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34 Abstract

35	Traces of Pu have been detected in material released from the Fukushima Daiichi Nuclear
36	Power Plant (FDNPP) in March of 2011; however, to date the physical and chemical form of the Pu
37	have remained unknown. Here we report the discovery of particulate Pu associated with cesium-rich
38	microparticles (CsMPs) that formed in and were released from the reactors during the FDNPP
39	meltdowns. The Cs-pollucite-based CsMP contained discrete $U(IV)O_2$ nanoparticles, $< \sim 10$ nm, one
40	of which is enriched in Pu adjacent to fragments of Zr-cladding. The isotope ratios, $^{235}U/^{238}U$,
41	240 Pu/ 239 Pu, and 242 Pu/ 239 Pu, of the CsMPs were determined to be ~0.0193, ~0.347, and ~0.065,
42	respectively, which are consistent with the calculated isotopic ratios of irradiated-fuel fragments.
43	Thus, considering the regional distribution of CsMPs, the long-distance dispersion of Pu from
44	FNDPP is attributed to the transport by CsMPs that have incorporated nanoscale fuel fragments prior
45	to their dispersion up to 230 km away from the Fukushima Daiichi reactor site.
46	
47	Keywords

48 Uranium, Plutonium, Fuel fragment, Fukushima Daiichi, Cs-rich microparticle

50 1. Introduction

51	The nuclear disaster that occurred at the Fukushima Daiichi Nuclear Power Plant (FDNPP)
52	in March 2011 released $\sim 10^{19}$ Bq of radioactivity into the environment; this included fission
53	products (Xe, I, and Cs) (Buesseler et al., 2017; Steinhauser et al., 2014) and trace quantities of
54	actinides, such as U and Pu (Sakaguchi et al., 2014; Yamamoto et al., 2014; Zheng et al., 2013,
55	2012). In the reactor units $\#1 - 3$, a large fraction of nuclear fuel, which is mainly composed of UO ₂ ,
56	melted and caused the failure of each unit's reactor pressure vessel (RPV) (Nagase et al., 2016). The
57	melting fuel mixed with various structural materials, such as concrete and steel, and cooled to
58	remain as "debris" inside the reactors. The amount of U and Pu released from the FDNPP have been
59	estimated to be ~ 6.1×10^{-5} and 2.0×10^{-5} % of the core inventory, respectively (Sakaguchi et al.,
60	2014; Zheng et al., 2013). The Pu radioactivity in surface soils in the vicinity of the FDNPP was
61	determined to range from 0.003 to 3.924 Bq/kg (Yamamoto et al., 2014). The comparatively small
62	amount of released Pu was previously ascribed to the low volatility of U and Pu in nuclear fuel
63	(Grambow and Poinssot, 2012). Despite the limited amount of Pu in the environment surrounding
64	the FDNPP, the occurrence of Pu isotopes (half-lives of ²³⁸ Pu, ²³⁹ Pu, ²⁴⁰ Pu, and ²⁴¹ Pu are 87.74 years,
65	2.411×10^4 years, 6563 years, and 14.35 years, respectively) have been a concern because of the
66	high effective dose conversion coefficient for these isotopes in the event of internal exposure (1.5 \times
67	10^{-5} Sv/Bq for ²³⁸ Pu, ²³⁹ Pu, and ²⁴⁰ Pu, 1.6×10^{-7} Sv/Bq for ²⁴¹ Pu, and 1.4×10^{-5} Sv/Bq for ²⁴² Pu)

68 (ICRP, 2012). Also, 32 of the 548 fuel assemblies in reactor unit 3 were a metal oxide fuel (MOX

69	fuel; U oxide mixed with \sim 3.9 wt % of Pu, (U,Pu)O ₂).
70	Before the FDNPP meltdowns, the average isotope ratios of ${}^{235}\text{U}/{}^{238}\text{U}$, ${}^{240}\text{Pu}/{}^{239}\text{Pu}$, and
71	242 Pu/ 239 Pu in the irradiated fuels of units 1 – 3 were calculated to be 0.0172 – 0.0193, 0.319 – 0.355,
72	and $0.0612 - 0.0654$, respectively (Nishihara et al., 2012). Note that these values are only
73	representative isotope compositions averaged across the entire fuel inventory before meltdowns. A
74	significant heterogeneity likely occurred in the isotopic and chemical composition of the fuel debris
75	during, and possibly after the meltdowns.
76	Although intrinsic U and Pu oxide microparticles have been previously characterized in
77	various contaminated environments utilizing micro-focused X-ray analysis (e.g. Batuk et al., 2015;
78	Eriksson et al., 2005; IAEA, 2011; Ikeda-Ohno et al., 2016; Salbu et al., 2001), reports on the form
79	of U and Pu released from the FDNPP has been limited by the extremely low bulk Pu concentration
80	in regional soils, which is at the level of global, radioactive fall out from the testing of nuclear
81	weapons (Igarashi et al., 2019). However, the recent discovery of U oxides and fuel debris
82	microparticles associated with FDNPP derived Cs-rich microparticles (CsMPs) in the vicinity of the
83	site (Ochiai et al., 2018) raises the possibility that Pu might have been released together with U in
84	particulate form to the surrounding environment. The CsMPs are generally composed of Si, Fe, Zn,
85	and Cs; they are sparingly soluble and have high radioactivity per unit mass ($\sim 10^{11}$ Bq/g), and their

