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journal or publication title	Nuclear Fusion
volume	50
number	8
page range	084005
year	2010-07-28
NAIS	5254
URL	<a href="http://hdl.handle.net/10655/00013077">http://hdl.handle.net/10655/00013077</a>

doi: 10.1088/0029-5515/50/8/084005



**Observation of Energetic-Ion Losses Induced by Various MHD Instabilities  
in the Large Helical Device (LHD)**

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**Abstract**

Energetic-ion losses induced by toroidicity-induced Alfvén eigenmodes (TAEs) and resistive interchange modes (RICs) were observed in neutral-beam heated plasmas of the Large Helical Device (LHD) at a relatively low toroidal magnetic field level ( $\leq 0.75$  T). The energy and pitch angle of lost ions are detected using a scintillator-based lost-fast ion probe. Each instability increases the lost ions having certain energy/pitch angle. TAE bursts preferentially induce

energetic beam ions in co-passing orbits having energy from the injection energy  $E=190\text{keV}$  down to  $130\text{keV}$ , while RICs expel energetic ions of  $E=190\text{keV}$  down to  $\sim 130\text{keV}$  in passing-toroidally trapped boundary orbits. Loss fluxes induced by these instabilities increase with different dependencies on the magnetic fluctuation amplitude: nonlinear and linear dependencies for TAEs and RICs, respectively.

## 1. Introduction

The self-sustained D-T burning plasma can be realized under the condition that fusion-born energetic alpha particles ( $\alpha$ 's) are confined long enough to heat the bulk plasma [1]. In addition to effective alpha heating of bulk plasma, loss of  $\alpha$ 's should be suppressed down to an acceptable level to prevent plasma-facing components from the localized damage due to the impact of escaping  $\alpha$ 's. Better understanding of transport and loss process of these energetic ions is therefore essentially required to realize a nuclear fusion reactor. The principal concern is that D-T produced  $\alpha$ 's and super-Alfvénic ions produced by neutral beam (NB) injection or ion cyclotron resonance heating may destabilize energetic-ion-driven magnetohydrodynamic (MHD) instabilities such as Alfvén eigenmodes (AEs) [2] and energetic-particle-continuum modes (EPMs) [3] because those instabilities can potentially cause radial transport and/or losses of energetic ions through wave-particle resonance processes. Effect of MHD instabilities driven

by the bulk plasma, such as sawtooth and tearing mode (TM) on energetic ion transport is also of great concern since transport of energetic ion may be affected through not only wave-particle resonance but also stochastization of energetic-ion orbits [4].

Energetic-ion losses induced by AEs have been so far studied experimentally in several tokamaks [5-7]. Previously, detailed loss processes of energetic ions due to AEs were studied in TFTR and JT60-U [7-9]. In TFTR, a scintillator-based lost fast ion probe (SLIP) was employed to simultaneously measure the energy  $E$  and pitch angle  $\chi$  of escaping fusion products and beam ions [10, 11]. In those experiments, decreases of neutron emission rate, and at the same time, increase of energetic-ion losses due to global Alfvén eigenmodes (GAEs) were observed [7]. Nowadays, efforts on numerical simulations are being made to reveal the energetic-ion transport due to energetic-ion-driven MHD instabilities [12, 13]. In addition, MHD instabilities driven by the bulk plasma can potentially induce redistribution and/or losses of energetic ions. It has been experimentally observed that significant redistribution of energetic ions is caused by sawteeth in tokamaks [14-16]. Also, it has been recognized that a TM leads to redistribution and/or loss of energetic ions. Reduction of the NB current drive efficiency caused by beam-ion losses due to TMs was observed in DIII-D [17] and also energetic-ion loss by neoclassical TM was recently discussed in ASDEX-U [18]. In both cases, beam-ion losses are interpreted to be due to stochastization of beam-ion orbits. At present, energetic-ion losses due

not only to energetic-ion-driven MHD modes but also pressure/current-driven MHD modes are intensively studied using the SLIP in NSTX, ASDEX-U and JET [19-21].

It is also of importance for helical/stellarator plasmas to understand anomalous radial transport and/or loss of energetic ions caused by various MHD instabilities. In experiments prior to this work, anomalous beam-ion losses induced by toroidicity-induced Alfvén eigenmodes (TAEs) and EPs were measured with the SLIP on medium-scale stellarators/helical devices, i.e. CHS[22, 23] and W7-AS [24]. In this paper, we present characteristics of beam-ion losses due to both MHD instabilities, i.e. energetic-ion driven TAEs and pressure-driven resistive-interchange (RIC) modes [25] in high  $\beta$  discharges with  $v_b > v_A$  beam ( $v_b$  and  $v_A$  indicate the velocity of beam and Alfvén speed) of a large-scale helical device where energetic ions are well confined, compared with these medium-sized devices. Beam-ion losses are studied by use of the SLIP, which has recently been installed at the outboard side of the Large Helical Device (LHD) to investigate behaviours of co-going beam ions [26-28].

This paper is organized as follows. In Sec. 2, the brief descriptive introduction of LHD and the SLIP is presented. Characteristics of TAE-induced beam ion loss, i.e. energy and pitch angles of escaping beam ions due to TAE burst, and high-frequency fluctuations seen on energetic-ion loss signals are presented in Sec. 3. Anomalous losses of energetic-ions induced by RIC mode are also reported in Sec. 3. Sec. 4 gives a summary. Finally, the equations to

calculate  $H\alpha$  fluctuations induced by TAE mode are given in Appendix 1. The dispersion relation of RIC is also given in Appendix 2.

## 2. Experimental Setups

LHD is a large helical device having number of toroidal periods  $M$  of 10 and multipolarity  $l$  of 2. The major radius  $R_0$  and plasma minor radius  $a$  are 3.9 m and  $\sim 0.6$  m, respectively. LHD is equipped with three negative-ion-source based NB injectors of which injection energy  $E_b$  is up to 190 keV. One of three tangentially injects NBs in the opposite direction of the equivalent-plasma current whereas the others tangentially inject NBs in the direction of the equivalent-plasma current in counter clockwise  $B_t$  which is defined from the top view of the torus. Injected beam ions are super-Alfvénic in many cases and often destabilize TAEs [29, 30].

The SLIP important in this work has been lately developed and installed at the outboard side of the LHD torus. It is actually a magnetic spectrometer using a set of apertures and a scintillator plate (ZnS:Ag), providing energy  $E$  and pitch angles  $\chi = \arccos(v_{\parallel}/v)$  of lost energetic ions simultaneously as a function of time, where  $v_{\parallel}$  is velocity of ion parallel to magnetic field and  $v$  is velocity of ion. The detailed description of the design concept and hardware of SLIP on LHD are available in Ref. 26. Figure 1a shows the installation position of

SLIP and typical orbit of co-going energetic ion reaching the SLIP. Figure 1b shows the relation between the pitch angle at the plasma surface and that of at the SLIP position. As seen from Fig.1b, the differences of  $\chi$  between the SLIP position and that at the plasma surface from which position the energetic ion is lost to the SLIP are in the range of  $\pm 10$  degrees.

Luminous image produced on the scintillator screen is transferred to the outside of vacuum vessel and subsequently is divided by a half mirror into two paths. Each scintillation light image is monitored with a 4×4-photomultiplier tube (PMT) array and a CMOS camera, simultaneously. Relative sensitivity of PMTs is calibrated with an electro-luminescence sheet emitting a blue-green light uniformly within 10 % error. The detection ranges of gyroradius centroid  $\rho_h = \sqrt{2Em_h} / qB_{SLIP}$  and  $\chi$  of lost-energetic ions on the scintillator screen of the SLIP are shown in Fig. 2 for the vacuum magnetic configuration of the magnetic axis position  $R_{ax}$  of 3.6 m. Here,  $m_h$ ,  $q$  and  $B_{SLIP}$  represent for the ion mass, elementary electronic charge and the magnetic field strength at the SLIP head position, respectively. Ranges of the detectable  $\rho_h$  and  $\chi$  are  $\sim 2$  cm to 23 cm and  $\sim 25$  deg. to 75 deg., respectively, where  $\rho_h \approx 17.0$  cm correspond to  $E \sim 190$  keV at the field strength  $B_{\parallel}=0.6$  T on the magnetic axis, that is,  $B_{SLIP}=0.37T$ .  $E$  and  $\chi$  shown in this paper stand for those at the SLIP position. The PMT array having high-time resolution up to 2  $\mu$ s is used to detect rapid events related to TAE and EPM. The camera system is slow in time response but can provide detailed information of  $E$  and  $\chi$  with a 353×353

pixels image. The typical frame rate is 500 frames/s.

In addition to the above-mentioned SLIP, the E//B-type neutral-particle analyser (NPA) having a tangential line of sight was employed to detect fast neutrals charge-exchanged with co-going energetic ions [31]. Magnetic fluctuations were detected using toroidal/poloidal magnetic probe (MP) arrays placed on the vacuum vessel. Toroidal/poloidal MP arrays consist of 6/13 MPs and are used for analysis of toroidal/poloidal mode numbers of magnetic fluctuations [32]. Information of radial structure of AEs was obtained with a fast-response  $H\alpha$  array system (FHA), of which system consists of interference filters (transmission wavelength is from 654 nm to 658 nm) and seven PMTs characterized by fast-time response up to 200 kHz [33]. The sight line of FHA is shown in Fig.1c. Note that the FHA is applicable to any ranges of density and magnetic field strength without rigorous limitations in these parameters like electron cyclotron emission diagnostic. Radial structure of RIC was derived with several soft-x ray arrays (SX) or absolute-extreme ultraviolet (AXUV) arrays [34], where each array consists of 20 SX/AXUV detectors (Fig. 1d).

