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Neutron Generation Time in Highly-Enriched Uranium Core at Kyoto University Critical Assembly

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ABSTRACT

At the Kyoto University Critical Assembly, experiments on kinetics parameters are carried out at near-critical configurations, super-critical and subcritical states, in the thermal neutron spectrum made with a highly-enriched uranium fuel. The main calculated kinetics parameters, the effective delayed neutron fraction (β_{eff}) and the neutron generation time (Λ) are used effectively for the estimation of experimental parameters, and the accuracy of experiments on prompt neutron decay constant (α) and subcriticality (ρ_s) in dollar units is attained by the numerical results of β_{eff} and Λ . Furthermore, the value of β_{eff} / Λ is experimentally deduced with the use of the experimental results of α and ρ_s , ranging between 250 and -80 pcm. Thus, the experimentally deduced values of β_{eff} / Λ that reveal good accuracy through a comparison with those by the MCNP6.1 calculations with JENDL-4.0 are then taken as an index of Λ by introducing an acceptable assumption of β_{eff} at near-critical configurations. From the results of experimental and numerical analyses, the experimental value of β_{eff} / Λ is important for the validation of Λ , since kinetics parameters are successfully obtained from the clean cores of near-critical configurations in the thermal neutron spectrum.

KEYWORDS: *Neutron generation time, Highly-enriched uranium core, KUCA, Near critical, Thermal neutron spectrum*

I. INTRODUCTION

Kinetics parameters in nuclear reactor physics that include the effective delayed neutron fraction (β_{eff}) and neutron generation time (Λ) are important in ascertaining safe operation of nuclear reactor cores. At the critical state in a reactor core, excess reactivity and control rod worth are experimentally attained by the positive period method [1] and the rod drop method [2], respectively, after obtaining the values of β_{eff} and neutron life time (ℓ) by numerical calculations. Also, at the subcritical state, prompt neutron decay constant (α) and subcriticality in dollar units (ρ_s) are measurable by the pulsed-neutron source (PNS) method [3] and the noise methods (the Feynman- α method [4] and the Rossi- α method [5]). Among these reactor physics parameters, however, both β_{eff} [6] and Λ are considered difficult to obtain directly by reactor physics experiments at super-critical, critical and subcritical states.

In the accelerator-driven system (ADS) experiments at the Kyoto University Critical Assembly (KUCA), real-time measurements [7]-[8] of α and ρ_s were made in attempts to satisfy sufficient reliability of the measured accuracy of experimental results. Also, at KUCA, experimental analyses of β_{eff} were conducted by Monte Carlo calculations with the use of ADS experimental results, demonstrating the dependence of external neutron source [9] and the measurement methodology [10] with respect to β_{eff} . Moreover, a variation in the ratio of β_{eff} and Λ (β_{eff} / Λ) obtained by the α -fitting method [11] was experimentally examined on subcriticality in hard neutron spectrum cores [12]-[14], with the combined use of the experimental results of α and ρ_s . Numerical analyses of β_{eff} and Λ were conducted with Monte Carlo calculations together with the use of experimental data obtained at other ADS facilities [15]-[16].

Among many benchmark problems of kinetics parameters, the GODIVA I experiments [17]-[20] are very useful in determining the numerical precision of kinetics parameters, particularly those including β_{eff} and Λ , by stochastic and deterministic approaches. In the experimental database [20], kinetics parameters obtained by the GODIVA I experiments are

interestingly prepared, particularly, for a very simple core configuration that is available to model easily by numerical calculations, and used for validation of the transport calculation codes. The main characteristics of the GODIVA I experiments are a sphere system with one region and a fast neutron spectrum core with highly-enriched uranium (HEU), for providing experimental results of near-critical configurations ranging between about 450 (excess reactivity) and -150 (subcriticality) pcm. Also, kinetics parameters $\beta_{eff} = 680$ pcm and $\Lambda = 6.2$ ns are different from those in a thermal spectrum core. Meanwhile, in ADS experiments at KUCA, kinetics parameters of α and ρ_s were successfully obtained with the combined use of the KUCA core and external neutron sources (14 MeV neutrons and 100 MeV protons). Here, the KUCA core was composed of the HEU fuel and a polyethylene moderator (**Table I**), with a thermal neutron spectrum. Interestingly, using parameters α and ρ_s in the KUCA core, the values of β_{eff} / Λ were deduced at near-critical configurations ranging between 250 and -80 pcm through experimental analyses, and compared well with Monte Carlo calculations.

To estimate the numerical precision of Λ , the value of β_{eff} / Λ was used as an index of Λ evaluation that is defined by a ratio of Λ values in super-critical and subcritical states. The main objectives of this study were to examine the validity of evaluating Λ with the use of the value of β_{eff} / Λ , and providing a relative value obtained by the two results of super-critical and subcritical states. The details of experimental settings are presented in Sec. II, and the results of the comparison between experiments and calculations are discussed in Sec. III. Finally, the conclusions are summarized in Sec. IV.

II. EXPERIMENTAL SETTINGS

II.A. Core Configurations

Among three cores (termed A, B and C) at KUCA, A and B are polyethylene

solid-moderated and solid-reflected cores, and C is a light water-moderated and water-reflected one. The three cores are operated at zero power in the normal operating state. Critical cores set in the A-core (**Figs. 1 and 2**) have polyethylene moderator and reflector rods, and three different fuel assemblies: normal “F” (**Fig. 3**), partial “8” and “4” (**Figs. 4 and 5**) corresponding to Figs. 1 and 2, respectively. The normal fuel assembly “F” is composed of 60 unit cells, and upper and lower polyethylene blocks about 23” and 21” long, respectively, in an aluminum (Al) sheath (2.1×2.1×60”). A unit cell is composed of two HEU fuel plates 2×2” square and 1/8” thick (1/16”×2), polyethylene (p) plate 2×2” square and 1/8” thick, for normal fuel plate “F.” Numeral “8” represents a partial fuel assembly composed of 8 unit cells, with two HEU fuel plates and a polyethylene plate as in the normal fuel assembly, providing 52 unit cells of two Al plates 2×2” square and 1/8” thick (1/16”×2), and 1/8” polyethylene plates. Also, numeral “4” corresponds to 4 unit cells of fuel assembly with 56 unit cells composed of Al and polyethylene plates.