86	size ranges from submicron to tens of microns (Furuki et al., 2017; Ikehara et al., 2018; Imoto et al.,
87	2017; Suetake et al., 2019; Utsunomiya et al., 2019) . The 134 Cs/ 137 Cs radioactivity ratio can be used
88	to assign the source reactor because of the homogenized isotopic composition of the volatilized Cs
89	(> ~670 °C) within the damaged reactors, where temperatures approached ~2200 K. The ${}^{134}Cs/{}^{137}Cs$
90	radioactivity ratio of CsMPs is typically slightly over ~1 (Furuki et al., 2017), which agrees well
91	with the ratios for units 2 (\sim 1.08) and 3 (\sim 1.05). The CsMPs formed inside the reactor were released,
92	and dispersed over an area $>10,000$ km ² in the Fukushima prefecture and Kanto region of eastern
93	Japan (Adachi et al., 2013; Furuki et al., 2017; Ikehara et al., 2020, 2018; Utsunomiya et al., 2019;
94	Yamasaki et al., 2016).
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104	signatures of Pu in this other type of radioactive particle have been recently investigated by
105	dissolution techniques (Igarashi et al., 2019). Based on a whole digestion and solution analysis, the
106	Pu isotopes were determined to be 240 Pu/ 239 Pu of 0.330 – 0.415 and 241 Pu/ 239 Pu of 0.162 – 0.178 with
107	large uncertainties; $9 - 23\%$ (Igarashi et al., 2019), and thus it was difficult to be used as an explicit
108	isotopic signature for Pu.
109	Compared with the large radioactive particle derived from unit 1 distributed only within a
110	limited area near FDNPP, CsMPs of a smaller size were distributed across a wide area in the Kanto
111	region (up to 230 km away from FDNPP). The relation between CsMPs and long-range Pu transport
112	has been inferred in a previous study (Yamamoto et al., 2014), but the previous studies failed to
113	detect Pu in the CsMPs (Igarashi et al., 2019; Yamamoto et al., 2014). The purpose of the present
114	study is to elucidate the dispersion and release mechanisms of FDNPP-derived Pu into the
115	environment and to better understand the properties of the debris, in particular any occurrence of Pu.
116	This is based on a combination of advanced analyses of Pu and U isotopic signatures, speciation and
117	elemental distribution measurements, and characterization of the internal texture and structural
118	characteristics of individual CsMPs that have been released from the FDNPP units 2 or 3.
119	

120 2. Samples and Methods

121 2.1. Samples

122	Soil samples were collected at an aqua culture center (AQC) and a paddy field in Ottozawa
123	(OTZ), Okuma town, within ~ 4 km of the FDNPP (Fig. 1). Surface soils of the dimension of 20 cm
124	\times 20 cm \times 2–5 cm depth were sampled from areas without vegetation. All of the collected soils were
125	air dried and sieved through 2 mm Tetron mesh to remove pebbles and plants. Soil particles <2 mm
126	were used for the present experiments. Isolation of three CsMPs was completed for three soil
127	samples collected from the two locations, which were labelled as AQC1, AQC2, and OTZ.
128	
129	2.2. Isolation of CsMPs
130	The procedure for separating CsMPs from soil samples followed the method developed in
131	our previous study (Furuki et al., 2017). First, the soil samples were sieved through a 114-µm mesh.
132	The samples were dispersed on an Al board and covered with a plastic sheet; an imaging plate (IP,
133	Fuji film, BAS-SR 2025) was then placed on the samples for 5-25 minutes. The autoradiograph
134	image was recorded with a pixel size of 100 μ m using an IP reader (GE, Typhoon FLA 5100). After
135	identifying the positions of intense radioactive spots, droplets of pure water were added to the
136	positions and then taken up using a pipette to make a suspension with a small amount of soil
137	particles. The procedure was repeated until the suspension did not contain a significant amount of
138	soil particles. CsMPs are sparingly soluble because of the Si glass matrix (Suetake et al., 2019), as
139	such, this procedure would not have affected the chemical properties of the CsMPs. Subsequently,

140	the position containing a highly radioactive spot was sampled using double-sided carbon tape. The
141	piece of carbon tape with the radioactive particle was cut into smaller pieces using a blade. The
142	small pieces were scanned again by autoradiography before a scanning electron microscopy (SEM)
143	observation. The pieces were placed on an Al plate and coated with carbon using a carbon coater
144	(SANYU, SC-701C) prior to SEM analysis. The CsMP was found using an SEM (Shimadzu, SS550
145	and Hitachi, SU6600) equipped with energy dispersive X-ray spectrometry (EDX, EDAX Genesis).
146	The acceleration voltage was $5-25$ kV for imaging details of the surface morphology and $15-25$ kV
147	for elemental analysis, including area analysis and elemental mapping.
148	
149	2.3. Gamma spectrometry
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Japan and Ge semiconductor detector GMX40 (SEIKO E&G) at the Center for Radioisotopes in
Kyushu University, Japan. The acquisition times were 4800 seconds for AQC using GMX30200 and
65878 s for OTZ using GMX40.
2.4. Secondary ion mass spectrometry
Isotopic ratio analysis was performed for the three CsMPs using secondary ion mass
spectrometry (SIMS, SHRIMP-II, Australian Scientific Instruments) at the National Institute of

- specification (onvis, original in, rustianal scientific instruments) at the ruttohar instruction
- 165 Polar Research, Tokyo. The SIMS specimens were put on Al plates or a Cu grid and fixed on a glass
- 166 slide with Cu tape. They were coated with Au at a 13.5 nm thickness prior to the analysis. An O_2^-
- 167 primary ion beam of 0.2–0.4 nA was used to sputter the specimen surface with a beam diameter of
- 168 5.0–7.0 μ m. The typical mass resolution is about 4,500 (M/ Δ M at 1 % of peak height). Masses of
- 169 237.0 (or 253.0) and 242.1 (or 258.1) that is 0.1 amu heavier than 242 Pu, were measured as
- background counts. The background ion counts were <0.04 cps in the CsMPs, which corresponds to
- 171 ~50 ppb. This value is considered to be the detection limit in the present study. The National
- 172 Institute of Standards and Technology (NIST) SRM610 was used as a standard specimen for mass
- 173 calibration, in which 461.5 mg/kg of depleted U was doped in a silicate glass matrix. Because
- 174 quantitative analysis using SIMS requires suitable standards, which have chemical compositions
- 175 similar to those of the target minerals, the absolute concentration was not obtained, but only the