### **3. Experimental results**

#### **3.1 Typical discharges with toroidal Alfvén eigenmodes and resistive interchange modes excited in NB-heated high beta plasmas of LHD**



Figure 3 shows a typical discharge with TAEs obtained at  $B_t=0.6\text{T}$  (CCW direction) in the magnetic configuration of  $R_{ax}=3.6\text{ m}$ . In this shot, three tangential NBs are tangentially injected. The electron temperature at the centre  $T_{e0}$  is  $\sim 0.8\text{ keV}$ , and line-averaged electron density  $\langle n_e \rangle$  is  $\sim 1.5 \times 10^{19}\text{ m}^{-3}$ . The volume-averaged beta value  $\langle \beta \rangle$  is relatively high  $\sim 2.0\%$ , as seen from Fig.3a. The ratio of gyroradius of beam ions to averaged plasma minor radius  $\rho_h/\langle a \rangle$  is fairly large. i.e.,  $\sim 0.15\text{ m}/0.6\text{ m}$ . The ratio of the initial energetic-ion velocity to the Alfvén speed is  $v_h/v_A \sim 2$ . The volume-averaged energetic-ion beta value  $\langle \beta_h \rangle$  is comparable to  $\langle \beta \rangle$ , estimated to be  $\sim 1\%$ , on the assumption of classical slowing down of beam ions for  $\sim 5\text{ MW}$  beam deposition. Strongly excited TAE and RICs are identified in the quasi-stationary phase ( $2.8\text{s} \leq t \leq 3.2\text{s}$ ) from the spectrogram analysis of magnetic probe signal. In high  $\beta$  plasmas in the inward-shifted configuration ( $R_{ax}=3.6\text{ m}$ ), TAE and low-frequency RICs often coexist. RICs are excited near the plasma edge due to the steep gradient of bulk plasma pressure and can generate a magnetic island in nonlinear evolution regime, while linearly unstable interchange has no magnetic island [35]. The radial structure of the RICs can be derived from the data of SX/AXUV detector arrays [36-38].

In this paper, the discharge at  $t \sim 2.82\text{ s}$  is mainly discussed. The radial profiles of electron temperature  $T_e$  and  $n_e$  measured with Thomson scattering diagnostic (TSD) at this time are shown in Fig.4a. Hydrogen atom density  $n_0$  is assumed to be scaled as  $1/n_e^2$ , of which

scaling was derived from the experimental result on the CHS having a similar magnetic configuration [39]. As seen from Fig.4a,  $n_o$  does not change significantly toward the last closed flux surface ( $r/a=1$ ) and would rapidly decrease from the value at the edge of ergodic layer where electron density and temperature are very low. The deposition profile of NB-generated energetic ions is shown in Fig. 4b. It is calculated using FIT code [40], which gives the deposition profile of beam ions averaged over the magnetic surface taking into account the prompt orbit loss. The MHD equilibrium is reconstructed using experimental data by VMEC2000 code [41] in the free boundary condition. The reconstruction of MHD equilibrium was done using the pressure profile evaluated from the experimentally obtained data on the assumption of  $T_e=T_i$  and  $n_e=n_i$  having a multiplication factor of energetic ion effect (in this case, the factor is  $\sim 1.2$ ) so that the calculated magnetic axis position agrees well with that predicted from  $T_e$ -profile.

The scintillator image in the time frame of  $t=2.820-2.822$  s captured by a CMOS camera is shown in Fig.5a. On this image, three-primary-loss domains, named D<sub>1</sub>, D<sub>2</sub> and D<sub>3</sub>, are clearly recognized. Lost energetic ions of [ $\rho_h(E), \chi$ ]=[11-15 cm(80-150keV), 25-35 deg.], [8-18 cm (50-190keV), 35-45 deg.] and [8-18 cm (50-190 keV), 45-60 deg.] would reach the domain D<sub>1</sub>, D<sub>2</sub> and D<sub>3</sub>, respectively. Here,  $\rho_h(E)$  is evaluated using  $B_{SLIP}= 0.37$  T. The scintillation image expands into the region of  $\rho_h>17$  cm ( $E>190$  keV), especially in D<sub>3</sub> domain,

although  $E_b$  is less than 190 keV. This is due to relatively low energy-resolution of the SLIP, typically  $\sim\pm 20$  keV for  $E/\chi = 150$  keV/60 degrees, even larger for higher energy ions. The orbit calculation indicated that energetic ions have passing, passing, and passing - toroidally trapped (or so-called “transition”) orbits would reach  $D_1$ ,  $D_2$ , and  $D_3$  domains, respectively. On the other hand, Fig.5b shows these loss fluxes monitored with 16 PMTs of which frequency responses are 20 kHz for No’s. 1~6, 8, 11, 12, 14, 15 and 16 PMTs and 200 kHz for No’s. 7, 9, 10 and 13 PMTs. As seen from Figs. 5a and 5b, TAE/RIC-induced losses are dominantly detected on the above-mentioned three domains  $D_1$ ,  $D_2$  and  $D_3$ . The RIC-induced loss and TAE-induced one are mainly detected on  $D_1$  and  $D_2$ , respectively. Loss flux detected on  $D_3$  is mainly caused by Coulomb collisions in the non-uniform magnetic field structure, since the  $D_3$  region is observed even in the phases without TAE and RICs. In subsequent sub-sections, effects of respective MHD instability, TAE or RIC on beam-ion losses will be discussed in detail.

### 3.2 Beam-Ion Losses due to TAE

Strongly excited TAE having 60~80 kHz is observed in the time window from  $t\sim 2.7$  s to 3.3 s by means of MPs, as shown in Fig.3. The toroidal and poloidal mode numbers are  $n=1$  and  $m\sim 1$ , respectively. The values of  $n$  and  $m$  were determined by MPs. The amplitude of TAE evaluated at the MP position is  $\sim 5\times 10^{-5}$  T at  $t=2.82$  s. Shear Alfvén continua calculated by

STELLGAP code [42] is shown in Figure 6a. A TAE eigenfunction is calculated using AE3D code [43] for  $m=0\sim 10$  assuming that ion density  $n_i = n_e$  and MHD equilibrium is reconstructed using VMEC2000 (Fig.4). As seen from Fig.6b, eigenfunction of the observed TAE is composed by two dominant Fourier components and have a similar character of a core-localized TAE which can exist in low magnetic shear region of a tokamak plasma. This TAE is the odd parity mode, and has the peak of the eigenfunction at normalized radial position  $r/a \sim 0.6$ .

Unlike tokamaks [44], the TAE with odd parity is often strongly excited having large amplitude in LHD. Although the reason is not clarified yet at this moment, it may be attributed to strong continuum damping at the very edge of the plasma having high magnetic shear for an even parity TAE, as seen from the character of the TAE gap near the edge shown in Fig.6a. In high- $n$  and analytic approximation, the continuum damping rate increases strongly with the increase in the magnetic shear strength [45]. Moreover, the continuum damping rate for low- $n$  TAE increases more rapidly with the increase in the magnetic shear strength, compared with that for high- $n$  TAE [46- 48]. This may be a dominant cause that even TAE is usually suppressed or only be very weakly destabilized. A detailed numerical calculation is required to draw a decisive conclusion on the difference in excitation of even and odd parity TAEs, and is left for a future work.

Fluctuations of  $n_e$  due to TAE are observed as H $\alpha$  fluctuations by FHA diagnostics in

LHD plasmas at fairly low  $B_t$  (typically  $<1$  T). The information of the radial structure of TAE can be derived through the cross correlation between the H $\alpha$  fluctuation  $\tilde{I}_{H\alpha}$  signal and the MP signal  $d\tilde{b}_\theta/dt$  [33]. The fluctuation components modulated by TAE in  $\tilde{I}_{H\alpha}$  was calculated by the product of the coherence  $\gamma_{H\alpha-MP}^2$  and the power spectral density of  $\tilde{I}_{H\alpha}$ , where the coherence between TAE induced fluctuations and background fluctuations due to edge turbulence is assumed to be negligibly small [49]. Thus derived TAE induced H $\alpha$  fluctuations integrated along the line of sight is shown in Fig.7c. On the other hand, the predicted H $\alpha$  fluctuations were calculated by TAE eigenfunctions obtained by AE3D code (Fig.6) and plasma parameters including assumed neutral density profile shown in Fig.4. The predicted electron density fluctuations were obtained based on ideal MHD theory. The plasma displacement due to shear Alfvén waves such as TAE was evaluated under the condition that the magnetic perturbations are expected to be purely transverse. Each local H $\alpha$  emission patterns (H $\alpha$  emissibility) induced by  $m=1$  and  $m=2$  Fourier components in TAE eigenfunction in circular plasma are shown in Fig.7a. The local H $\alpha$  emission induced by whole TAE eigenfunction is given in Fig.7b, on the assumption that the mode pattern of local H $\alpha$  emission is uniform in the toroidal direction. The Z-profile of H $\alpha$  emission integrated along the sight lines shown in Fig.1c in the Z-direction is shown as the profile of the normalized minor radius (Fig.7c). The detailed equations used for this calculation are given in Appendix 1. Thus calculated profile agrees well

with the structure obtained using FHA. This result indicates that this FHA diagnostics newly introduced on LHD has a potentiality to give useful information on the internal structure of TAEs excited in high beta plasmas characterized by relatively high density at considerably low toroidal field.