II.B. Experiments

In the two critical cores shown in Figs. 1 and 2, criticality was reached at positions of control (C1, C2 and C3) and safety (S4, S5 and S6) rods for the total number of HEU fuel plates: 3016 and 3008, as shown in **Tables II and III**, respectively. Excess reactivity and control rod worth in pcm units were then experimentally obtained by the positive period method and the rod drop method, respectively, with the use of two kinetics parameters β_{eff} and ℓ estimated by numerical calculations. Here, excess reactivity obtained experimentally was about 250 and 100 pcm in Figs. 1 and 2, respectively, and control rod worth of C1, C2 (Fig. 1 or C3 in Fig. 2) and C3 (Fig. 2 or C2 in Fig. 1) was about 880, 150 and 520 pcm, respectively.

In the ADS core (3000 HEU fuel plates) at the subcritical state, deuteron beams were injected onto a tritium target set at (14-15, X) in **Fig. 6**. In the same subcritical core, another external neutron source of 100 MeV proton beams was injected onto a lead-bismuth (Pb-Bi)

target set at (15, H) in **Fig. 7**. Two different external neutron sources (14 MeV neutrons; 100 MeV protons: spallation neutrons) were separately injected into the subcritical core: deuteron beams, 0.1 mA current, 20 Hz repetition rate, 97 μs width and $1 \times 10^5 \text{ s}^{-1}$ neutron yield; 100 MeV protons, 0.1 nA current, 20 Hz repetition rate, 100 ns width and $1 \times 10^7 \text{ s}^{-1}$ neutron yield. The Pb-Bi target was 50 mm in diameter and 18 mm thick. In a series of ADS experiments and during the injection of external neutron source, α and ρ_s were measured by the least-square fitting method and the extended area ratio method [21], respectively.

Moreover, to examine the effects of detector position dependence [22] and external neutron source spectrum on measurement results, three optical fiber detectors [23] were set at (14-15, L-M); (13-14, K-L); (12-13, J-K) in **Fig. 6** and (15-16, O-Q); (16-17, Q-R); (18-19, S-T) in **Fig. 7**; also, three BF-3 detectors were at (15, H); (11, I); (11, M) in **Fig. 6** and (15, U); (11, T); (10, O) in **Fig. 7**.

II.C. Kinetics Parameters

For the α -fitting method [11], α is easily deduced by combining subcriticality ($-\rho_{\text{pcm}}$) in pcm units, β_{eff} and Λ as follows:

$$\alpha = \frac{\beta_{\text{eff}} - \rho_{\text{pcm}}}{\Lambda} \quad (1)$$

The relationship between ρ_{pcm} and ρ_s is expressed with the use of β_{eff} as follows:

$$\rho_{\text{pcm}} = \rho_s \beta_{\text{eff}} \quad (2)$$

Substituting Eq. (2) for Eq. (1), $\beta_{\text{eff}} / \Lambda$ can be expressed as follows:

$$\frac{\beta_{eff}}{\Lambda} = \frac{\alpha}{1-\rho_s} \quad (3)$$

The value of β_{eff} / Λ is easily deduced from the experimental results of α and ρ_s .

III. RESULTS AND DISCUSSION

III.A. Eigenvalue Calculations

For transport, numerical calculations were performed by the Monte Carlo transport code, MCNP6.1 [24] together with nuclear data libraries, JENDL-4.0 [25] and ENDF/B-VII.1 [26]. Here, in MCNP6.1, since the effects of reactivity by neutron detectors (optical fiber detectors, BF-3 detectors, fission chambers and uncompensated ionization chambers) and control (safety) rods are not negligible, neutron detectors and control (safety) rods were modeled precisely in the simulated geometry and transport calculations. The precision of numerical criticality in pcm units was attained by the eigenvalue calculations with a total of 5×10^8 histories (1×10^5 histories per cycle, 5×10^3 active cycles and 1×10^2 skip cycles) and a statistical error of about 4 pcm. Also, main kinetics parameters, β_{eff} , Λ and Rossi- α (termed β_{eff} / Λ in MCNP6.1) values, were obtained by the eigenvalue calculations, when obtaining the effective multiplication factor by the k-code option.

To confirm the numerical precision of eigenvalue calculations by MCNP6.1, excess reactivity and control rod worth (C1, C2 and C3) were compared with those obtained from the experiments by the positive period method and the rod drop method, respectively. The MCNP eigenvalue calculations in critical state are not always unit, differing from the experimental results in critical state obtained by nuclear data accuracy and uniformed number density

approximation in core materials, although a critical core configuration is more closely simulated by MCNP. Thus, excess reactivity $\rho_{\text{excess}}^{\text{cal}}$ in pcm units was numerically deduced by the difference between two effective multiplication factors k_{critical} and $k_{\text{super-critical}}$, in critical and super-critical (clean) cores, respectively, as follows:

$$\rho_{\text{excess}}^{\text{cal}} = \frac{1}{k_{\text{critical}}} - \frac{1}{k_{\text{super-critical}}} . \quad (4)$$

Also, control rod worth was numerically deduced by the difference between the critical and the subcritical (rod insertion) cores, as in Eq. (4).

