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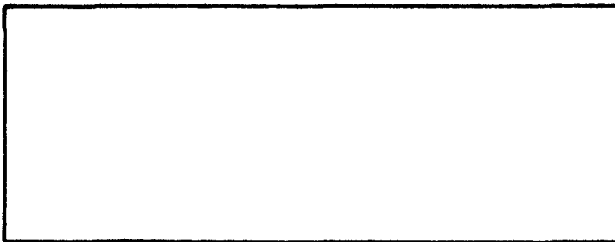
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DOCUMENT TITLE

AVAILABILITY OF ENDF/B NEUTRON CROSS
SECTION DATA

AUTHOR

H. ALTER



CLASSIFICATION TYPE
(RD OR DI)

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ATOMICS INTERNATIONAL A Division of North American Aviation, Inc. TECHNICAL DATA RECORD		AI-AEC-TDR TDR NO 12639	APPROVALS
		PAGE 1 OF 50	<i>H. Alter</i>
AUTHOR H. Alter	DEPT & GROUP NO 736-644	DATE 1/8/68	<i>S. J. Carpenter</i>
TITLE Availability of ENDF/B Neutron Cross Section Data		GO NO 7701	
		S/A NO 13130	TWR
PROGRAM Reactor Development		SECURITY CLASSIFICATION	
		(CHECK ONE BOX ONLY)	
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DISTRIBUTION See Attached List		CONF. <input type="checkbox"/>	<input type="checkbox"/>
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		AUTHORIZED CLASSIFIER SIGNATURE _____ DATE _____	
STATEMENT OF PROBLEM			
Prepare a summation of neutron cross section data distributed through ENDF/B and available at Atomics International.			
ABSTRACT			
Over the past several months Brookhaven National Laboratory has been distributing ENDF/B Neutron Cross Section Data to participating members of the Cross Section Evaluation Working Group (CSEWG). This report summarizes data received and available for use at Atomics International as of January 1, 1968			

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Over the past several months BNL has been distributing ENDF/B neutron cross section data to participating members of the Cross Section Evaluation Working Group (CSEWG). This report lists the nuclides for which data has been received and filed. With the exception of ENDF/B tape No. 999, all data is considered to be category 1 data, that is, checked and found free of mechanical error. Category 2 data has not been so certified. The data listed is generally identified in terms of an ENDF/B identification number (MAT), nuclide name, magnetic tape number and the number of BCD cards per nuclide on the tape. A set of CRT's for a typical nuclide is appended. Listings and some writeups of the data are available from the author.

ENDF/B Data Tape 114

MAT	Nuclide	No. of Cards	MAT	Nuclide	No. of Cards
1005	Li-6	1263	1025	Mo	570
1006	Li-7	1140	1030	Gd	601
1009	B-10	456	1035	Ta-181	721
1010	C-12	1938	1053	Pu-240	807
1012	N-14	3674	1059	Na	1330
1014	Mg	502	1060	W-182	585
1016	Ti	650	1061	W-183	667
1017	V	716	1062	W-184	568
1019	Mn	1326	1063	W-186	530
1042	U-233 FP	87	Rapidly saturating fission products		
1066	"	76	Slowly	"	"
1067	"	72	Non	"	"
1045	U-235 FP	87	Rapidly	"	"
1068	"	76	Slowly	"	"
1069	"	72	Non	"	"
1052	Pu-239 FP	87	Rapidly	"	"
1070	"	76	Slowly	"	"
1071	"	72	Non	"	"

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NO. AI-AEC-TDR-12639DATE 1/8/68PAGE 3 OF 50ENDF/B Data Tape 115

MAT	Nuclide	No. of Cards	MAT	Nuclide	No. of Cards
1001	H	205	1043	U-234	684
1003	D	263	1046	U-236	675
1013	O-16	2380	1047	U-238	953
1018	Cr	2296	1051	Pu-239	1008
1020	Fe	3299	1054	Pu-241	2145
1021	Ni	3082	1056	Am-241	656
1026	Xe-135	386	1057	Am-243	436
1027	Sm-149	897			

ENDF/B Data Tape 116

MAT	Nuclide	No. of Cards	MAT	Nuclide	No. of Cards
1007	Be	639	1037	Au-197	1081
1024	Nb	924	1044	U-235	2670
1028	Eu-151	859	1048	Np-237	655
1029	Eu-153	858	1050	Pu-238	697
1031	Dy-164	875	1055	Pu-242	695
1032	Lu-175	800	1058	Cm-244	680
1033	Lu-176	828			

ENDF/B Data Tape 117

MAT	Nuclide	No. of Cards
1015	Al	1559

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ENDF/B Data Tape 999

MAT	Nuclide	MAT	Nuclide
1034	Hf	1074	Hf-178
1041	U-233	1075	Hf-179
1072	Hf-176	1076	Hf-180
1073	Hf-177	1082	Hf-174

Thermal cross sections and scattering law data for H₂O, D₂O, Be, BeO, C, CH₂ and ZrH have also been received and filed on magnetic tape.

For Be, C, BeO and ZrH the data is tabulated at the following temperatures, °K, 296, 400, 500, 600, 700, 800, 1000 and 1200. Data for H₂O and D₂O is tabulated for temperatures of 296, 350, 400, 450, 500 and 600 while data for CH₂ is available at temperatures of 296 and 350.

MOLYBDENUM CROSS SECTIONS MATERIAL 1025

THE PRIMARY SOURCE FOR THIS EVALUATION IS KFK-12D BY J.J.SCHMIDT. THIS DATA WAS PREPARED BY E.PENNINGTON, A.N.L., IN OCTOBER 1966. THE FOLLOWING MODIFICATIONS WERE MADE OF THE DATA FROM THE PRIMARY SOURCE-

RESONANCE PARAMETERS FROM JETP 17,803,(1963), PLUS SOME SMOOTH CAPTURE WERE USED FROM 4 EV. TO 1 KEV.

INELASTIC SCATTERING CROSS SECTIONS FROM 200 KEV. TO 1.5 MEV. FOR FOUR RESOLVED LEVELS WERE OBTAINED FROM A.B.SMITH AND R.HAYES (TO BE PUBLISHED).

N-2N AND N-3N CROSS SECTIONS WERE OBTAINED FROM S.PEARLSTEIN. SEE N.S.E.23,238-250(1965).

THE TOTAL CROSS SECTION WAS EXTENDED FROM 10 TO 15 MEV. USING BNL-325 DATA.

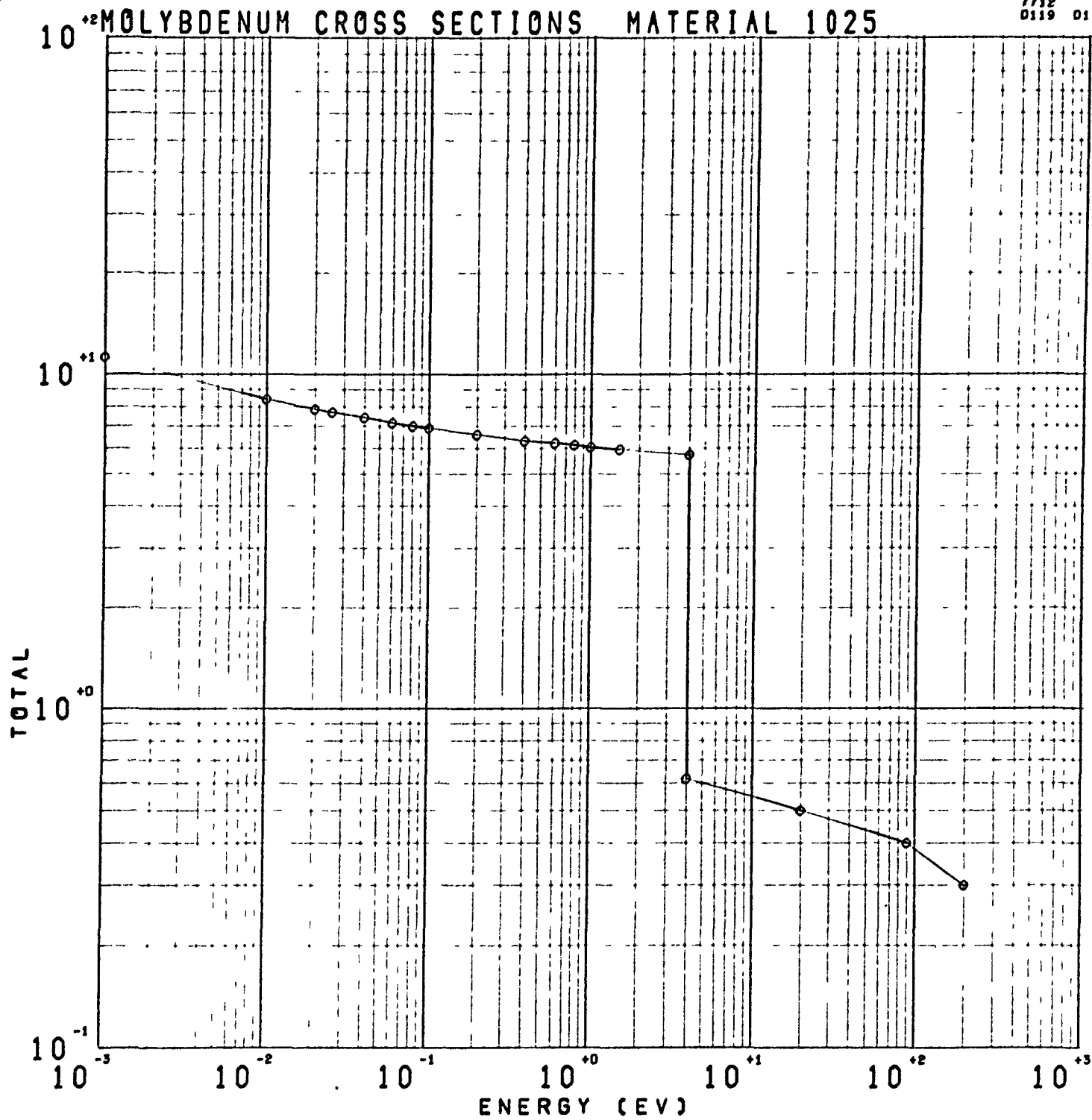
THE N-GAMMA CROSS SECTION WAS EXTENDED FROM 10 TO 15 MEV. USING $1/E$ BEHAVIOR.

THE REACTION CROSS SECTION WAS ASSUMED CONSTANT FROM 10 TO 15 MEV. THE INELASTIC SCATTERING CROSS SECTION WAS OBTAINED ABOVE THE N-2N THRESHOLD BY SUBTRACTING OTHER REACTIONS FROM THE REACTION CROSS SECTION.

NUCLEAR TEMPERATURES WERE ESTIMATED FOR CONTINUUM INELASTIC SCATTERING AS IN YIFTAH-OKRENT-MOLDAUER.