As shown in Fig.8a, the sharp drop and relatively slow rise in the charge-exchanged fast-neutral fluxes  $\Gamma_{\text{neutral}}$  in the range of energy from 180 keV to 190 keV are observed, correlating with each TAE burst, while  $\Gamma_{\text{neutral}}$  in the range of  $E=130$  keV to 160 keV exhibits a rapid rise and slow decay. The temporal evolution of charge-exchanged neutral fluxes will indicate the effect of the clump-hole pair creation in velocity space caused by TAE burst [50]. The time evolution of  $\Gamma_{\text{neutral}}(E=130-160\text{keV})$  related to the clump formation indicates the rapid radial transport of energetic ions, suffering from rapid changes in ion energy and pitch angle, and would be lost outside the plasma during the process. The loss flux of energetic ions detected on the D<sub>2</sub> domain of SLIP has a similar shape of the pulse. The characteristic decay time of the beam-ion loss flux to SLIP  $\Gamma_{\text{SLIP}_{10}}$  is evaluated to be  $\sim 4.5$  ms that is comparable to the estimated slowing down time through Coulomb collision of beam ions with electrons ( $\sim 5$  ms) at the peak of the TAE eigenfunction without TAE. During the slowing down of the formed clump of energetic ions between TAE bursts, some amount of the ions will be lost and detected by SLIP. This process would lead to the slow decay of the SLIP signal  $\Gamma_{\text{SLIP}_{10}}$  shown in Fig.8a.

These observations tell us that substantial transport/loss of energetic ions is induced by TAEs. That is, the peak just after the TAE burst is thought to be caused by the TAE burst. It should be noted that the signal  $I_{SLIP\_10}$  is also modulated more frequently, compared with the change by TAE burst. This frequent modulation is caused by the RIC mode excited near the plasma edge [25], and will be discussed in more detail in the next sub-section. After the frequent modulations due to RIC modes in  $I_{SLIP\_10}$  was removed using numerical band-pass filter, the increment of the beam-ion loss rate due to TAE bursts  $\Delta I_{SLIP}$  was evaluated. Moreover, time averaging every 10  $\mu$ s time interval was also applied to remove remaining white noise in  $I_{SLIP\_10}$ . Thus derived smoothed  $I_{SLIP\_10}$  is also overlaid in the trace of  $I_{SLIP\_10}$ . In Fig. 8b,  $\Delta I_{SLIP}$  evaluated with the difference of  $I_{SLIP\_10}$  between the pulse peak and that just before each TAE burst is plotted versus the magnetic fluctuation amplitude measured at the MP position. The error bar corresponds to the level of the white noise. In this figure, the curve of  $\Delta I_{SLIP} \propto (\tilde{b}_{TAE})^2$  is also shown. As seen from Fig.8b,  $\Delta I_{SLIP}$  induced by TAE increases non-linearly or approximately quadratically with the increase in the TAE magnetic fluctuation amplitude, although the data points are scattered appreciably. This may suggest a diffusive-type loss rather than a convective one [51]. These energetic ions may not be lost promptly from the plasma region by TAE. Presumably, first, energetic ions would be transported by TAE excited around  $r/a \sim 0.6$  to the plasma edge region, as discussed in refs. [50, 52], being scattered into the loss orbit.

The temporal changes of  $E$  and  $\chi$  associated with TAE burst may be obtained through comparison among the loss flux signals ( $I_{\text{SLIP}_1}, I_{\text{SLIP}_2}, \dots, I_{\text{SLIP}_{16}}$ ) having various  $E$  and  $\chi$ . Temporal evolutions of  $I_{\text{SLIP}_{14}}, I_{\text{SLIP}_{13}}, I_{\text{SLIP}_{10}}$  and  $I_{\text{SLIP}_6}$  are shown together with NPA signals in Fig. 9a. The signals  $I_{\text{SLIP}_{10}}$  and  $I_{\text{SLIP}_6}$  and charge-exchanged fast neutral having  $E=130$  keV  $I_{\text{neutral}_E=130\text{keV}}$  are rapidly increase and slowly decay by TAE burst exhibiting similar wave forms, although  $I_{\text{SLIP}_6}$  is appreciably deformed by RICs. In contrast to them, the signals  $I_{\text{SLIP}_{13}}$  and  $I_{\text{SLIP}_{14}}$  and charge-exchanged fast neutral having  $E>180$  keV  $I_{\text{neutral}_E>180\text{keV}}$  suddenly decrease and slowly recover by TAE burst, having similar wave forms. That is, the hole formation in the velocity space by TAE burst is seen as a sharp drop on the wave of  $I_{\text{neutral}_E>180\text{keV}}$  which will be proportional to source signal of energetic ions supplied by NBI. Energetic ions in this energy range will also be lost by TAE burst suffering from a certain change of the pitch angle, which will be detected as  $I_{\text{SLIP}_{14}}$  and  $I_{\text{SLIP}_{13}}$ . At the same time with the hole formation, the formed clump with a certain energy drop is recognized as a pulsed increase in the signal  $I_{\text{neutral}_E=130\text{keV}}$ . Thus generated energetic ions with some energy drop will be lost and be detected as  $I_{\text{SLIP}_6}$  and  $I_{\text{SLIP}_{10}}$ . The above mentioned characters in these signals indicate clearly that the  $E$  and  $\chi$  of lost ions are quickly changed during each TAE burst. This is also clearly seen on the scintillation pattern shown in Fig.9b. If fast ions are kicked by TAE into another position, this may lead to another loss orbit. The increase in  $I_{\text{SLIP}_6}$  by TAE burst



shows a certain time lag for that in  $I_{SLIP\_10}$ , although the signal seems to be still deformed by RIC induced losses. This time lag suggests that further pitch-angle scattering by TAE bursts needs a certain time.

Interestingly, high-frequency fluctuations of  $I_{SLIP}$  correlated with TAE mode frequency ( $\sim 70$  kHz) were observed using correlation analysis between  $I_{SLIP}$  and the MP signal, as seen from the spectrogram of the coherence shown in Fig.10a. High-frequency ion loss fluxes correlated with TAE are often observed in the ranges of the energy and the pitch angle of lost ions:  $E/\chi \sim 180$  keV/45-55 degrees and 6 - 23 keV/25-35 degrees. The  $E$  and  $\chi$  ranges match to the regions indicated by red circles in Fig.10b. The energy of lost energetic ions in the regions #5 and #15 shown in Fig. 10b respectively satisfies the fundamental and the sideband resonance conditions of TAE, that is  $v_{||} \geq v_A$  and  $\geq v_A/3$  at TAE mode peak [53]. The Alfvén speed at the peak position of the TAE eigenfunction ( $r/a \sim 0.6$ ) is  $\sim 2.2 \times 10^6$  m/s. The  $v_{||}$  of lost beam ions at the region #5  $\sim 3.4 \times 10^6$  m/s ( $\rho_h \sim 15$  cm,  $\chi \sim 50$  degrees and  $B_{SLIP}$  at SLIP position is 0.37 T) corresponds to  $v_{||}/v_A \sim 1.5$ . The velocity at # 15  $\sim 9.2 \times 10^5$  m/s ( $\rho_h \sim 5$  cm,  $\chi \sim 30$  degrees) corresponds to  $v_{||}/v_A \sim 0.4$ . The fluctuations in  $I_{SLIP}$  suggest the existence of fundamental and side-band resonant interactions between TAE and energetic ions.

### 3.3 Energetic-Ion Losses due to RIC

As mentioned briefly in the previous sub-section, these low frequency instabilities also induce energetic ion losses, which are clearly detected by SLIP. In this high beta shot RIC with  $m/n=1/1$  is strongly excited. The mode frequency of about 2 kHz is related to mainly poloidal plasma rotation (Fig.3b). Fluctuations in AXUV signals in the frequency range of 1.5~2.5 kHz would give the information of the eigenfunction of  $m/n=1/1$  RIC mode. The radial profiles of the AXUV fluctuation amplitude related to  $m/n=1/1$  RIC mode in the time window of  $t=2.75\sim 2.83$  s are shown in Fig.11a, together with the fitted AXUV fluctuation data obtained using a model profile of the local emissivity of AXUV. In this analysis, the parameters  $x_0$  and  $w$  in the model function  $\exp[\{(r/a-x_0)/w\}^2]$  were adjusted so that the calculated AXUV fluctuation profile taking account the line of sights of the AXUV detector array (Fig.1d) should best-fit to the experimental data [37]. Here,  $x_0$  and  $w$  correspond to the location of the rational surface and the island width. The model function of the local emissivity corresponds to the radial structure of the  $m/n=1/1$  RIC mode. The best fit result was obtained with  $x_0=0.9$  and  $w=0.2$ . Figure 11b shows the thus obtained radial profile of the emissivity fluctuation caused by  $m/n=1/1$  RIC mode. The peak position  $x_0$  agrees well with the position of  $t/2\pi = 1$  rational surface shown in Fig. 4b. This approximately corresponds to the eigenfunction of the RIC mode. On the other hand, the dispersion relation of RIC is given in Appendix 2 and eigenmode function can be derived from the eigenmode equation. In the present paper, however, we did not make direct

comparison of the RIC-induced fluctuation profile derived from experimental data (Fig.11b) with the eigenfunction calculated from the RIC eigenmode equation, because the experimentally obtained eigenfunction is applicable to qualitative discussion on interaction of the RIC mode with energetic ions.