176	isotopic ratio was determined in the present study. Ten scans were completed for each analytical spot
177	on the CsMPs, meaning that the sequence of ten analyses represents a depth profile of the variation
178	in the isotopic ratios; thus, SIMS analyses provide the isotopic signatures inside the CsMPs. The
179	effects of hydride on the U ion counts is considered to be negligibly small in the present study,
180	because isotope ratios of U ions are almost same as those of UO ions during measurement of
181	SRM610 despite the fact that probability of hydride formation from a UO ion is less than that of U
182	ion (Morrall et al., 2007). In addition, a mass ²³⁷ (²³⁶ UH) was also measured in the present analysis
183	and the count was at the background level in all measurements.

2.5. Synchrotron Analysis 185

After performing SIMS analysis, X-ray fluorescence (XRF) mapping and discrete-area 186X-ray absorption near-edge spectroscopy (XANES) was completed at beamlines I18, at 187Diamond Light Source, UK, and X05LA at the Swiss Light Source. In both instances, AQC1 188was mounted on carbon tape and double contained in Kapton windowed radioactive sample 189 190 holders for analysis.

191	At beamline I18 (double-crystal monochromator), the synchrotron micro-beam was
192	focused to a horizontal width of ~2 $\mu m.$ XRF mapping was performed with an incident beam
193	energy of 18.1 keV. Uranium and Pu L_{III} -edge XANES data were collected in fluorescence mode

194	at ambient temperature with minimum step sizes of 0.1 eV and 0.5 eV over the absorption edges
195	of U and Pu, respectively. Fluorescence data were collected using the beamline's 4 element
196	Vortex Silicon Drift Diode (SDD). Uranium (IV/VI) reference standards were the research
197	teams own, with data collected on beamline B18 at the Diamond Light Source (calibrated using
198	in-line Y foil). The U(IV) standard is a crystalline uraninite (UO ₂) powder and the U(VI)
199	standard is a crystalline metaschoepite (UO ₃ •nH ₂ O) powder. Plutonium standards were not run.
200	At beamline X05LA (double-crystal monochromator), the synchrotron micro beam
201	was focused to a horizontal width of ~ 1 μ m. On-the-fly XRF maps, windowed for Fe, Zn, Zr,
202	Mn, Sr, and U were collected at incident beam energies of 18.0 and 18.1 keV. Pu L_{III} -edge
203	XANES data were collected in fluorescence mode at ambient temperature with a step size of 0.5
204	eV over the absorption edge. Fluorescence data was collected using the beamlines'
205	single-element Ketek SDD. At both beamlines, Y and Zr foils were used to calibrate the U and
206	Pu absorption edges, respectively. XRF mapping data were processed using PyMCA (Solé et al.,
207	2007). Although μ XRF analysis was not used for quantitative elemental analysis in the present
208	study, the synchrotron-based XRF facilities used at both beam-lines can typically analyze U and
209	Pu down to concentrations of several 10s of ppm. Maps were normalized for comparability by
210	subtracting the base noise level from the maps and dividing the maps by the maximum pixel
211	intensity. XANES data were aligned and normalized using Athena (Demeter Suite) (Ravel and

212 Newville, 2005).

214	2.6. Transmission electron microscopy (TEM) analysis and the specimen preparation
215	High-resolution TEM (HRTEM) with energy dispersive X-ray analysis (EDX) and
216	high-angle annular dark-field scanning transmission electron microscopy (HAADF-STEM)
217	were performed using a JEOL JEM-ARM200F with an acceleration voltage of 200 kV at the
218	Ultramicroscopy Research Center (URC) of Kyushu University. The spatial resolution is ~0.11
219	nm. The JEOL Analysis Station software was used to control the STEM-EDX mapping. To
220	minimize the effect of sample drift, a drift-correction mode was used during acquisition of the
221	elemental map. The detection limit of EDX analysis is typically \sim 0.1 wt%. The STEM probe
222	size was ~0.13 nm, generating ~140 pA of current when 40 μm of the condenser lens aperture
223	was inserted. The collection angle of the HAADF detector was ~97-256 mrad.
224	A focused ion beam (FIB) system (FEI, Quanta 3D FEG 200i Dual Beam) was utilized
225	to prepare thin foils of AQC1 after SIMS and synchrotron X-ray analysis. The ion source was
226	Ga, while W deposition was used to minimize damage by the ion bombardment covering a
227	rectangle area of $\sim 3 \times 8 \ \mu m$ size (Fig. S1a). The current and acceleration voltage of the ion beam
228	were adjusted to be 100 pA to 30 nA and 5–30 kV depending on the progress of thinning and
229	sample properties such as hardness and size. The thinned piece of size of ${\sim}2\times6\times5~\mu m$ was

230	attached to the semilunar-shaped Cu grid for FIB and further thinned to < 200 nm thickness by
231	an ion beam at 5 or 8 kV.
232	
233	The above analysis was performed in the order of SIMS > μ -focus X-ray analysis > TEM, so that the
234	CsMPs containing U and Pu can be efficiently analyzed. Due to the limited size of the samples
235	compared with the SIMS ion probe, AQC2 and OTZ were entirely consumed during SIMS analysis
236	and the further analyses could not be performed. The μ -focus X-ray analysis and TEM were
237	competed only for sample OTZ1.
238	
239	3. Results and Discussion
240	3.1. Occurrence of U and Pu in the CsMPs revealed by the advanced nano-scale analyses
241	In the present study, three CsMPs, labelled as AQC1, AQC2, and OTZ, were isolated from
242	the surface soils sampled within 3.9 km of the FDNPP (Fig. 1 and Table 1). Secondary electron
243	images from scanning electron microscopy (SEM), energy dispersive X-ray analysis (EDX)
244	elemental maps, and the resulting spectra revealed a morphology and major element composition (Si,
245	Fe, Zn, Cs, and O) typical of CsMPs reported in the literature (Furuki et al., 2017; Ikehara et al.,
246	2018; Ochiai et al., 2018) (Fig. 2a - c). AQC1 occurs as an aggregate of smaller CsMP particles,
247	while AQC2 and OTZ are individual microparticles. In all EDX spectra, Al concentrations may have

