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Experimental Uncertainty and Covariance Information in EXFOR Library

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Abstract. Compilation of experimental uncertainty and covariance information in the EXFOR Library is discussed. Following the presentation of a brief history of information provided in the EXFOR Library, the current EXFOR Formats and their limitations are reviewed. Proposed extensions for neutron-induced reaction cross sections in the fast neutron region and resonance region are also presented.

1 Introduction

Detailed documentation of experimental uncertainty and covariance information is crucial for evaluators who wish to provide their evaluated cross sections with the corresponding covariance matrices based on the experimental knowledge. A typical example is uranium-235 and other actinide neutron-induced fission cross section evaluations performed by least squares fitting to experimental cross sections. Evaluators may use generalized least square fitting codes developed for nuclear data evaluation (*e.g.*, GMA [1], SOK [2]), and can obtain the evaluated cross sections from experimental cross sections and their covariances. Also various approaches used to combine covariances of experimental results and nuclear model calculation results have been proposed [3]. Detailed experimental uncertainty information can often be lengthy and thus not suitable for some types of publications (*e.g.*, journals, conference proceedings). Consequently, the EXFOR Library [4], maintained by the International Network of Nuclear Reaction Data Centres (NRDC) [5] under the auspices of the International Atomic Energy Agency Nuclear Data Section (IAEA-NDS), can serve to provide storage for the relevant information.

In 1978, F.G. Perey showed the importance of documentation [6] by referring to the 2080 keV resonance energy in the neutron induced reaction on carbon measured at Harwell [7]. In this experiment, two energies (2078.31 and 2079.2 keV) were obtained from two time-of-flight paths (50 and 100 m). The independent and common parts of the estimated uncertainties in two flight-paths and two time-of-flight measurements are missing in the original publication. Fortunately, Perey was able to obtain estimates of these partial uncertainties

personally from the author, and could construct a 2x2 covariance matrix. The off-diagonal element of the matrix lies between two diagonal elements (*i.e.*, $V_{11} < V_{12} < V_{22}$), and the resultant least squares solution (2078.27 keV) is lower than the two measured resonance energies thereby demonstrating the PPP phenomenon (Pelle's Pertinent Puzzle). Perey concluded as follows: "We urge experimentalists to report the uncertainties in their measurements in such a fashion that the covariance matrix of their results can be generated. We hope that data compilers in the future will expand their data compilation formats such that this valuable information can be made available".

Now, three decades later, neutron-induced reaction cross sections are often evaluated along with their covariances for general purpose evaluated data libraries. A standardized Guide to the Expression of Uncertainty in Measurement (GUM) [8] has been prepared to provide useful information about estimating uncertainties in data. Also, good text books are available that deal particularly with nuclear data covariances [9-10]. However, the current contents of the EXFOR Library are still not satisfactory for evaluation applications. Recently, at the IAEA Technical Meeting on Neutron Cross Section Covariances (September 2010 [11]), data centres received various recommendations from experts familiar with experiments and evaluations on how to improve the situation, and EXFOR compilers are urged to address this matter in collaboration with experimentalists.

In this report, limitations of the current EXFOR Formats [12] and their extensions for documentation of experimental uncertainties and covariances are discussed for neutron-induced reaction data in the resonance and fast-neutron regions. Detailed discussions with more

examples will be published separately [13]. Note that a portion of the extended formats introduced in this article is still under consideration and will be discussed further at the next NRDC Meeting (April, 2012).

2 Current EXFOR Formats

In 1980, M. Bhat submitted a working paper entitled “Proposal to include detailed information on systematic errors in the international data exchange files” to the 1980 NRDC meeting [14]. The proposal was adopted after a slight modification in 1982 [15] and it has been continuously used until now by the EXFOR compilers without major revisions.