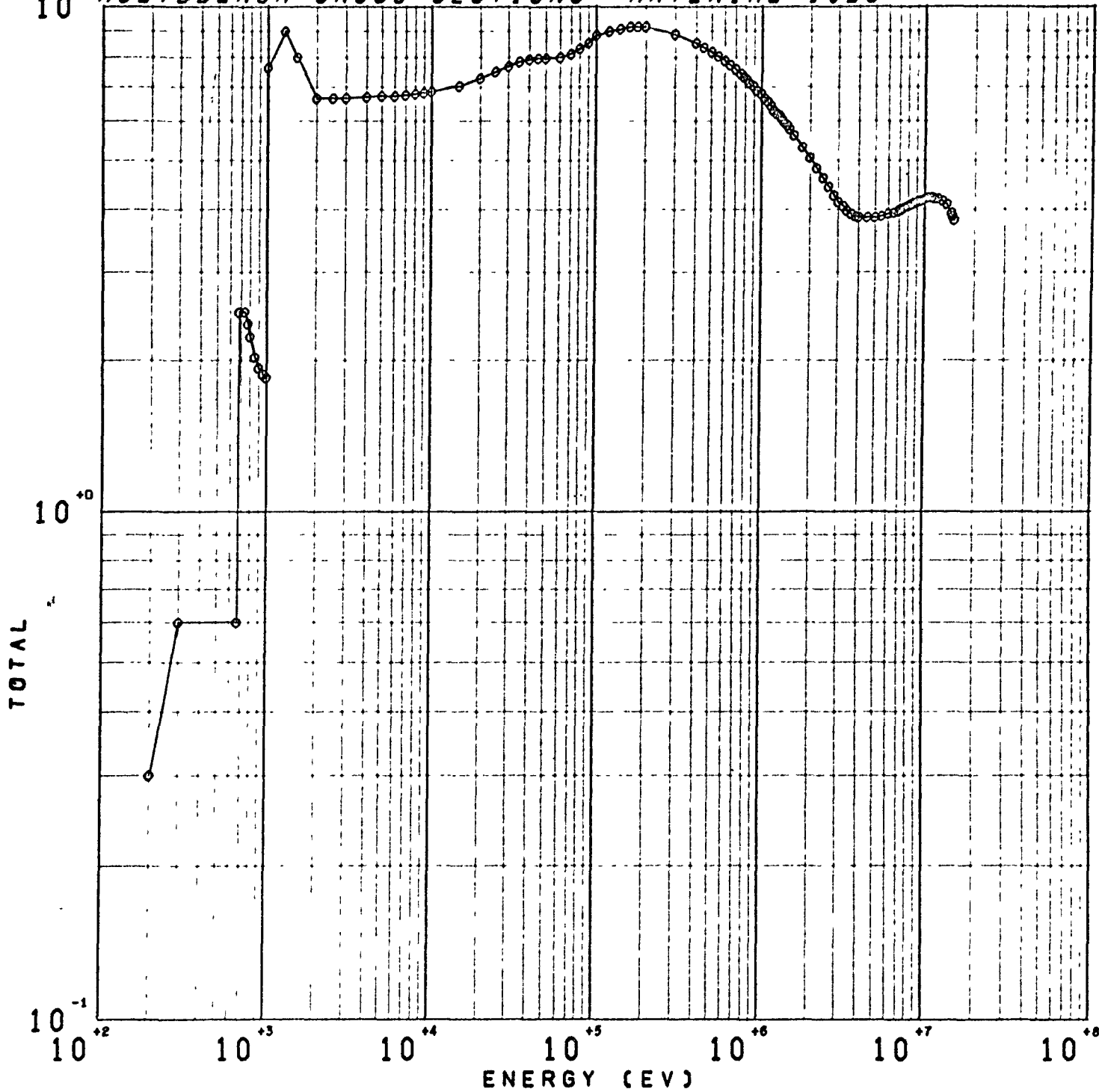
LEGENDRE COEFFICIENTS FOR ELASTIC SCATTERING WERE OBTAINED FROM H.ALTER UP TO 6 MEV. (SEE NAA-SR-11980 VOL.IV). ABOVE 6 MEV. ABACUS OPTICAL MODEL CALCULATIONS PROVIDED THE COEFFICIENTS.

VALUES OF MU BAR LAB,XI,AND GAMMA WERE CALCULATED FROM THE
LEGENDRE COEFFICIENTS.
PARAMETERS FOR THE FREE GAS THERMAL SCATTERING LAW WERE INCLUDED.

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0119 0119

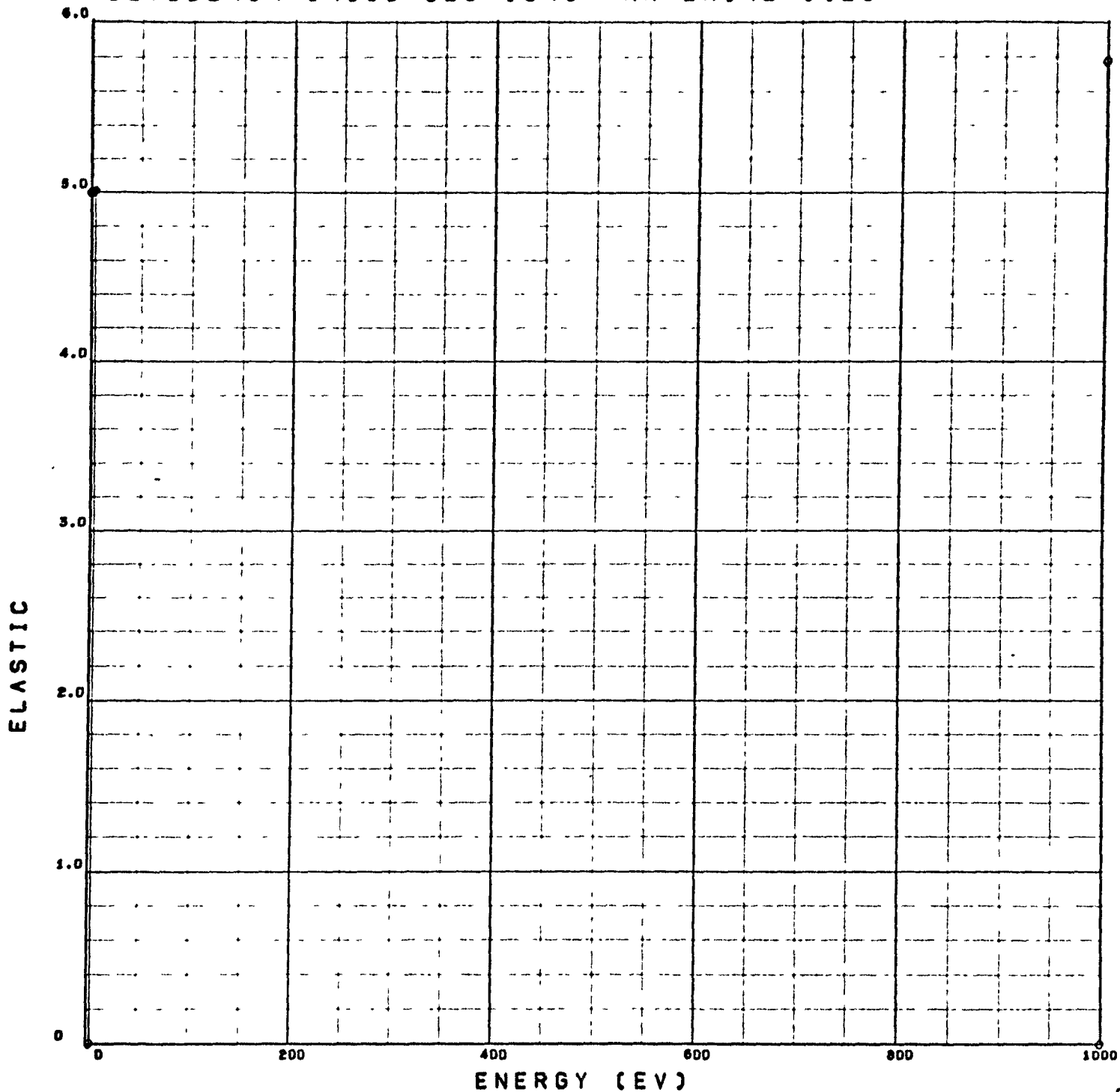
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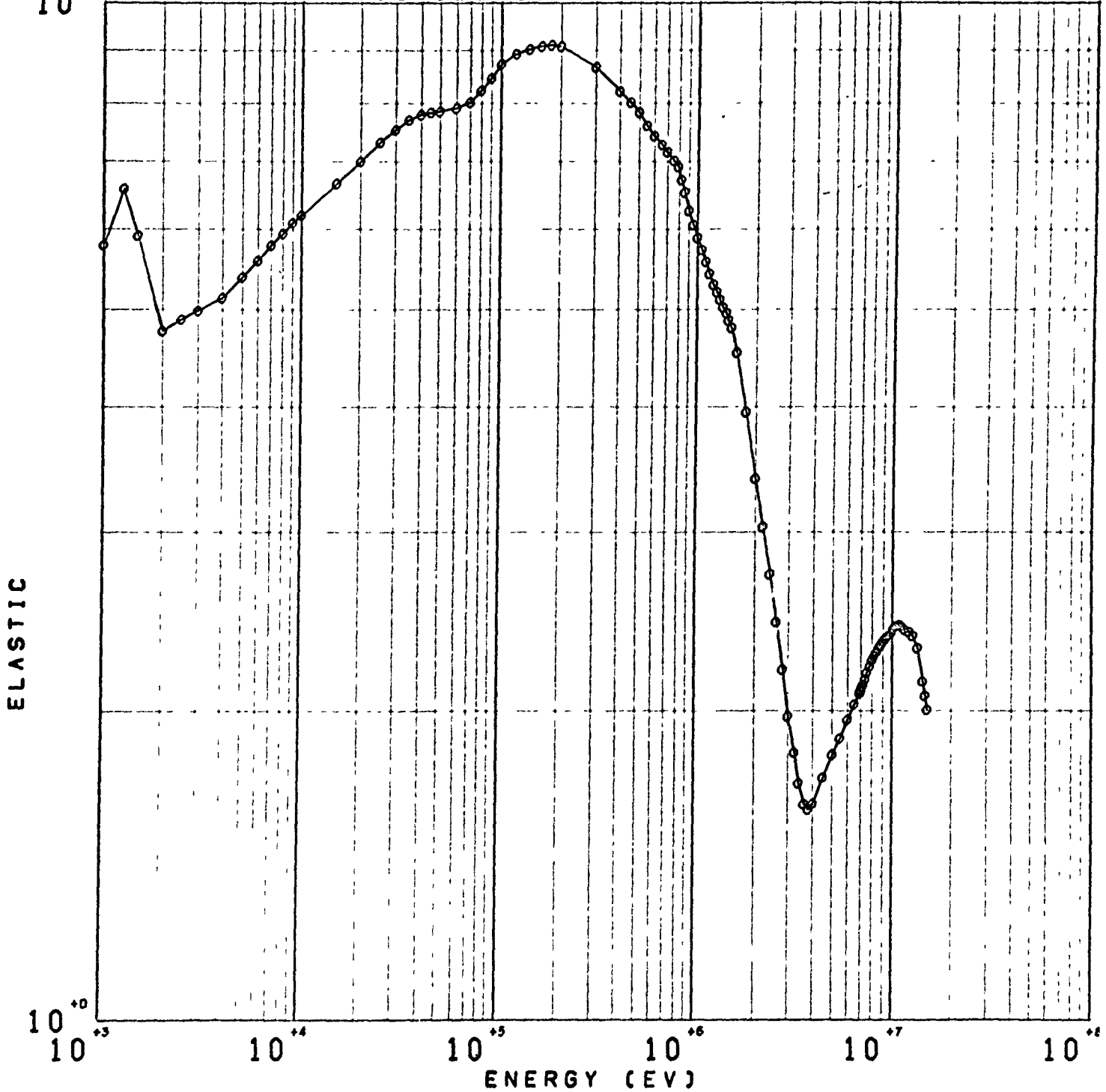
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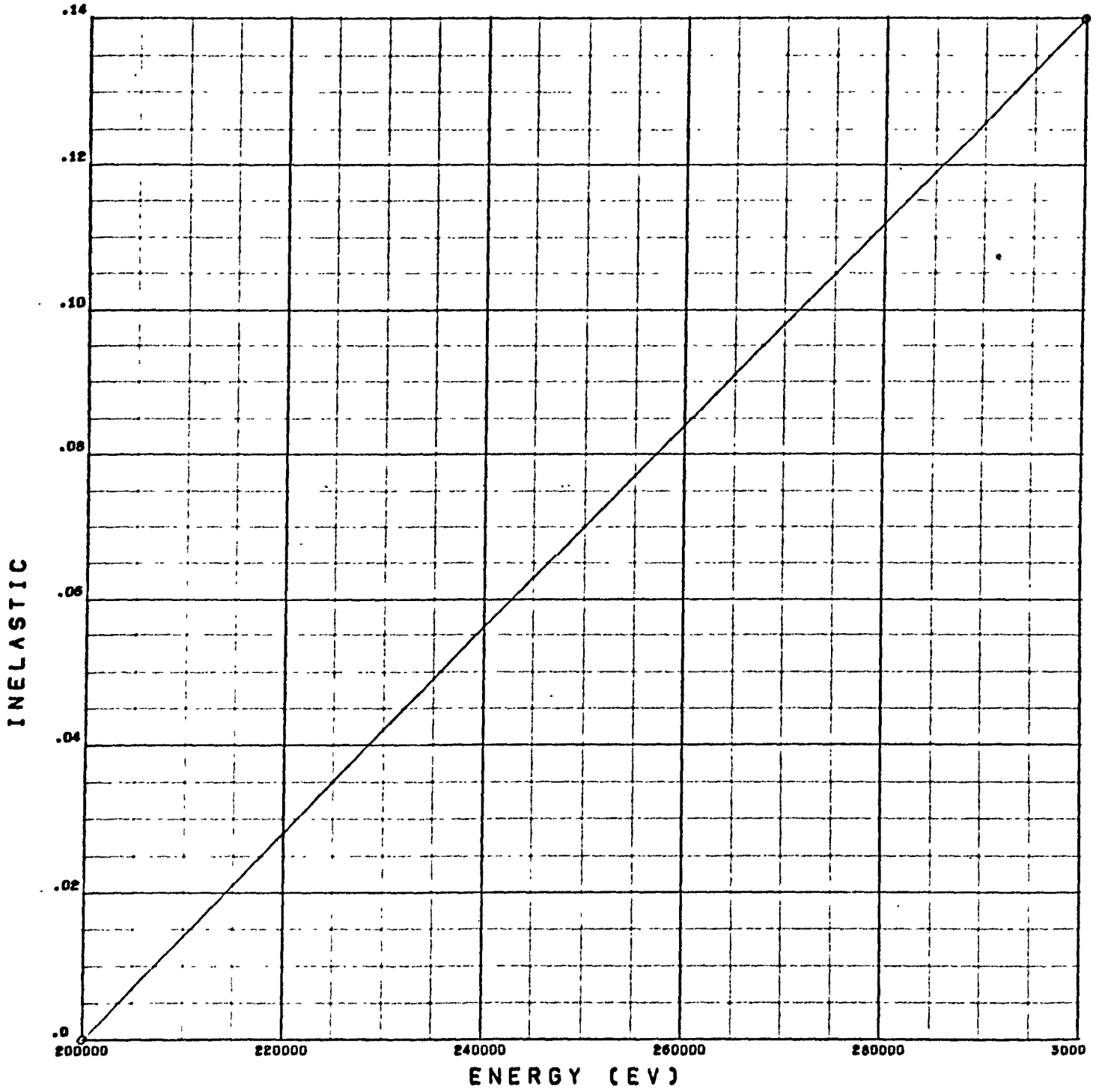
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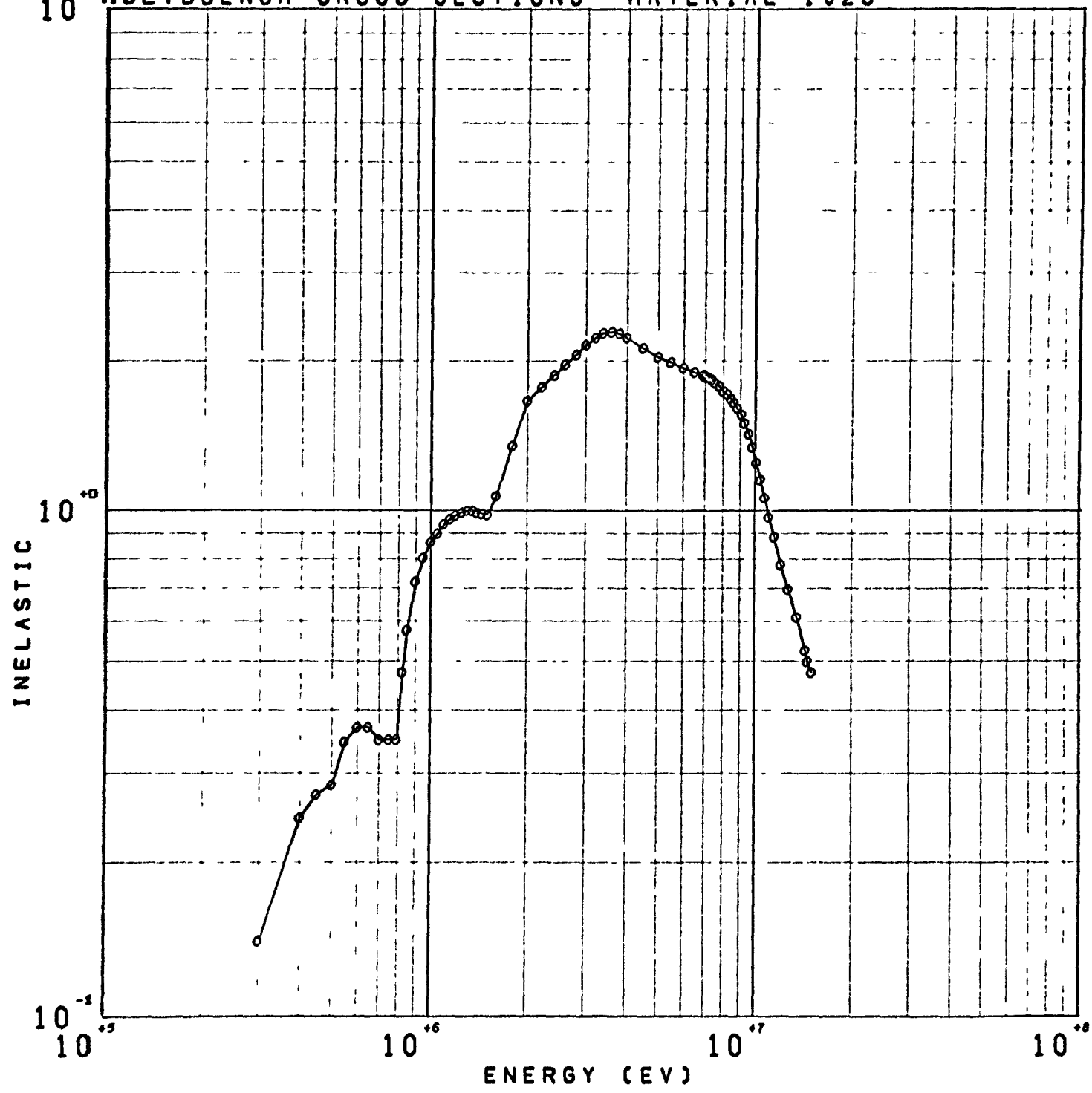
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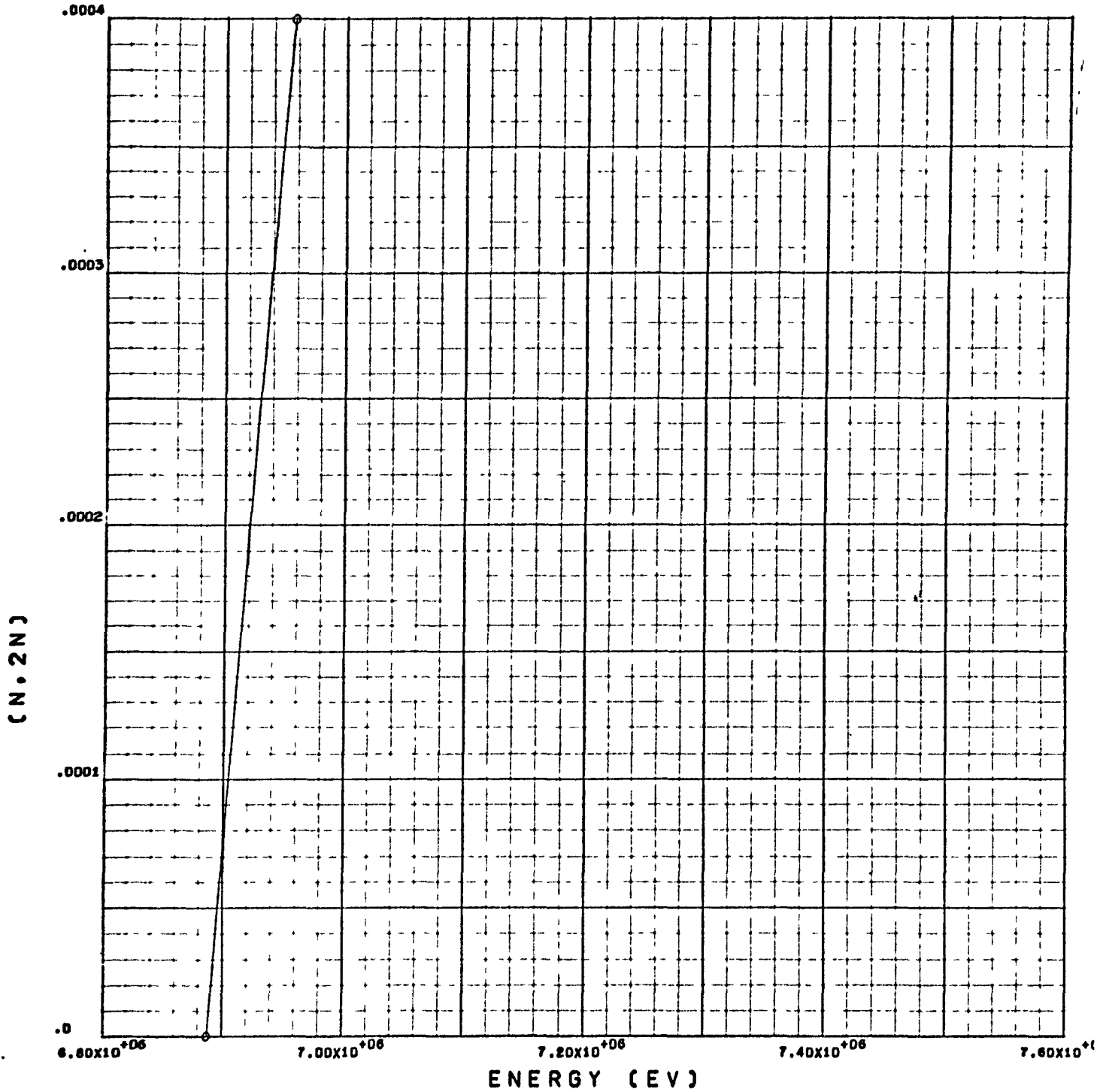