248	been enhanced due to background soil materials. The amount of the other major and minor elements
249	such as Fe, Zn, and Sn are similar to that of CsMPs previously reported (Furuki et al., 2017; Imoto et
250	al., 2017; Utsunomiya et al., 2019), and they are derived from structural materials such as stainless
251	steel, Zr-Sn alloy claddings, and primary cooling water (that contained Zn) (Abe et al., 2014). In the
252	CsMPs, the total radioactivities of 134 Cs and 137 Cs per unit mass are in the order of 10^{11} Bq/g, and the
253	¹³⁴ Cs/ ¹³⁷ Cs radioactivity ratios range from 1.05 to 1.17 after decay-correction to March 11 th , 2011.
254	This demonstrates that the CsMPs originated from the volatilized Cs stemming from molten fuel
255	inside the damaged FDNPP reactor units 2 or 3 (Table 2). In general, the CsMPs were considered to
256	form through molten core-concrete interaction at the concrete pedestal of the primary containment
257	vessel, where the volatilized Cs, stemming from molten fuel, was condensed with vaporized silica
258	(Furuki et al., 2017).
259	Synchrotron micro-focus X-ray fluorescence (µXRF) mapping was successfully performed
260	on AQC. Mapping of the whole particle showed that the distribution of Fe was slightly different to
261	Zn (Fig. 3a). Further detailed mapping of the area outlined by the white dotted rectangle in the Fe
262	map of Fig. 3a also revealed heterogeneously localized distributions of U, Zr, Sr (Sr masked any Pu
263	signal) and Mn within AQC, implying that the melted fuel fragment reacted heterogeneously with
264	Zr-containing fuel cladding at the micron-scale. Indeed, prior work has characterized nanoparticles
265	of (U, Zr) oxide solid solution containing a wide range of U/Zr ratios (Ochiai et al., 2018).

266	Iterative-target factor analysis (ITFA) (Rossberg et al., 2003) of the U L_3 -edge μ XANES (Fig. 3b),
267	collected from the point indicated by red arrow (Fig. 3a), and comparison with U(IV) and U(VI)
268	standards, suggests that the U predominantly occurs as U(IV) oxide (i.e. there was no meaningful
269	spectral contribution from U(VI)) (Fig. 3b).
270	Plutonium XRF mapping was attempted; however, it was hampered due to interference
271	from Sr. Within the U distribution map, the three spots with highest U intensity (indicated by the
272	arrows in Fig. 3a) were further analyzed in an attempt to obtain X-ray absorption spectra at the Pu L_3
273	edge. In the three areas of highest U, two spots (indicated by the white arrows) did not show a Pu
274	absorption edge; in contrast, the area indicated by the red arrow had enough Pu to permit collection
275	of an absorption edge, albeit of low quality (Fig. 3c). Given the noise in the data and to ensure
276	reproducibility, the Pu L_3 edge XANES spectrum was collected from the same point in the sample at
277	both micro-focus beamlines used in this study (Fig. 3c). This highlights Pu occurrence within the
278	CsMP and represents the first Pu XANES spectrum collected from Fukushima soils, where Pu is
279	present at levels similar to global fallout. Whilst the low signal to noise ratio precludes confident
280	evaluation of the Pu speciation in the CsMP inclusion, the shape of the spectrum at least indicates
281	that the Pu occurs as a Pu oxide (e.g. Ankudinov et al., 1998; Conradson et al., 2004), and not as Pu
282	metal, which has a reduced XANES amplitude (e.g. Clark and Hobart, 2019; Ikeda-Ohno et al.,
283	2016).

284	A thin foil for TEM observation was then made from the part of AQC1 indicated by a red
285	rectangle in Fig. S1a using a focused ion-beam system to investigate textural and structural
286	characteristics. A cross sectional view in HAADF-STEM images and EDX elemental maps of major
287	elements revealed that AQC consists of a variety of nano-phases including Cs-pollucite (Cs-Fe
288	silicate), ilmenite (Fe-Ti oxide), magnetite (Fe oxide), and chromite (Fe-Cr oxide), based on the
289	diffraction pattern and composition (Fig. 2b and Fig. S1b – e). Although a typical CsMP comprises a
290	SiO ₂ glass matrix with Fe, Cs, and Zn, Cs-pollucite is a zeolite phase that can form within CsMPs
291	when the Cs concentration is extremely high (> 10 wt.% ;Imoto et al., 2017). Closely associated with
292	the Fe oxides, several U oxide nanoparticles were also found as indicated by yellow and red arrows
293	in Fig. 4a, which are labeled from U1 to U4 (Fig. 4b and Fig. S2). A high-resolution HAADF-STEM
294	image of U4 indicated by the red arrow revealed a periodic array of U atoms corresponding to the
295	(111) plane of uraninite (Fig. 4c), indicating that U dioxide nanoparticles were encapsulated during
296	the formation of the CsMP, as reported in our previous study (Ochiai et al., 2018) and as indicated
297	from the U L3-edge XANES (Fig. 3b). The scattered occurrence of uraninite nanoparticles within
298	AQC is consistent with the heterogeneous U distribution observed in the μ XRF map (Fig. 3a), which
299	does not correlate with the distribution of other elements within CsMP.
300	Based on the results obtained from TEM, XRF elemental mapping, and XANES analyses,