A coding sample based on the format agreed upon at this meeting is shown in Fig. 1. In this example, two constant “systematic” uncertainties $\epsilon_1(E_i)$ and $\epsilon_2(E_i)$ are coded under the data headings ERR-1 and ERR-2. The incident energy E_i and its uncertainty ΔE_i are coded with the corresponding cross section $\sigma(E_i)$ under the data headings EN, EN-ERR and DATA, respectively. The energy dependent “statistical” uncertainty $\epsilon_s(E_i)$ and “systematic” uncertainties $\epsilon_3(E_i)$ are coded under the data headings ERR-S and ERR-3 with the energy dependent total uncertainty $\epsilon_t(E_i)$ under ERR-T. For the last component (ERR-4), only the lower and upper boundaries ϵ_{4min} and ϵ_{4max} are coded. All partial uncertainties must be defined under the keyword ERR-ANALYS.

```

BIB
...
ERR-ANALYS (ERR-T)      total uncertainty (et)
              (ERR-S)      statistical uncertainty (es)
              (ERR-1)      1st systematic uncertainty (e1)
              (ERR-2)      2nd systematic uncertainty (e2)
              (ERR-3,,1.)  3rd systematic uncertainty (e3)
              (ERR-4, e4min, e4max) 4th systematic uncertainty (e4)
...
ENDBIB
COMMON
ERR-1          ERR-2
PER-CENT      PER-CENT
e1            e2
ENDCOMMON
DATA
EN            EN-ERR  DATA  ERR-T  ERR-S  ERR-3
MEV          MEV    MB      PER-CENT PER-CENT PER-CENT
E1           ΔE1    σ(E1)  et(E1)  es(E1)  e3(E1)
E2           ΔE2    σ(E2)  et(E2)  es(E2)  e3(E2)
...          ...    ...    ...    ...    ...
Ei           ΔEi    σ(Ei)  et(Ei)  es(Ei)  e3(Ei)
...          ...    ...    ...    ...    ...
En           ΔEn    σ(En)  et(En)  es(En)  e3(En)
ENDDATA

```

Fig. 1. Current EXFOR format for uncertainties.

The major data headings available for specifying uncertainties of cross sections are summarized in Table 1. Note that only uncertainties that are 1σ standard deviations are coded under these headings, and other values (e.g., 2σ) are converted to 1σ values before entering these data into the EXFOR Library.

Experimentalists often report their cross sections with the “total” uncertainties and fail to specify partial uncertainties. These total uncertainties are useless for covariance analysis, and the compiler is asked to keep the uncertainties under the data heading DATA-ERR (i.e., uncertainties for which details are unknown). However, if the EXFOR compiler is fortunate enough to find all the

partial uncertainties as well as the total uncertainties, then the total uncertainties are coded under the data heading ERR-T with their components under ERR-1, ERR-2 etc. In the past, the importance of the partial uncertainties was not well recognized by some EXFOR compilers, and sometimes they kept only the total uncertainties even if the authors gave partial uncertainties in their report. Following the recommendation from the 2010 IAEA Technical Meeting [11], EXFOR compilers are now attempting to revise the affected EXFOR entries retroactively.

Table 1. Major data headings for uncertainties (present).

Heading	Definitions
ERR-T	Total uncertainty
ERR-S	Statistical uncertainty
ERR-SYS	Total systematic uncertainty
ERR-1	1st systematic uncertainty
ERR-2	2nd systematic uncertainty
DATA-ERR	Data uncertainty (details unknown)
MONIT-ERR	Uncertainty in monitor cross sections

