CONF-M80222-7

PNL-SA-6492

Perspective on the Fusion-Fission Energy Concept

MASTER

5th Energy Technology Conference and Exposition Washington, D. C. February 27-March 1, 1978

R. C. Liikala R. T. Perry V. L. Teofilo Battelle, Pacific Northwest Laboratories Richland, Washington 99352

.

Prepared for the U.S. Department of Energy under Contract EY-76-C-06-1830

DISTRIBUTION OF THIS DOCUMENT IS UNLIMITED

DISCLAIMER

This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor any agency Thereof, nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government or any agency thereof.

DISCLAIMER

Portions of this document may be illegible in electronic image products. Images are produced from the best available original document.

PNL-SA-6492

PERSPECTIVE ON THE FUSION-FISSION ENERGY CONCEPT

NOTICE — This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Department of Energy, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

R. C. Liikala, R. T. Perry, and V. L. Teofilo Battelle, Pacific Northwest Laboratories Richland, Washington 99352

ABSTRACT

A concept which has potential for near-term application in the electric power sector of our energy economy is combining fusion and fission technology. The fusion-fission system, called a hybrid, is distinguished from its pure fusion counterpart by incorporation of fertile materials (uranium or thorium) in the blanket region of a fusion machine.

The neutrons produced by the fusion process can be used to generate energy through fission events in the blanket or produce fuel for fission reactors through capture events in the fertile material. The performance requirements of the fusion component of hybrids is perceived as being less stringent than those for pure fusion electric power plants. The performance requirements for the fission component of hybrids is perceived as having been demonstrated or could be demonstrated with a modest investment of research and development funds. This paper presents our insights and observations of this concept in the context of why and where it might fit into the picture of meeting our future energy needs.

I. INTRODUCTION

Energy is vitally important to the economy of the United States (U.S.) and we are running out of critical resources to supply the energy needed to support our economy. The electric utility industry is a major user of primary resources. Since electric utility companies are committed to providing reliable electric service and to deliver this service as cheaply as possible they are justifiably concerned with depletion of primary resources.

There are many forecasts of electric power generation which indicate a short-fall between supply and demand near the turn of the century. The question is, what energy technology can be developed in time to fill this gap? A concept which has potential for electric power application early in the twentieth century is combining fusion and fission technology. The fusion-fission system, called a hybrid,* could produce electric power directly, or generate fuel for existing nuclear power plants (Light Water Reactors) or generate both electric power and fuel.

In this paper we briefly describe the principal features of the pure fusion and hybrid processes and attempt to show where hybrids might fit in the energy picture. Performance-cost targets which the hybrid must meet to compete with other major electrical generation technologies are briefly outlines. Currently, there is much concern

Hereafter we simply refer to fusion-fission (hybrid) systems as hybrid systems or hybrids.

about fission technology in the context of weapons proliferation. hybrid concept is related to the nonproliferation scenarios being considered for fission technology. Major near-term technological requirements for hybrid systems are identified. In the summary we offer our judgments regarding the development of this concept.

THE HYBRID CONCEPT II.

The common feature of the fusion and fission processes is that they both generate neutrons. The distinguishing feature is their number and the energy liberated in the process.

A. Fusion Process

The fusion process is shown in Figure 1. Atoms of deuterium and tritium are heated and confined in a plasma where they fuse and split into helium atoms and neutrons, as shown in Equation 1.

$${}^{2}_{1}D + {}^{3}_{1}T + \text{Energy} - {}^{4}_{2}He + {}^{1}_{0}n + \text{Energy}$$
(1)

In doing so, about 17.6 MeV of energy is released. The major portion of the energy released in the reaction (Energy on the right side of Equation 1) is in the energy of the neutron (n), about 14 MeV. This neutron travels to the blanket region surrounding the plasma where it eventually strikes a lithium atom, deposits its energy, and creates tritium in the process through these reactions

The first reaction in Equation 2 occurs with fast neutrons (i.e., MeV range) whereas the other reaction occurs when the neutron energy is substantially degraded from 14 MeV. The tritium produced by the reactions shown in Equation 2 is extracted and used to refuel the reactor plasma.

B. Fusion-Fission (Hybrids) Process

The hybrid is distinguished from its pure fusion counterpart by incorporation of fertile material (uranium or thorium) in the blanket region of a fusion reactor. The fusion-fission process is depicted in Figure 2. The 14 MeV neutron produced in the plasma region travels to the blanket where it becomes absorbed by the fertile material and deposits energy. Subsequent reactions, neutron reemission, fission or capture, can take place which depends upon the energy of the absorbed neutron. If the neutron energy is high enough the neutron multiplying (Equation 3) and fission (Equation 4) reactions are dominant.

> n + U (of Th) _____2 or 3 neutrons (3)n + U (or Th) ------2 Fission Fragments + multiple neutrons (4)

About 180 MeV of energy is released in the fission reaction and more neutrons are released. If the neutron energy is degraded below \sim 2 MeV, the principal absorption reaction is capture. The capture reactions are

$$\frac{1}{0}n + \frac{238}{92}U - \frac{239}{92}U - \frac{\beta}{92}U - \frac{239}{93}Np - \frac{\beta}{94}Pu$$

$$\frac{1}{0}n + \frac{232}{90}Th - \frac{233}{90}Th - \frac{\beta}{91}Pu - \frac{\beta}{91}Pu - \frac{\beta}{92}U$$
(5)

The end product of these reactions are ²³⁹Pu and ²³³U. These isotopes are both fissile materials thereby condidate fuels for fission power plants.

The

)

Comparing the fusion process with the process in a hybrid shows that more energy is released in the hybrid when fission occurs. The fusion process yields 18 MeV of energy whereas fission in the hybrid blanket yields 180 MeV, roughly ten times more energy release. In the high energy reemission and fission processes, additional neutrons are also released. Thus, in the hybrid both energy and neutron multiplication take place which are considered desirable features for reaction applications.

C. Why Hybrids?

There are two principal reasons why hybrids appear interesting. The first is that the fusion plasma requirements for hybrids are generally less stringent than those for pure fusion power plants. The second reason is that the hybrid appears to be able to play multiple roles in the nuclear power economy.

A projection of electric generation mix, ⁽¹⁾ shown in Figure 3, is used here to put the hybrid concept in perspective of the potential roles it could play. With those technologies shown on the right side of Figure 3, this projection predicts a potential shortfall between electricity supply and demand shortly after the year 2000. One of the interests in hybrids stem from the thought that an electric power producing hybrid might be developed in time to ease or eliminate this potential shortfall.

The middle of Figure 3 illustrates the sensitivity of nuclear power generation to estimates of uranium supply. The shaded area labeled IV shows the nuclear contribution based upon a uranium supply limit of 1.8 million tons of U₃0g and where we do not reprocess and recycle fissile material. The shaded area labeled III shown the added contribution to electricity supply if the uranium supply limit is 3.5 million tons of U₃0g and spent fuel is reprocessed to recover and recycle fissile material (i.e., plutonium). Because of the uncertainty in uranium supply, electric utility companies owning nuclear power plants are interested in concepts, such as the hybrid, which can produce fissile material the nuclear increment might grow substantially above that shown in Figure 3. Hence, utility companies having (or expecting to have) nuclear power plants are interested in the concept of a hybrid as a fuel factory for fission reactors.

The hybrid concept is also looked upon as a step along the pathway to pure fusion power. It is conceivable that many uncertainties in plasma physics, plasma engineering, and blanket engineering performance of pure fusion systems could be resolved through the development of hybrids. Thus, as shown in Figure 4, hybrids could be a step on the road to achieving the benefits of pure fusion technology.

III. ECONOMIC AND NONPROLIFERATION PERSPECTIVES

In this section we attempt to put the hybrid concept in perspective of hybrids competing in the electric energy marketplace and in relation to the nonproliferation considerations of fission power systems.

A. Economic Perspective

We utilize the results of previous studies (2,3) to develop a perspective on the performance-cost targets for hybrids. In these studies, the capitalized costs for hybrids which allow the hybrid to compete with alternatives were computed for systems producing varying degrees of fissile fuel and electricity for sale. Thus, the key parameters in these analyses are the hybrid:

- capital cost
- fuel production rates
- electrical energy production

The alternative electric generation plants included: fossil power plants, LWR's Liquid Metal Fast Breeder Reactors (LMFBRs), High Temperature Gas Cooled Reactors (HTGRs), and pure fusion power plants. The base cases included hybrids along with fossil and LWR power plants. Sensitivity cases were run with and without the other reactor types. We have extracted a limited amount of the data in References 2 and 3 to provide a perspective. We refer the reader to these references for additional detail.