301 U predominantly occurs as U(IV) oxide in the CsMP, with small Pu oxide-enriched areas also

302	present. As such, the particle appears to be a debris fragment with fuel composition that has been
303	minimally affected by the subsequent alteration owing to its encapsulation in the CsMP. With
304	increasing burn-up, actinides such as Np, Pu, and Am substitute for U in UO ₂ matrix (Konings et al.,
305	2015); thus, the trace Pu is distributed within the UO_2 matrix. The maximum value of burn-up: 40
306	GWd/tHM for both units 2 and 3 (Nishihara et al., 2012), is high enough to have Pu concentrates at
307	fuel edge sites due to the higher burn-up (Bruno and Ewing, 2006; Burns et al., 2012; Ewing, 2015).
308	Hence, the slightly enhanced Pu concentration, likely at ppm level, detected by μ -focus XANES
309	within the UO_2 matrix is plausibly derived from a Pu concentrate at the rim of irradiated fuel, where
310	a U, Zr oxide eutectic formed as inferred from localized Zr occurrence (Fig. 3) and also
311	characterized in another CsMP utilizing TEM (Ochiai et al., 2018). Because the nuclear fuel
312	fragment was not subjected to significant alteration, the initial state of Pu(IV) that substitutes for U
313	in the irradiated UO ₂ (Ewing, 2015; Konings et al., 2015) has remained in its initial state, which is
314	consistent with the XANES data that indicates Pu is present as an oxide.
315	
316	3.2. Plutonium and U isotope ratios in the CsMPs
317	Secondary ion mass spectrometry (SIMS) was successfully applied to determine U and Pu
318	isotopes in these three CsMPs (Table 3). During SIMS analysis, the ionized isotopes were detected
319	either in non-oxide ion or oxide ion form. Oxide form means an element bound to oxygen such as

320	$^{235}\text{U}^{16}\text{O}$, of which the total mass of 251 is measured. The non-oxide form of ionized isotopes was
321	detected for two analytical spots in AQC1, one spot in AQC2, and one spot in OTZ, whilst oxide
322	ions were only detected for two spots in AQC1 and one spot in AQC2, due to the limited sample
323	volume for OTZ (the OTZ CsMP was consumed by the first set of analysis detecting the non-oxide
324	ion form and could not be used for the second set of analysis detecting the oxide ion form). As
325	revealed in Fig. 5 and Table 3, isotope ratios in non-oxide and oxide form revealed almost identical
326	values for ${}^{235}\text{U}/{}^{238}\text{U}$ and ${}^{239}\text{Pu}/{}^{238}\text{U}$, but the greater number of ions were counted for the oxide form.
327	The ${}^{235}\text{U}/{}^{238}\text{U}$ isotope ratios of the four analyzed spots are 0.01935 (± 0.00009), 0.01935 (± 0.00006),
328	0.01935 (± 0.00011), and 0.01925(± 0.00014), and the 238 U ion count ranges from 835 to 975
329	(cps/nA) (Fig. 5a, Table 3). The isotope ratio is nearly identical to that calculated for the irradiated
330	fuel of units 2 or 3 based on the burnup using ORIGEN2 code; ~0.019 (Nishihara et al., 2012). In
331	our previous study (Imoto et al., 2017), the 235 U/ 238 U isotope ratio of the analyzed CsMPs was
332	determined to be ~0.029 with low 238 U counts; 11 – 14 cps/nA. These ratios plotted between that of
333	non-irradiated fuel (0.039) and irradiated fuel (0.019), which was ascribed to volatilization of a
334	partially oxidized form of the less-irradiated fuel, as evidenced by the low ²³⁸ U counts. In contrast,
335	the total ²³⁸ U counts detected in the present study are nearly two orders of magnitude greater than
336	that derived from U volatilization, with the 235 U/ 238 U isotope ratio identical to that of the irradiated
337	fuel (Fig. 5a). This is a strong indication that the SIMS analysis spots included discrete fuel particles

338 within the CsMPs as revealed by the TEM results (Fig. 4, Fig. S2) rather than trace and diluted U 339 sorbed to the CsMP matrix, subsequent to volatilization, as reported in a previous study (Imoto et al., 340 2017). Nanoparticulate uranium dioxides and (U, Zr)-oxide solid solutions were also identified to be 341associated with CsMPs (Ochiai et al., 2018). Despite that the burnup of the irradiated fuel at the time 342of nuclear disaster ranged from 3.3 to 40 GWd/tHM for unit 2 and from 4.5 to 40 GWd/tHM for unit 3 (Nishihara et al., 2012), the ${}^{235}U/{}^{238}U$ isotope ratios determined in the present study are identical to 343 344 the isotope ratios averaging the entire fuel assemblies in each reactor. This indicates either the 345formation of large-scale melting of fuel rods so that the U isotopic ratio was homogenized, or the 346 burnup of U oxide analyzed by SIMS in the present study is coincidentally close to the average value of the irradiated fuels in FDNPP at the time of meltdowns. 347

In general, a series of Pu isotopes are produced mainly through the following sequence of
 nuclear reactions in reactors (Choppin et al., 2002);

351 , in which the time in parentheses represents the half-life of the respective radionuclide. The

analyzed Pu isotopes are listed in Table 3. Because the isotope ratio of ²³⁹Pu/²³⁸U was almost
identical between non-oxide and oxide forms (Fig. 5b), the following discussion describes Pu-oxide
ion ratios simply as a Pu isotope ratio. Plutonium isotope ratios of two spots in AQC1 are plotted in