Sometimes EXFOR compilers find reports which are well-suited for covariance evaluation (e.g., [16-17]). Unfortunately the current EXFOR formats cannot accommodate all the information given in such reports. In general, the neutron cross section covariance between two neutron energies E_i and E_j is expressed in terms of the correlation coefficients and partial uncertainties as follows:

$$\begin{aligned}
 \text{cov}(\sigma_i, \sigma_j) &= \sum_{k=1}^n \text{cov}_k(\sigma_i, \sigma_j) \\
 &= \sum_{k=1}^n c_{k,ij} \cdot \Delta_k \sigma_i \cdot \Delta_k \sigma_j
 \end{aligned}
 \tag{1}$$

where $\text{cov}(\sigma_i, \sigma_j)$ and $\text{cov}_k(\sigma_i, \sigma_j)$ are the total covariance and partial covariance due to the k -th source of uncertainty, $c_{k,ij}$ is the micro-correlation coefficient of the k -th partial uncertainty between the two cross sections σ_i and σ_j , and $\Delta_k \sigma_i$ stands for the k -th partial uncertainty for the cross section σ_i . This equation reveals that the evaluators need the correlation coefficients $c_{k,ij}$ in addition to the partial uncertainties $\Delta_k \sigma_i$. For some specific sources of uncertainties, evaluators can assign a proper value to $c_{k,ij}$. For instance, uncertainties coded under the data heading ERR-S may be always treated as uncorrelated (i.e., $c_{k,ij} = 0$). Uncertainty due to the intensity of a gamma line adopted from a decay data table for activation cross section determination can always be regarded as fully correlated (i.e., $c_{k,ij} = 1$). In many cases, however, it is not a trivial task to assign proper values to $c_{k,ij}$ without further experimental information. Correlation due to the sample mass may be treated as fully correlated

only when the evaluator knows the same sample was used at all incident energies. When a correlation coefficient is a constant for a given source, the value can be coded under the keyword `ERR-ANALYS` in accordance with the current rule. But this information has rarely been given by authors, and only a few EXFOR entries adopt this option for compilation of cross sections (*e.g.*, those measured at the ANL fast neutron generator (FNG) [18-19]).

Experimental total covariances or corresponding macro-correlation coefficients,

$$c_{ij} = \text{cov}(\sigma_i, \sigma_j) / (\Delta\sigma_i \Delta\sigma_j), \quad (2)$$

($\Delta\sigma_i$ is the total uncertainty of σ_i) can be coded under the keyword `COVARIANCE`. In the current format rule, however, the matrix elements are treated as free text and therefore are not suitable as inputs to computer programs, especially when the matrix is huge (*e.g.*, covariances of high resolution time-of-flight data in the resolved resonance region). Also, updating of the matrix to account for changes of the partial uncertainties (*e.g.*, revision of the standard cross section with its covariance) is not easy if covariance matrices of partial uncertainties are not explicitly given in the EXFOR Library.

3 Extensions of the EXFOR Formats

In order to improve the capability of the EXFOR Formats to adequately represent cross section data uncertainties and covariances, various extensions of EXFOR formats are under consideration. Hereafter, we limit our discussion to neutron cross sections in the fast neutron and resonance regions. The reader can refer to [13] for more details.

3.1. Covariances in the Fast Neutron Region

As discussed in the previous section, the EXFOR formats must be extended to record the correlation properties of the partial uncertainties. The following two extensions were proposed in the last NRDC Meeting (Vienna, May 2011) [20].

1. An indication should be provided for the correlation property for each source of the partial uncertainty by flags (`F` for “fully correlated”, `U` for “uncorrelated”, `P` for “partially correlated”, `C` for “correlated” where the range is unknown). Note that `F`, `U` and `P` are also known as “long energy range correlation (LERC)”, “short energy range correlation (SERC)” and “medium energy range correlation (MERC)” in neutron-induced reaction cross section uncertainties [10].
2. The total and partial covariances (or corresponding macro and micro-correlation coefficients) should be included in computer-readable form.

The EXFOR users are able to derive the full covariance matrix if the experimentalists provide all the necessary

information, and the EXFOR compilers record it in this extended format.