Using the kinetics parameters obtained by MCNP6.1, experimental reactivity ρ^{exp} was deduced and compared with numerical reactivity ρ^{cal} obtained by JENDL-4.0 and ENDF/B-VII.1, as shown in **Tables IV** through **VII**, with the consideration of statistical errors of neutron counts obtained from the neutron detectors and processing of error propagation caused by experimental analyses. For the critical cores (3016 and 3008 HEU plates in Tables II and III) shown in Figs. 1 and 2, respectively, numerical reactivity obtained by JENDL-4.0 revealed a fairly good agreement with the experimental reactivity within a relative difference around 5% through the C/E (calculation/experiment) value of excess reactivity and control rod worth, as shown in Tables IV and VI. In terms of ENDF/B-VII.1, the difference between numerical and experimental reactivity was relatively large over 10%, as shown in Tables V and VII. The difference between β_{eff} values by JENDL-4.0 and ENDF/B-VII.1 was attributable to numerical results by the MCNP calculations in fraction β_i ($i = 1$ to 6: precursor group of delayed neutrons) and decay constant λ_i of delayed neutrons, as shown in **Figs. 8** and **9**, respectively.

As shown in Tables IV through VII, numerical results (eigenvalue calculations) by MCNP6.1 with JENDL-4.0 demonstrated good agreement with the experimental data of

reactivity in the critical cores. JENDL-4.0 was taken as a reference library by comparing it with ENDF/B-VII.1.

III.B. Experimental Analyses of β_{eff} / Λ

Kinetics parameters β_{eff} , Λ and Rossi- α , and k_{eff} values were obtained by the MCNP eigenvalue calculations and compared with those of JENDL-4.0 and ENDF/B-VII.1, for critical and near-critical states, as shown in **Tables VIII, IX and X**. For the two critical cores shown in Figs. 1 and 2, JENDL-4.0 demonstrated a small difference between the super-critical (Table VIII) and the critical (Table IX) states, because of the slight difference in the number of fuel plates: 3016 and 3008. Also, in terms of ENDF/B-VII.1, the three kinetics parameters were almost the same in the two states.

At the subcritical state with 3000 fuel plates, three kinetics parameters showed a meaningful change from the critical state, although the subcriticality was very small and around the criticality (Table X). Kinetic parameters α and ρ_s were then obtained by the extended area ratio method and the least-square fitting method, respectively, by varying the external neutron source: 14 MeV neutrons (**Table XI**) and spallation neutrons (**Table XII**). Also, as shown in **Tables XI and XII**, detector position dependency revealed interestingly: the results of Fiber #1 and BF-3 #3 were rather good, and on the contrary, those of Fiber #2, BF-3 #1 and BF-3 #3 looked problematic.

On the basis of the experimental results of α and ρ_s , the value of $(\beta_{eff} / \Lambda)^{exp}$ was experimentally deduced by Eq. (3) for 14 MeV neutrons (Table XI) and spallation neutrons (Table XII), and compared with that of $(\beta_{eff} / \Lambda)_{J40}^{cal}$ obtained by MCNP6.1 together with JENDL-4.0. With the 14 MeV neutrons, since subcriticality in pcm units was found to be near-critical, about 16.7 ± 4.5 pcm, the value of $(\beta_{eff} / \Lambda)_{J40}^{cal}$ was observed the same as that of α .

The result was considered valid at a shallow subcritical state. Among optical fibers and BF-3 detectors, Fiber #1 revealed a fairly good agreement with the MCNP calculation within a relative difference of 4% in the C/E value (Table XI), which was attributable to the location of Fiber #1 near the center of the core (Fig. 1): the position dependence caused by the spatial effect was very small on the experimental results of α and ρ_s . Also, with the spallation neutrons (Table XII), Fiber #1 demonstrated the same accuracy about 2% in the C/E value as with 14 MeV neutrons, although the subcriticality in pcm units was about 78.4 ± 2.2 pcm. The value of $(\beta_{eff} / \Lambda)^{exp}$ was found to be experimentally valid in the deduction of kinetics parameters, through a comparison between the experiments and the MCNP6.1 calculations with JENDL-4.0 (Tables XI and XII).

III.C. Discussion

Experimental analyses of the ADS with spallation neutrons at KUCA have clearly demonstrated that the value of β_{eff} has a little effect on the evaluation of subcriticality in pcm units converted from that in dollar units, when compared with the results of numerical subcriticality in pcm units [4]. In the present study, considering that the values of β_{eff} are almost the same in the near-critical configurations, particular attention was directed to kinetic parameter Λ obtained by combining α and ρ_s .

The values of β_{eff} by MCNP6.1 with JENDL-4.0 in the super-critical (3008 HEU fuel plates) and subcritical (3000 plates) configurations, were 812 ± 10 and 806 ± 10 pcm, as shown in Tables IX and X, respectively. Since the values of β_{eff} are almost the same and within the allowance of experimental uncertainty, the ratio of $(\beta_{eff} / \Lambda)_{super-critical}^{cal}$ and $(\beta_{eff} / \Lambda)_{subcritical}^{cal}$ in the near-critical configurations is approximated as follows:

$$\left(\beta_{eff} / \Lambda\right)_{super-critical}^{cal} / \left(\beta_{eff} / \Lambda\right)_{subcritical}^{cal} \approx \left(\Lambda_{subcritical} / \Lambda_{super-critical}\right)^{cal} . \quad (5)$$

Here, Eq. (5) is defined as “Lambda ratio” that is a relative value of Λ in the near-critical configurations. From the results of Tables XI and XII, assuming that the value of $(\beta_{eff} / \Lambda)_{subcritical}^{cal}$ in the subcritical state by MCNP6.1 is equal to that of $(\beta_{eff} / \Lambda)^{exp}$ by the experiment, Eq. (5) can finally be written as follows, by substituting $(\beta_{eff} / \Lambda)_{subcritical}^{cal}$ for $(\beta_{eff} / \Lambda)^{exp}$ and applying again the assumption with respect to β_{eff} in Eq. (5):

$$(\beta_{eff} / \Lambda)_{super-critical}^{cal} / (\beta_{eff} / \Lambda)^{exp} \approx \Lambda^{exp} / \Lambda_{super-critical}^{cal} \quad (6)$$