Energetic-ion losses induced by RIC are mainly observed in the domains  $D_1$  and  $D_3$  on the  $\rho_h - \chi$  plane shown in Fig.5a. Energetic-ion loss fluxes are increased and modulated synchronizing with the RIC mode oscillation as seen from Figs. 5 b. The zoomed time evolutions of the magnetic fluctuation of RIC and the beam-ion loss flux signals  $\Gamma_{\text{SLIP}_6}$  and  $\Gamma_{\text{SLIP}_{13}}$  are shown in Fig.12a. The pitch angle  $\chi$  of these lost-beam ions is close to that of the so-called transition particles of which orbit randomly changes between co-passing and toroidally trapped orbits[54]. It should be noted that the particle gyro radius  $\rho_h$  correspond to the ions with slightly reduced energy from the injection energy of NB. Fluctuations of  $\Gamma_{\text{SLIP}}$  correlated with RIC were observed using coherence analysis (Fig 12 b). It should be noted that  $\Gamma_{\text{SLIP}}$  fluctuation is not only correlated with fundamental RIC mode of  $\sim 2\text{kHz}$  but also the second harmonic having  $\sim 4\text{ kHz}$  (Fig 12 b and Fig 3 b). Moreover, it is clearly found from the phase analysis that the phase difference between  $\Gamma_{\text{SLIP}_{13}}$  and RIC is approximately fixed at  $\sim 185^\circ$  having appreciable variation of about  $\pm 50^\circ$  (Fig 12 c). The reason why the phase does not stay rigorously constant is thought that RIC induced losses would be induced through

stochastization of energetic ion orbit due to large amplitude RIC oscillations. The increment of loss fluxes due to RICs increases linearly rather than non-linearly with the magnetic fluctuation amplitude, as shown in Fig. 12d. This dependence suggests a convective-type loss [51].

In edge region of LHD, plasma rotates mainly in the poloidal direction due to high toroidal viscosity. The  $m/n=1/1$  RIC mode frequency of  $\sim 2$  kHz will reflect the poloidal plasma rotation. In this situation, we discuss whether or not trapped energetic ions could interact with this mode. The resonance condition for helically trapped ion is  $\omega \sim N\Omega$  where  $\omega$  and  $\Omega$  indicate the frequency of the mode and the toroidal precession frequency, and  $N$  is the harmonics number [4]. Note that in LHD configuration, once energetic ions are deeply trapped in a helical ripple well, those ions can drift toroidally while they rotate poloidally along the valley of the two helical winding coils [55, 56]. The poloidal precession of helically trapped ion  $\omega_{hp}$  is expressed

as [57],  $\omega_{hp} = -2l \left( \frac{E(\kappa)}{K(\kappa)} - \frac{1}{2} \right) \varepsilon_h \omega_0$ , where  $\kappa^2 \approx \frac{1}{2\varepsilon_h} \frac{v_{//}^2}{v_{\perp}^2}$ ,  $\omega_0 = \frac{-E}{B_0 r^2} (l / 2\pi)$ , and

$E = \frac{1}{2} m_h v_{\perp}^2$ . Here,  $E(\kappa)$  and  $K(\kappa)$  represent the first and the second kinds of the complete elliptic integrals, respectively. The parameter  $\varepsilon_h$  stands for the ratio of amplitude of fundamental helical perturbation component  $B_{21}$  to magnetic field strength  $B_{00}$ , where two subscripts of the field amplitude stand for the poloidal and toroidal numbers of the helical field. The  $B_0$  is the strength of the resultant magnetic field strength, i.e.,  $B_0 \approx B_{00}$ . For simplicity, we discuss the deeply helically trapped limit, that is,  $v_{//} \sim 0$ . The minor radius and magnetic field

strength are chosen as the peak position of RIC:  $r = 0.54$  m and  $B_0 = 0.6$  T, respectively. Since the toroidal precession frequency of helically trapped particle  $\omega_{ht}$  is approximately expressed as  $\omega_{ht} = (l/M) \cdot \omega_{hp}$ ,  $\omega_{ht}$  is estimated to be  $\sim 10$  kHz for helically trapped ion of  $E = 130$  keV using  $\varepsilon_h = 0.2$ , which is the value at  $r/a = 0.9$  in the present plasma condition [53]. Since  $\omega_{hp}$  is much higher than the mode frequency ( $\sim 2$  kHz), trapped ion would not to be a candidate for the resonant interaction. However, this kind of resonant interaction may not be ruled out completely, because the above discussion is still crude and more detailed particle orbit calculations are needed. Again, it should be noted that toroidally trapped energetic ions is an extremely small fraction of energetic ions in LHD because of tangential beam injection and the relatively large aspect ratio of the plasma ( $R_0/r \sim 6$ ).

As another more likely mechanism, energetic ions travelling near the plasma edge as shown in Fig.1a may suffer from falling into an escaping orbit easily by the large amplitude of magnetic fluctuations of RIC mode near the edge, of which magnetic fluctuations are related to the eigenfunction shown in Fig.11b ( $r/a \sim 0.9$ ) [37]. This loss is thought to be caused by stochastization of the orbit of energetic ion by RICs.

#### 4. Summary

In NB-heated plasmas in LHD, anomalous losses of energetic-ions due to TAE and

RIC modes were observed using the SLIP placed at the outboard side of LHD. The radial profile of H $\alpha$ -fluctuations due to TAE was derived through correlation analysis between H $\alpha$  fluctuations and magnetic fluctuations. The radial structure of H $\alpha$  fluctuations calculated using the eigenfunction of TAE obtained with AE3D code agrees well with the experimentally obtained H $\alpha$  fluctuation profile. This comparison has demonstrated that the FHA newly installed on LHD is a powerful diagnostic tool to get information of the TAE internal structure in high beta plasmas obtained at considerably low toroidal field. Energetic ions lost by TAE bursts belong to a class of co-passing particles. From the temporal analysis of SLIP signals obtained by multi PMTs, it was shown that  $E$  and  $\chi$  of the lost-energetic ions are quickly changed by each TAE burst. The relation between the magnetic fluctuation amplitude and the increment of loss flux by TAE is approximately quadratic and suggests diffusive type loss. High-frequency fluctuation components in energetic ion loss flux correlated with TAE oscillations were clearly observed at the energy range in co-passing orbits where the resonance conditions of the fundamental and the sideband excitation of TAE are satisfied. In addition to TAE, enhanced losses of energetic ions due to RIC were recognized in the vicinity of passing and toroidally trapped boundary for energetic ions, where the RIC has the peak near the plasma edge. The dependence of the RIC induced losses on the magnetic fluctuation amplitude is nearly linear and suggests convective type loss. Numerical simulations are required to clarify the details of loss

processes of energetic ions induced by TAEs or RICs. Moreover, synergistic effects of TAEs and RICs on energetic ion losses are also important for LHD plasmas, and would be also important for tokamak plasmas. This is one of future interesting research targets.

### Acknowledgements

This work is supported in part by the Grant-in-Aid for Scientific Research from MEXT, No. 16082209 and from JSPS No. 21360457 and No. 21340175, and the LHD project budget (NIFS09ULHH508). This research is also supported by the JSPS-CAS Core-University Program in the field of “Plasma and Nuclear Fusion”.

### Appendix 1

The amplitude of H $\alpha$  fluctuations  $\tilde{I}_{H\alpha-AE}$  induced by AE is expressed as,

$$\tilde{I}_{H\alpha-AE} \cong \int \left( \overline{n_0 n_e} \langle \sigma_{ex} v_e \rangle \right)_{AE} dl \quad (A1)$$

where,  $n_0$ ,  $n_e$ ,  $l$ , and  $\langle \sigma_{ex} v_e \rangle$  indicate neutral density, electron density, the path length along the sight line of an H $\alpha$  detector, and electron-impact excitation rate coefficient averaged over the Maxwell distribution function. The subscript AE stands for the contribution from AE activities.

In this formula, fluctuations of neutral density and  $\langle \sigma_{ex} v_e \rangle$  is assumed to neglected because of  $T_e$  is the range from 10 eV to 700 eV in the present experimental condition, then

$$\tilde{I}_{H\alpha-AE} \cong \int n_0 \tilde{n}_{e-AE} \langle \sigma_{ex} v_e \rangle dl \propto \int n_0 \tilde{n}_{e-AE} dl \quad (A2)$$

In Eq. (A2)  $\langle \sigma_{ex} v_e \rangle$  is almost constant in this  $T_e$  range. Accordingly, H $\alpha$  fluctuations will reflect electron density fluctuation and profiles of neutral density.