A coding sample for the fast neutron region is given in Fig. 2. This sample shows a compilation of the $^{241}\text{Am}(n,2n)^{240}\text{Am}$ cross section measured at the IRMM Van de Graaff laboratory [17,21]. In this example, the total uncertainty and 9 partial uncertainties (`MONIT-ERR`, `ERR-1`, `ERR-2`, ..., `ERR-8`) are defined under the keyword `ERR-ANALYS` with their correlation properties (`F`, `U`, `P`). There are 2 partially correlated uncertainties (`MONIT-ERR`, `ERR-5`), and their micro-correlation coefficients are coded under the keyword `COVARIANCE` explicitly, along with the macro correlation coefficients.

```

BIB
REACTION (94-AM-241(N,2N)94-AM-240,,SIG)
...
ERR-ANALYS (MONIT-ERR,,P) 27Al(n,a) standard cross section (1.6-5.4%)
(ERR-1,,U) Counting of 240Am activity (1.4-6.3%)
(ERR-2,,U) Counting of 24Na activity (0.7-2.0%)
(ERR-3,,F) Intensity of 240Am gamma line (1.2%)
(ERR-4,,U) Number of 27Al in sample (0.1%)
(ERR-5,,P) Number of 241Am in sample (0.3%)
(ERR-6,,F) 24Na/240Am efficiency ratio (3.0%)
(ERR-7,,F) Correction for decay of 240Am (0.4-0.9%)
(ERR-8,,U) Correction for secondary neutron (<1.4%)
COVARIANCE (COR,ERR-T,PER-CENT) macro-correlation
100
35 100
37 42 100
38 43 53 100
40 45 57 58 100
41 45 57 59 84 100
21 24 30 31 39 39 100
30 34 44 45 58 59 51 100
20 22 29 30 40 42 39 65 100
(COR,MONIT-ERR,PER-CENT) correlation due to standard
100
43 100
0 0 100
0 0 6 100
0 0 9 12 100
0 0 11 12 100 100
0 0 11 11 40 40 100
0 0 11 11 40 40 100 100
100 0 11 11 40 40 100 100 100
(COR,ERR-5,PER-CENT) micro-correlation due to sample mass
100
0 100
0 100 100
0 100 100 100
0 0 0 0 100
100 0 0 0 0 100
0 0 0 0 100 0 100
0 0 0 0 0 0 0 100
100 0 0 0 0 100 0 0 100

ENDBIB
COMMON
ERR-3 ERR-4 ERR-5 ERR-6
PER-CENT PER-CENT PER-CENT PER-CENT
1.2 0.1 0.3 3.0
ENDCOMMON
DATA
EN DATA ERR-T MONIT-ERR ERR-1 ERR-2
ERR-7 ERR-8
MEV MB PER-CENT PER-CENT PER-CENT PER-CENT
PER-CENT PER-CENT
8.34 96.8 6.5 1.9 5.0 1.0
0.9
9.15 162.9 5.7 1.9 4.0 1.0
0.6
13.33 241.8 4.6 1.6 2.5 1.0
0.4 0.3
16.1 152.4 4.6 2. 2.1 1.0
0.6 0.3
17.16 116.1 4.4 2. 1.5 1.0
0.6 0.3
17.9 105.7 4.4 2.2 1.3 0.7
0.7 0.3
19.36 89.5 8.2 3.1 6.3 2.0
0.6 1.3
19.95 102.1 5.8 4.1 1.4 1.0
0.6 1.4
20.61 77.9 8.8 5.4 5.7 1.6
0.6 1.4
ENDDATA

```

Fig. 2. Coding sample of uncertainties and covariances using the extended EXFOR formats (fast neutron region).

Fig. 3 compares macro-correlations c_{ij} derived from the full information of micro-correlation (upper panel) provided by the authors with the results obtained by

assuming the total systematic error is fully correlated (lower panel). In other words, in the latter case the assumption is adopted that $c_{k,ij}=1$ except for the uncertainty due to counting statistics. The main correlations are well represented by the simplifying assumption for this specific example.

