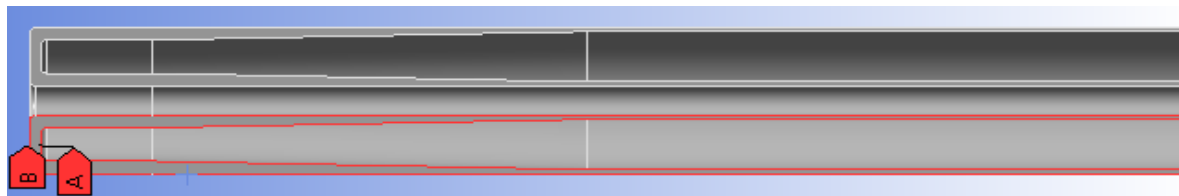


- Creep-Fatigue-Assessment tool [2] used to assess different design options (2 bar vs 80 bar purge gas)



2 bar purge gas

- Along the indicated paths, most regions failed to withstand the required 7787 cycles



80 bar purge gas

- Along the indicated paths, most regions succeeded to withstand the required 7787 cycles

- New design able to improve lifetime.

[1] J. Aktaa et al., Fusion Eng. Des. 157 (2020) 111732.

[2] M. Mahler, J. Aktaa. Nucl. Mat. Energ. 15 (2018) 85-91.

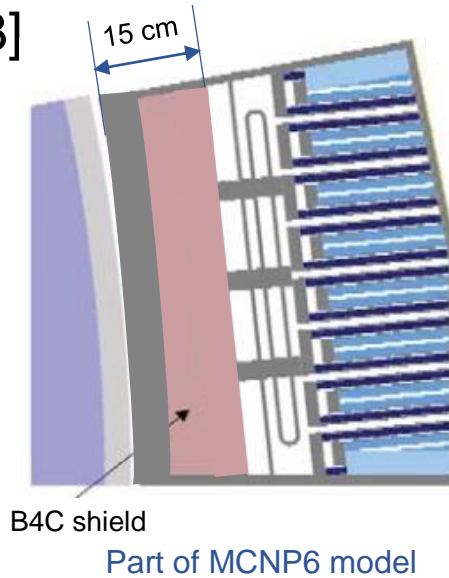
Shielding design (1/2)



- Parametric neutronics analysis [3]

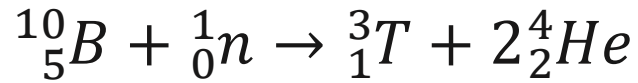
3D MCNP model by SuperMC

- *Baseline*: 15 cm Eurofer
- *v1*: 1 cm B₄C, 14 cm Eurofer
- *v2*: 2 cm B₄C, 13 cm Eurofer
- ...
- *v5*: 5 cm B₄C, 10 cm Eurofer
- ...
- *v10*: 10 cm B₄C, 5 cm Eurofer



Cases	Nuclear heating at 1st cm of TFC (limit: 5e-5)	Neutron flux at 1st cm of TFC (limit: 1e9)	dpa/fpy at 1st cm of TFC (limit: 1.5e-5)	dpa/fpy at 1st cm of VV (limit: 4.5e-1)	He production at 1st cm of VV (limit: 0.15)
	W/cm ³	n/cm ² /s			appm/fpy
Baseline	8.69e-5	2.21e9	1.81e-5	1.53e-1	0.56
v1	7.36e-5	2.07e9	1.69e-5	1.28e-1	0.42
v2	6.83e-5	2.29e9	1.24e-5	9.27e-2	0.35
v3	5.37e-5	1.82e9	1.42e-5	9.43e-2	0.29
v4	5.16e-5	1.74e9	1.50e-5	8.58e-2	0.27
v5	4.72e-5	1.66e9	1.40e-5	7.70e-2	0.24
v6	4.16e-5	1.57e9	1.41e-5	6.94e-2	0.22
v7	3.69e-5	1.47e9	1.41e-5	6.29e-2	0.18
v8	3.32e-5	1.43e9	1.24e-5	5.76e-2	0.17
v9	3.30e-5	1.41e9	1.27e-5	5.52e-2	0.16
v10	3.24e-5	1.40e9	1.24e-5	5.27e-2	0.15
v5_inverted	4.06e-5	1.65e9	1.28e-5	7.46e-2	0.19
v10_inverted	2.81e-5	1.33e9	1.16e-5	5.07e-2	0.14