355	Fig. 5c and exhibit clear difference from the values of global fallout (Kelley et al., 1999). The
356	isotope ratio of Pu, 240 Pu/ 239 Pu, was determined to be 0.3492 (± 0.0040) for AQC1-spot1, 0.3441 (±
357	0.0033) for AQC1-spot2, and 0.3313 (\pm 0.0114) for AQC2, which are close to those of the average
358	Pu vector calculated using ORIGEN2 for the irradiated fuels at the time of disaster in the FDNPP
359	(Nishihara et al., 2012). This is consistent with the results analyzed for bulk soils and litter samples
360	in previous studies (Zheng et al., 2013, 2012). The other results calculated using ORIGEN ARP
361	(Kirchner et al., 2012; Schwantes et al., 2012) and CASMO5 (Yamamoto et al., 2018) revealed
362	slightly higher 240 Pu/ 239 Pu isotope ratios. The isotopic ratio of 242 Pu/ 239 Pu was determined to be
363	0.0668 (± 0.0033), 0.0651 (± 0.0011), and 0.0622 (± 0.0059), which are plotted within the range of
364	the calculated ratios by ORIGEN2 (Nishihara et al., 2012), ORIGEN ARP (Kirchner et al., 2012;
365	Schwantes et al., 2012), and CASMO5 (Yamamoto et al., 2018). For comparison, the Pu isotope
366	ratios were calculated using ORIGEN ARP for the MOX fuels irradiated for 171 days prior to the
367	meltdowns with input data of the fuel at the time of loading (Nishihara et al., 2012), because the Pu
368	concentration was as high as ~2.8 wt% (Nishihara et al., 2012) and occupied 32 of the 548 fuel
369	assemblies in the unit 3 (Nuclear Emergency Response Headquarters and Government of Japan,
370	2011) leading to the serious concerns of radiotoxicity. The relationship between Pu isotopes clearly
371	indicates that the Pu is not derived from MOX.

372 The ratio of the 239 Pu/ 238 U ion count was calculated to be 0.00488 (± 0.00020) and 0.00490

373	(\pm 0.00010) for two spots in AQC1, 0.01011 (\pm 0.00031) for AQC2, and 0.00534 (\pm 0.00037) for
374	OTZ (Fig. 5b, Table 3). Figure 5b also indicates that the ionization efficiency of the non-oxide is
375	essentially the same as that of the oxide forms for U and Pu in the CsMP matrix. The wide range in
376	the ratios for the three analytical points from 0.0048 to 0.01011 also indicates that the Pu content in
377	the U oxide matrix can vary within the volume of SIMS microanalysis (~4 – 5 $\mu m)$ and between the
378	different CsMPs.
379	
380	3.3. Release process of fuel fragments, environmental effects, and insights to debris
381	In early studies, the release of Pu from the FDNPP was considered to occur through
382	volatilization and the amount of Pu released was estimated relative to that of Cs. This was based on
383	the assumption that Pu isotopes and ¹³⁷ Cs were released from the reactors following the same
384	general fate in the meltdown environment (Zheng et al., 2013). Schwantes et al. (2012) proposed that
385	the relative amount of released Pu was proportional to its volatility, which in turn is correlated with
386	its Gibbs free energy of formation at 1000 K (Schwantes et al., 2012). However, experimental
387	studies have revealed that the volatility of Pu is lower than that of U (Pontillon et al., 2010; Pontillon
388	and Ducros, 2010), and the amount of Pu released through volatilization has been considered to be
389	negligible in the FDNPP (Grambow and Poinssot, 2012). In addition, ambiguity remained for the
390	wide range of Pu radioactivity in bulk soil samples (Sakaguchi et al., 2014; Yamamoto et al., 2014).

391	This is partly ascribed to the difference in the form of the released Cs; soluble Cs and CsMPs, which
392	formed through different processes and timing (Furuki et al., 2017). The distribution of Pu is not
393	identical to that of Cs. Rather, the scattered occurrence of trace Pu in the surrounding environments
394	reported in the previous studies (Sakaguchi et al., 2014; Yamamoto et al., 2014) is attributed to the
395	Pu transport in the form of microparticles as inferred in the previous study (Kirchner et al., 2012;
396	Zheng et al., 2012). As shown in the micro XRF maps in Fig. 3, Zr occurrence is heterogeneous,
397	indicating that the UO_2 in the CsMP was not subject to the homogenization or volatilization and that
398	the fuel fragment itself was captured by the CsMP and released to the environment through the same
399	mechanisms reported previously (Ochiai et al., 2018). This is the direct evidence of Pu release along
400	with fuel fragments that has never been reported for the FDNPP.
401	In general, the particulate occurrence of Pu affects the mobility, bioavailability,
402	transportation, and dose conversion factors, depending on the physicochemical properties (Salbu and
403	Lind, 2016). As shown in previous studies, the scattered distribution of Pu derived from the FDNPP
404	was observed in the large area at >50 km of radial distance affected by radioactive Cs released from
405	units 2 and/or 3 (Sakaguchi et al., 2014; Yamamoto et al., 2014), while CsMPs were ubiquitously
406	distributed in the area (Ikehara et al., 2018). This suggests that even though fuel fragments were
407	found to be associated with CsMPs in the present study, not all CsMPs contained the fragments
408	when they were released and transported over large distances. In addition, it is highly probable that

409	other fuel fragments were also released directly without association with CsMPs during the
410	explosive events; however, the size must be small enough to allow them to be transported over long
411	distances. Although the discovery of a particulate form of Pu released from the FDNPP may raise the
412	potential risk of Pu uptake, even at large distances from the FDNPP, the Pu radioactivity in bulk
413	samples does not exceed the range of radioactivity from global fallout from the testing of nuclear
414	weapons (MEXT and GSI, 2012; Zheng et al., 2013). Such a low concentration of Pu may not have
415	significant health effects (Taylor, 1995); however, the actual interaction mechanism can be more
416	complicated depending on the occurrence of fuel fragments in CsMPs; either attaching onto the
417	surface or encapsulated by the CsMP.
418	The present study highlights that the fuel fragments containing Pu were released from the
419	reactors to the surrounding environment associated with micron-scale CsMPs. These fuel fragments
420	provide partial but direct information on the processes that the nuclear fuels experienced during the
421	meltdowns, as well as on the current status of debris. Notably, other fissionogenic radionuclides,
422	such as Sr are also associated with the fuel fragment, which is also consistent with the very
423	low-volatility of Sr (Pontillon and Ducros, 2010). These results also suggest that the concentrations
424	of Pu isotopes in the fuel composition of the debris can be reasonably estimated by using a code such
425	as ORIGEN. Currently, the radiation doses are still too high to access inside the reactors; ~11 and
426	~80 Sv/h within primary containment vessels (PCVs) of units 1 and 2 in 2017, respectively (Tokyo