According to Ideal MHD theory, fluctuation of electron density due to plasma displacement  $\xi$  is expressed as

$$\tilde{n}_e \cong -n_e \nabla \cdot \xi - \xi \cdot \nabla n_e. \quad (A3)$$

From Eqs. (A2) and (A3), the H $\alpha$  fluctuation induced by the displacement related to AE  $\xi_{AE}$  is expressed as,

$$\tilde{I}_{H\alpha-AE} = \int n_0 (-n_e \nabla \cdot \xi_{AE} - \xi_{AE} \cdot \nabla n_e) dl \quad (A4)$$

On the other hand, shear Alfvén wave may be approximated by a transverse magnetic perturbations and the condition is expressed as,

$$\mathbf{B}_0 \cdot \delta \mathbf{B} = -\mathbf{B}_0^2 (\nabla \cdot \xi) - \mathbf{B}_0 \cdot (\xi \cdot \nabla) \mathbf{B}_0 + \mathbf{B}_0 \cdot (\mathbf{B}_0 \cdot \nabla) \xi = 0 \quad (A5)$$

where,  $B_0$  and  $\delta B$  indicate equilibrium magnetic field and the magnetic perturbation induced by shear Alfvén wave, respectively. On the assumption of strong toroidal field,  $\nabla \cdot \xi_{AE} \cong \frac{2}{R} \xi_{R-AE}$  is derived from Eq.(A5), where  $R$  is the major radius. This relation agrees with that shown in the ref [58] through more rigorous treatment. Taking into account that the electron density



gradient is  $r$  (radial) direction, the equation (A4) is modified as,

$$\tilde{I}_{H\alpha-AE} = \int n_0 \left( -\frac{2}{R} n_e \xi_R - \frac{dn_e}{dr} \xi_r \right) dl \quad (\text{A6})$$

The displacement induced by TAE is approximately derived from the scalar potential obtained from AE3D code as,

$$\xi_{AE} = \int \mathbf{v}_k dt \approx -\int \frac{(\mathbf{e}_k \times \mathbf{B})}{B^2} dt = -\int \frac{(\nabla \phi_k \times \mathbf{B})}{B^2} dt = -\frac{i}{\omega} \frac{\phi_k (\mathbf{k} \times \mathbf{B})}{B^2} \quad (\text{A7})$$

where  $k$ ,  $\mathbf{v}$ ,  $\mathbf{e}$ ,  $\phi$ ,  $\mathbf{B}$  and  $\omega$  indicate wave number, velocity, electrical field induced by TAE, the electrostatic potential, magnetic field strength and the TAE frequency. With large aspect ratio approximation ( $R \gg r$ ), the displacement for TAE in cylindrical coordinate ( $r$ ,  $\phi$ ,  $\theta$ ) with low  $n$  can be simply expressed as,

$$\xi_R \sim \frac{1}{\omega} \left( \frac{m \cos \theta}{B_{\phi 0}} \frac{\phi}{r} - i \frac{\sin \theta}{B_{\phi}} \frac{\partial \phi}{\partial r} \right) \quad (\text{A8})$$

$$\xi_r \sim i \frac{m \phi}{r \omega B_{\phi}} \quad (\text{A9})$$

where  $i = \sqrt{-1}$  then, (A6) is rewritten as

$$\tilde{I}_{H\alpha-AE} \sim \frac{1}{\omega} \int \frac{n_0}{B_{\phi}} \left\{ -\frac{2}{R} n_e \left( m \cos \theta \frac{\phi}{r} - i \sin \theta \frac{\partial \phi}{\partial r} \right) + \frac{dn_e}{dr} \frac{m \phi}{r} \right\} dl \quad (\text{A10})$$

Hence, the amplitude of H $\alpha$  fluctuation is expressed as

$$\tilde{I}_{H\alpha-AE\_rms} \sim \frac{1}{\omega} \int \frac{n_0}{B_{\phi}} \left\{ \sqrt{\frac{m^2 \phi^2}{r^2} \left( \frac{dn_e}{dr} - \frac{2}{R} n_e \cos \theta \right)^2 + \left( \frac{2}{R} n_e \sin \theta \frac{\partial \phi}{\partial r} \right)^2} \right\} dl. \quad (\text{A11})$$

## Appendix 2

The dispersion relation of RIC mode in cylindrical plasma is given as follows [59]

$$2\rho \left( 1 + \frac{g - \omega_*}{\omega} + \rho^2 \frac{m^2}{r^2} \right) = \pi e^{i3\pi/4} \frac{(v\omega)^{1/2}}{\varepsilon |k'_{||}|} \left( \frac{g\omega_*}{\omega^2} + \rho \frac{m^2}{r^2} \right) \quad (\text{A12})$$

$$\rho = \rho_s / a \quad (\text{A13})$$

$$g = \frac{m}{r} \frac{d\Omega}{dr} \quad (\text{A14})$$

$$\omega_* = -\frac{m}{r} \frac{1}{n_0} \frac{dn_0}{dr} \quad (\text{A15})$$

$$k'_{||} = (ml - n) / (r - r_0) \quad (\text{A16})$$

where,  $v$ ,  $\varepsilon$ ,  $\omega$ ,  $\rho_s$ ,  $a$ ,  $\Omega$ ,  $r$ ,  $r_0$ ,  $m$ , and  $n$  indicate ion-electron collision frequency normalized by electron cyclotron frequency, inverse of aspect ratio, frequency of the mode, bulk ion Larmour radius evaluated with the electron temperature, plasma minor radius, the curvature of magnetic field, radial position, the location of the rotational surface, poloidal mode number and toroidal mode number, respectively. Index of “0” indicates the rational surface. The mode is interchange type and therefore localizes around the rotational transform ( $v/2\pi = 1/q$ :  $q$  is safety factor).

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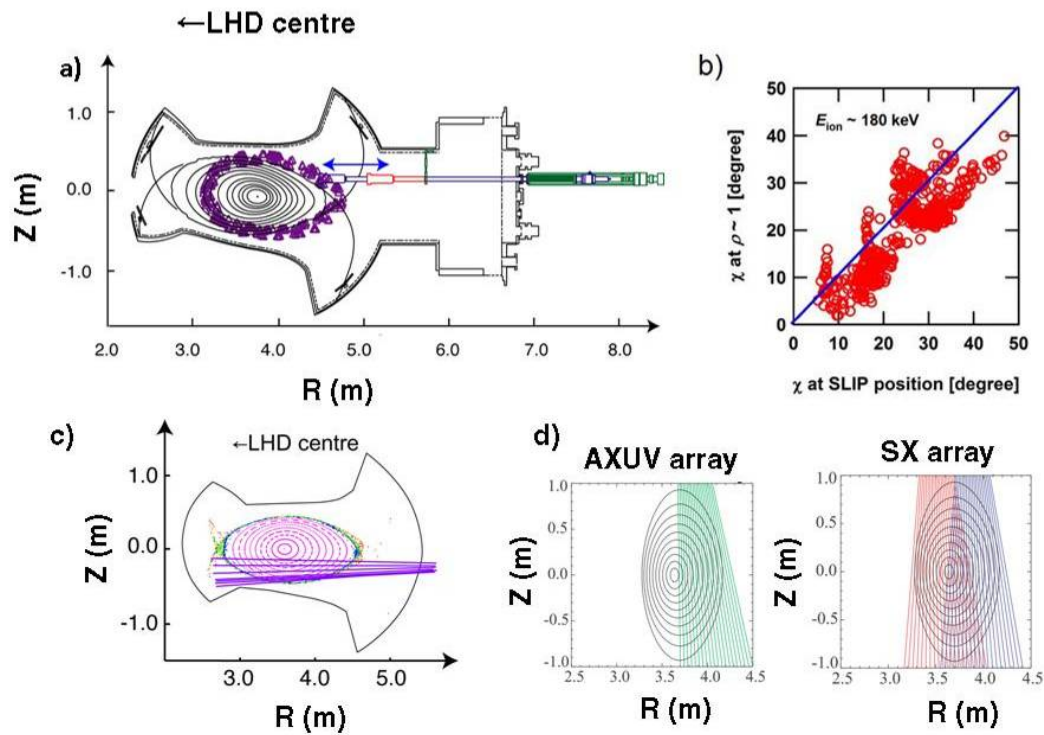


Fig. 1 a) Schematic drawing of the SLIP installed on the outboard side of the LHD and a Poincaré plot of the typical orbit which can be detected with the SLIP. The orbit of the detectable ion deviates from magnetic flux surfaces. b) The relationship between the pitch angle of energetic ions at the SLIP position and that on the plasma surface. The pitch angle on the plasma surface was evaluated by calculating the orbit of energetic ions launched from the SLIP in the vacuum field. c) The sight lines of FHA, where FHA consists of seven PMTs having fast time response up to 5  $\mu$ s. d) Sight lines of AXUV array and SX array, each array consists of twenty detectors.

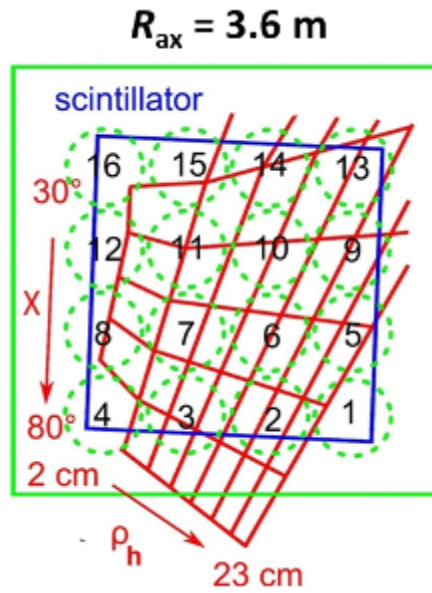


Fig. 2  $E/\chi$  map on the scintillator screen. The circles 1 to 16 on the screen indicate the area that each PMT views.