- Tritium and helium production in B4C



Negligible, 120 kg T/fpy in EU-DEMO \rightarrow $1e-28$ [Pa·m³/(s·m²)] \ll Outgassing limit $1e-11$

Maximum T and He production is in v10, 1.84 mole (5.52 g) T per FPY, 500 mole (2 kg) He per FPY in EU-DEMO

Due to fragmentation of B4C, container of B4C is needed.

Nuclear heating in B4C and Eurofer used as input for structural design of the shield.

Shielding design (2/2)



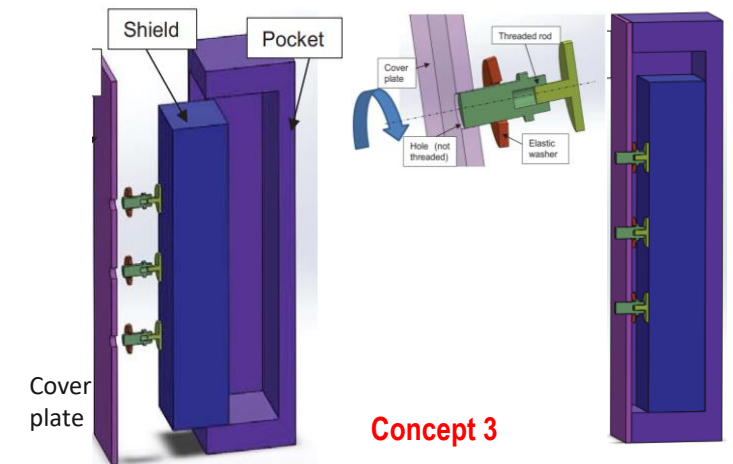
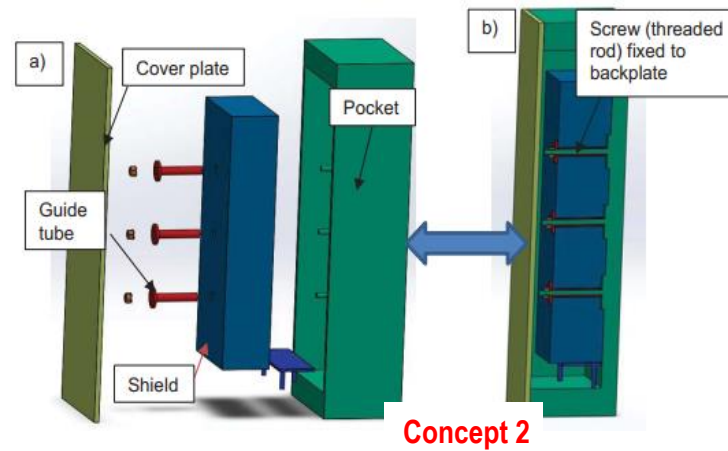
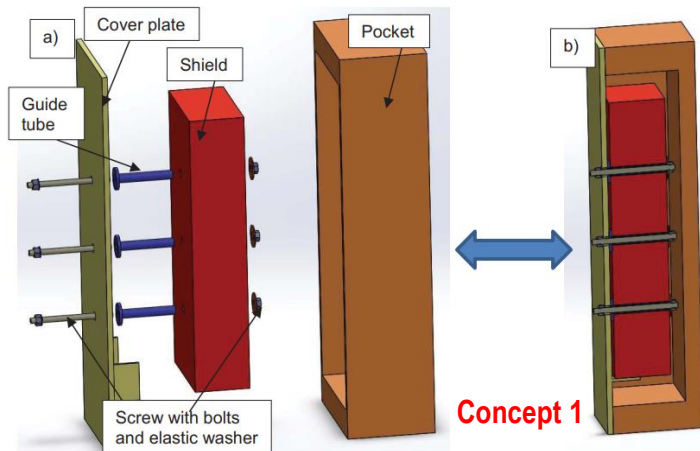
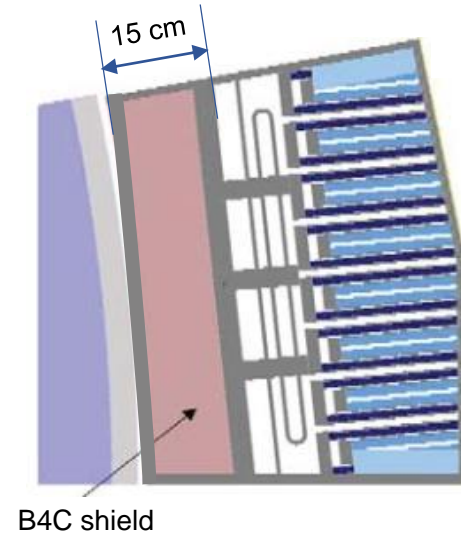
- Structural design