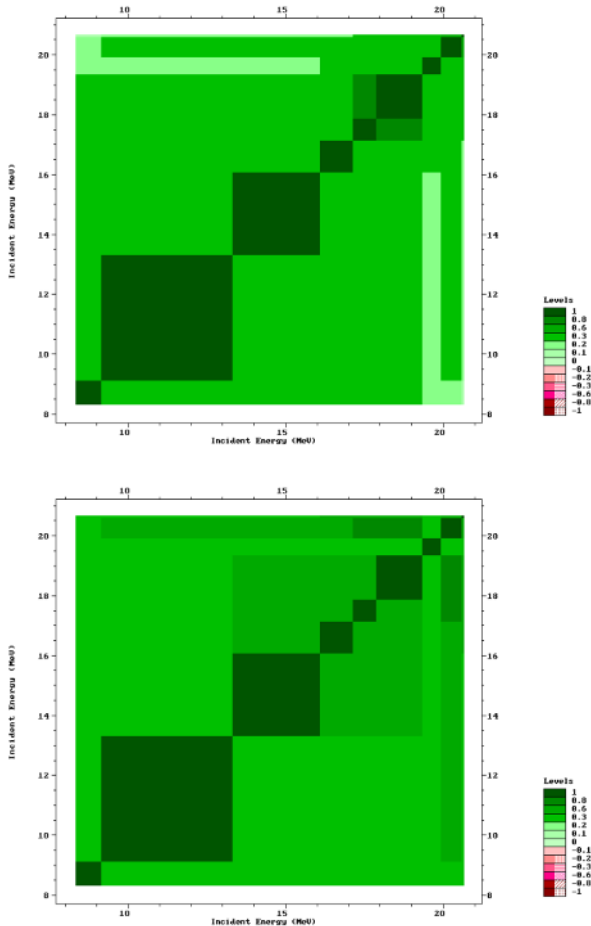


Fig. 3. Macro-correlations constructed using the full information obtained from the authors (upper panel) and with assumption of full correlation ($c_{k,ij}=1$) for total systematic uncertainties (lower panel).

Table 2. Major data headings for uncertainties (future).

Heading	Definitions
ERR-T	Total uncertainty
ERR-S	Total statistical uncertainty
ERR-SYS	Total systematic uncertainty
ERR-1	1st <i>partial</i> uncertainty
ERR-2	2nd <i>partial</i> uncertainty
DATA-ERR	Data uncertainty (details unknown)
MONIT-ERR	Uncertainty in monitor cross sections

There has been considerable variety in the uncertainty terminology used by experimentalists, and this makes it difficult to achieve consistency in EXFOR entries. EXFOR has categorized uncertainty components as “statistical” (ERR-S) and “systematic” (ERR-1, ERR-2, ...), but this approach is not recommended in GUM [8]. From the perspective of evaluating uncertainties and covariances, the specification of micro-correlation properties ($c_{k,ij}$) is crucial, as is evident from Eq. (1), and the current formats must store uncorrelated component under ERR-S and correlated components under ERR-1, etc, effectively in line with [8]. In the extended formats, the correlation properties will be indicated by the new flags (F, U, P, C) while the data headings will function as indicated in Table 2.

3.2. Covariance in the Resonance Region

Covariances of time-of-flight spectra Z (transmission, reaction yield, self indication) are fundamental experimental information needed to evaluate covariances of resonance parameters. The number of high resolution time-of-flight bins m is typically 10^4 . Its covariance matrix becomes huge, and it is not realistic to store all partial covariance matrices (or corresponding micro correlation coefficients) in the EXFOR Library. Clearly, a different approach is required.