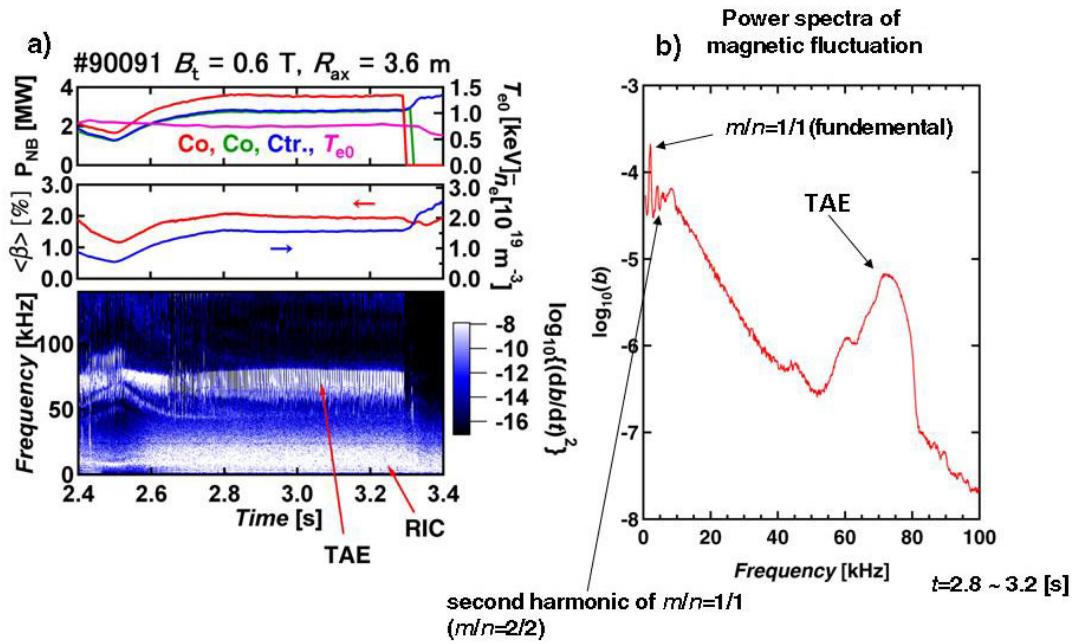


Fig. 3 a) Time evolutions of NBI absorbed powers,  $T_{e0}$ , the bulk plasma beta  $\langle\beta\rangle$ , the line-averaged electron density  $\langle n_e \rangle$  and spectrogram of magnetic probe signal, where TAE ( $m \sim 1$ ,  $n=1$ ) and RIC fluctuations (frequency  $<10$  kHz) are identified. b) Power spectra of magnetic fluctuation. The dominant peak at 2 kHz frequency corresponds to RIC ( $m/n = 1/1$ ). The smaller peak at  $f \sim 4$  kHz is the second harmonic of the RIC.

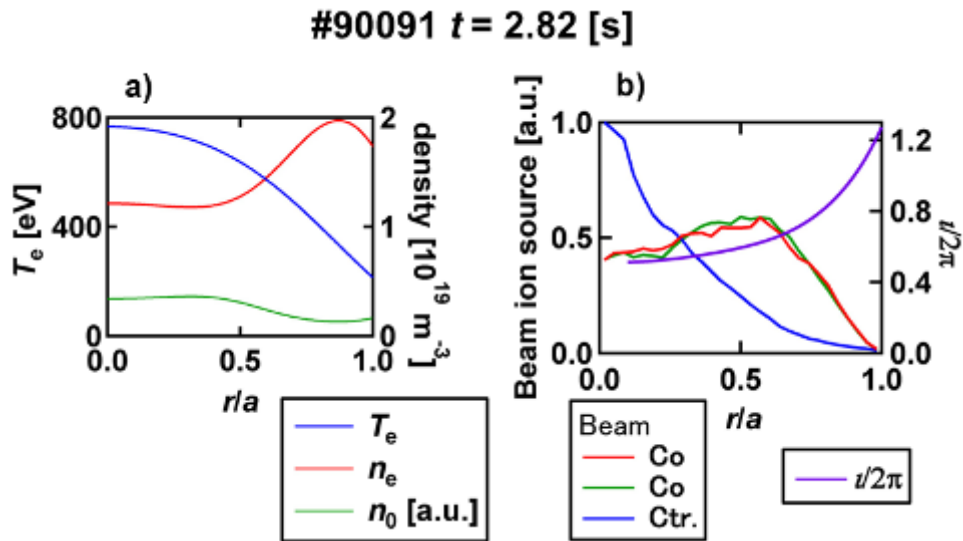


Fig. 4 a) Profiles of electron temperature, electron density, and neutral density at  $t = 2.82$  s.

Neutral density is inferred from the scaling of  $1/n_e^2$ . b) Radial profiles of deposited beam ions and the rotational transform  $\nu/2\pi$  (inverse of safety factor) at  $t = 2.82$  s, where the beam deposition profile is estimated using FIT code, and the  $\nu/2\pi$  is calculated using VMEC2000 code.

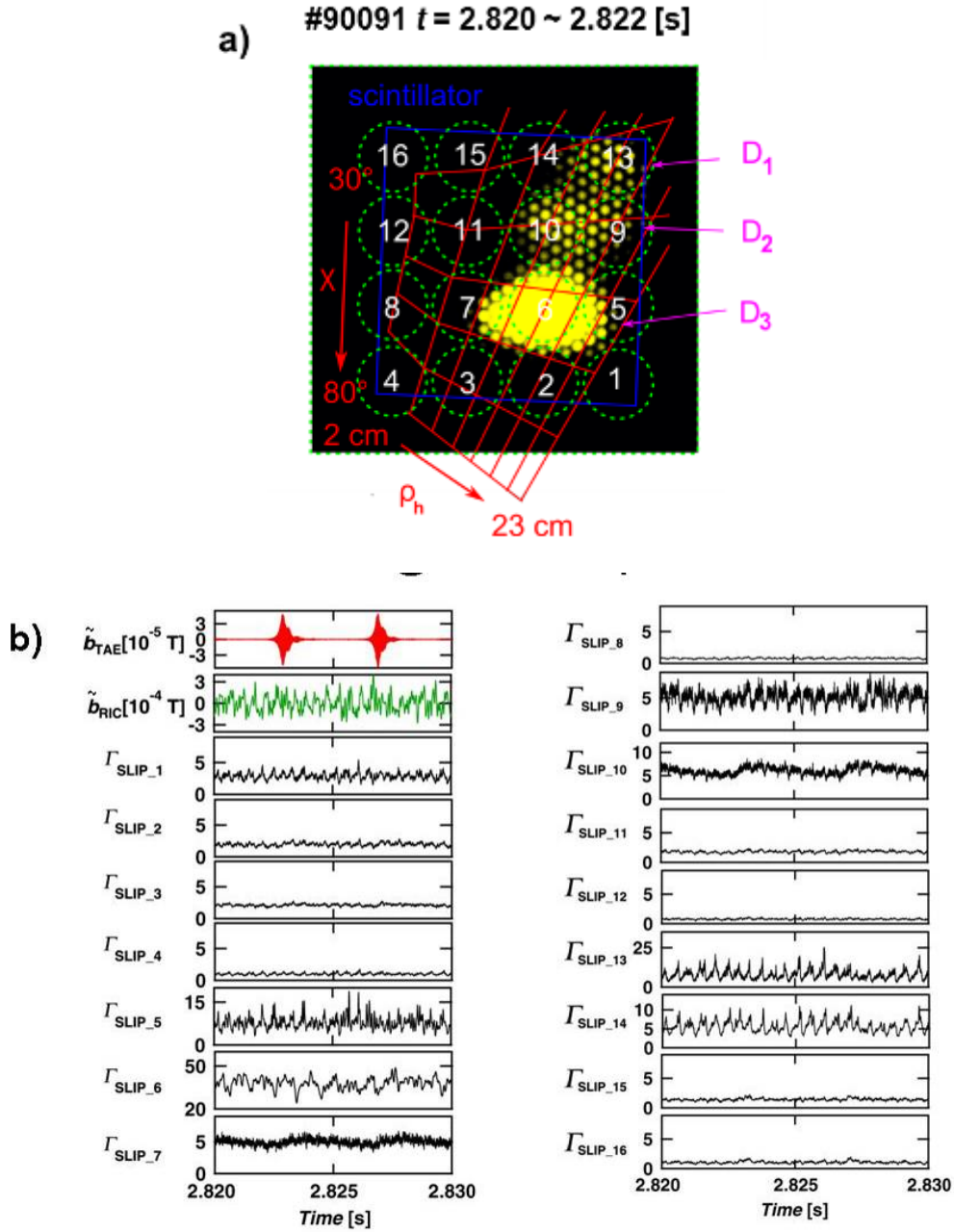


Fig. 5 a) Scintillation pattern captured with a CMOS camera. Three-dominant-loss domains ( $D_1$ ,  $D_2$  and  $D_3$ ) are observed. The spot in the domain  $D_3$  appears even if TAE/RIC is not excited. b) Time traces of magnetic fluctuations by TAE and RIC, and 16 SLIP signals. TAE-induced loss is

clearly observed in  $I_{\text{SLIP}_7}$ ,  $I_{\text{SLIP}_9}$  and  $I_{\text{SLIP}_{10}}$ , of which detail is discussed in the subsection 3.2.