To confine the fragmentation, B4C is designed to be contained.

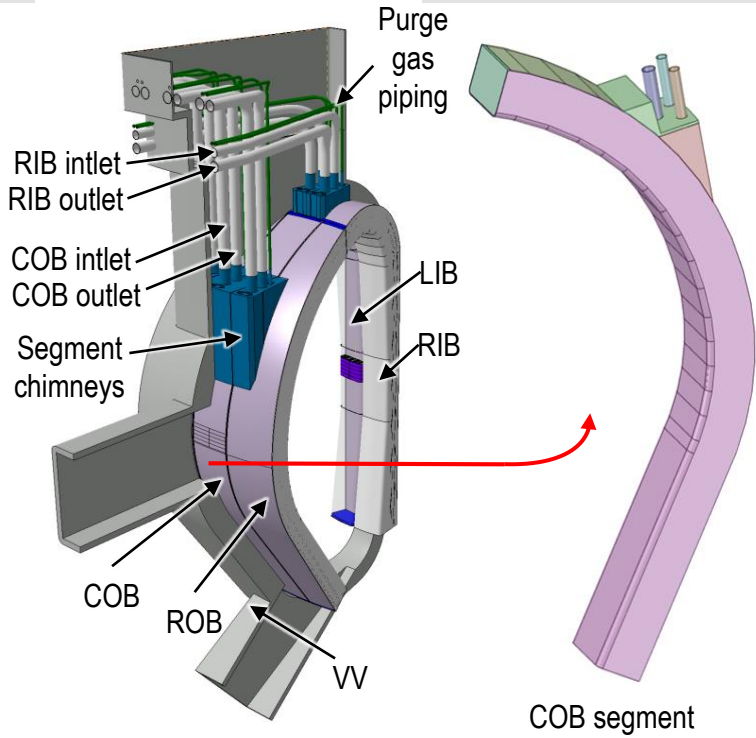
Concept 1: Radiation, shield fixed to cover plate

Concept 2: Contact, shield fixed to BSS backplate

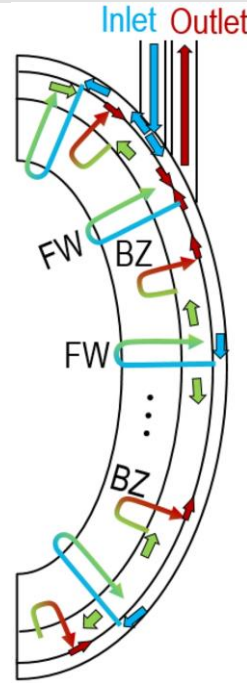
Concept 3: Contact, shield fixed to BSS backplate with external clamping