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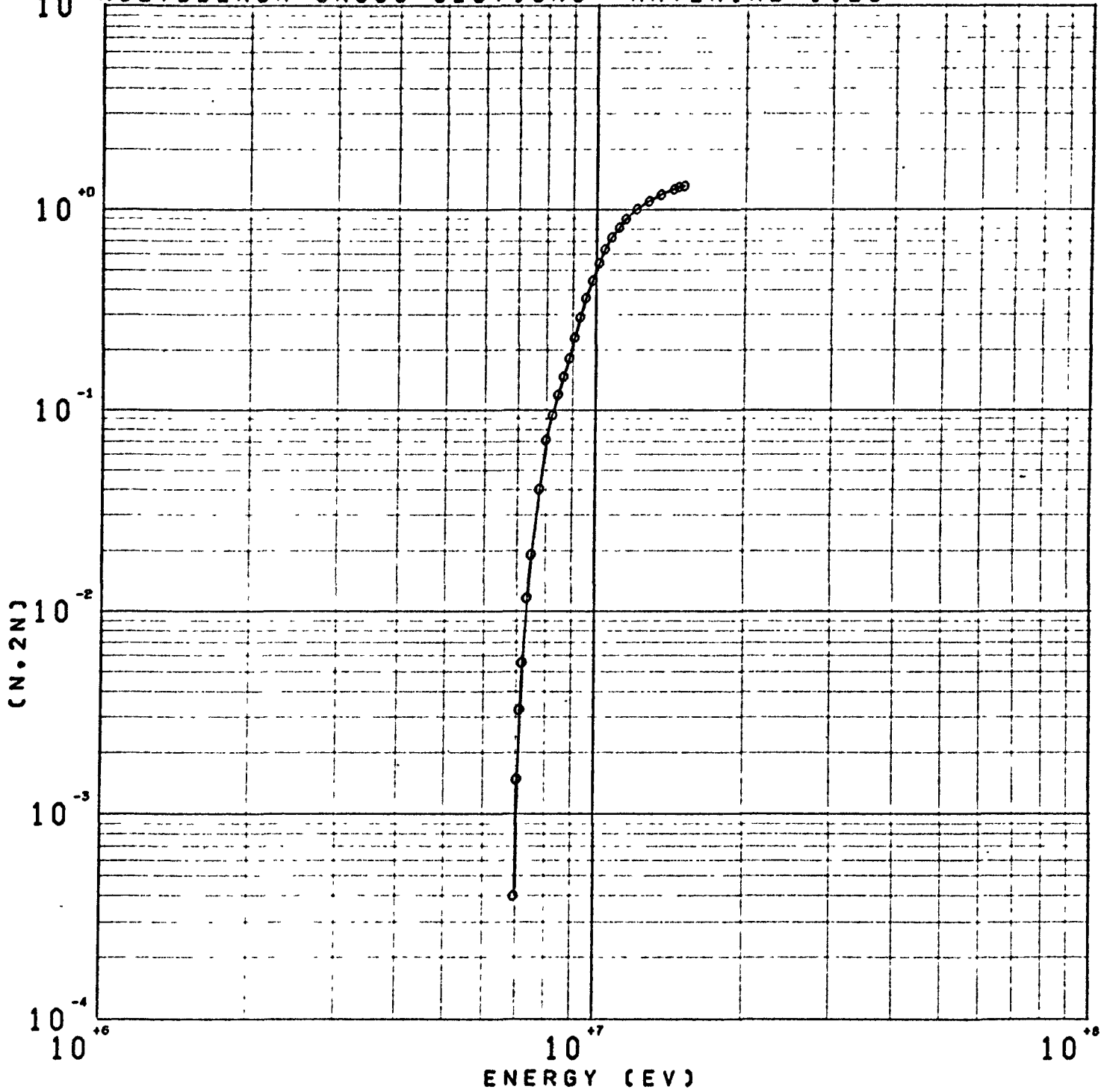