RIC-induced loss is clearly observed as frequent spikes in  $I_{\text{SLIP}_5}$ ,  $I_{\text{SLIP}_6}$ ,  $I_{\text{SLIP}_{13}}$  and  $I_{\text{SLIP}_{14}}$ , of

which detail is discussed in the subsection 3.3.

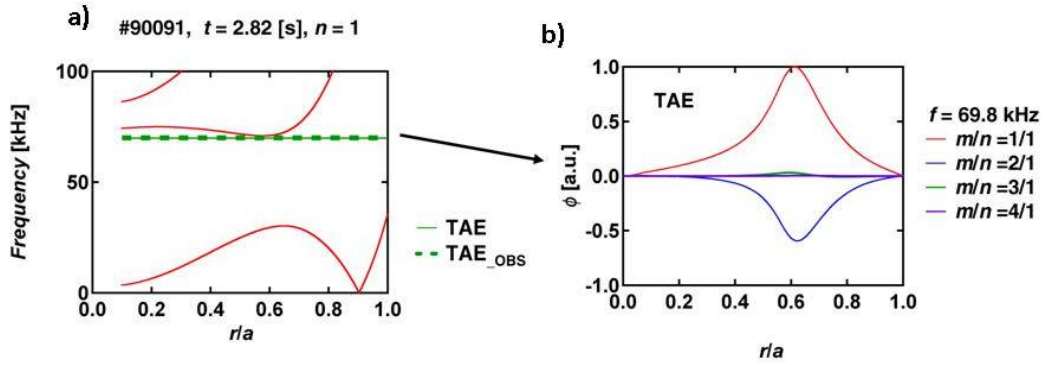


Fig. 6 a) Shear Alfvén spectra of  $n=1$  calculated by the STELLGAP code, where  $m = 0 \sim 10$  Fourier modes are included. In the calculation, profiles of  $n_e$ , and  $v/2\pi$  shown in Fig4 are used. The calculated and observed TAE frequencies are shown with thin and broken horizontal lines, respectively. b) Eigenfunction of the TAE mode calculated with AE3D. This mode has the odd radial parity.

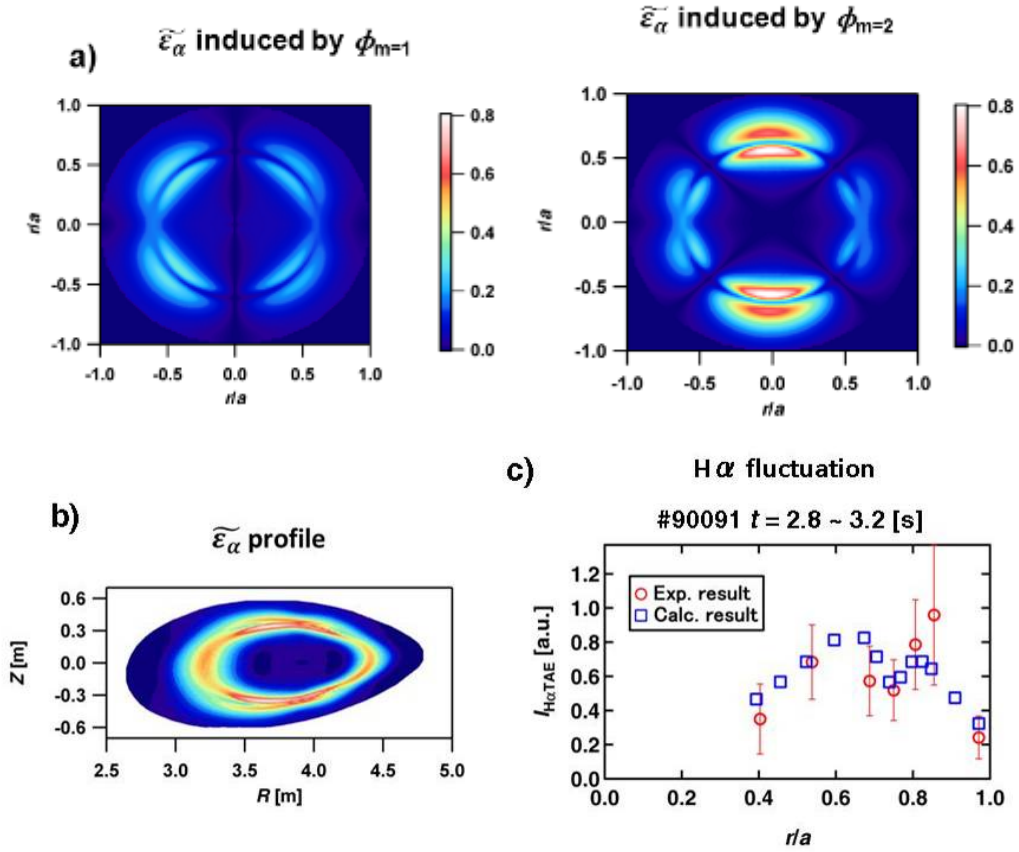


Fig. 7 a) Profiles of the local H $\alpha$  (H $\alpha$  emissivity) fluctuation induced by  $m=1$  and  $m=2$  Fourier components of TAE eigenfunction calculated by AE3D where  $n_0 \propto n_e^{-2}$  is assumed. b) Profile of local H $\alpha$  fluctuation induced by TAE in the cross section of the LHD plasma viewed by the FHA diagnostic, where the mode pattern is uniform in the toroidal direction. The eigenfunction of TAE is converted from that in the flux coordinate using in Fig. 7a to the real one based on VMEC2000 calculation. c) The H $\alpha$  fluctuation profile obtained experimentally with FHA (open circles) and that calculated from the result shown in Fig.7 b that was calculated with AE3D code

(open squares). The mode pattern shown in Fig.7 b is rotated 10 times having 1000 steps for each rotation.

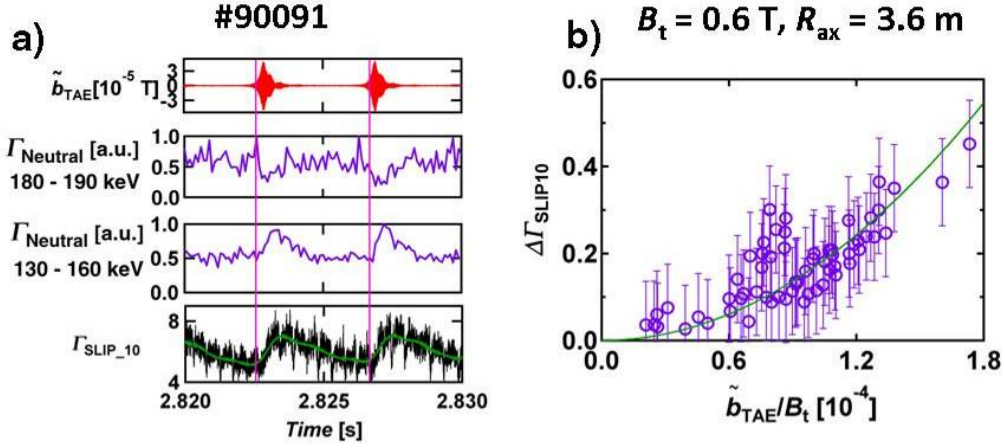


Fig. 8 a) Time traces of magnetic fluctuations of TAE, fast neutral flux measured with a tangential NPA ( $\Gamma_{neutral}$ ) and  $\Gamma_{SLIP}$ . The fluxes  $\Gamma_{neutral}$  in the range of  $E=130-160$  keV and  $E=180-190$  keV are shown. In the trace of  $\Gamma_{SLIP_{10}}$ , the smoothed signal of  $\Gamma_{SLIP_{10}}$  is overlaid. The smoothing is done by numerical band pass filter to remove the modulations by RICs and also by moving average technique to suppress white noise. b) Dependence of the increment of the loss flux induced by TAE on magnetic fluctuation amplitude of TAE.



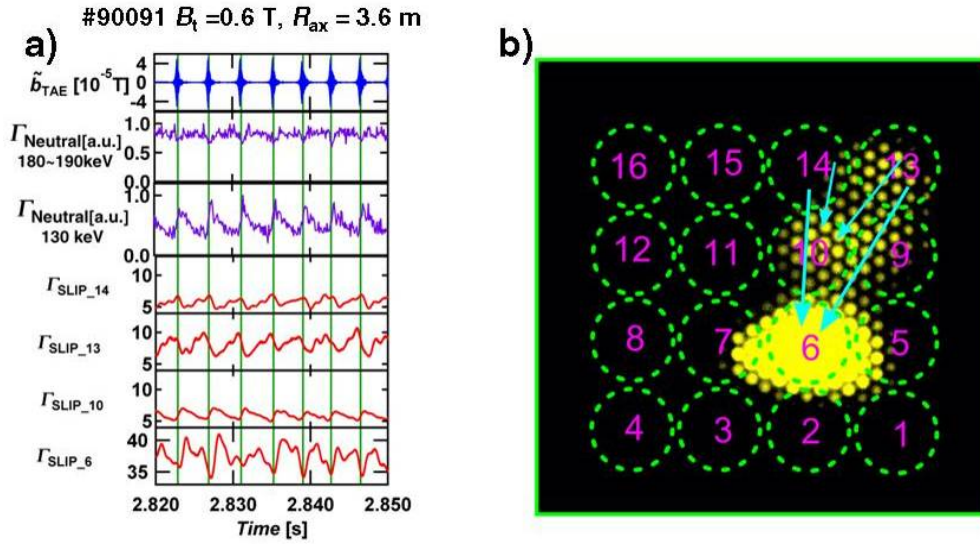


Fig. 9 a) Time traces of TAE magnetic fluctuations,  $\Gamma_{neutral}$ , and  $\Gamma_{SLIP}$  that RIC-induced modulations are removed. b) Scintillation pattern obtained by a CMOS camera in the phase of  $t=2.820$ s to 2.822s. Directions of the pitch angle scattering and the energy drop of energetic ions by TAE bursts are indicated with arrows.