428	released to the environment (or collected from preliminary investigations inside the reactors using
429	robots) is of crucial importance for understanding the properties of the debris prior to its removal in
430	the course of decommissioning, which is expected to require several decades.
431	
432	

Electric Power Company Holdings, 2018, 2017). Direct characterization of fuel fragments that were

Author Contributions

435	S.U. conceived the idea and designed all experiments. E.K. and S.U. wrote the manuscript.
436	M.S., T.K., K.M., and R.I. performed separation of CsMPs and SEM analysis. T.O. provided
437	navigation during field research in Fukushima. S.Y. performed gamma spectroscopy. K.H. and M.T.
438	performed SIMS analysis. G.T.W.L., W.R.B., J.F.W.M., and P.W. performed the SR-µXRF and
439	µXAFS analysis. G.T.W.L, W.R.B., B.G., R.C.E., J.F.W.M participated in the discussion and
440	interpretation of the results and helped to write the manuscript.

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626 Figure caption

627 Figure 1. Location of Fukushima Daiichi Nuclear Power Plant and sampling location. OTZ and AQC

stand for the Ottozawa and aqua culture center in Okuma town, respectively.

- 630 Figure 2. Results of scanning electron microscopy including elemental maps and EDX spectra from
- three CsMPs; (a) OTZ, (b) AQC1, and (c) AQC2. The images are secondary electron images in (a)
- and (b), and a back-scattered electron image in (c). The red square in (b) indicates the position of the
- EDX analysis with a rastered electron beam. The area outlined by the white dotted line indicates the
- etched hole that resulted from SIMS analysis.
- 635
- 636 Figure 3. Results of synchrotron X-ray analysis. (a) Synchrotron micro-focus X-ray fluorescence
- 637 (µXRF) elemental maps for the entire area of AQC1 outlined by the orange dotted rectangle in Fig.
- 638 2(b), revealing the distribution of Fe and Zn, collected at beamline X05LA at the Swiss Light
- 639 Source. The horizontal beam size was ~ 1 μ m. High-resolution (step size 0.5 μ m) μ XRF
- elemental maps of U, Zr, Mn, and Sr for the area indicated by the white dashed square in the Fe
- 641 map. (b) Uranium L_3 -edge X-ray absorption near-edge structure (XANES) of a discrete point
- 642 within AQC, indicated by the red arrow in the U Lα map. The spectrum is plotted alongside
- 643 U(IV) and U(VI) oxide standards. (c) Discrete-area Pu L_3 -edge XANES collected from the point

644	within AQC indicated by the red arrow in the U $L\alpha$ map. The XANES spectra from the point
645	indicated by the white arrows in the U $L\alpha$ map did not show any Pu absorption edge. The two
646	spectra, one collected from each beamline, displayed after normalization and energy calibration.
647	
648	Figure 4. Nano-structure of AQC1 and distribution of discrete U nanoparticles within the CsMP. (a)
649	HAADF-STEM image of FIB thin foil prepared from the area indicated by the red rectangle in Fig.
650	S1a. Yellow arrows and red arrow indicate the position of U nanoparticles labeled U1 – 3 (displayed
651	in Fig. S2) and U4, respectively. STEM-EDX mapping was obtained from the area outlined by the
652	white dotted square and presented accompanied with the HAADF-STEM image. (b) Enlarged
653	HAADF-STEM image of a U nanoparticle (U4) and U elemental map associated with an EDX
654	spectrum obtained with spot analysis on U4. (c) High resolution HAADF-STEM image of U4.
655	
656	Figure 5. Uranium and Pu isotopic signature of CsMPs released from units 2 or 3 at FDNPP. (a)
657	235 U/ 238 U isotopic ratio of the CsMPs as a function of ion counts (cps/nA) as compared with the
658	values of irradiated and non-irradiated fuels installed in the FDNPP at the time of meltdowns
659	(horizontal lines), which were calculated based on the burnup using the ORIGEN code (Nishihara et
660	al., 2012). For comparison, 235 U/ 238 U of the volatilized U in the CsMPs (Imoto et al., 2017) is also
661	plotted. (b) The ratios of 239 Pu/ 238 U and 239 PuO/ 238 UO ion counts on four spots of analysis for the

three CsMPs. Only the non-oxide form is included for OTZ because of the limited volume of the

- sample. (c) 240 Pu/ 239 Pu and 242 Pu/ 239 Pu isotope ratios of the AQC1 and AQC2 compared with the
- ratios of bulk soils and litter samples (Zheng et al., 2012), calculated inventory using ORIGEN2
- 665 (Nishihara et al., 2012), ORIGEN ARP (Kirchner et al., 2012; Schwantes et al., 2012), and
- 666 CASMO5 (Yamamoto et al., 2018), the MOX fuels only installed in the FDNPP unit 3 (ORIGEN2
- 667 calculation displayed as dashed lines) (Nishihara et al., 2012), and global fallout (Kelley et al., 1999).
- 668 Error bars exist within the symbols unless otherwise noted.

1 Table 1. Information on the sampling for CsMPs characterized in the present study. KU-CsMP

2 number and KU-soil number represent a label of CsMP and soil samples in Kyushu University

3 sample archive.