In Figures 5 through 7 we show the capitalized cost for a hybrid producing varying amounts of fissile material which allows them to compete in the U.S. electric generation economy. The capital cost of a LWR is shown as a point of reference. These data can be thought of as giving the designers of hybrids some guidance on plant performance projections necessary to be competitive. We first show (in Figures 5 and 6) the results where the only plants in the electric power economy are fossil, LWR, and hybrid plants. The effect of having LMFBRs and pure fusion power plants in the economy is shown in Figure 7.

The case for a hybrid which produces only fissile material for sale to LWRs is shown in Figure 5. The hybrid produces enough power however to break even, i.e., meet its own needs. A production rate of 1.0 kg/MW_t supports approximately five (5) LWR's. These results show that the capital cost of a hybrid producing less than ~ 1.3 kg/MW_t must be less than the capital cost of a LWR. For production rates above this, the hybrid plant could cost more than the capital cost of a LWR. It's questionable that a hybrid could be built for less capital cost than a LWR. Thus, the performance target area for fuel producing hybrids which breakeven on power is the shaded area. We point out, however, that the location of this curve vertically in the figure is sensitive to many input data, particularly electrical growth rate and prices for U_3O_8 and these results are a year to two out of date. An off-line fuel producing hybrid is of high interest to LWR power plant owners because it doesn't have to be hooked to the grid and be dependent on the many utility operations variables for plants which are. Therefore, a comprehensive and thorough analyses of this type of system is needed to put this hybrid concept in better perspective on filling this role in the electric generation economy.

The allowable capitalized costs for a hybrid which produces both electricity and fissile fuel for sale is shown in Figure 6. In this situation the costs range from near LWR capital cost for a hybrid producing no fissile material for sale to about 2.5 times LWR capital cost for a hybrid producing 1.5 kg/MWt. The effect of having the LMFBR and the pure fusion reactor competing with the hybrid is shown in Figure 7. As shown, the capital cost of a hybrid must be cheaper if it's to compete with LMFBRs and pure fusion power plants.

It's clear from these results that a dual purpose (fuel and electricity production) hybrid might be an economically viable concept in the electric generation marketplace. The case for the fuel producing hybrid needs further study.

B. Nonproliferation Perspective

There is concern that reprocessing spent LWR fuel based on the current solvent extraction process involving isolation of plutonium might allow the diversion of plutonium for weapons purposes. The U.S. is stressing assessment and development programs for alternative fuel cycles which might reduce or eliminate risk of nuclear weapons proliferation. The U.S. nonproliferation concerns have resulted in the Department of Energy (DOE) Nonproliferation Alternative Systems Assessment Program (NASAP)⁽⁴⁾ and the International Fuel Cycle Evaluation (INFCE). Approximately forty nations are participating in the INFCE.

To put hybrids in context with nonproliferation policy, we choose scenarios of not reprocessing spent fuel and reprocessing spent fuel to recover and recycle fissile materials in fission reactors. In the reprocessing scenario, we examine two cases, first where reprocessing is restricted to recovery and recycle of denatured ²³³U in fission reactors and second; where plutonium is recovered and recycled in fission reactors.

1. No Reprocessing Scenario

The current once-through LWR fuel cycle is shown in Figure 8. The spent LWR fuel is shown going to storage (SURF) where it stays until such time that a decision is made to dispose of it or recycle. We use the dashed arrow to indicate the uncertainty in future decisions, whether recycling is in the national interest.

In the no-reprocessing scenario, the hybrid role is limited to producing power for sale. The hybrid fuel cycle analagous to the oncethrough LWR cycle is shown in Figure 9. The depleted uranium, which comes from the tails of enriching the ²³⁵U content of natural uranium, is used as blanket material for a hybrid. The blanket is irradiated, the uranium fissions, and power is generated. The spent blanket is discharged and stored awaiting ultimate disposition. Natural uranium and/or thorium could also be used as a source of feed for fabricating hybrid blankets.

A concept which has only recently been briefly studied⁽⁵⁾, based upon the no-reprocessing scenario, is the "refresh cycle". This is shown in Figure 10. Natural thorium is mined and refined to produce thorium for fabricating a thorium blanket for the hybrid. The blanket is irradiated in the hybrid where neutrons are captured in 232 Th to produce 233 U. The bred 233 blanket material is inserted in High Temperature Gas Cooled (HTGR) fission reactor to produce power. After the 233 U is depleted in the HTGR it is sent back to the hybrid to be "refreshed" in 233 U. Upon refreshing, the fuel is again used in the HTGR for power production. After this cycle, the spent fuel is stored and ultimately disposed of.

The concept of refreshing spent LWR fuel in a hybrid reactor has also been studied briefly.⁽⁶⁾ In the concept studied, reprocessing and refabrication of the U-Pu fraction was considered.

None of the above concepts have been studied to any extent. Hence assessments of their technical feasibility and/or economic viability are premature at this point in time.

2. <u>Restricted Reprocessing - Denatured</u> ²³³U

Denaturing a fissile isotope such as 233 U means diluting it with another isotope of uranium to the extent that a nuclear weapon cannot be made directly from the material. In the case of the fissile uranium isotopes (235U and 233U) 238U serves as the diluent. No corresponding isotope of plutonium can denature the fissile isotopes of plutonium (239pu and 241pu).

This scenario assumes uranium (rather the fissile component, 235 U) is in short supply. The limitation of 235 U supply can be alleviated through utilization of thorium to generate 233 U (which is denatured) and thereby extend the supply of fissile material for fission reactors. The LWR thorium cycle is shown in Figure 11. Raw materials bearing thorium are refined to produce ThO₂ which is mixed with enriched UO₂ to fabricate ThO₂-UO₂ fuel for an LWR. The spent fuel is reprocessed to recover denatured 233 U which is refabricated into new fuel. Since this

is not a breeder cycle, an external source of 233 U is needed to sustain the system and allow it to grow. The hybrid could be the external source.

The hybrid concept based on this scenario is shown in Figure 12. Mined thorium is refined and a thorium blanket for the hybrid is fabricated. Irradiating this blanket in the hybrids builds in 233 U which is reprocessed and denatured (238 U added during reprocessing). The denatured uranium is mixed with thorium during fabrication to produce LWR fuel. Once the spent fuel is discharged from the LWR, the steps shown in Figure 11 would be followed. This concept has not been examined to date.

3. Reprocessing for Plutonium Utilization

Since plutonium cannot be denatured, other means must be found to make plutonium proliferation resistant. The technical and institutional fixes being examined in NASAP and INFCE include

- Keeping the plutonium and uranium together at all times (e.g., coprocessed U-Pu)
- Making the plutonium fuel highly radioactive (e.g., having highly radioactive materials in the fuel)
- Restricting plutonium to fuel cycle centers

All of these are variants on the once-through cycle depicted in Figure 1 where spent fuel would be reprocessed and recycled rather than disposing of it. The technical modifications listed in the first two bullets above would be made as a part of reprocessing and/or refabrication. The fuel cycle center would probably contain both the reprocessing plant and the refabrication plant, and possibly those reactors using the plutonium fuel. The plutonium produced in hybrid blankets would probably be subject to the same restrictions as that produced in fission reactor fuels.

IV. NEAR-TERM TECHNOLOGY REQUIREMENTS

For near-term application, the technology requirements for a hybrid are dictated by the development pathway for pure fusion because hybrids should be able to piggyback the fuel cycle technology development for fission reactors. The hypothetical pathways to reach commercial application for pure fusion and hybrids are shown in Figure 13. It is presently perceived that at some point along the pure fusion pathway that there is a jumping-off point for hybrids and that commercial application for a hybrid could be attained sooner than for pure fusion. To put this in better perspective, we briefly compare the technology requirements for pure fusion and hybrid Reactors.

Figure 14 was constructed to aid the discussion of the technology requirements. Basically, certain properties of the plasma must be achieved and to achieve these requires confining forces and other energy inputs. Plasma confinement is accomplished by using one of two methods, either magnetic or inertial forces. The high temperature plasma requirement dictates that the plasma be restrained from contact with the vacuum wall which would cool the plasma and probably damage the wall. Strong magnetic fields are used to exert pressure on the plasma and keep the plasma away from the wall in magnetic confinement. In inertial confinement, an incident pulse of high energy intensity (such as from a laser) is used for compression. The power output from any power plant must exceed the input to be viable. The engineering of the vacuum wall and the blanket are important factors in assuring the desired power output is attained from fusion and hybrid plants. Much of the information given below was extracted from References 7-11.