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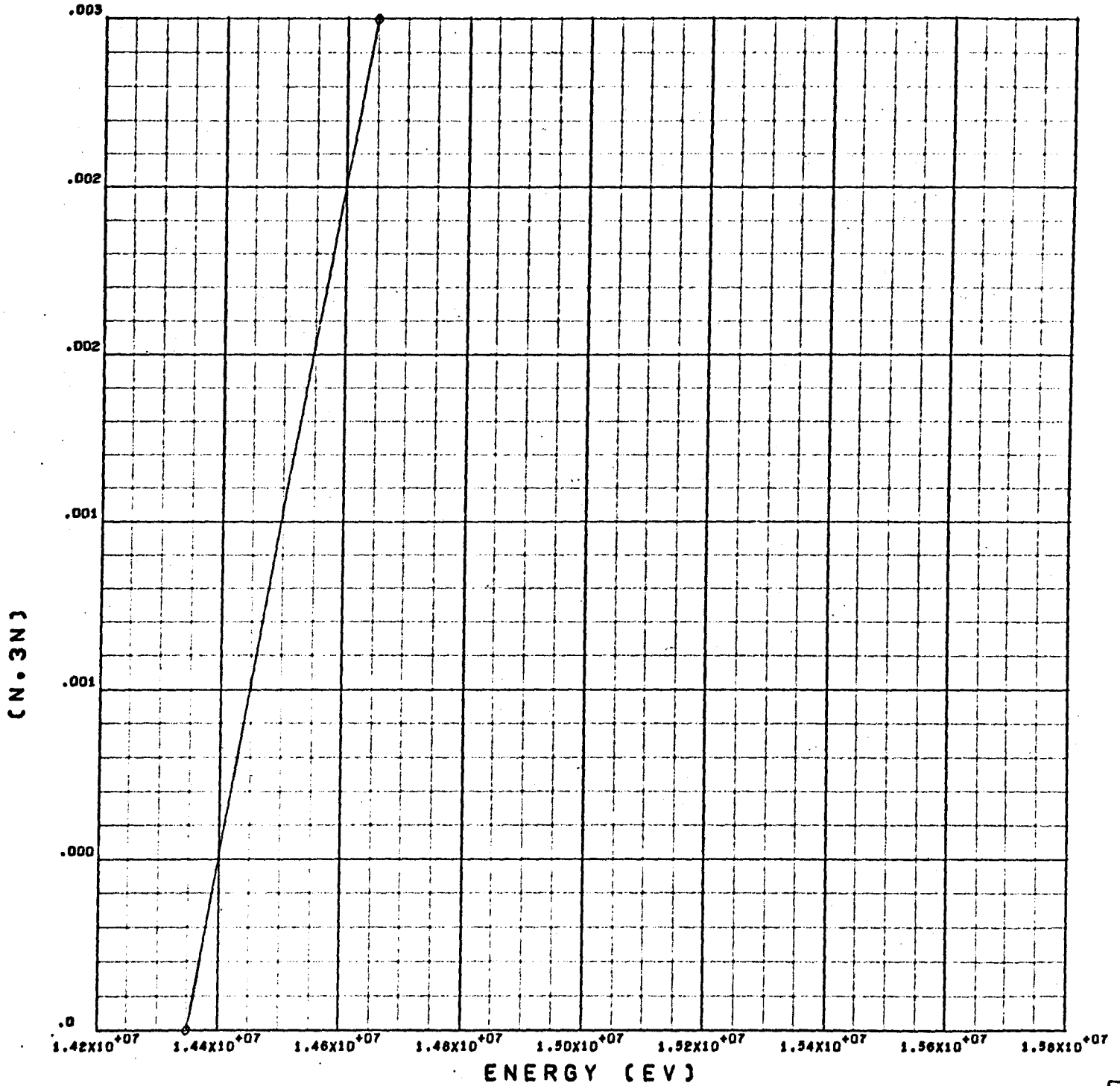
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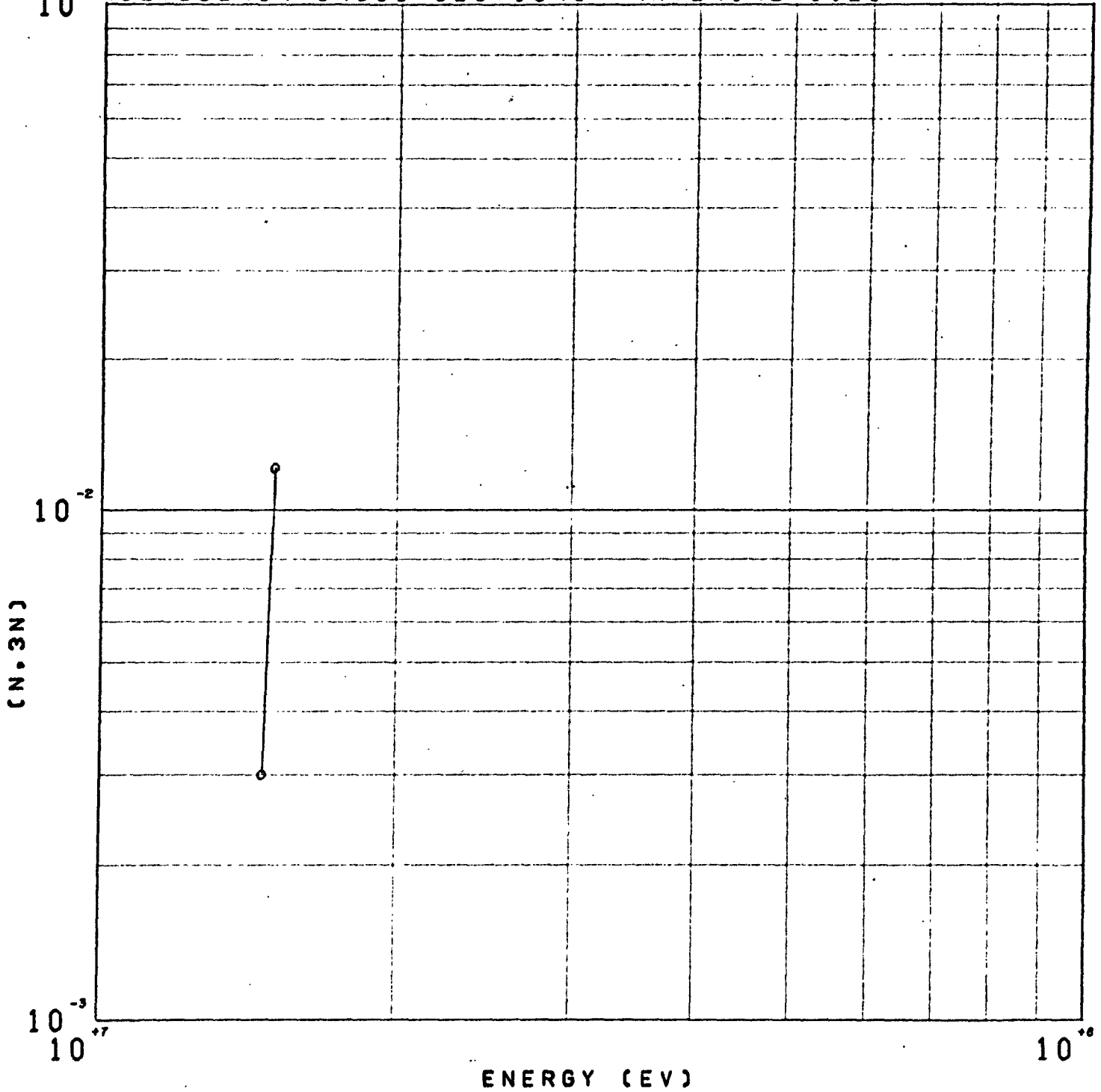
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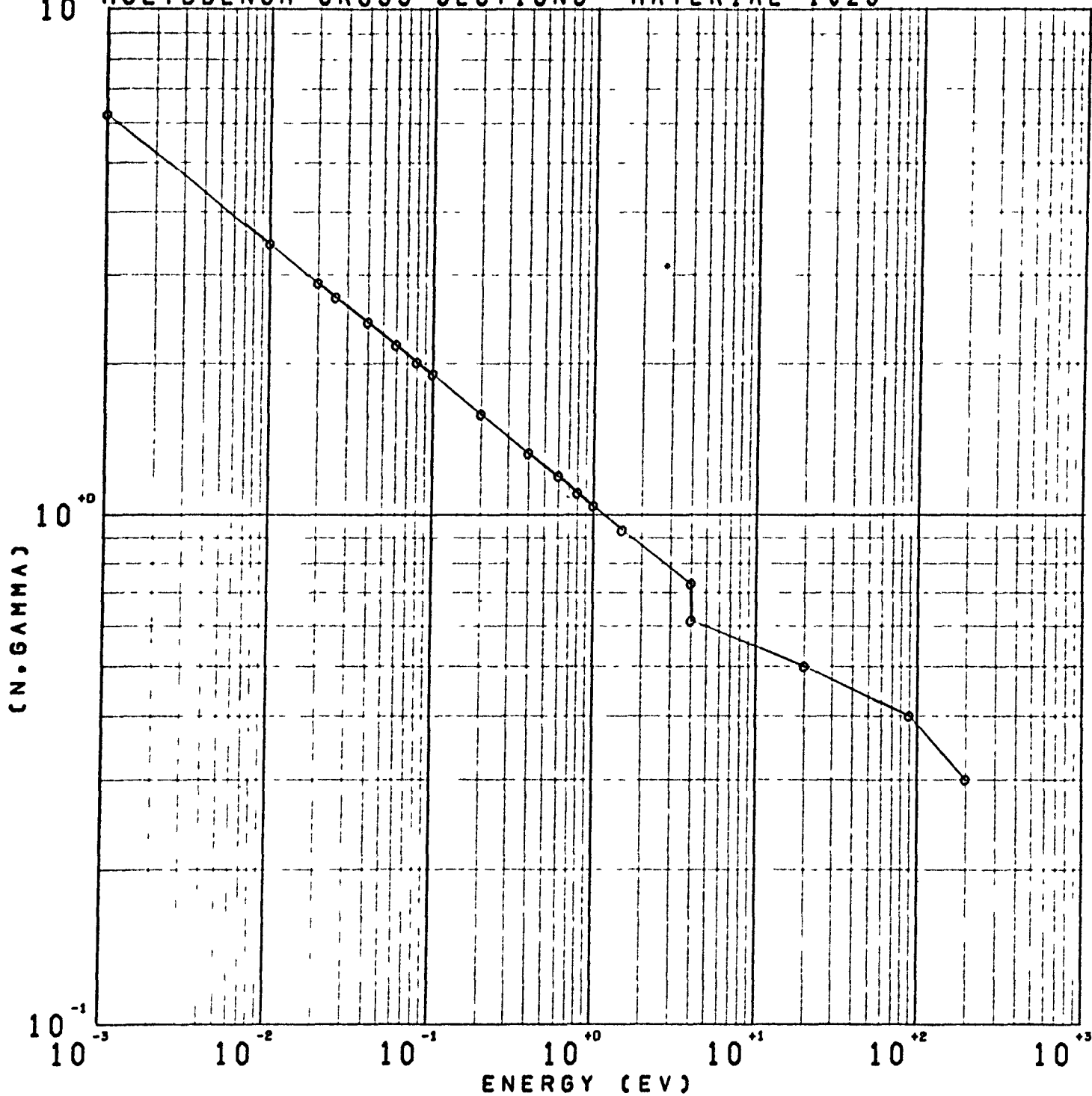
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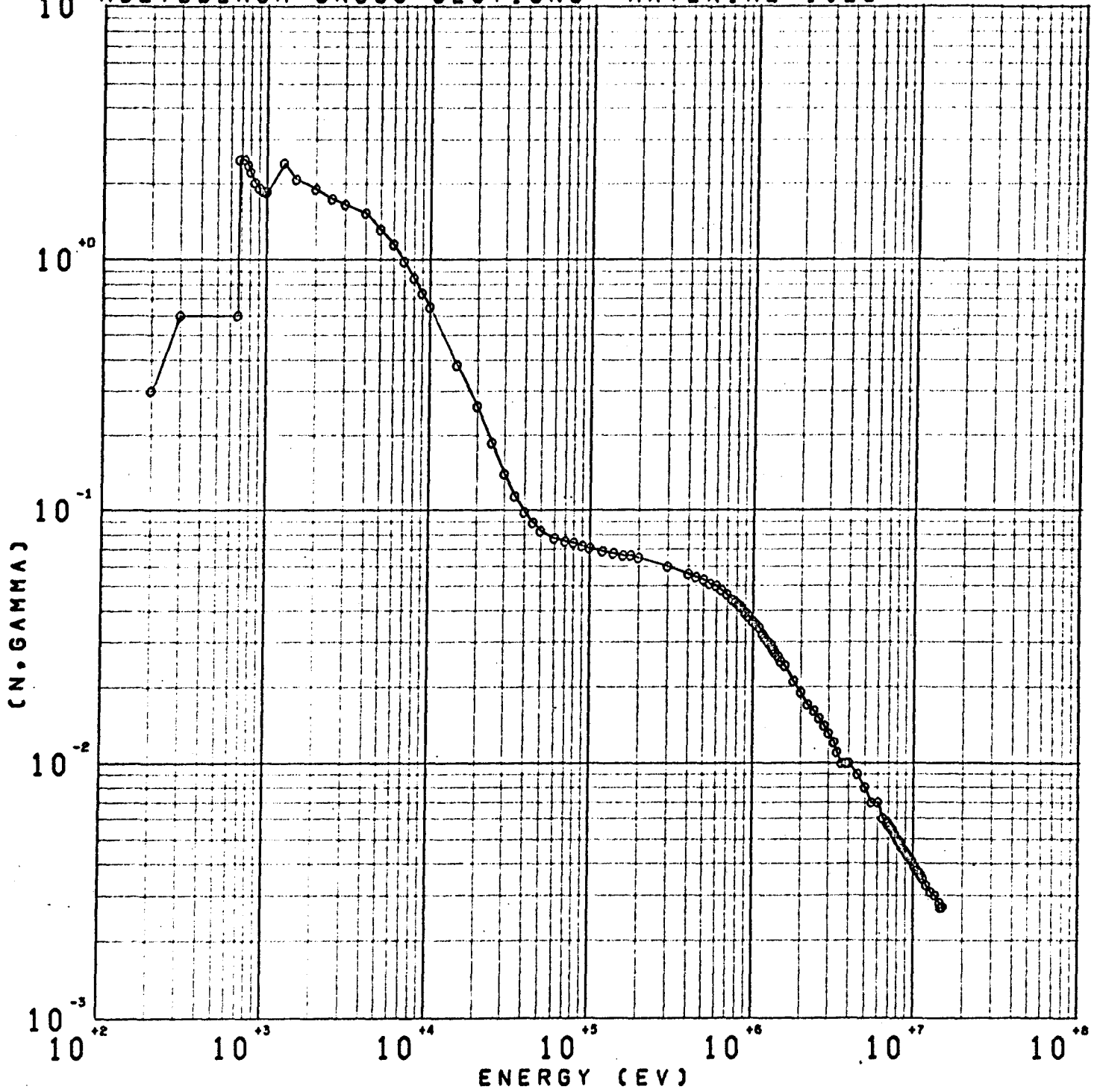
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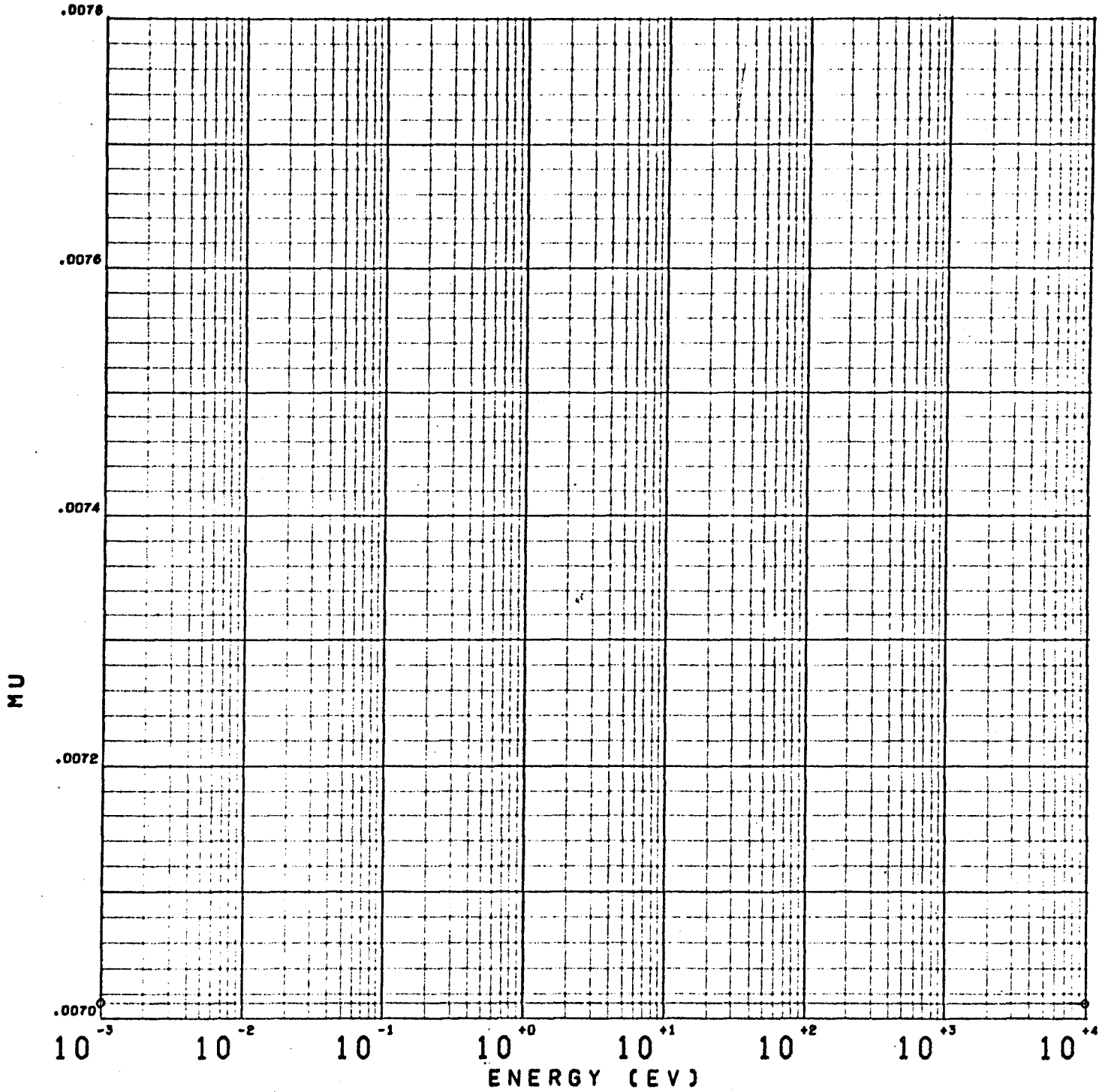
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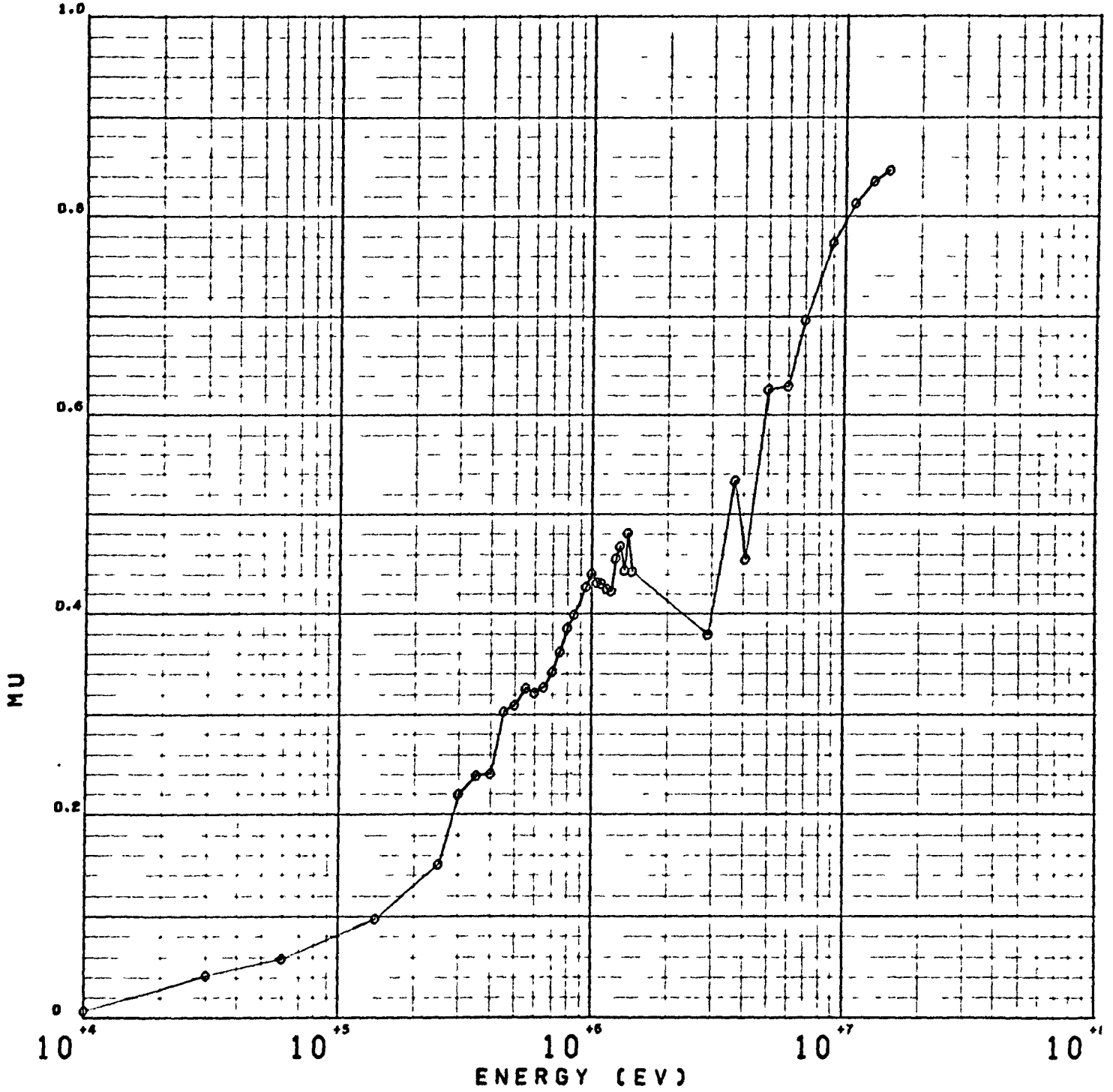