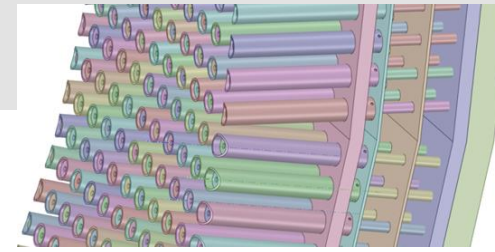
Global segment hydraulics



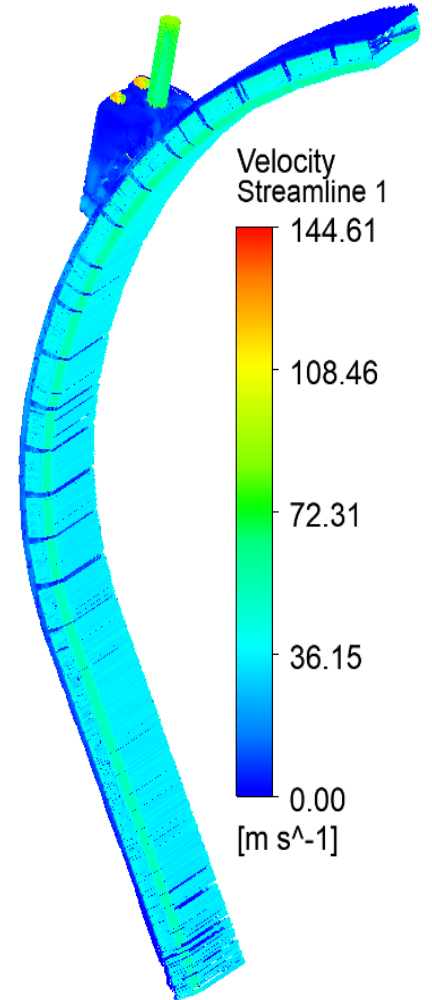
COB segment



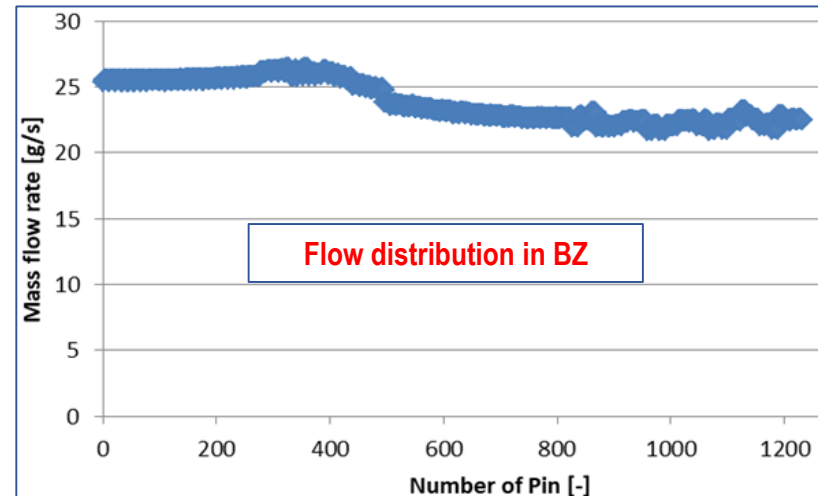
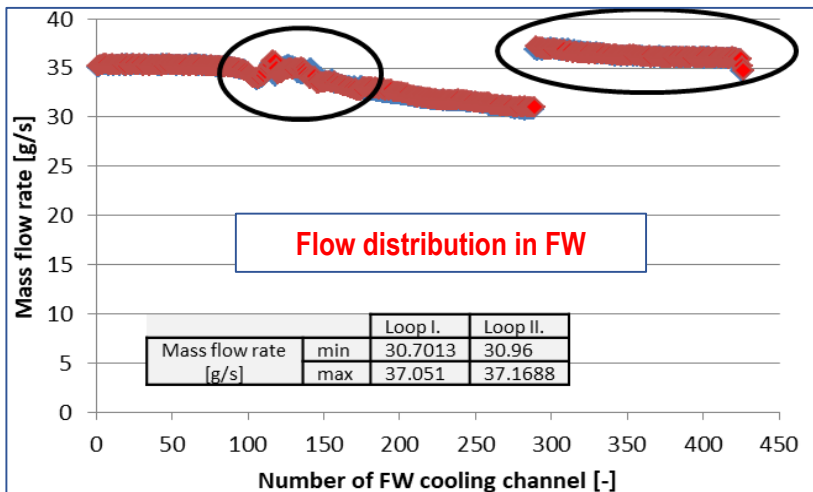
COB segment w/o FW