A. Plasma Requirements

The required plasma properties for reactors are given in Table 1. The property $n\tau$ is the product of the plasma density (n) and the plasma confinement time (τ). For a fusion reactor this must be $\sim 10^{14}$ seconds/cm³. The hybrid can get by with around 10^{13} which is near the current state-of-the-art. The required ion temperature for fusion is also higher than for a hybrid and again the current state-of-the-art is coming close to meeting the hybrid requirement.

B. Confinement Force Requirements

The principal confinement force requirements are listed in Table 2. For magnetic confinement the field strength exceeds 5 tesla for fusion, is approximately 4 for hybrids, and the current status is nearer 3. For inertial confinement the hybrid requirements are near to being met whereas pure fusion requires an order of magnitude improvement.

C. Power Input Requirements

The power input requirements are listed in Table 3. For magnetic systems, though the hybrid requirements are less than those for fusion, both are substantially larger than the current state-of-the-art. For inertial confinement, the input power for a hybrid reactor is an order of magnitude less than for a fusion reactor. The hybrid requirements are close to being met with current state-of-the-art. The rep rate is the rate at which the compression energy must be delivered. Both the fusion and hybrid requirements are high compared to the current state-of-the-art.

D. Power Output Requirements

The output power requirements are given in Table 4. In a fusion reactor the power is produced in the plasma whereas in the hybrid the power is produced in the blanket. These numbers describe the basic difference between a fusion reactor and a hybrid reactor, namely the power density in the blanket. However, as shown, useful power has not been achieved in either case.

E. Vacuum Wall and Blanket Engineering Aspects

The differences cited in Table 4, on where the power is produced, translate into differences in engineering problems in the vacuum wall and the blanket. The power produced in the plasma of a fusion reactor must be deposited in the blanket for subsequent recovery. To accomplish this means transferring the power to and through the vacuum wall. This high power level at the vacuum wall impairs the structural integrity of the wall. As shown in Table 5, the neutron wall loading at the first vacuum wall is about twice as high for a fusion reactor as it is for a hybrid reactor. However, since the power density in the blanket is higher in the hybrid the blanket cooling requirements are correspondingly higher (however, well within the technology of fission reactors).

Considering the near-term requirements of hybrid reactors we look to using the technology developed in fission research and development programs wherever possible in hybrid applications. This means we would select blanket materials which have (or will have when needed) established technical bases (for the performance of fuels, coolants, and structural materials used in the hybrid blanket). In addition, the uranium and/or thorium blankets would be based on existing technology for fabrication, and reprocessing of blankets. Otherwise, if substantial new research and development investments are needed for the hybrid blanket, then the timeliness and cost incentives of employing the concept commercially are comprised.

V. SUMMARY AND CONCLUSIONS

The concept of the hybrid has been outlined and reasons why this concept is interesting are given. The interest stems mainly from fuel supply uncertainties for fission reactors and the possiblity that hybrids might be a step along the pathway to pure fusion. We described the technological requirement differences between pure fusion and hybrid reactors. Indeed it does appear that the hybrid requirements are less than those for pure fusion and are attainable in the nearfuture. On the basis of looking ahead to where magnetic and inertial confinement technology is going we speculate that the jumping off point might be somewhere between 1983 and 1985 as shown in Figure 15. Accepting this, then when could we expect commerical application of the hybrid? The phases of research and development leading to commercializing a new technology⁽¹²⁾ are shown in Figure 16. This figure illustrates that about 25 years are needed following scientific feasibility to arrive at a commercial plant. Assuming the jumping off point in Figure 16 is 1985, this would say that a commercial hybrid plant could be expected in the year 2010. If indeed, as shown in Figure 1, if the shortfall starts in 2000 and the hybrid is looked upon as being thrown in the breach to meet this need, then clearly the study and development of hybrids warrants acceleration.

To date there has not been enough investment in hybrid research and development to be able to reliably ascertain the technical feasibility and the economic viability of the concept. This leads us to the conclusion that hybrids have not received proper emphasis in planning future U.S. energy systems.

REFERENCES

- Research and Development Program for 1978-1982 Overview, Electric Power Research Institute, Palo Alto, California, September 1, 1977.
- R. L. Engel and D. E. Deonigi, <u>Evaluations of Fusion-Fission (Hybrid)</u> <u>Concepts: Market Penetration Analysis for Fusion-Fission Hybrids</u>, <u>EPRI ER-469</u>, <u>Electric Power Research Institute</u>, January 1976.
- D. E. Deonigi and R. L. Engel, <u>Performance Targets for Fusion-Fission</u> (Hybrid) Reactors, BNWL-2139, Battelle, Pacific Northwest Laboratories, January 1977.
- 4. <u>Nonproliferation Alternative Systems Assessment Program Plan</u>, Department of Energy, October 1977.
- 5. K. R. Schultz, et. al., <u>A U-233 Fusion-Fission Power System Without</u> <u>Reprocessing</u>, GA-A14635, General Atomic Company, September 1977.
- L. F. Hansen and J. A. Maniscalco, "Laser Driven Fusion Fission Hybrids," Conf. Am. Institute of Chem. Engrs, Nov. 13-17, 1977, New York, NY, and U.S. DOE Report UCRL-79653, Rev. 1, November 1977.
- R. W. Moir, "Mirror Fusion-Fission Reactor Designs," Proceedings of the US-USSR Symposium on Fusion-Fission Reactors, Livermore, CA, July 13-16, 1976.
- J. A. Maniscalco, J. Hovingh, R. R. Buntzen, "A Development Scenario for Laser Fusion", presented at the 1975 Winter Meeting of American Nuclear Society, San Francisco, California, November 1975. UCRL-76980, March 30, 1976.
- 9. K. A. Brueckner, Assessment of Laser-Driven Fusion, Electric Power Research Institute, EPRI ER-203, September 1976.
- L. A. Booth and T. G. Frank, <u>Commercial Applications of Inertial</u> Confinement Fusion, LA-6838-MS, May 1977.

- 11. R. W. Conn, T. G. Frank, R. Hancox, G. L. Kulcinski, K. H. Schmitter and W. M. Stacey, Jr., "Fusion Reactor Design-II" from the IAEA Technical Committee Meeting and Workshop, Madison, WI, October 10-21, 1977.
- 12. P. Bos, "Utility Perspectives," Proceedings of Second MFE Fusion-Fission Energy Systems Review Meeting, 2-3 November 1977, Washington, DC.

2

TABLE 1. Required Plasma Properties for Reactors

| · . | Fusion | Hybrid | Current State |
|-------------------------------|---------------------|----------------|-------------------|
| nt (Seconds/cm ³) | <lo<sup>14</lo<sup> | $\sim 10^{13}$ | ~10 ¹³ |
| Ion Temperature (keV) | | 5-10 | < 5 |

TABLE 2. Confinement Force Requirements

| | Fusion | Hybrid | Current State |
|---------------------------------|--------|--------|---------------|
| Magnetic | | | |
| Magnetic Field Strength (Tesla) | >5 | ∿4 | ∿3 |
| Inertial | | | |
| Energy on Target (MJ) | >1 | >0.1 | <0.1 |

TABLE 3. Power Input Requirements

| | Fusion | Hybrid | Current State |
|----------------------------|---------------|------------------|-------------------|
| Magnetic | | | |
| Input Power (MW) | ~500 | ~400 | . 5 |
| Pulse Length (sec) | $\sim 10^{4}$ | ∿10 ³ | <1 |
| Inertial | | | |
| Input-Power (TW) | >1000 | >100 | <100 |
| Rep Rate (H _z) | 1-10 | ∿1 | ~10 ⁻³ |

TABLE 4. Power Output Requirements

| Power Produced | Fusion | Hybrid | Current State |
|----------------|--------|--------|---------------|
| Plasma | ∿3000 | ∿300 | 0 |
| Blanket | ∿500 | ∿3200 | 0 |

TABLE 5. Vacuum Wall and Blanket Engineering Aspects

| | Fusion | Hybrid |
|---|-----------------|----------|
| Structural Integrity | High | Moderate |
| - Neutron Wall Loading (MW/m ²) | ∿5 [`] | ∿2 |
| Blanket Cooling | Moderate | High |



Fusion Process







FIGURE 3. Projected Electricity Generation Mix, 1975 to 2020 (Reference 1)

_...,



FIGURE 4. Where the Hybrid Concept Seems to Fit in

· · · · · · · · · ·



FIGURE 5. Market Penetration for Hybrids Producing Only Fissile Fuel for Sale





•

HYBRID CONCEPTS

HYBRID MARKET PENETRATION



ANNUAL FISSILE MATERIAL PRODUCTION, kg/MWt

FIGURE 7. Effect of Competition on Hybrid Market Entry





FIGURE 9.