On the basis of the validity of $(\beta_{eff} / \Lambda)^{exp}$ mentioned in Sec. III.B., Eq. (6) can be interpreted as a kind of Lambda ratio between the experiments and the calculations, with the combined use of $(\beta_{eff} / \Lambda)_{super-critical}^{cal}$ and $(\beta_{eff} / \Lambda)^{exp}$. Actually, the value of $(\beta_{eff} / \Lambda)^{exp}$ obtained by the ADS experiments at KUCA can be used as an index of Λ in the near-critical configurations, and is, indeed, expected to be applied for the validation of Λ , by stochastic and deterministic calculations, at near-critical configurations in a thermal neutron spectrum core.

IV. CONCLUSIONS

Main kinetics parameters, α and ρ_s , were experimentally obtained from the KUCA core, and β_{eff} and Λ , were numerically validated by MCNP6.1, at the near-critical configurations: super-critical and subcritical states. The numerical result of $(\beta_{eff} / \Lambda)^{cal}$ by MCNP6.1 together with JENDL-4.0 compared well with the experimental result of $(\beta_{eff} / \Lambda)^{exp}$ with fairly good accuracy within a relative difference of 4%, while a discrepancy over 10% was shown in other results. The experimental value of $(\beta_{eff} / \Lambda)^{exp}$ was then available for use as an index of Λ in the near-critical configurations, with an attempt at validation of Λ by the numerical calculations.

From the results of experimental and numerical analyses, the importance of the experimental value of $(\beta_{eff} / \Lambda)^{exp}$ was emphasized for the verification of Λ , since the kinetics parameters were successfully obtained from the clean cores of near critical configurations (super-critical and subcritical states) in the thermal spectrum core. Also, a series of experimental data obtained at KUCA, including super-critical, critical and subcritical configurations, was potentially conducive to estimating statics and kinetics parameters obtained by the numerical calculations, for nuclear design calculations, such as for thermal spectrum cores.

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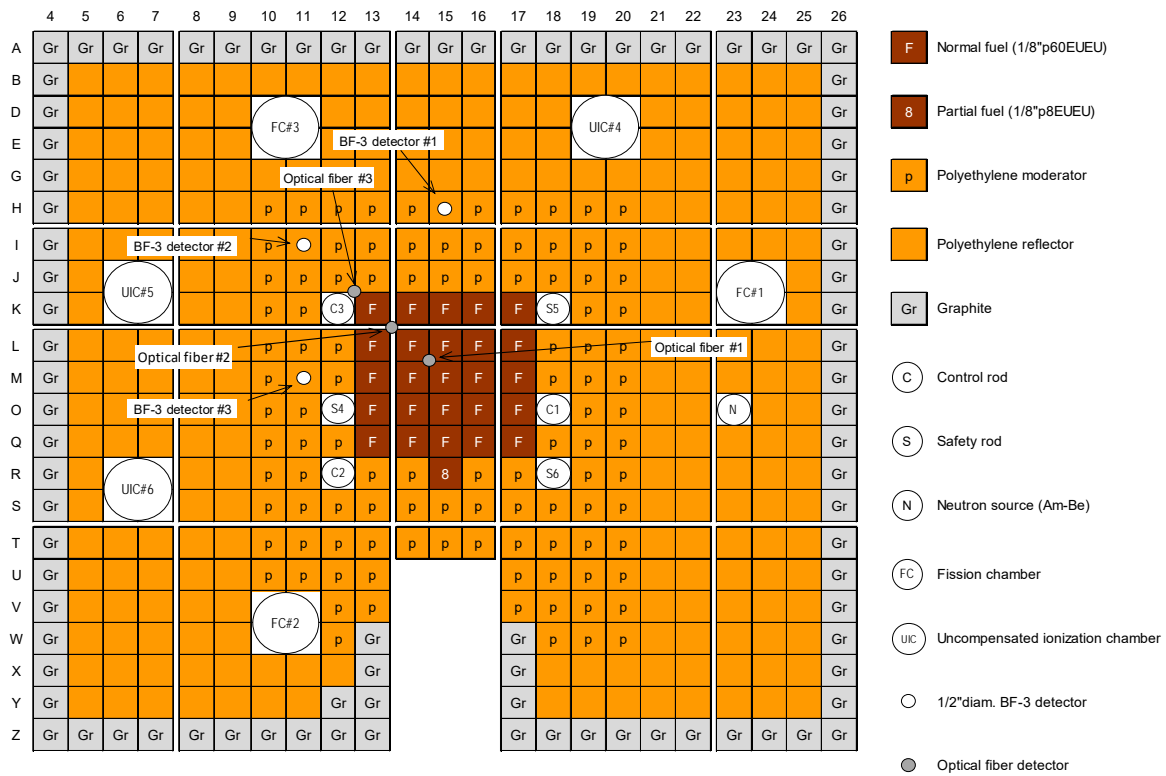


Fig. 1 Top view of KUCA A-core
 (# of HEU plates: 3016; critical core in Table II)