Porous media approach [4]
 Totally in COB
 1232 pins
 860 FW channels
 Pressure drop: 0.96 bar
 Flow distribution relative OK



[4] G. Zhou et al. Nucl. Fusion 60 (2020) 096008.



Tritium Extraction and Removal (TER) system



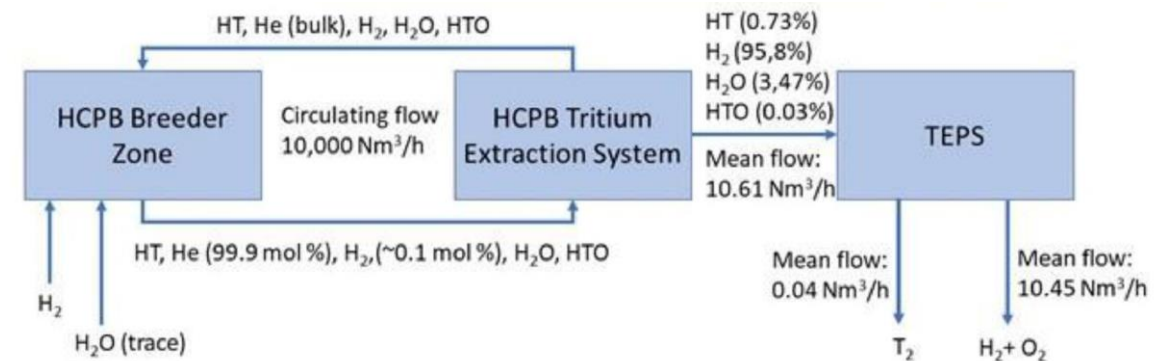
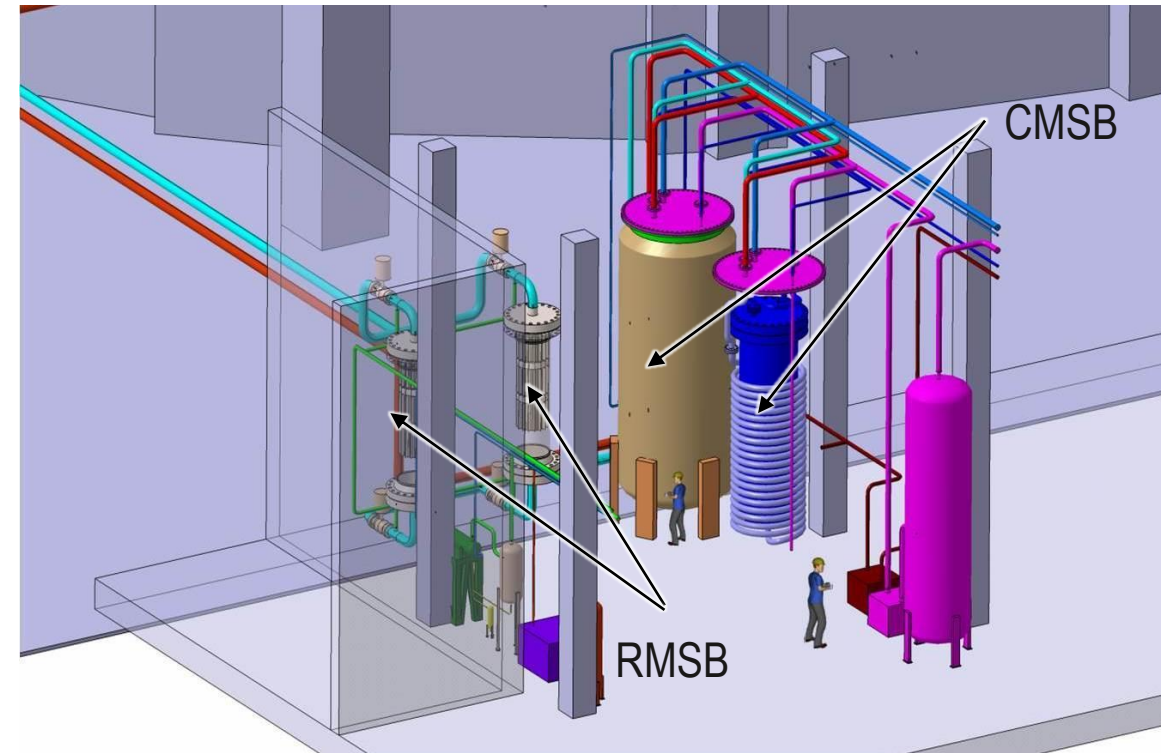
• Reference design