وروان المراجع والمراجع

Throwaway Blanket Concept for No Reprocessing Scenario

HYBRID

ENERGY

₹-



FIGURE 10. Refresh Cycle Hybrid Concept for No Reprocessing Scenario



FIGURE 11. Thorium LWR Fuel Cycle



FIGURE 12. Hybrid Concepts for Restricted Reprocessing Scenario



TECHNOLOGY

. . .



TIME

FIGURE 13. Hypothetical Fusion Technology Pathway



FIGURE 14. Technology Requirements



.

TECHNOLOGY



.



PURE NUBRID HYBRID COMMERCIAL APPLICATION

1983-85 TIME





FIGURE 16. Commercializing a New Technology

BIBLIOGRAPHY OF FUSION-FISSION HYBRID PUBLICATIONS

B. R. Leonard, Jr.

BATTELLE-NORTHWEST

Revised February 7, 1978

٠

LAWRENCE LIVERMORE LABORATORY

MIRROR HYBRID

D. J. Bender, J. D. Lee and R. W. Moir, "Preliminary Assessment of a Symbiotic Fusion-Fission Power System Using the Th/U Refresh Fuel Cycle." US ERDA Report UCID-17607, Oct. 1977.

D. J. Bender, J. D. Lee, K. R. Schultz and S. B. Rao, "Fuel Design Considerations for the Mirror Hybrid Reactor," TANSAO 27, 341, 1977.

D. J. Bender, J. D. Lee, W. S. Neef, R. S. Devoto, T. R. Galloway, W. L. Dexter, M. A. Hoffman, J. H. Fink, K. R. Schultz, D. Culver, R. Rao, S. Rao and W. E. Kastenberg, "A Reference Mirror Hybrid Fusion-Fission Reactor Design," TANSAO <u>26</u>, 55 (1977) and US ERDA Report UCRL-79093, June 8, 1977.

R. W. Moir, W. L. Bair, D. J. Bender, R. J. Burleigh, G. A. Carlson, R. S. Devoto, J. N. Doggett, J. H. Fink, G. W. Hamilton, J. D. Lee, M. E. Rensink, C. E. Taylor, R. W. Werner and K. R. Schultz, "Mirror Reactor Studies," <u>Proc. Sixth Int'l Conf. on Plasma Physics and Controlled Nuclear Fusion Research, October 6-13, 1976</u>, Berchtesgaden, FRG and US ERDA Report UCRL-78122, August 10, 1976.

J. D. Lee, D. J. Bender, R. W. Moir and K. R. Schultz, "Mirror Hybrids -A Status Report," <u>Technology of Controlled Nuclear Fusion</u>, <u>September 21-23, 1976</u>, Richland, WA, CONF-760935-P2, 689 and US ERDA Report UCRL-78079.

J. D. Lee, "Blanket Design for the Mirror Fusion/Fission Hybrid Reactor," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 27 (1976) and US ERDA Report UCRL-78515.

D. J. Bender, "Mirror Hybrid Reactor Optimization Studies," <u>US-USSR</u> <u>Symposium on Fusion-Fission Reactors, July 13-16, 1976</u>, Livermore, CA, CONF-76073, 37, 1976.

D. J. Bender and J. D. Lee, "The Potential for Fissile Breeding with the Fusion-Fission Hybrid Reactor," US ERDA Report UCRL-77887, June 10, 1976, and TANSAO 23, 24, 1976.

R. W. Moir, J. D. Lee, R. J. Burleigh, W. L. Barr, J. H. Fink,
G. W. Hamilton, D. J. Bender, G. A. Carlson, W. L. Dexter, J. Holdren,
C. L. Folkers, M. A. Peterson, M. E. Rensink, H. W. Patterson,
R. L. Nelson, and C. E. Taylor, "Progess on the Conceptual Design of a
Mirror Hybrid Fusion-Fission Reactor," US ERDA Report UCRL-51797,
June 25, 1975.

J. D. Lee, Ed., "Mirror Hybrid Reactor Studies - July 1975 through September 1976," US ERDA Report UCRL-50043-1, 1976.

J. D. Lee, "Neutronic Analysis of a 2500 MW_{th} Fast Fission Natural Uranium Blanket for a D-T Fusion Reactor," in <u>Proc. First Topical Meeting</u> on the Technology of Controlled Nuclear Fusion, 1974, San Diego, CA, 1974, CONF-740402-P1, Vol. 1, p. 233; also Lawrence Livermore Laboratory Rept. UCRL-75304 (1974).

J. D. Lee, "Neutronics of Sub-Critical Fast Fission Blankets for D-T Fusion Reactors,: in Proc. 7th Conf. Intersociety Energy Conversion Engineering, 1972, American Chemical Society, 1972, p. 1294. and US ERDA Report UCRL-73952.

D. J. Bender, "A Review of the Mirror Hybrid Reactor," US ERDA Report UCRL-79548, May 4, 1977.

LLL MIRROR HYBRID (CONT'D)

R. C. Haight and J. D. Lee, "Calculations of a Fast Fission Blanket for D-T Fusion Reactors with Two Evaluated Data Libraries," in <u>Proc. First</u> <u>Topical Meeting on the Technology of Controlled Nuclear Fusion, 1974,</u> San Diego, CA, 1974, CONF-740402-Pl, Vol. 1, p. 271; also US ERDA Report UCRL-75627 (1974).

R. W. Moir, J. D. Lee, R. J. Burleigh, W. L. Barr, J. H. Fink, G. W. Hamilton, D. J. Bender, G. A. Carlson, W. L. Dexter, C. L. Folkers and M. A. Peterson, "Major Features of a Mirror Fusion - Fast Fission Hybrid Reactor," DCTR Fusion-Fission Energy Systems Review Meeting, Dec. 3, 4, 1974, Germantown, MD, US ERDA Report ERDA-4, 173 (1975) and US ERDA Report UCRL-76525 Feb. 4, 1975.

L. D. Hansborough and R. W. Werner, "A Modular Fission-Fusion Hybrid Blanket," <u>First Topical Meeting on the Technology of Controlled Nuclear</u> <u>Fusion, April 16-18, 1974</u>, San Diego, CA, CONF-740402-P1, 211 (1974) and US ERDA Report UCRL-51610 (July 1, 1974).

R. W. Moir, J. D. Lee, R. C. Burleigh, W. L. Barr, J. H. Fink, G. W. Hamilton, D. J. Bender, G. A. Carlson, W. L. Dexter, J. Holdren, C. L. Folkers, M. A. Peterson, "Conceptual Design of a Mirror Hybrid Fusion-Fission Reactor," TANSAO 21, 55 (1975).

D. J. Bender and J. D. Lee, "Neutronic and Thermal-Hydraulic Analysis of a Fast Fission Blanket for a Fusion Reactor," TANSAO 21, 64 (1975).

R. W. Moir, "Physics and Engineering Design Considerations for Mirror Fusion Reactors and Devices," TANSAO 19, 3 (1974).

D. J. Bender, "Performance Parameters for Fusion-Fission Power Systems," Third ANS Topical Technology of Controlled Thermonuclear Fusion, May 9-11, 1978, Santa Fe, NM, and US DOE Report UCRL-80589.

GENERAL ATOMIC

K. R. Schultz, G. A. Backus, C. B. Baxi, J. B. Dee, E. A. Estrine, R. Rao, and A. R. Veca, ":Mirror Hybrid Reactor Blanket and Power Conversion System Conceptual Design." US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 45 (1976) and US ERDA Report GA-A 14008 (June 29, 1976).

"Conceptual Design of the Blanket and Power Conversion System for a Mirror Hybrid Fusion-Fission Reactor," GA-A-14021, 12 Month Progress Report, July 1, 1975 to June 30, 1976, and Addendum 1: Alternate Concepts (Nov. 15, 1976).

M. R. Jonzen, "An Energetics Assessment of the Potential of the Fusion-Fission Hybrid Reactor System," TANSAO <u>26</u>, 50 (1977).

K. R. Schultz, "A Review of Hybrid Reactor Fuel Cycle Considerations," TANSAO 27, 344 (1977) and US ERDA Report GA-A-14475 (July 1977).

K. R. Schultz, R. H. Brogli, G. R. Hopkins, M. Jonzen, and G. W. Shirley, "A U-233 Fusion-Fission Power System Without Reporcessing." US ERDA Report GA-A-14635 (September 1977).

P. Fortesque, "Comparative Breeding Characteristics of Fusion and Fast Reactors," Science $\underline{196}$, 1326 (1977).

M. R. Jonzen, "Mass Energy Analyses for GCFR and Fusion-Fission Hybrid Reactor Systems." US ERDA Report GA-13762 (April 1977).