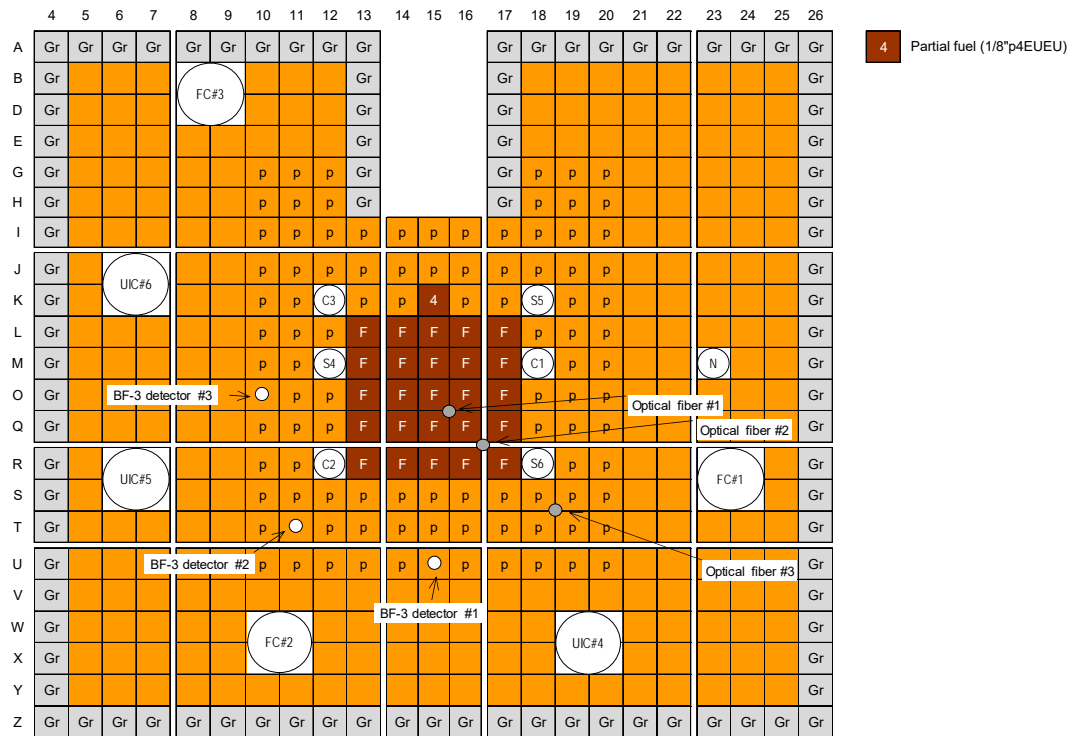


Fig. 2 Top view of KUCA-A core
(# of HEU plates: 3008; critical core in Table III)