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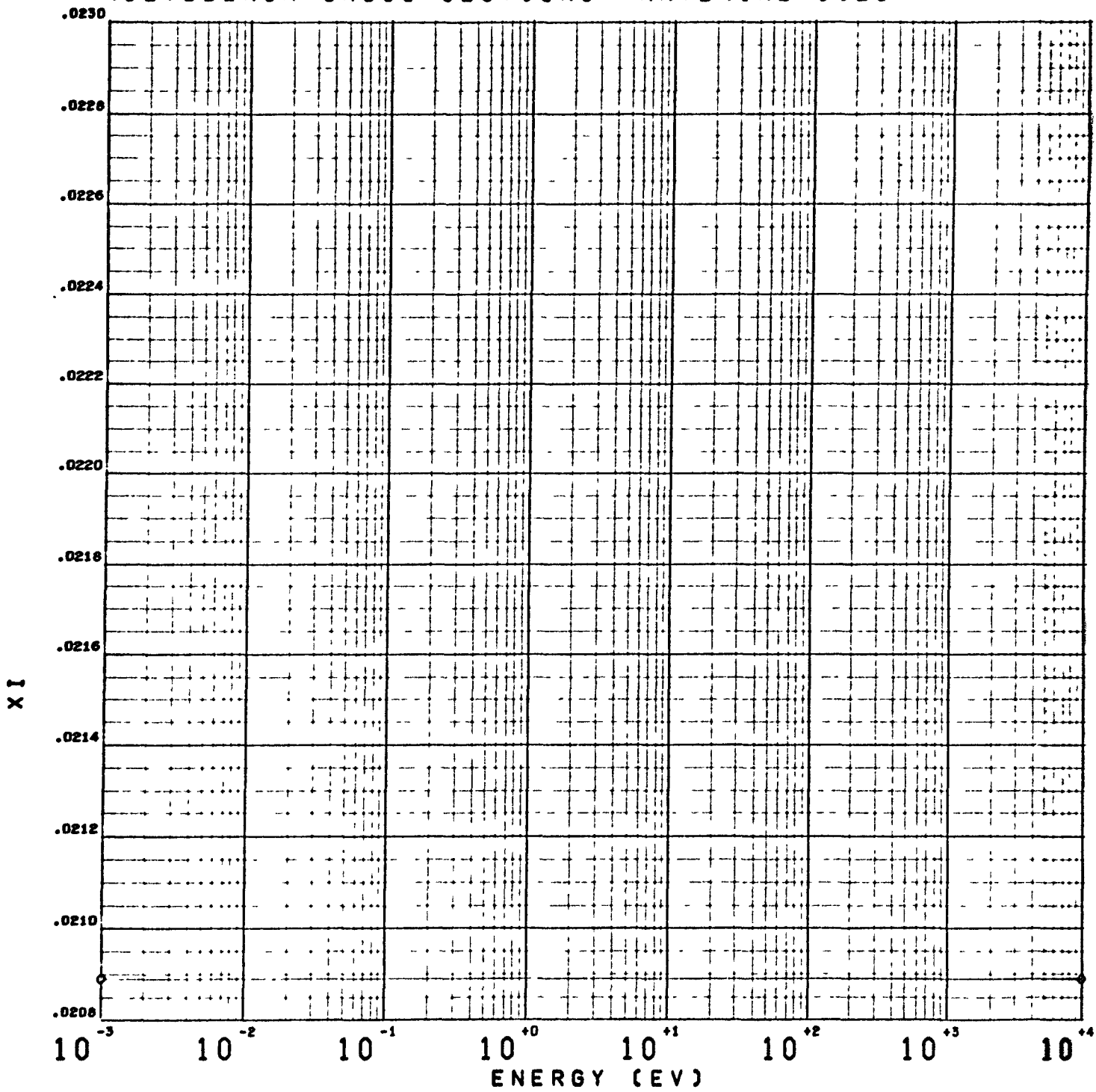