D. W. Culver and W. S. Neef, "Mechanical Structure of the Mirror Hybrid Reactor Power Plant, "TANSAO 27, 68 (1977).

K. R. Schultz, "Review of the Current Status of Linear Hybrid Reactor Concepts," US ERDA Report GA-Al4538 (July 1977).

PACIFIC NORTHWEST LABORATORIES

MIRROR HYBRID

B. R. Leonard, Jr. and U. P Jenquin, "The Quality of Fissile Fuel Bred in a Fusion Reactor Blanket." <u>Second Topical Meeting on the Technology</u> of Controlled Nuclear Fusion, ERDA Report CONF-750935, Septtember 21-23, 1976.

R. C. Liikala, B. R. Leonard, Jr., W. C. Wolkenhauer, and D. T. Aase, "Review of Battelle-Northwest Technical Studies on Fusion-Fission (Hybrid) Energy Systems." In DCTR Fusion-Fission Energy System Review Meeting, US ERDA Report ERDA-4, pp 117-175, 1975.

B. R. Leonard, Jr. and W. C. Wolkenhauer, "Progress Toward the Development of a Mirror Hybrid (Fusion-Fission) Reactor." <u>Plasma Physics and Con-</u> <u>trolled Nuclear Fusion Research 1974</u>, Vol. III, pp 649-655, IAEA, Vienna, 1975.

R. W. Werner, J. D. Lee, R. W. Moir, G. A. Carlson and M. A. Peterson, LLL; and W. C. Wolkenhauer and B. R. Leonard, Jr., Battelle, Pacific Northwest Laboratories, "The Interesting Possibilities of Fusion-Fission." Plasma Physics and Controlled Nuclear Fusion Research 1974, Vol. III, pp 641-648, IAEA, Vienna, 1975.

B. R. Leonard, Jr., U. P. Jenquin, D. L. Lessor, D. F. Newman and K. B. Stewart, "A Sensitivity Study of Data Deficiencies, Weighting Functions, and 14 MeV Neutron Source Spectrum Effects in a ²³⁸U Fueled Fusion-Fission Hybrid Blanket" <u>Bull. Am. Phys. Soc. Ser. 11, 20, 2,</u> p. 163, February 1975 and Nuclear Cross Sections and Technology, NBS Special Publication 425, Vol. II, 680, October 1975.

W. C. Wolkenhauer, B. R. Leonard, Jr., and U. P. Jenquin, "Advances in the Development of the Mirror-Hybrid (Fusion-Fission) Reactor," 8th Symposium on Fusion Technology 1974, June 17-21, 1974, Noordwijkerhout, Netherlands.

W. C. Wolkenhauer, B. R. Leonard, Jr., U. P Jenquin, D. F. Newman, A. M. Sutey, C. W. Stewart, D. L. Prezbindowski, R. W. Moir, J. D. Lee and R. W. Werner, "Status Report: Mirror Hybrid Reactor Studies," BNWL-1835, Battelle, Pacific Northwest Laboratories, Richland, WA, June 1974.

A. M. Sutey and C. W. Stewart, <u>A Thermal Hydraulic Analysis of the</u> <u>Pacific Northwest Laboratory Fusion-Fission Hybrid Reactor Concept</u>, Battelle Pacific Northwest Laboratories Rept. BNWL-B-283, 1973.

W. C. Wolkenhauer, B. R. Leonard, Jr., A. M. Sutey and R. W. Moir, "Conceptual Design of a Fusion-Fission Hybrid Reactor Based on a Mirror Fusion Reactor with a Subcritical Gas Cooled Fission Blanket," in Proc. First Topical Meeting on the Technology of Controlled Nuclear Fusion, 1974, San Diego, CA, 1974, CONF-740402-P1, Vol. 1, p. 238.

W. C. Wolkenhauer, C. W. Stewart, R. W. Werner and J. D. Lee, "Some Safety Considerations of Hybrid Reactors in Comparison with Fission and Fusion Reactors," in Proc. First Topical Meeting on the Technology of Controlled Nuclear Fusion, 1974, San Diego, CA, 1974, CONF-740402-Pl, Vol. 1, p. 588.

PRINCETON PLASMA PHYSICS LABORATORY

D. L. Jassby, "Beam Driven Tokamak Fusion-Fission Hybrid," DCTR Fusion-Fission Energy Systems Review Meeting, Dec. 3-4, 1974, Germantown, MD. US ERDA Report ERDA-4, 221 (1974) and US ERDA Report TM-280 (Nov. 1974).

F. H. Tenney, "A Brief Review of the Fusion-Fission Hybrid Reactor," Technology of Controlled Nuclear Fusion, Sept. 21-23, 1976, Richland, WA, CONF-760935-P2, 641 (1976).

D. L. Chapin and R. G. Mills, "Optimization of Fusion-Driven Fissioning Systems," U.S-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 63 (1976).

F. H. Tenney, "A Tokamak Hybrid Study, "US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 71 (1976).

D. L. Jassby and J. D. Lee, "Counterstreaming-Ion-Tokamak Fissile Breeder," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, Livermore, CA, CONF-760733, 81 (1976).

D. L. Chapin, "Molten Salt Blanket Calculations for a Tokamak Fusion-Fission Hybrid Reactor, Princeton Plasma Physics Laboratory Report MATT-1236 (1976).

D. L. Jassby, "Optional Plasma Conditions for Driven Tokamak Fusion-Fission Hybrid Reactors," TANSAO 21, 60 (1975).

F. H. Tenney, C. G. Bathke, D. L. Chapin, S. L. Gralnick, R. G. Mills, W. G. Price, Jr., A. M. M. Todd, W. H. Bohlke and R. T. Perry, The Princeton Beam-Driven Tokamak Fusion-Fission Hybrid Reactor, "TANSAO 24, 38 (1976).

C. G. Bathke, "The Optimization Process in the PPL Hybrid Study," TANSAO 26, 52 (1977).

C. G. Bathke, W. H. Bohlke, D. L. Chapin, S. L. Gralnick, R. G. Mills, W. G. Price, Jr., F. H. Tenney and A. M. M. Todd, "Can TFTR Be a Prototype for a Fissile Fuel Producer?" Unpublished Manuscript (August 1976).

PACIFIC NORTHWEST LABORATORIES-PRINCETON PLASMA PHYSICS LABORATORY

TCT HYBRID

D. T. Aase, M. C. C. Bampton, T. J. Doherty, B. R. Leonard, R. A. McCann, D. F. Newman, R. T. Perry and C. W. Stewart, "TCT Hybrid Preconceptual Blanket Design Studies," PNL-2304, January 1978.

D. T. Aase and R. A. McCann, "Preconceptual Evaluation of a Helium Cooled Molten Salt Hybrid Blanket." In Proceedings of 7th Symposium on Engineering Problems of Fusion Research, October 1977.

T. J. Doherty and C. W. Stewart, "Preconceptual Evaluation of a Pressure Tube Converter Region for a Hybrid Blanket." In Proceedings of 7th Symposium on Engineering Problems of Fusion Research, October 1977.

M. C. C. Bampton and R. A. McCann, "Structural Analysis of a Two-Component Torus (TCT) Hybrid Reactor First Wall." In Proceedings of 7th Symposium on Engineering Problems of Fusion Research, October 1977.

PACIFIC NORTHWEST LABORATORIES

TDHR

V. L. Teofilo, "Preliminary Conceptual Design of a TDHR." Presented at Second DMFE Fusion-Fission Energy Systems Review Meeting, Washington, DC, November 1977.

V. L. Teofilo, et al., "TDHR Conceptual Design Study." BNWL- , draft to be issued January 1978; Final Report expected to be published in February 1978.

V. L. Teofilo, et al., "TDHR - A Tokamak Demonstration Hybrid Reactor." ANS Transactions, 27, 29 (1977).

B. R. Leonard, Jr. and V. L. Teofilo, "Fissile Fuel Breeding and Hybrid Blanket Power Production." ANS Transactions <u>27</u>, 339 (1977).

R. T. Perry, V. L. Teofilo and U. P. Jenquin, "Blanket Neutronic Analysis for a TDHR." ANS Transactions <u>27</u>, 338 (1977).

V. L. Teofilo, et al., "A Tokamak Hybrid Blanket Design." In Proceedings of 7th Symposium on Engineering Problems of Fusion Research, October 1977.

R. T. Perry, V. L. Teofilo, and M. A. McKinnon, "Neutronics and Thermal Hydraulics of a Tokamak Hybrid Blanket." In Proceedings of 7th Symposium on Engineering Problems of Fusion Research, October 1977.