One approach that can be taken under these circumstances involves what is known as a Cholesky decomposition. In general, the full covariance matrix V_Z for a spectrum Z obtained from an analysis involving a parameter vector $\vec{\alpha}$ with dimension n is expressed as

$$V_Z = M_Z M_Z^T + S_\alpha V_\alpha S_\alpha^T, \quad (3)$$

where M_Z is a diagonal $m \times m$ matrix consisting of uncorrelated uncertainty components, while V_α ($n \times n$ matrix) and $S_\alpha = \partial Z / \partial \vec{\alpha}$ ($m \times n$ matrix) are the covariance matrix and the sensitivity (functional) matrix of Z to $\vec{\alpha}$. Because the covariance matrix V_α is positive definite, there is an $n \times n$ triangular matrix L_α which satisfies $V_\alpha = L_\alpha L_\alpha^T$ (Cholesky decomposition). Finally, the covariance matrix can be written as

$$V_Z = M_Z M_Z^T + D_\alpha D_\alpha^T \quad (4)$$

with an $m \times n$ matrix $D_\alpha = S_\alpha L_\alpha$. Such a decomposition of the covariance matrix offers an efficient tool to compile time-of-flight covariance matrices in EXFOR (*i.e.*, the AGS format [22-25]). Actually M_Z and D_α are $m \times 1$ and $m \times n$ matrices, and they can be treated as typical partial uncertainties of $(1+n)$ sources at m neutron energies in the EXFOR formats.

```

BIB
REACTION (48-CD-0(N,TOT),,TRN)
...
ERR-ANALYS (ERR-T) Total uncertainty (1 sigma)
           (ERR-S) Uncorrelated uncertainty (1 sigma)
           (ERR-1) Correlation dead time correction (sample)
           (ERR-2) Correlation background correction (sample)
           (ERR-3) Correlation dead time correction (open beam)
           (ERR-4) Correlation background correction (open beam)
COVARIANCE (CHLSK) Compiled in ERR-1 to ERR-4 in the AGS format

```

```

...
ENDBIB
DATA
TOF-MIN      TOF-MAX      DATA      ERR-T      ERR-S      ERR-1
ERR-2      ERR-3      ERR-4
NSEC      NSEC      NO-DIM      NO-DIM      NO-DIM
NO-DIM      NO-DIM      NO-DIM
873301.2      873429.2      1.1478      6.65562E-2      6.65376E-2      1.33447E-5
-9.12646E-4      -1.37145E-5      1.28552E-3
873429.2      873557.2      0.97025      5.63176E-2      5.63021E-2      1.09942E-5
-8.47529E-4      -1.16680E-5      1.00913E-3
...
ENDDATA

```

Fig. 4. Coding sample of uncertainties and covariances in the extended EXFOR formats (resonance region). Only two lines of a spectrum of 25 288 time-of-flight bins are shown.

A coding sample for the covariances of time-of-flight spectra in the resonance region is shown in Fig. 4, where the Cd+n transmission spectra with their uncertainties measured at the IRMM GELINA facility [26] are given. Uncertainties coded under `ERR-S` and `ERR-i` ($i = 1$ to 4) in this coding sample correspond to M_z and D_α in Eq. (4), respectively, and the EXFOR data users can treat them as uncorrelated ($c_{k,ij} = 0$) and fully correlated ($c_{k,ij} = 1$) uncertainties in Eq. (1).

4 Summary

The present paper has discussed the current EXFOR formats, as well as their extension to enable better documentation and more complete representation of experimental uncertainties and covariances than was possible with the existing scheme. In order to provide useful information on experimental uncertainties and covariances to neutron cross section evaluators and other EXFOR data users, within the framework of the extended formats, experimentalists are encouraged to provide detailed information on energy dependent partial uncertainties and their correlation properties. Primary publications of experimental works published as journal articles and conference proceedings are often not suitable to record such detailed information because the numerical data tables may be too lengthy. Reports published by laboratories or data centres (e.g., the INDC report series published by IAEA Nuclear Data Section) may be adequate to publish such supplemental numerical data tables and other information which can be included in the EXFOR entries. Close collaboration between experimentalists and EXFOR compilers may be required for this to be feasible in practice. Furthermore, the development of useful software tools to process information compiled in the extended formats is important. To this end, the IAEA Nuclear Data Section is developing a web-based system to construct and visualize macro-correlation coefficients based on partial uncertainties compiled in the EXFOR Library [27].

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