- Two stages in series
First the adsorption of Q2O on the Reactive Molecular Sieve Bed (RMSB), thereafter the adsorption of Q2 on the Cryogenic Molecular Sieve Bed (CMSB) at 77 K.
- Tritium recovered via isotope exchange on RMSB and by heating-up of the CMSB.
- Extrapolated to DEMO scale is realized with industry.

Q = H, D, T

• Outlook

- 80 bar purge gas, introduced to improve reliability of BB, results show that TER operating at 80 bar not a issue.
- CMSB requires large amount of liquid N2, getter bed is explored as alternative.
- Wetted purge gas to have a higher isotopic exchange rate compared to H2 and oxidized Q2, reducing permeation.



Tritium permeation analysis

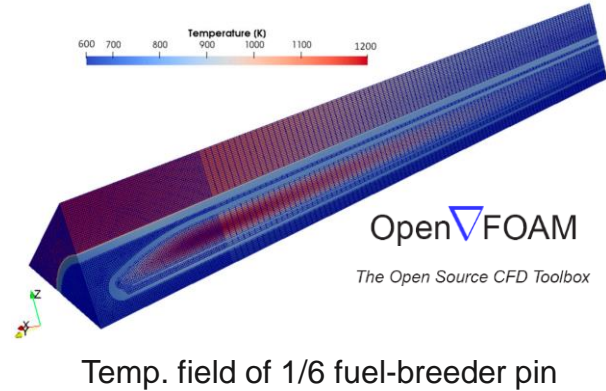


- 3D component level solver [6]

- Developed based on the OpenFOAM and benchmarked with TMAP 7

- T release model

Grain surface release model based on irradiation T release experiment [7]