PACIFIC NORTHWEST LABORATORIES

ТОКАМАК

B. R. Leonard, Jr., and W. C. Wolkenhauer, "Fusion-Fission Hybrids: A Subcritical Thermal Fission Lattice for a DT Reactor," in <u>Proc. on</u> <u>Controlled Thermonuclear Fusion, 1972</u>, Univ of Texas, 1972, CONF 721111, p 918; also, Pacific Northwest Laboratories Rep. BNWL-SA-4390 (1972).

B. R. Leonard, Jr., "A Review of Fusion-Fission (Hybrid) Concepts." Nucl. Tech. <u>20</u>, 161 (Dec. 1973).

WESTINGHOUSE FUSION POWER SYSTEMS

R. P. Rose, J. W. H. Chi, R. E. Gold, J. Jedruch, and J. L. Kelley, "Blanket Fuel Selection for a Tokamak Fusion-Fission Breeder," TANSAO 27, 337 (1977).

R. P. Rose, J. W. H. Chi, A. H. Colman, R. E. Gold, R. R. Holman,
H. R. Howland, D. L. Jassby, J. Jedruch, S. Kellman, D. Klein,
M. Raymund and E. W. Sucov, "Tokamak Actinide Burner Design Study,"
US-USSR Symposium on Fusion-Fission Reactors, July 13-16, Livermore, CA,
CONF-760733, <u>91</u> (1976).

R. P. Rose, "Design of a Beam-Driven Tokamak for Depletion of Actinide Wastes," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 723 (1976).

R. P. Rose, "Westinghouse Tokamak Fusion-Fission Design Studies," US-USSR Workshop on Fusion-Fission Reactors, March 14-April 2, 1977, Moscow/Leningrad USSR.

Westinghouse Electric Corporation, Fusion Power Systems Department, "Fusion-Driven Breeder Reactor Design Study," First Quarterly Report WFPS-TME-035 (August 1976), Second Quarterly Report WFPS-TME-037, (November 1976), Third Quarterly Report WFPS-TME-040, (February 1977).

J. L. Kelley, "Cursory Conceptual Study of a U.S. Blanket Test Module for the USSR T-20 Reactor," Westinghouse Fusion Power Systems Report WFPS-TME-047 (October 1977).

R. P. Rose, "Status of Westinghouse Tokamak Hybrid Studies," Second DMFE Fusion-Fission Energy Systems Review Meeting, Washington, DC, November 2-3, 1977, and Report WFPS-TME-073, December 1977.

R. P. Rose, et al., "Fusion-Driven Breeder Reactor Design Study," Final Report, WFPS-TME-043, May 1977.

W. P. Kovacik, "Laser Fusion Power Reactor System (LFPRS)," WFPS-TME-070, December 1977.

R. P. Rose, et al., "Fusion-Driven Actinide Burner Design Study," Final Report, EPRI Research Project 473-1, EPRI Report ER-451, Vols. I and II (May 1976).

MASSACHUSETTS INSTITUTE OF TECHNOLOGY

L. M. Lidsky, "Fission-Fusion Systems: Hybrid, Symbiotic and Augean," Nucl. Fusion <u>15</u>, 151 (1975).

L. M. Lidsky, "Fission-Fusion Systems: Classification and Critique," DCTR Fusion-Fission Energy Systems Review Meeting, Dec. 3-4, 1974, Germantown, MD, US ERDA Report ERDA-4, 63 (1974).

G. W. Braun and L. M. Lidsky, "Geometry and Performance of a Theta Pinch Power Reactor," First Topical Meeting on the Technology of Controlled Nuclear Fusion, April 16-18, 1974, San Diego, CA, CONF-740402-P1, 199.

L. M. Lidsky, "Fission-Fusion Symbiosis; General Considerations and a Specific Example," in Proc. Nuclear Fusion Reactors Conference, 1969, (Culham, England, 1969), paper 1.2.

Laszlo N. Lontai, <u>Study of A Thermonuclear Reactor Blanket with Fissile</u> <u>Nuclides</u>, MS thesis, Technical Report No. 436, Massachusetts Institute of Technology (1965).

A. G. Cook and L. M. Lidsky, "The Effect of Processing Fabrication Costs on Design Choices for Hybrid and Symbiotic Fusion Reactor Blankets," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 757 (1976).

L. M. Lidsky, "Symbiosis: Optimization of the Nuclear Fuel Cycle," TANSAO 21, 60 (1975).

CANADA

C. W. Gordon and A. A. Harms, "Parameter Matching for the Breeding Blanket," TANSAO <u>26</u>, 38 (1977).

G. LaVergne, J. E. Robinson and J. G. Martel, "On the Matching of Fusion Breeders to Heavy-Water Reactors," Nucl. Techn. 26, 12 (1975).

A. A. Harms, "Upper Bounds of Fissile Fuel Yield with Fusion Breeders," Can. J. Phys. <u>54</u>, 1637 (1976).

A. A. Harms, "Hierarchical Systematics of Fusion-Fission Energy Systems," Nucl. Fusion 15, 5, 939 (1975).

C. W. Gordon and A. A. Harms, "The Basic Characteristics of an Efficient Fusion Breeder," Atomkernenergie (ATKE) Bd. 29, 213 (1977).

A. A. Harms and C. W. Gordon, "Energy Yield and Fuel Dynamics of the Fusion Breeder," Proc. 9th Symposium on Fusion Technology, 399, Pergamon Press Oxford and New York (1976).

C. W. Gordon and A. A. Harms, "Comparative Energetics of Three Fusion-Fission Symbiotic Nuclear Reactor Systems," Nucl. Engr. and Design, 34, 269 (1975).

0

UNIVERSITY OF WASHINGTON

TOKAMAK

Shang-Fou Su, G. L. Woodruff and N. J. McCormick, "A High-Gain Fusion-Fission Reactor for Producing 233 U," Nucl. Techn. 29, 392 (June 1976).

ĩ.

UNIVERSITY OF ROCHESTER

R. N. Horoshko, H. Hurwitz and H. Zmora, "Application of Laser Fusion to the Production of Fissile Materials," Annals of Nuc. Sci. Engr., Vol $\underline{1}$, 223 (1974).

4

UNIVERSITY OF TEXAS

Won-Guk Hwang, and T. A. Parish, "The Economic Feasibility of a Fusion-Based Energy Storage System," TANSAO <u>26</u>, 49 (1977).

T. A. Parish, "Fusion-Fission Studies at the University of Texas at Austin," US-USSR Symposium on Fusion-Fission Reactors July 13, 14, 15, 16, 1976, Livermore, CA, CONF-760733, p. 24.

E. L. Draper, Jr. and S. J. Gage, "The Fusion-Fission Breeder: Its Potential in a Fuel Starved Thermal Reactor Economy," Symposium on the Technology of Controlled Thermonuclear Fusion Experiments and the Engineering Aspects of Fusion Reactors, Austin, TX (November 1972), CONF-721111, p. 132.

T. A. Parish, and E. L. Draper, Jr., "Neutronic and Photonic Analyses of Fusion Reactor Blankets Containing Thorium," Fifth Symposium on Engineering Problems in Fusion Research, Princeton, New Jersey (November 1973).

T. A. Parish and R. G. Spangler, "The Production of ²³³U in Hybrid Fusion Reactor Blankets," Transactions of the American Nuclear Society, TANSAO 21, 58 (June 1975).

T. A. Parish and E. L. Draper, Jr. "Neutronic and Photonic Analyses of Fusion Reactor Blankets Containing Natural Uranium," Symposium on the Engineering Problems of Fusion Research, San Diego, CA (April 1974).

T. A. Parish and E. L. Draper, Jr., "Neutronic and Photonic Analyses of Simulated Fusion Reactor Blankets Containing Thorium and Natural Uranium," Technical Report ESL-16, Univ. of Texas at Austin, (October 1973).

T. A. Parish and E. L. Draper, Jr., "Neutronic and Photonic Analyses of Fusion Reactor Blankets Containing Natural Uranium," Proc. First Topical Meeting on the Technology of Controlled Nuclear Fusion, April 1974, San Diego, CA, CONF-740402-P1, p. 256. S. S. Rozhkov and G. E. Shatalov, "Thorium in the Blanket of a Hybrid Thermonuclear Reactor," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 143 (1976).

V. V. Kotov and G. E. Shatalov, "Gas-Cooled Blanket of a Hybrid Thermonuclear Reactor with Solid Lithium-Containing Materials," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 129 (1976).

V. V. Gur'ev, A. M. Epinat'ev, B. N. Kolbasov, V. V. Kotov, E. M. Kuz'min, M. E. Netecha, V. P. Smetannikov, G. E. Shatalov, and O. L. Shchipakin, "Experimental Gas-Cooled Hybrid Blanket Module for a Tokamak Demonstration Reactor," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 119 (1976).

V. I. Pistunovich, "Plasma Physics Basis for the Tokamak Hybrid Reactor with Injection," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 57 (1976).