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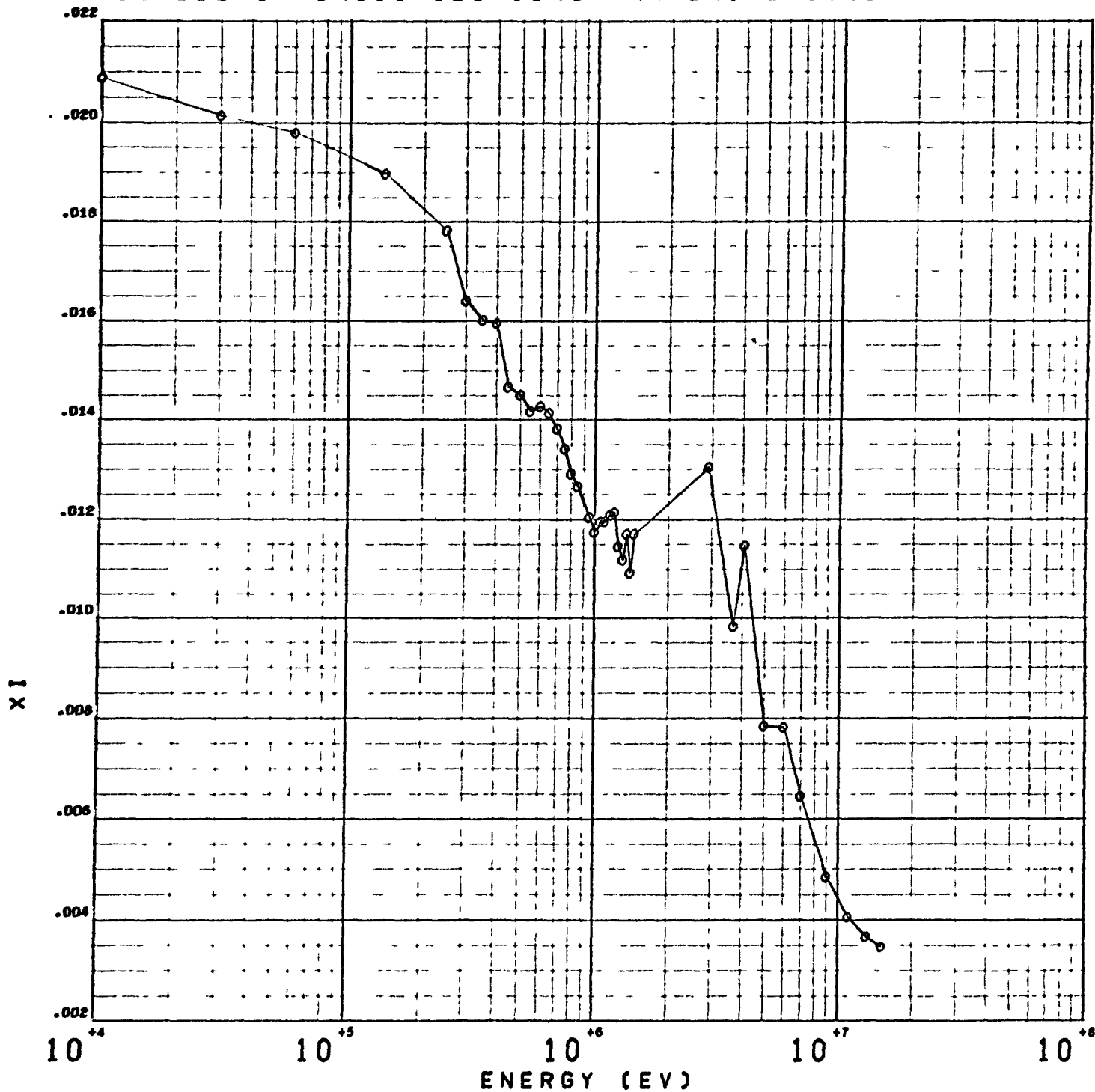
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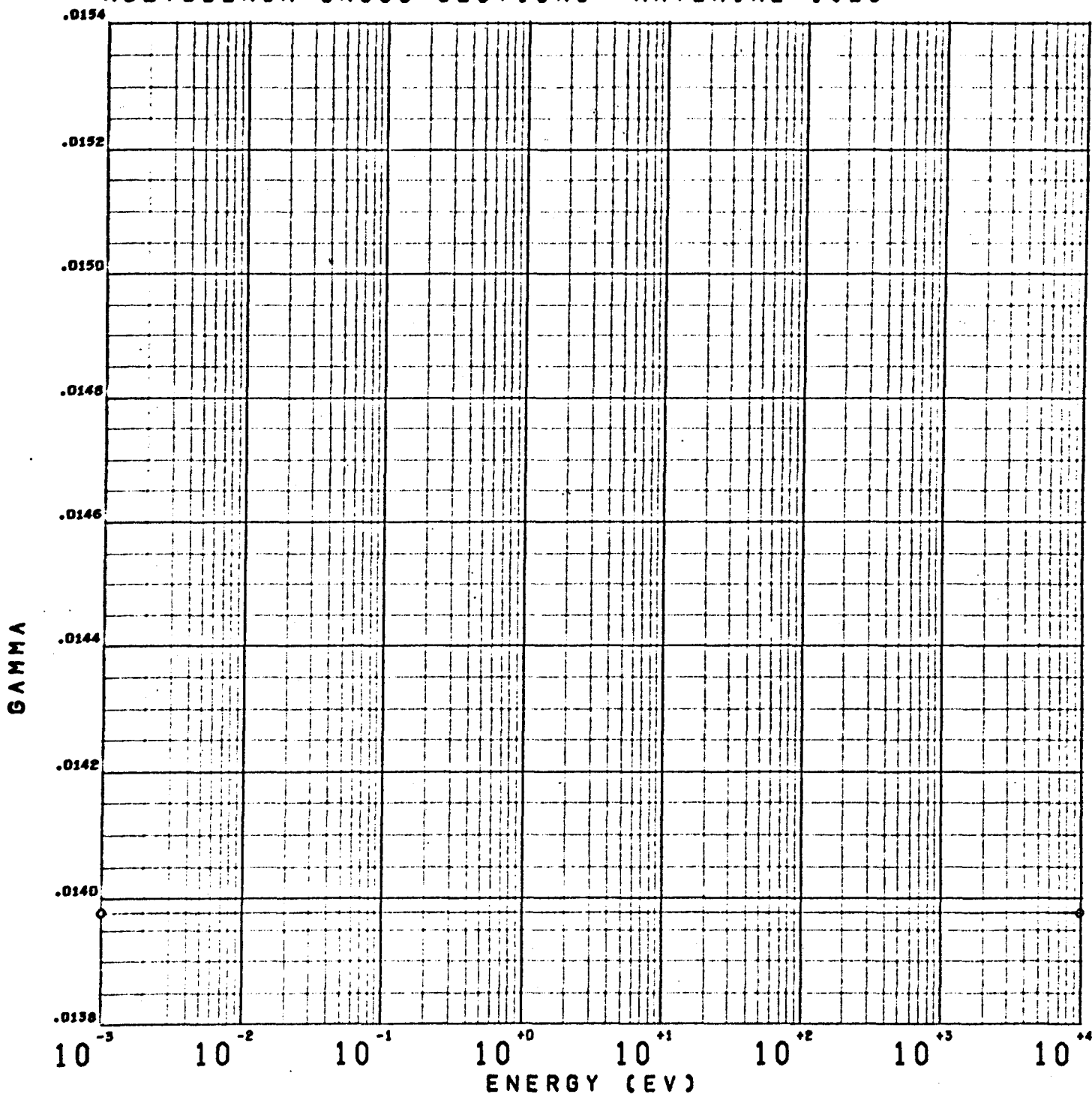
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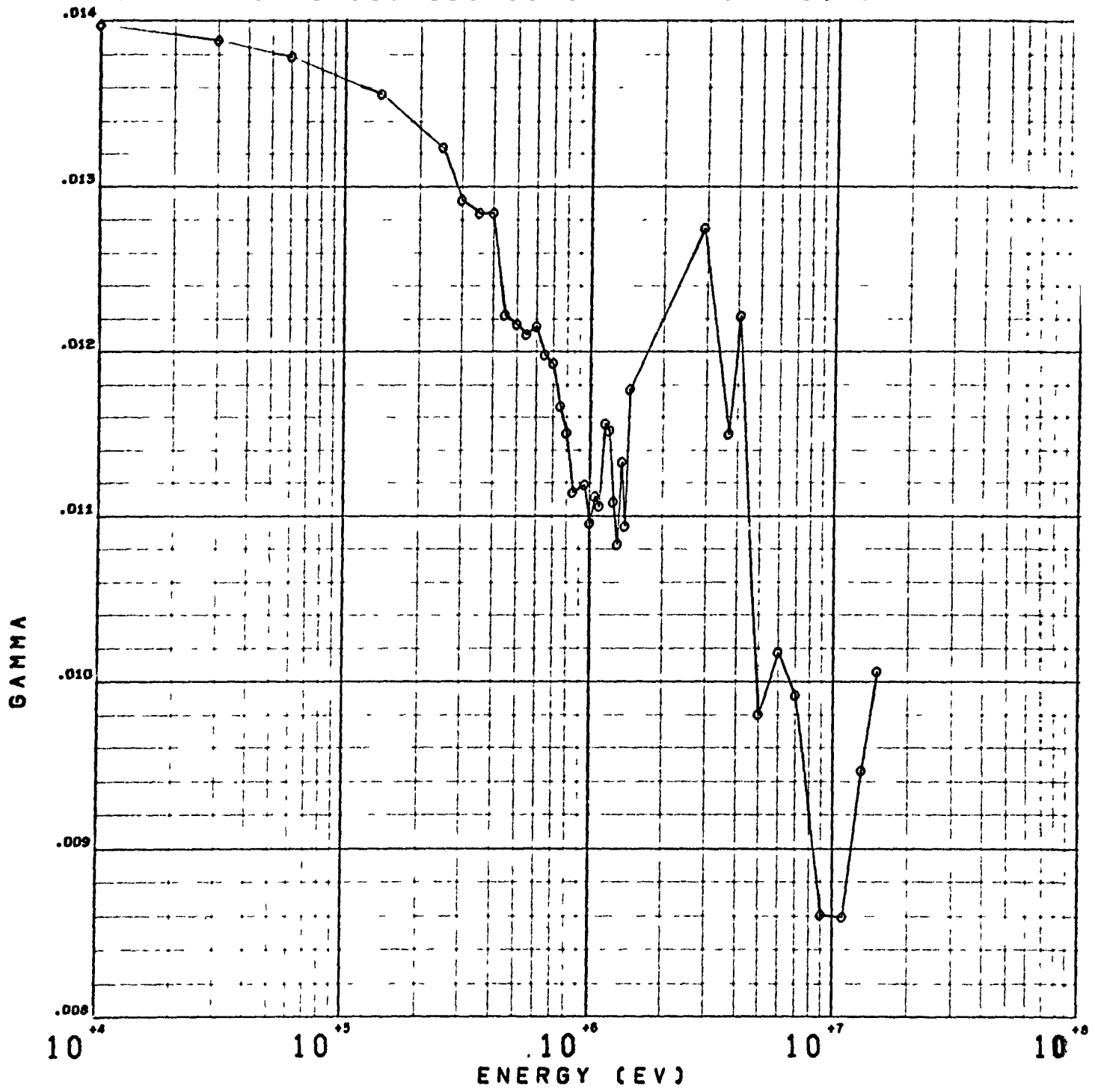
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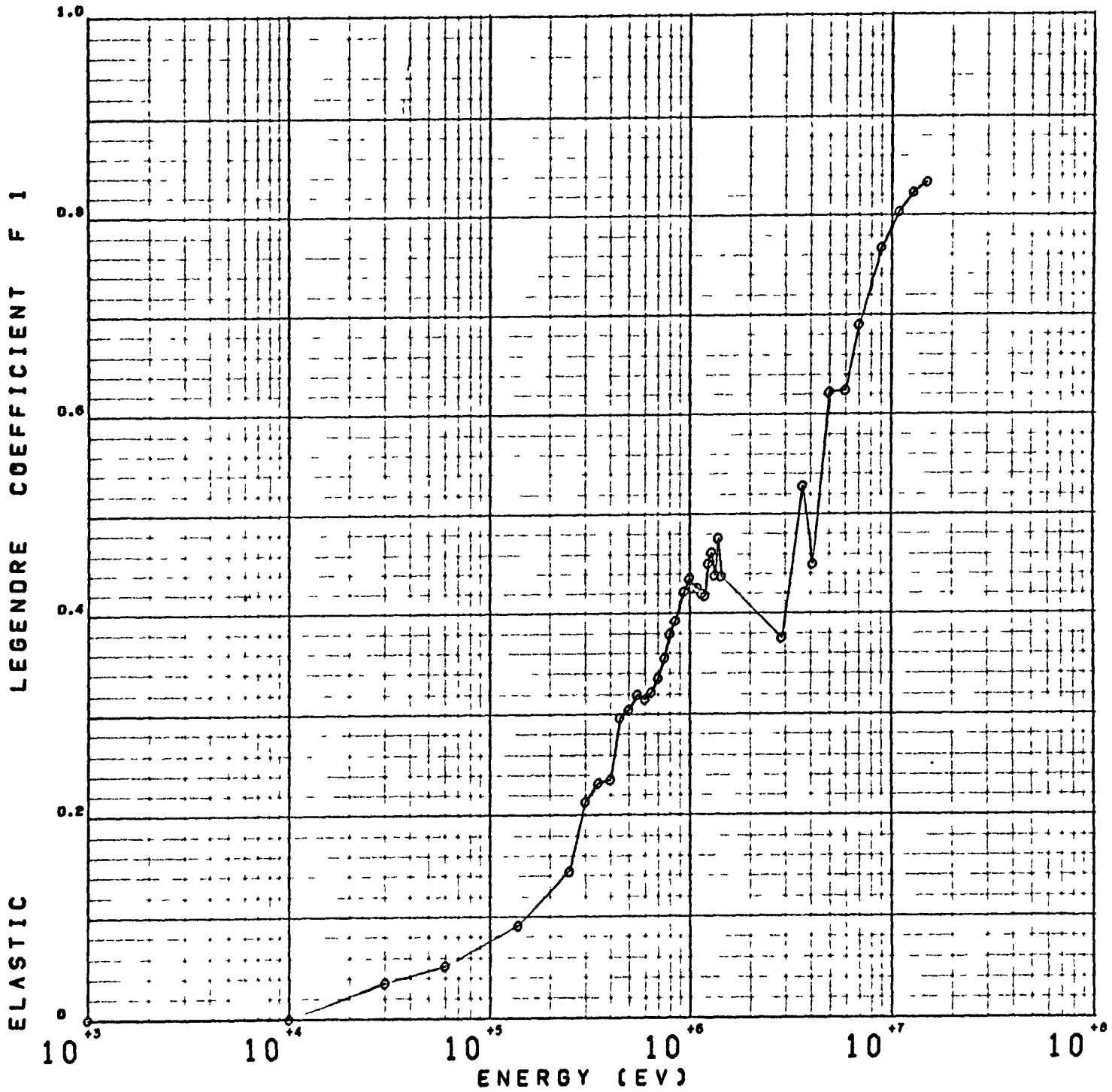