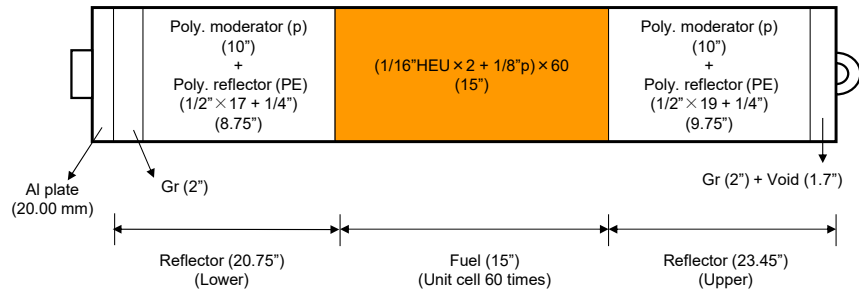


Fig. 3 Schematic drawing of “F” normal fuel rod (1/8”p60EUEU) in Fig. 1

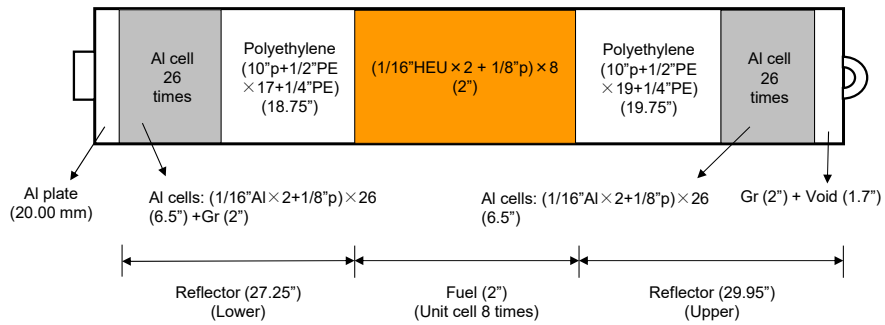


Fig. 4 Schematic drawing of “8” partial fuel rod (1/8”p8EUEU) in Fig. 1

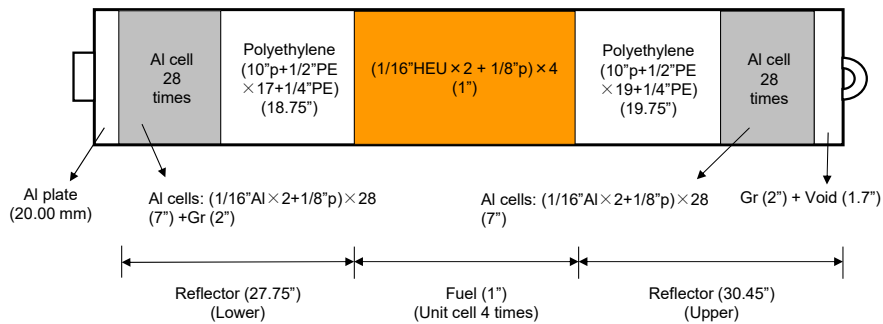


Fig. 5 Schematic drawing of “4” partial fuel rod (1/8”p4EUEU) in Fig. 2

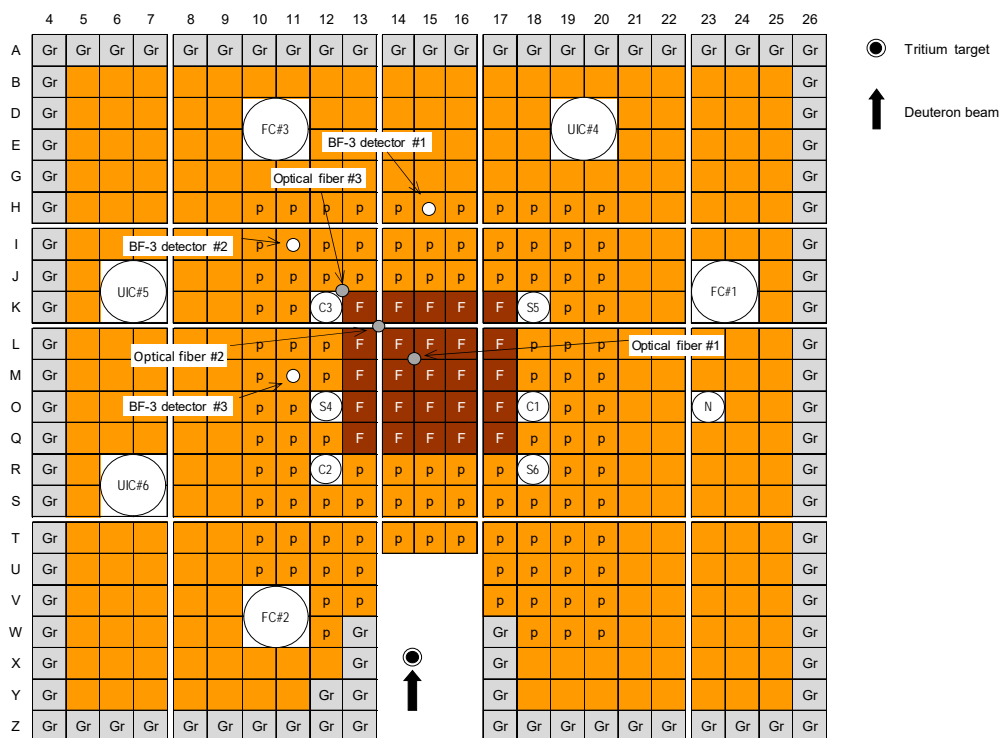


Fig. 6 Top view of ADS core with 14 MeV neutrons
(# of HEU plates: 3000; subcritical core)

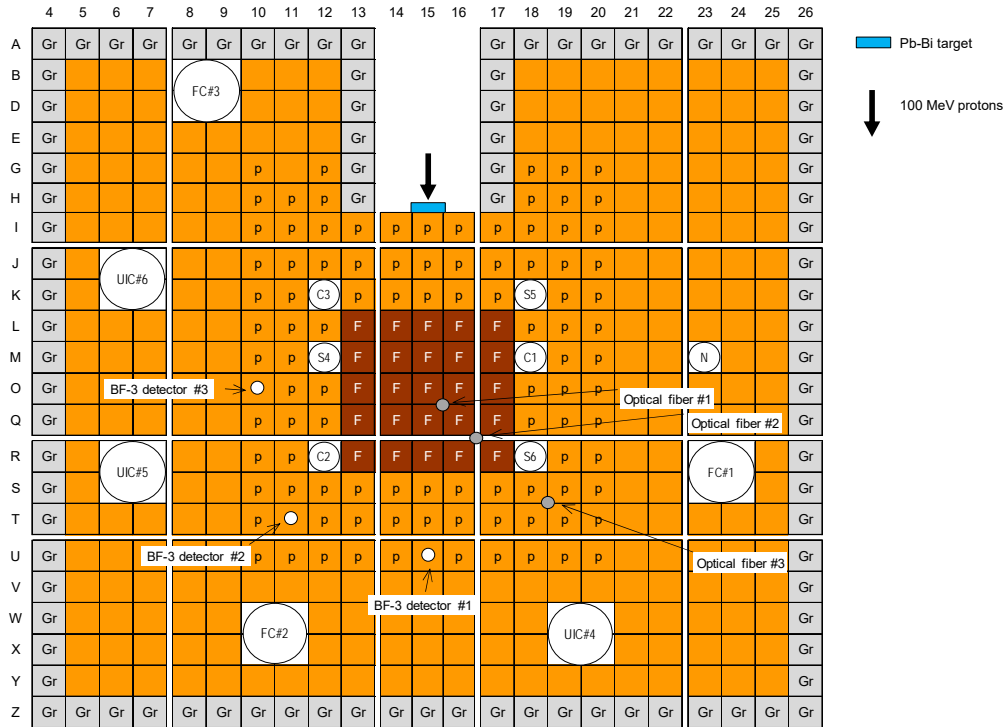


Fig. 7 Top view of ADS core with 100 MeV protons (Pb-Bi target)
(# of HEU plates: 3000; subcritical core)

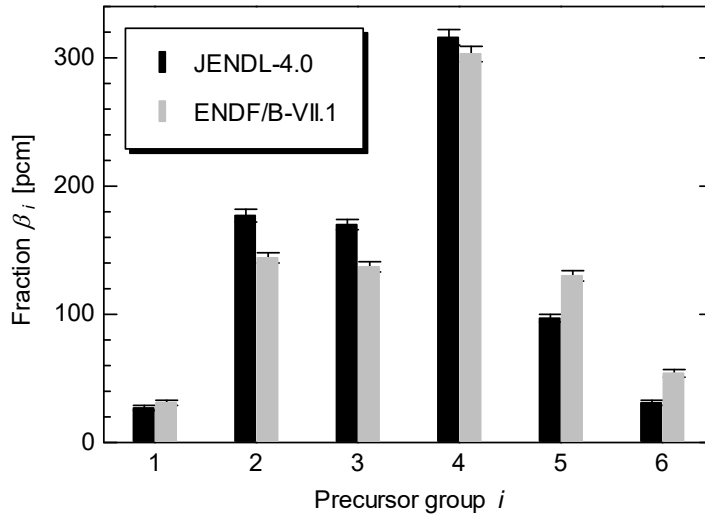


Fig. 8 Comparison between fraction β_i ($i = 1$ to 6) by JENDL-4.0 and ENDF/B-VII.1
(# of HEU plates: 3016; critical state in Fig. 1 and Table II)