G. E. Shatalov, "Calculation of Neutron Fluxes and Energy Releases in the Thermonuclear Reactor Blanket." Izv. akademii nauk SSSR, Energetika i transport, No. 6 (1975).

I. N. Golovin, "The Place of Hybrid Reactors in the World Power System," Atomnaya energiya (1975).

I. N. Golovin, G. E. Shatalov, and B. N. Kolbasov, "Some Problems of Hybrid Thermonuclear Reactors," Izvestiya AN SSSR, Energetika i transport $\underline{6}$, 28 (1975).

Ya I. Kolesnichenko and S. N. Reznik, "D-T Plasma as a Source of Neutrons for the Combustion of 238 U," Nucl. Fusion <u>14</u>, 114 (January 1974).

V. L. Blinkin and V. M. Novikov, "Optimal Symbiotic Molten-Salt Fission-Fusion System," Kurchatov Institute Report IAE-2819 (1977), UCRL-TRANS-11288.

Ya. I. Kolesnichenko and S. N. Reznik, "The D-D Nuclear Fusion Reaction in a Hybrid Reactor," Nuclear Fusion <u>16</u>, 97 (1976).

SYSTEMS ANALYSIS AND DESIGN OF THERMONUCLEAR STATIONS, papers Delivered at a Joint USSR-USA Seminar, 9-20 December 1974. US-ERDA Report ERDA-TR-17. Contents:

I. N. Golovin, "The Place of Hybrid Reactors in the World Energy System"

S. V. Marin, D. V. Markovskiy, L. P. Smirnov, G. Ye. Shatalov, "Certain Parameters of the Blanket of a Thermonuclear Reactor with a Fissionable Substance."

V. V. Kotov, Ch. W. Maynard, D. V. Markovskiy and G. E. Shatalov, "Sensitivity Analysis of Parameters of Hybrid Reactor to Nuclear Data," Kurchatov Institute Report IAE-2817 (1977).

USSR

LAWRENCE LIVERMORE LABORATORY

LASER FUSION

L. F. Hansen, R. R. Holman, and J. A. Maniscalco, "Scoping Studies of Blanket Designs for a Power-Generating Laser Fusion Hybrid Reactor," TANSAO <u>26</u>, 506 (1977).

R. R. Holman and W. P. Kovacik, "Conceptual Approach to Laser Fusion Hybrid Reactor," TANSAO $\underline{26}$, 56 (1977).

L. F. Hansen and J. A. Maniscalco, "Fuel Breeding and Power Production in Laser Fusion-Fission Hybrid Reactors," TANSAO <u>27</u>, 343 (1977).

J. A. Maniscalco, "A Conceptual Design Study for a Laser Fusion Hybrid," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 177 (1976).

J. A. Maniscalco, "A Conceptual Design Study for a Laser Fusion Hybrid," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 657 (1976).

L. F. Hansen and J. A. Maniscalco, "Neutronic Study of a Laser Fusion Hybrid Reactor Design," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 677 (1976) and US ERDA Report UCRL-78069, September 1976.

A. G. Cook and J. A. Maniscalco, $"^{233}$ U Breeding and Neutron Multiplying Blankets for Fusion Reactors," Nucl. Tech. <u>30</u>, 5 (July 1976) and TANSAO 23, 25 (1976).

J. Maniscalco, "Fusion-Fission Hybrid Concepts for Laser-Induced Fusion," Nucl. Tech. 28, 98, (January 1976) and US ERDA Report UCRL-76763 (May 1, 1975).

J. A. Maniscalco and L. Wood, "Advanced Concepts in Fusion-Fission Hybrid Reactors, "US ERDA Report UCRL-75835 (July 1974) and TANSA0 <u>19</u>, 6 (1974).

Bechtel Corporation, "Laser Fusion Hybrid Reactor System Study." Bechtel Corporation Report P08777205 (July 1976).

J. A. Blink and J. S. Chin, "Laser Fusion Development Scenario," TANSA0 27, 16 (1977).

J. Hovingh, "Design Considerations in Interially-Confined Fusion Reactors," Faculty Institute on Fusion Materials and Curriculum Development, Argonne, IL, August 9-13, 1976, US ERDA Report UCRL-78499.

J. Maniscalco, "Hybrid Concepts for Laser-Induced Fusion," TANSA0 21, 17 (1975).

Bechtel Corporation, "Laser Fusion-Fission Reactor Systems Study," U.S. ERDA Report UCRL-13796, (July 1977).

J. A. Blink and J. S. Chin, "Laser Fusion Development Scenario," TANSAO 27, <u>16</u> (1977).

J. Hovingh, "Design Considerations in Interially-Confined Fusion Reactors," Faculty Institute on Fusion Materials and Curriculum Development, Argonne, IL, August 9-13, 1976, US ERDA Report UCRL-78499.

J. Maniscalco, "Hybrid Concepts for Laser-Induced Fusion," TANSAO 21, 17 (1975).

LASER FUSION (CONT'D)

Bechtel Corporation, "Laser Fusion-Fission Reactor Systems Study," U.S. ERDA Report UCRL-13796, (July 1977).

J. Hovingh, "First Wall Studies of a Laser-Fusion Hybrid Reactor Design," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 765.

J. A. Blink, P. E. Walker and H. W. Meldner, "Energy Partition and Neutron Spectra from Laser Fusion Reactor Targets," TANSA0 27, 70 (1977).

÷

L. F. Hansen and J. A. Maniscalco, "Laser Driven Fusion Fission Hybrids," Conf. Am. Institute of Chem. Engrs, November 13-17, 1977, New York, NY, and US DOE Report UCRL-79653, Rev. 1., November 1977.

E-BEAM HYBRIDS

R. Cooper, V. Bailey, J. Benford, D. Oliver and M. DiCapua, "Electron Beam Heated Solenoid Reactors for Breeding Fissile Fuels," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 207 (1976).

Ŀ

J. Benford, V. Bailey, D. Oliver, M. DiCapua and R. Cooper, "Electric Breeding of Fissile Materials with Low Q, Non-Mainline Fusion Drivers," Physics International Report PIFR-863.

J. L. Mitchener and S. G. Varnado, "An Economic Model of a Relativistic Electron Beam Hybrid Reactor," TANSAO <u>27</u>, 17 (1977).

D. Oliver, R. Cooper and L. Lidsky, "Breeding of Fissile Fuel with Linear Fusion Sources," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 775 (1976).

J. Benford, V. Bailey and D. Oliver, "E-Beam Heated Linear Solenoid Reactors," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935-P2, 383 (1976).

UNIVERSITY OF WASHINGTON

LASER SOLENOID

Ŀ.

C

A. Hoffman, P. Rose, and L. Steinhauer, "Status of Laser Solenoid Fusion Concept," TANSAO 27, 49 (1977).

L. C. Steinhauer and R. T. Taussig, "Laser Solenoid Fusion-Fission Design," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, 197 (1976).

L. C. Steinhauer, R. T. Taussig and D. C. Quimby, "Fusion-Fission Hybrid Reactors Based on the Laser Solenoid," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935, 371 (1976).

C. L. Woodruff and D. C. Quimby, "Fusion-Fission Neutronics Calculations for the Laser Solenoid," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935, 787 (1976).

G. L. Woodruff and D. C. Quimby, "Neutronics Calculations for a Laser-Heated Solenoid," TANSAO 22, 14 (1975).

L. C. Steinhauer, et al., "A Feasibility Study of a Linear Laser Heated Solenoid Fusion Reactor," Electric Power Research Institute Report EPRI ER-171 (February 1976).

LOS ALAMOS SCIENTIFIC LABORATORY

LASER .

L. A. Booth and T. G. Frank, "Commercial Applications of Inertial Confinement Fusion," US ERDA Report LA-6838-MS (May 1977).

Û.

2

UNIVERSITY OF ILLINOIS

G. H. Miley, "Energy Balances for Fusion-Fission Hybrids," DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown, MD US ERDA Report-4 359 (1974).

G. H. Miley, "Plasma Requirements for Fusion-Fission Hybrids," TANSAO 21, 63 (1975).

0

F. H. Southworth and H. D. Campbell, "Neutron Downscattering in Dense Fusioning Plasma," TANSAO $\underline{21}$, 15 (1975).

6

)

ISRAEL

E. Greenspan, A. Schenider, D. Gilai and P. Levine, "Natural-Uranium Light-Water Breeding Hybrid Reactors," The Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935, p-3, 1061.

Ehud Greenspan, Catalyzed Deuterium Fusion-Fission Hybrid Reactors, TANSAO 27, 50 (1977) and US DOE Report PPPL-1399.