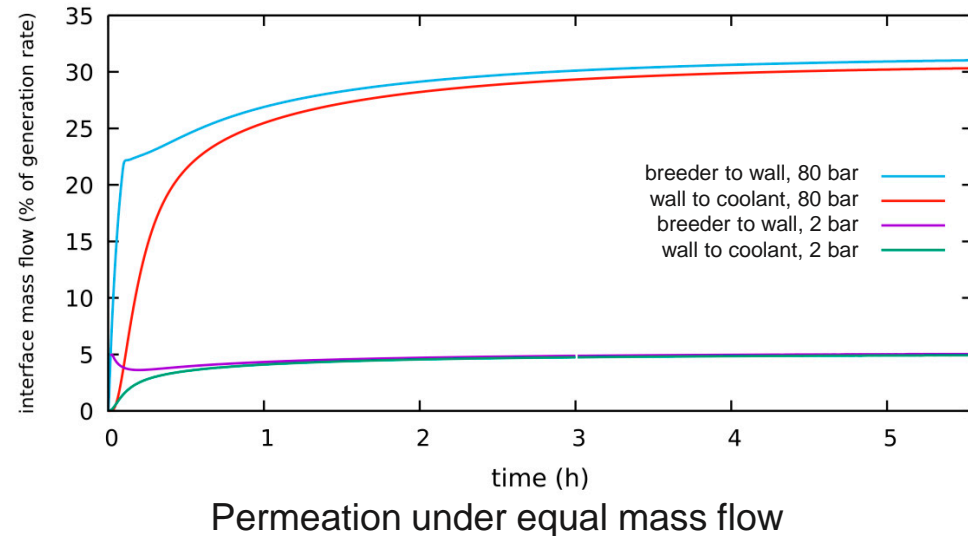
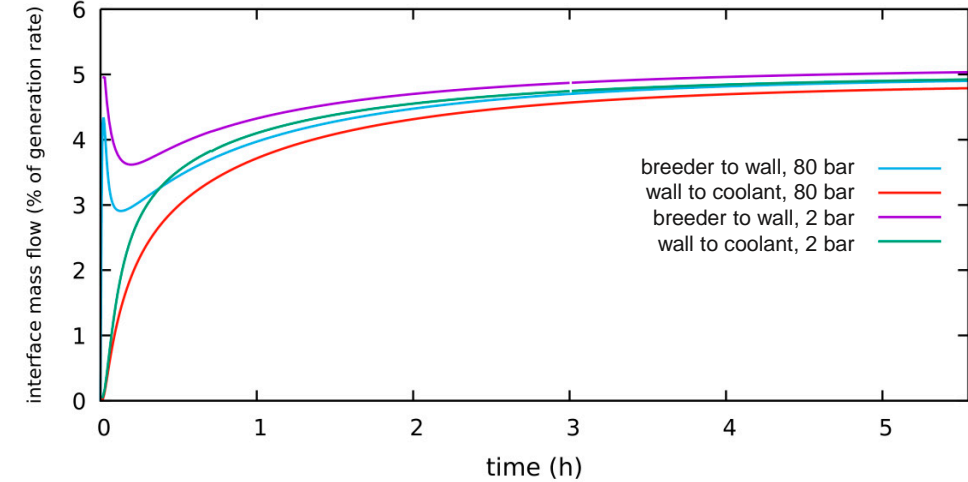


- T permeation analysis

- T permeation analysis under 2 bar pressure purge gas vs 80 bar pressure purge gas, with same H₂ partial pressure

- Wetted purge gas vs dry purge gas

Purge gas	Permeation to coolant	Wall T inventory
200Pa H ₂ , no H ₂ O	0.077% of T generation 290 mg/d	65 ng
200Pa H ₂ + 200Pa H ₂ O	0.022% of T generation 83 mg/d	19.2 ng



[6] V. Pasler et al., Applied Sciences 11 (2021) 3481.

[7] T. Kinjyo et al. Fusion Engineering and Design 81 (2006) 573-577.



- At end of 2022, the milestone of preliminary conceptual design of the HCPB blanket shall be reached.
- At second half of 2024, the milestone of reference conceptual design for the HCPB blanket shall be reached, together with R&D programme.
- At the end of 2024, the driver blanket for EU-DEMO will be selected from the HCPB and WCLL concepts.
- From 2025 to 2027, the selected blanket will be further consolidated and qualified via design and R&D activities.

Contributors & Acknowledgements



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Thank you for your attention!