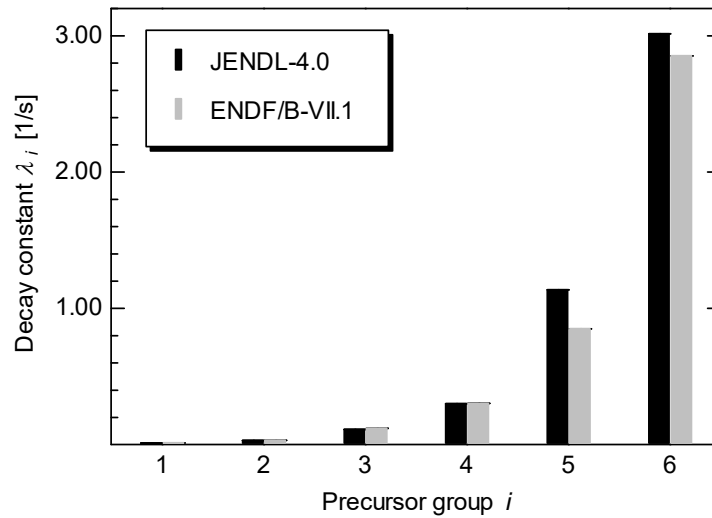


Fig. 9 Comparison between decay constant λ_i ($i = 1$ to 6) by JENDL-4.0 and ENDF/B-VII.1
(# of HEU plates: 3016; critical state in Fig. 1 and Table II)

Table I Dimensions of components in fuel assemblies

Component	Thickness and length [mm]
1/16"HEU	1.5875
1/8"p	3.158
10"p	254.00
1/4"PE	6.300
1/2"PE	12.500
2"Gr	50.80
1/16"Al	1.5875

Table II Positions of control and safety rods at critical state [mm]

(# of HEU plates: 3016; critical core in Fig. 1)

Control rod			Safety rod		
C1	C2	C3	S4	S5	S6
1200.00	1200.00	630.01	1200.00	1200.00	1200.00

(1200 [mm]: Position of upper limit)

Table III Positions of control and safety rods at critical state [mm]

(# of HEU plates: 3008; critical core in Fig. 2)

Control rod			Safety rod		
C1	C2	C3	S4	S5	S6
1200.00	723.31	1200.00	1200.00	1200.00	1200.00

(1200 [mm]: Position of upper limit)

Table IV Comparison between measured and calculated reactivity [pcm] with JENDL-4.0
 (# of HEU plates: 3016; critical core in Fig. 1 and Table II)

	ρ_{J40}^{cal}	ρ_{J40}^{exp}	C/E ($\rho_{J40}^{\text{cal}} / \rho_{J40}^{\text{exp}}$)
Excess reactivity	259.9 ± 5.7	271.7 ± 0.1	0.96 ± 0.02
C1 rod worth	876.1 ± 5.7	867.4 ± 3.9	1.01 ± 0.01
C2 rod worth	154.8 ± 5.7	142.8 ± 3.5	1.08 ± 0.05
C3 rod worth	529.2 ± 5.7	506.4 ± 4.1	1.05 ± 0.01

MCNP6.1 with JENDL-4.0: $\beta_{\text{eff}} = 813 \pm 10$ [pcm], $\Lambda = 31.67 \pm 0.07$ [μs]

Table V Comparison between measured and calculated reactivity [pcm] with ENDF/B-VII.1
 (# of HEU plates: 3016; critical core in Fig. 1 and Table II)

	ρ_{E71}^{cal}	ρ_{E71}^{exp}	C/E ($\rho_{E71}^{\text{cal}} / \rho_{E71}^{\text{exp}}$)
Excess reactivity	253.9 ± 5.6	229.8 ± 0.3	1.10 ± 0.02
C1 rod worth	862.5 ± 5.7	718.8 ± 3.2	1.20 ± 0.01
C2 rod worth	138.2 ± 5.6	118.3 ± 2.9	1.17 ± 0.05
C3 rod worth	513.1 ± 5.7	419.6 ± 4.5	1.22 ± 0.01

MCNP6.1 with ENDF/B-VII.1: $\beta_{\text{eff}} = 801 \pm 10$ [pcm], $\Lambda = 31.26 \pm 0.07$ [μs]

Table VI Comparison between measured and calculated reactivity [pcm] with JENDL-4.0

(# of HEU plates: 3008; critical core in Fig. 2 and Table III)

	ρ_{J40}^{cal}	ρ_{J40}^{exp}	C/E ($\rho_{J40}^{\text{cal}} / \rho_{J40}^{\text{exp}}$)
Excess reactivity	106.1 ± 5.7	100.2 ± 1.7	1.06 ± 0.06
MCNP6.1 with JENDL-4.0: $\beta_{\text{eff}} = 812 \pm 10$ [pcm], $\Lambda = 31.57 \pm 0.07$ [μs]			

Table VII Comparison between measured and calculated reactivity [pcm] with ENDF/B-VII.1

(# of HEU plates: 3008; critical core in Fig. 2 and Table III)

	ρ_{E71}^{cal}	ρ_{E71}^{exp}	C/E ($\rho_{E71}^{\text{cal}} / \rho_{E71}^{\text{exp}}$)
Excess reactivity	97.6 ± 5.6	84.7 ± 1.5	1.15 ± 0.07
MCNP6.1 with ENDF/B-VII.1: $\beta_{\text{eff}} = 802 \pm 10$ [pcm], $\Lambda = 31.30 \pm 0.07$ [μs]			