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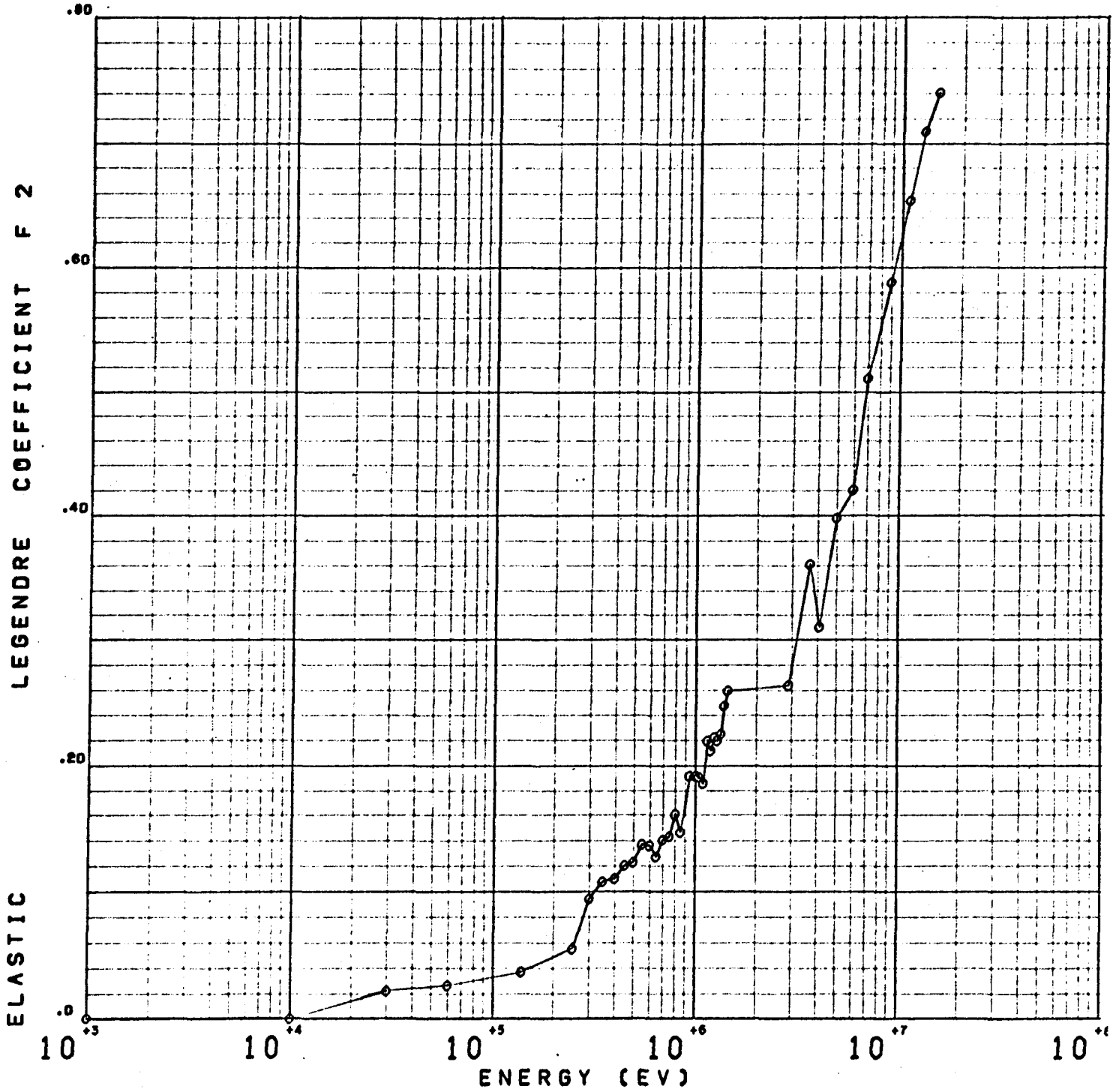