4

Sample	KU-CsMP No.		Distance from		
name	KU-soil No.	Latitude-longitude	FDNPP (km)	Soil type	Sampling date
AQC1	AQC6	37°24'27.626"N,	1.0	surface soil	2012/3/16
	KU-161	141°01'55.801"E		of field	
AQC2	AQC8	37°24'27.626''N,	1.0	surface soil	2012/3/16
	KU-171	141°01'55.801"E		of field	
OTZ	OTZ25	37°25'05.570"N,	3.9	surface soil	2017/7/27
	KU-309	141°00'21.661"E		of field	

1

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6

- 10 Table 2. Size and radioactivity of CsMPs used in this study. The radioactivity of CsMPs was
- 11 decay-corrected to March 11, 2011.

		Radioactivity (Bq)		¹³⁴ Cs/ ¹³⁷ Cs	Radioactivity
CsMP	Particle size	134.0	137.0	radioactivity	per unit mass
	(μm)	134Cs	137Cs	ratio	(Bq/g)
AQC1	17.8 - 27.9	1176.0 (± 13.0)	1124.2 (± 2.43)	1.05	3.6 × 10 ¹¹
AQC2	4.6 - 5.0	13.24 (± 0.18)	11.34 (± 0.04)	1.17	1.6 × 10 ¹¹
OTZ	2.6 - 2.8	12.72 (± 0.035)	11.39 (± 0.195)	1.12	1.1 × 10 ¹¹

17	Table 3. Uranium and Pu counts ionized from the CsMPs by SIMS analysis. (a) Ion counts
18	detected as non-oxide. (b) Ion counts detected as oxide. Non-oxide ionized isotopes were measurable
19	at two spots in AQC1, one in AQC2, and one in OTZ. Ions in the oxidized form were successfully
20	determined only for AQC1 and AQC2. Each value is the average of ten scans. The standard
21	deviation is give in the parenthesis. Note that isotope ratio was deduced for each scan leading to
22	better precision than the ion counts. The analytical error in isotope ratio was determined using NIST
23	standard SRM610 and the representative results are given in Supplementary Information Table S1.
24	These isotope ratios are not decay-corrected. Note that these data were subsequently decay-corrected
25	to March 11, 2011 and plotted in Fig. 5
26	

28 (a) Non-oxide id	ons			
Sample	AQC1- spot1	AQC1- spot2	OTZ	AQC2
	Ι	Detection non-oxide		
²³⁵ U (cps)	3.22 (± 0.22)	3.78 (± 0.32)	1.73 (± 0.26)	3.84 (± 1.08)
²³⁶ U (cps)	0.69 (± 0.14)	0.87 (± 0.05)	0.41 (± 0.13)	0.67 (± 0.35)
²³⁸ U (cps)	167 (± 11)	195 (± 16)	90 (± 14)	192 (± 48)
²³⁹ Pu (cps)	0.81 (± 0.1)	0.96 (± 0.08)	0.49 (± 0.15)	1.78 (± 0.68)
Primary beam intensity	0.2	0.2	0.1	0.2
(nA)	0.2	0.2	0.1	0.2
²³⁵ U/ ²³⁸ U	0.01935 (± 0.00009)	0.01935 (± 0.00006)	0.01925 (± 0.00014)	0.01935 (± 0.00011)
²³⁶ U/ ²³⁸ U	0.00415 (± 0.00026)	0.00429 (± 0.00008)	0.00447 (± 0.00035)	0.00411 (± 0.00019)
²³⁹ Pu/ ²³⁸ U	0.00488 (± 0.00020)	0.00490 (± 0.00010)	0.00534 (± 0.00037)	0.01011 (± 0.00031)

31 (b) Oxide ions

Sample	AQC1-spot1	AQC1-spot2	AQC2		
Detection oxide					
²⁵¹ (²³⁵ UO) (cps)	43.89 (± 1.37)	47.89 (± 1.39)	7.77 (±0.73)		
²⁵⁴ (²³⁸ UO) (cps)	2268 (± 63)	2470 (± 59)	400 (± 37)		
²⁵⁵ (²³⁹ PuO) (cps)	10.89 (± 0.56)	12.05 (± 0.36)	4.18 (± 0.34)		
²⁵⁶ (²⁴⁰ PuO) (cps)	3.800 (± 0.186)	4.145 (± 0.152)	1.140 (± 0.620)		
²⁵⁶ (²⁴¹ PuO) (cps)	1.290 (± 0.227)	0.935 (± 0.129)	0.380 (± 0.080)		
²⁵⁷ (²⁴² PuO) (cps)	0.730 (± 0.134)	0.785 (± 0.785)	0.260 (± 0.080)		
Primary beam intensity					
(nA)	0.2	0.2	0.2		
²⁵¹ (²³⁵ UO)/ ²⁵⁴ (²³⁸ UO)	0.01935 (± 0.00006)	0.01938 (± 0.00012)	0.01941 (± 0.00006)		
²⁵⁵ (²³⁹ PuO)/ ²⁵⁴ (²³⁸ UO)	0.00480 (± 0.00008)	0.00488 (± 0.00005)	0.01047 (± 0.00018)		
²⁵⁶ (²⁴⁰ PuO)/ ²⁵⁵ (²³⁹ PuO)	0.3492 (± 0.0040)	0.3441 (± 0.0033)	0.3313 (± 0.0114)		
²⁵⁷ (²⁴¹ PuO)/ ²⁵⁵ (²³⁹ PuO)	0.1184 (± 0.0060)	0.0777 (± 0.0036)	0.0904 (± 0.0052)		
²⁵⁸ (²⁴² PuO)/ ²⁵⁵ (²³⁹ PuO)	0.0668 (± 0.0033)	0.0651 (± 0.0011)	0.0622 (± 0.0059)		









Fig. 4



Fig. 5

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