Table VIII Comparison between kinetic parameters by MCNP6.1 with JENDL-4.0 and ENDF/B-VII.1 (# of HEU plates: 3016 in Fig. 1)

	Parameter	JENDL-4.0	ENDF/B-VII.1
Super-critical core (clean core)	β_{eff} [pcm]	813 ± 10	801 ± 10
	Λ [μ s]	31.67 ± 0.07	31.26 ± 0.07
	Rossi- α [s^{-1}]	256.89 ± 3.11	256.18 ± 3.11
Critical core (partial insertion of C3 rod)	β_{eff} [pcm]	818 ± 10	800 ± 10
	Λ [μ s]	30.91 ± 0.07	30.69 ± 0.07
	Rossi- α [s^{-1}]	264.55 ± 3.23	260.66 ± 3.17

Table IX Comparison between kinetic parameters by MCNP6.1 with JENDL-4.0 and ENDF/B-VII.1 (# of HEU plates: 3008 in Fig. 2)

	Parameter	JENDL-4.0	ENDF/B-VII.1
Super-critical core (clean core)	β_{eff} [pcm]	812 ± 10	802 ± 10
	Λ [μ s]	31.57 ± 0.07	31.30 ± 0.07
	Rossi- α [s^{-1}]	257.22 ± 3.11	256.35 ± 3.11
Critical core (partial insertion of C2 rod)	β_{eff} [pcm]	821 ± 10	801 ± 10
	Λ [μ s]	31.33 ± 0.07	30.03 ± 0.07
	Rossi- α [s^{-1}]	262.22 ± 3.17	258.09 ± 3.15

Table X Comparison between kinetic parameters by MCNP6.1 with JENDL-4.0 and ENDF/B-VII.1 (# of HEU plates: 3000; subcritical core in Figs. 6 and 7)

	Parameter	JENDL-4.0	ENDF/B-VII.1
Subcritical core	β_{eff} [pcm]	806 ± 10	796 ± 10
(clean core)	Λ [μ s]	31.71 ± 0.07	31.42 ± 0.07
	Rossi- α [s^{-1}]	254.05 ± 3.07	253.35 ± 3.09

Table XI Measured prompt neutron decay constants α [s^{-1}] deduced by least-square fitting method, subcriticality ρ_s [\$] (dollar units) by extended area ratio method, and β_{eff}/Λ [s^{-1}] by α -fitting method (# of HEU plates: 3000; subcritical core with 14 MeV neutrons in Fig. 6)

Detector	α [s^{-1}]	ρ_s [\$]	$(\beta_{eff}/\Lambda)^{exp}$ [s^{-1}]	C/E
BF-3 #1	294.78 ± 29.27	0.0228 ± 0.0070	288.20 ± 93.30	0.88 ± 0.29
BF-3 #2	226.96 ± 23.31	0.0158 ± 0.0054	223.44 ± 79.47	1.14 ± 0.41
BF-3 #3	258.58 ± 12.34	0.0194 ± 0.0027	253.66 ± 37.32	1.00 ± 0.15
Fiber #1	248.38 ± 39.04	0.0207 ± 0.0056	243.36 ± 72.07	1.04 ± 0.31
Fiber #2	229.95 ± 14.51	0.0200 ± 0.0032	225.45 ± 38.73	1.13 ± 0.19
Fiber #3	273.99 ± 26.24	0.0261 ± 0.0056	267.02 ± 62.56	0.95 ± 0.22

MCNP6.1 with JENDL-4.0: $\beta_{eff} = 806 \pm 10$ [pcm], $\Lambda = 31.71 \pm 0.07$ [μs],

$$(\beta_{eff}/\Lambda)_{J40}^{cal} (\text{Rossi-}\alpha \text{ value in MCNP6.1}) = 254.05 \pm 3.07 [s^{-1}]; C/E = (\beta_{eff}/\Lambda)_{J40}^{cal} / (\beta_{eff}/\Lambda)^{exp}$$

Table XII Measured prompt neutron decay constants α [s^{-1}] deduced by least-square fitting method, subcriticality ρ_s [\$] (dollar units) by extended area ratio method, and β_{eff}/Λ [s^{-1}] by α -fitting method (# of HEU plates: 3000; subcritical core with spallation neutrons in Fig. 7)

Detector	α [s^{-1}]	ρ_s [\$]	$(\beta_{eff}/\Lambda)^{exp}$ [s^{-1}]	C/E
BF-3 #1	276.25 ± 7.17	0.0900 ± 0.0011	253.43 ± 7.25	1.00 ± 0.03
BF-3 #2	300.62 ± 7.03	0.0966 ± 0.0011	274.13 ± 7.15	0.93 ± 0.03
BF-3 #3	259.96 ± 6.00	0.0899 ± 0.0011	238.53 ± 6.18	1.07 ± 0.03
Fiber #1	283.79 ± 15.38	0.0973 ± 0.0025	258.62 ± 15.49	0.98 ± 0.06
Fiber #2	360.05 ± 64.85	0.1139 ± 0.0087	323.24 ± 63.19	0.79 ± 0.15
Fiber #3	269.83 ± 26.36	0.1392 ± 0.0039	236.85 ± 24.04	1.07 ± 0.11

MCNP6.1 with JENDL-4.0: $\beta_{eff} = 806 \pm 10$ [pcm], $\Lambda = 31.71 \pm 0.07$ [μs],

$$(\beta_{eff}/\Lambda)_{J40}^{cal} \text{ (Rossi-}\alpha \text{ value in MCNP6.1)} = 254.05 \pm 3.07 \text{ [s}^{-1}\text{]; } C/E = (\beta_{eff}/\Lambda)_{J40}^{cal} / (\beta_{eff}/\Lambda)^{exp}$$