Ū.

Ĵ

E. Greenspan, A. Schneider, A. Misolovin, and D. Gilai, "Energy Multiplication and Tritium Breeding in Light-Water Hybrid Reactors (LWHR)." TANSAO <u>26</u>, 511 (1977).

G. Shani, "Tailoring the Breeding-Fission Ratio in a Hybrid Fusion Reactor Blanket by Neutron Moderation with Iron," Nucl. Sci. Engr. <u>65</u>, 183, (1978).

E. Greenspan, "Prospects for Natural Uranium Light-Water Hybrid Reactors," Trans. Israeli Nucl. Soc. 4, 35 (1976).

E. Greenspan, "Power Generation Versus Fuel Production in Light-Water Hybrid Reactors," Princeton University Plasma Physics Laboratory PPPL-1319 (1977).

E. Greenspan, A. Schneider and A. Misolovin, "On the Feasibility of Plutonium Separation-Free Nuclear Power Economy with LWHRs," Trans. Amer. Nucl. Soc., <u>26</u>, 305 (1977).

D. Gilai, E. Greenspan, P. Levin, A. Misolovin and A. Schneider, "Advances in Light-Water Hybrid Reactors (LWHR)," Trans. Israeli Nuc. Soc., 5, 60 (1977).

LOS ALAMOS SCIENTIFIC LABORATORY

THETA PUNCH

C.

R. A. Krakowski, D. J. Dudziak, T. A. Oliphant, K. I. Thomassen, G. E. Bosler and F. L. Ribe, "Prospects for Converting ²³²Th to ²³³U in a Linear Theta-Pinch Hybrid Reactor (LTPHR)," DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown MD, US ERDA Report ERDA-4, 249 (1974).

R. A. Krakowski, R. L. Miller and R. L. Hagenon, "Engineering and Physics Considerations for a Linear Theta-Pinch Hybrid Reactor," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, CONF-760733, 155 (1976).

G. E. Bosler, D. J. Dudziak and W. R. Ellis, "A Preliminary Appraisal of a Fusion/Fission (Hybrid) Reactor Based on the Linear Theta Pinch" Ninth Energy Conversion Engineering Conference (IECEC) August 26-30, 1974, San Francisco, CA, and US ERDA Report LA-UR-74805.

R. A. Krakowski, D. J. Dudziak, T. A. Oliphant and K. I. Thomassen, "An Engineering Design of a Linear Theta-Pinch Hybrid Reactor (LTPHR)," TANSAO <u>21</u>, 61 (1975).

W. R. Ellis, "Fusion Reactor Applications of the High-Density Linear Theta Pinch," First Topical Meeting on the Technology of Controlled Nuclear Fusion, April 16-18, 1974, San Diego, CA, CONF 740402-P1, 175 (1974).

K. I. Thomassen, R. A. Krakowski, T. A. Oliphant, G. E. Bosler, and D. J. Dudziak, "Prospects for Converting $232_{Th}-233_U$ in a Linear Theta Pinch," TANSAO <u>19</u>, 6 (1974).

UCLA

W. E. Kastenberg and D. Okrent, "Some Safety Considerations for Conceptual Fusion-Fission Hybrid Reactors," UCLA Report PFG-322 (October 1977).

A. Z. Ullman, "Blanket Cooling Under Accident Conditions in Fusion-Fission Hybrid Power Reactors," TANSAO <u>26</u>, 30 (1977).

V. Badham, G. C. Pomraning, W. E. Kastenberg and D. Okrent, "Accidental Criticality of Fusion/Fission Hybrid Blanket Designs," TANSAO <u>26</u>, 33 (1977).

¢

ECONOMIC AND GENERAL

D. E. Deonigi, "Energy Supply System Modeling," TANSAO 26, 43 (1977).

Ċ

)

2

D. E. Deonigi and R. L. Engel, "Performance Targets for Fusion-Fission Hybrid Reactors," US ERDA Report BNWL-2139 (January 1977).

D. E. Deonigi and R. L. Engel, "The Economics of Fusion-Fission Systems," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733. p. 231 (1976).

R. C. Liikala, U. P. Jenquin, R. L. Engel, B. R. Leonard, Jr., D. E. Deonigi, "Evaluations of Fusion-Fission (Hybrid) Concepts," EPRI ER-469-SY, Research Project 268-1, Summary Report (June 1976).

R. L. Engel and D. E. Deonigi, "Evaluations of Fusion-Fission (Hybrid) Concepts: Market Penetration Analysis for Fusion-Fission Hybrids," EPRI-ER-469, Project 268, Topical Report Part A (January 1976).

D. E. Deonigi, "The Potential Sale of Fission-Fusion Systems in the Electric Economy," TANSAO <u>21</u>, 58 (1975).

D. E. Deonigi, "Economic Regimes for Fission-Fusion Energy Systems," DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown, MD, ERDA-4, 93 (1974).

B. R. Leonard, Jr., "Discussion of Fuel Cycles and Economics," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, p. 241 (1976).

R. P. Rose "Summary of Discussion Session on Hybrid Blanket Designs," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, p. 247 (1976).

R. W. Bussard, "Some Systems Economic Considerations of Pure Fusion Breeders of Fissile Nuclear Fuel," Technology of Controlled Nuclear Fusion, September 21-23, 1976, Richland, WA, CONF-760935, p-2, 741 (1976).

R. N. Kostoff, "Hybrids: Promise, Problems and Progress," US-USSR Workshop on Fusion-Fission Reactors," Moscow/Leningrad, USSR, March 14-April 2, 1977.

R. N. Kostoff, "The ERDA DMFE Systems Studies Program," Work shop on Systems Studies of Fast Pulsed Reactors, October 12-23, 1976. Novosibirsk, Siberia, USSR.

R. N. Kostoff, "Comments on Hybrids," Information Meeting on Accelerator-Breeding, January 18-19, 1977, Upton, NY, CONF-770107, 381 (1977).

W. Seifritz, "The Symbiosis Between Beam-Driven Hybrid DT-Fusion Reactors and Near-Breeder HTGRs," TANSAO 26, 27 (1977).

W. C. Gough, "EPRI/Kurchatov Institute Joint Program on Fusion-Fission," TANSAO 27, 340 (1977).

W. C. Gough, "Prospects for Utility Applications of Fusion Power," American Power Conference, April 18-20, 1977, Chicago, IL.

W. C. Wolkenhauer, "Selection of Optimum Hybrid (Fusion-Fission) Reactors," TANSAO <u>21</u>, 55 (1975).

W. C. Wolkenhauer, "The Role of a Hybrid (Fusion-Fission) Reactor in a Nuclear Economy," DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown, MD, ERDA-4, 327 (1974).

M. Lotker, "Commercializating Fusion: From Plasma Lab to Power Line," TANSAO 24, 41 (1976).

ECONOMIC AND GENERAL (CONT'D)

G. Schlueter, "Uranium Demand of Symbiotic Reactor Strategies," TANSAO 24, 217 (1976).

R. A. Huse, J. M. Burger and M. Lotker, "Fusion-Fission Energy Systems -Some Utility Perspectives," DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown, MD, ERDA-4, 27 (1974).

R. G. Mills, "Discussion of Overall System Designs," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976, Livermore, CA, CONF-760733, p. 261 (1976).

J. P. Holdren, "Summary of the Discussion Session on Fusion-Fission Reactor Safety," US-USSR Symposium on Fusion-Fission Reactors, July 13-16, 1976 Livermore, CA, CONF-760733, p. 267 (1976).

J. Holdren, "The Relevance of Environmental Concerns in Contemplating Development of Fission-Fusion Hybrids - A Personal View." DCTR Fusion-Fission Energy Systems Review Meeting, December 3-4, 1974, Germantown, MD, ERDA-4, 209 (1974).

G. L. Woodruff, "Advantages and Limitations of High-Gain, Mixed-Cycle Hybrid Reactors," Unpublished Manuscript (January 1978).

B. Augenstein, "Fusion-Fission Hybrid Breeders--Economic and Performance Issues, Role of Advanced Converters, Interdependence between Fission and Fusion Programs," Rand Corp, Report P-6047, (December 1977).

D. Dreyfuss, B. Augenstein, W. Mooz, and P. Sher, "An Examination of Alternative Nuclear Breeding Methods, Rand Corp., US ERDA Report WN-9799 (May 1977).

R. A. Krakowski and A. S. Tai, "A Simple Economics Parametric Analysis of Fissile Fuel Production by Fusion-Fission Reactors," Second DMFE Fusion-Fission Energy Systems Review Meeting, Washington DC, November 2-3, 1977 (also USDOE Rept, LA-UR-77-2879).

J

¢