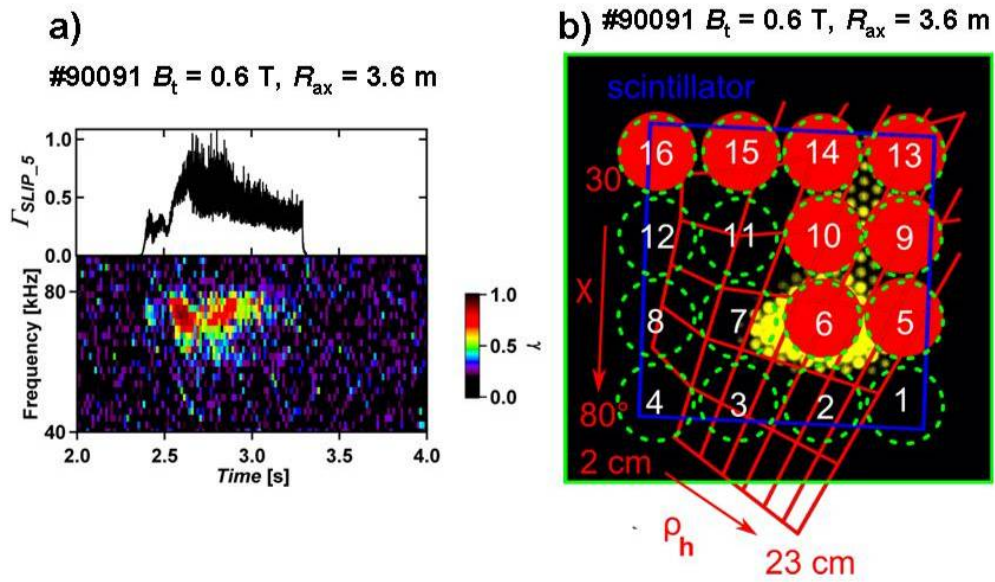


Fig. 10 a) Spectrogram of the coherence  $\gamma$  between  $\Gamma_{SLIP\_5}$  and magnetic probe signal. Losses correlated with TAE oscillations are observed with coherence analysis. b) Regions where high correlation between magnetic probe signal and  $\Gamma_{SLIP}$  are observed (red circles). Especially, #15 and #5 regions have higher coherence.

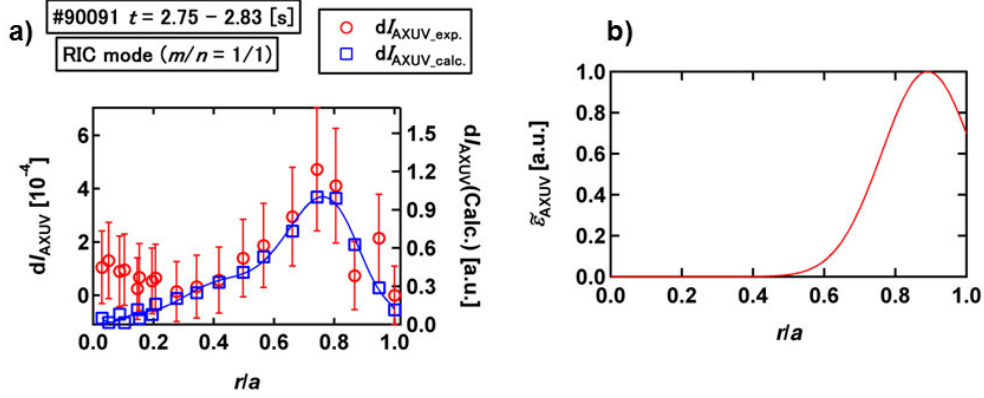


Fig. 11 (a) Radial profiles of the amplitude of RIC fluctuations measured with AXUV array (open circles) and that from the calculation (open squares). Note that the signal  $dI_{AXUV}$  obtained by the AXUV array corresponds to the signal integrated along the sight lines shown in Fig.1d. The parameters  $x_0$  and  $w$  in the model profile of the local emissivity  $\exp[\{(r/a-x_0)/w\}^2]$  was adjusted so that both profiles should have good agreement, where  $x_0$  and  $w$  indicate the location of the rational surface and the island width. Here,  $x_0=0.9$  and  $w=0.2$  give the good agreement as shown in this figure. (b) Radial profile of local fluctuation of  $m/n=1/1$  RIC derived from the AXUV fluctuation data in Fig.11a through the above model fitting. The  $x_0$  agrees well with the predicted location of the  $1/2\pi=1$  rational surface.

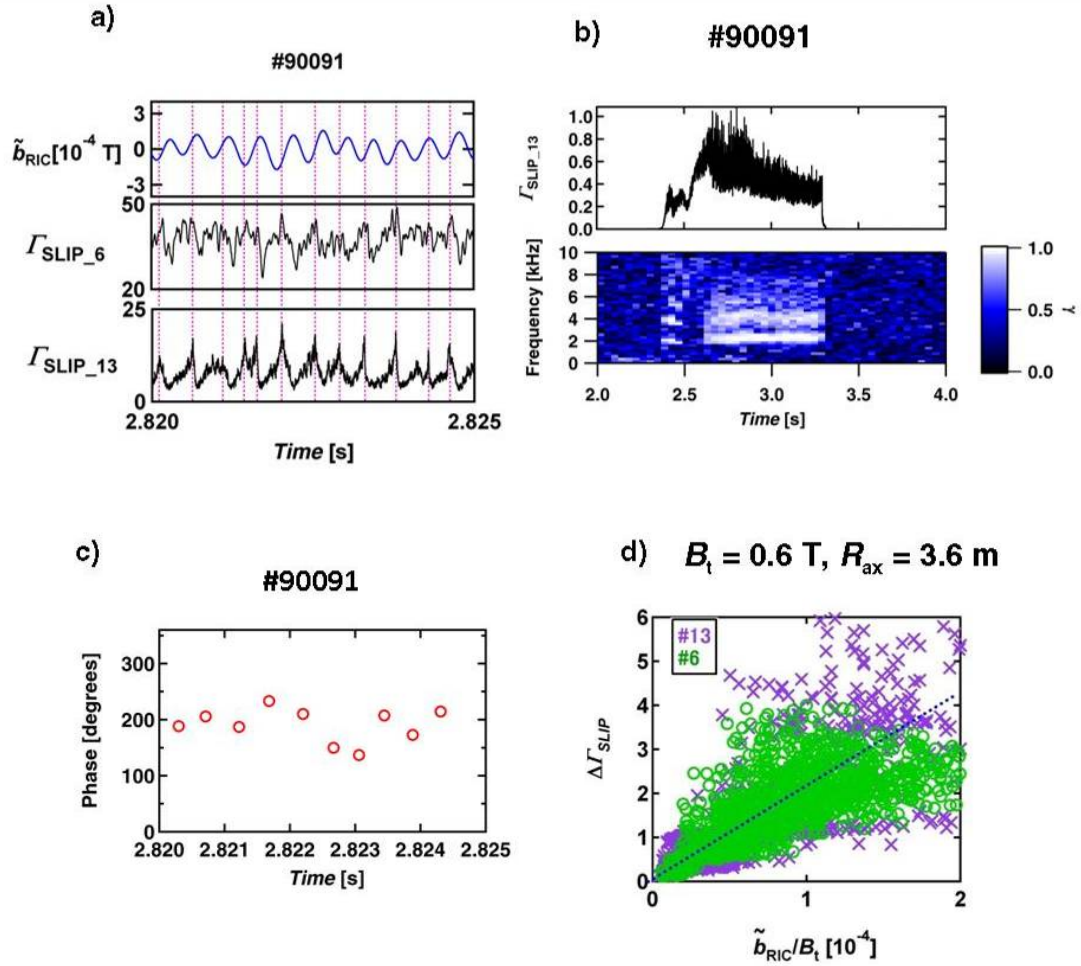


Fig. 12 a) Zoomed time evolution of RIC modes and  $\Gamma_{\text{SLIP}}$  signals. RIC magnetic fluctuations are derived from MP signal by using numerical band-pass filter of 1.5 to 3 kHz. b) Spectrogram of the coherence between  $\Gamma_{\text{SLIP}_{13}}$  and magnetic probe signal. The losses correlate with the RIC-range oscillations ( $f = 2 \text{ kHz}, 4 \text{ kHz}$ ). c) Time evolution of the phase difference between  $\Gamma_{\text{SLIP}_{13}}$  in the range of 1.5 kHz to 3.0 kHz and magnetic fluctuation of RIC mode. The average of the phase is  $185^\circ \pm 50^\circ$ . d) Increment of losses due to RIC modes vs. the relative amplitude of RIC

magnetic fluctuations.