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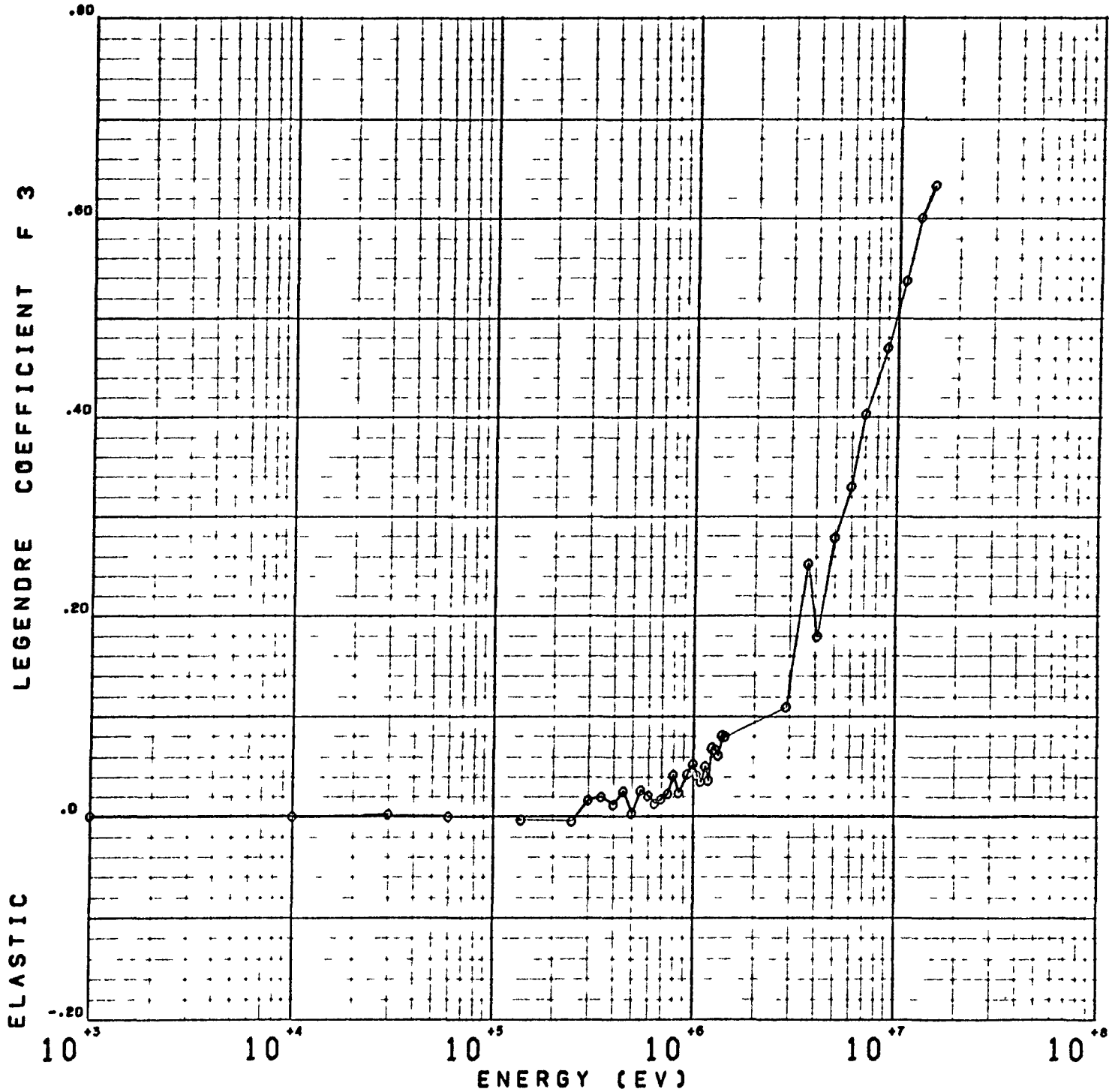
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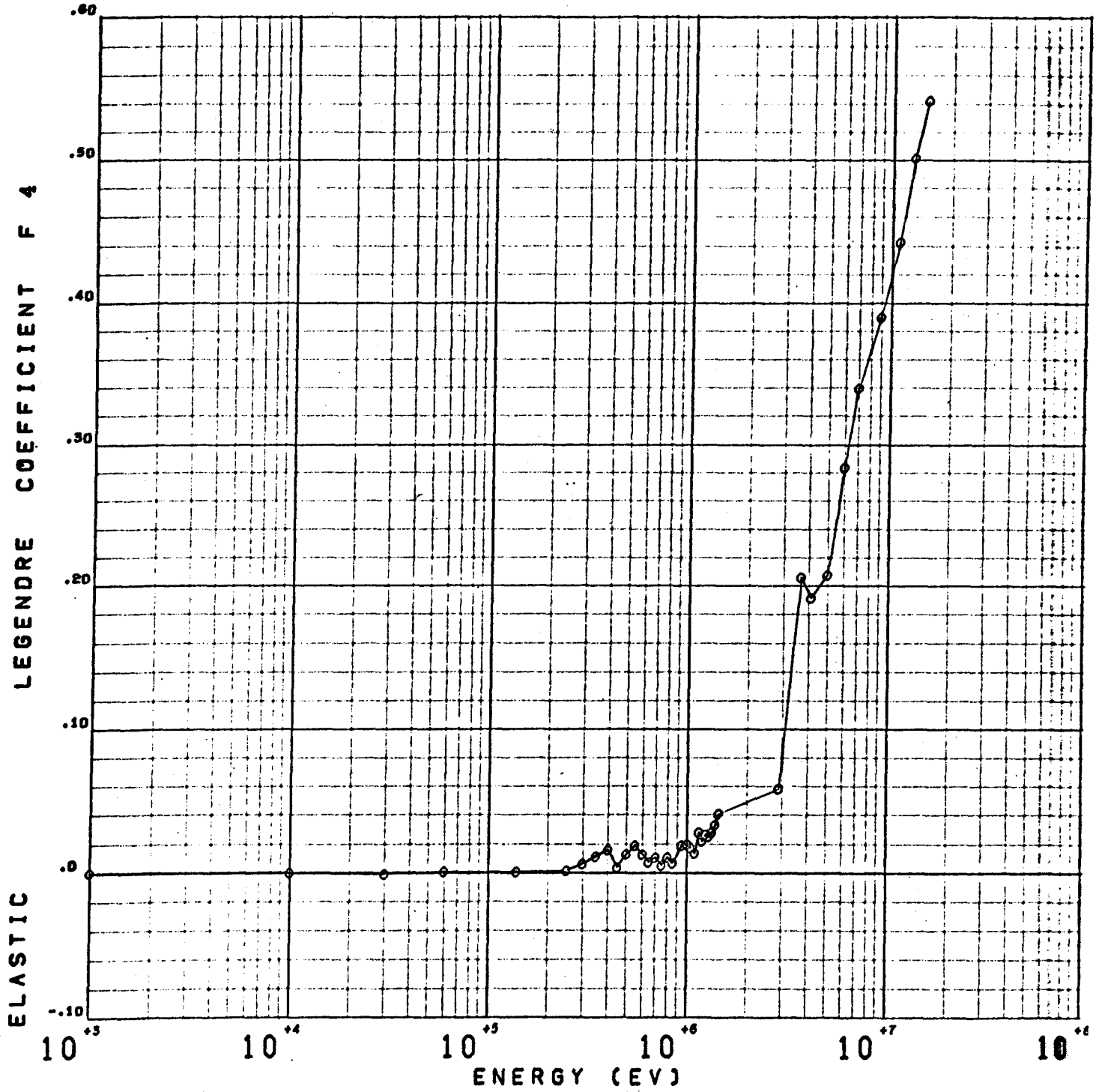
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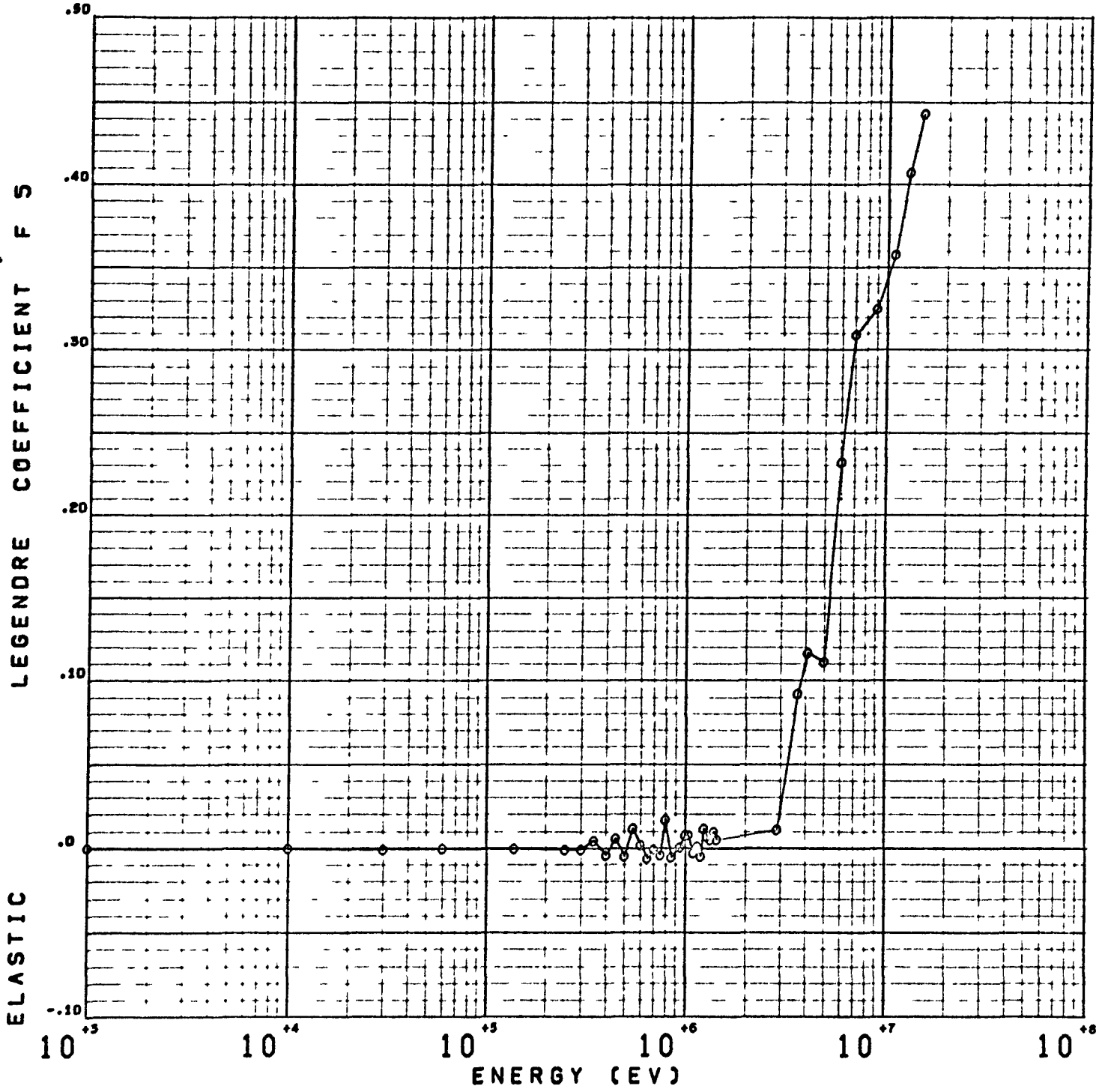
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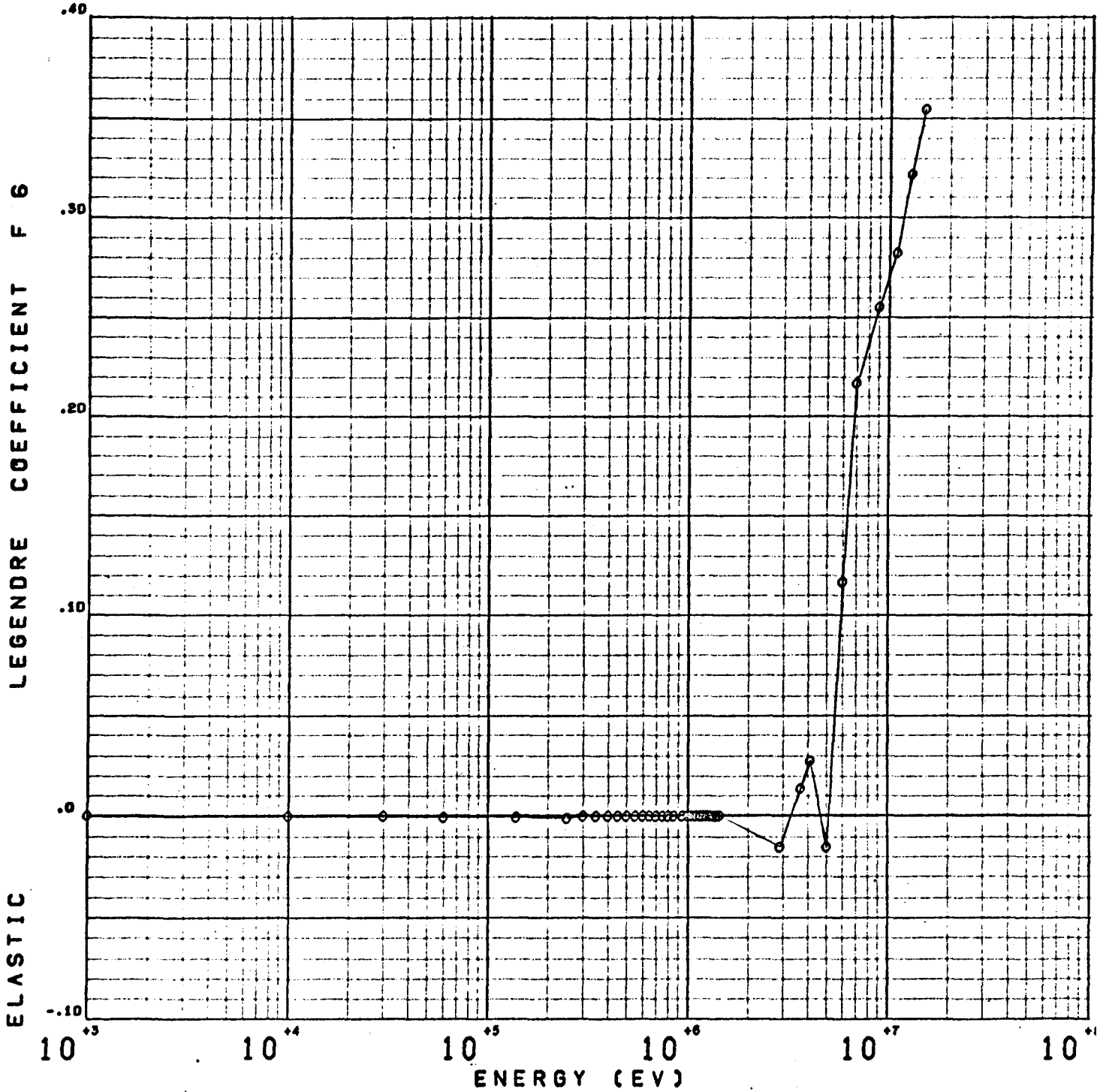


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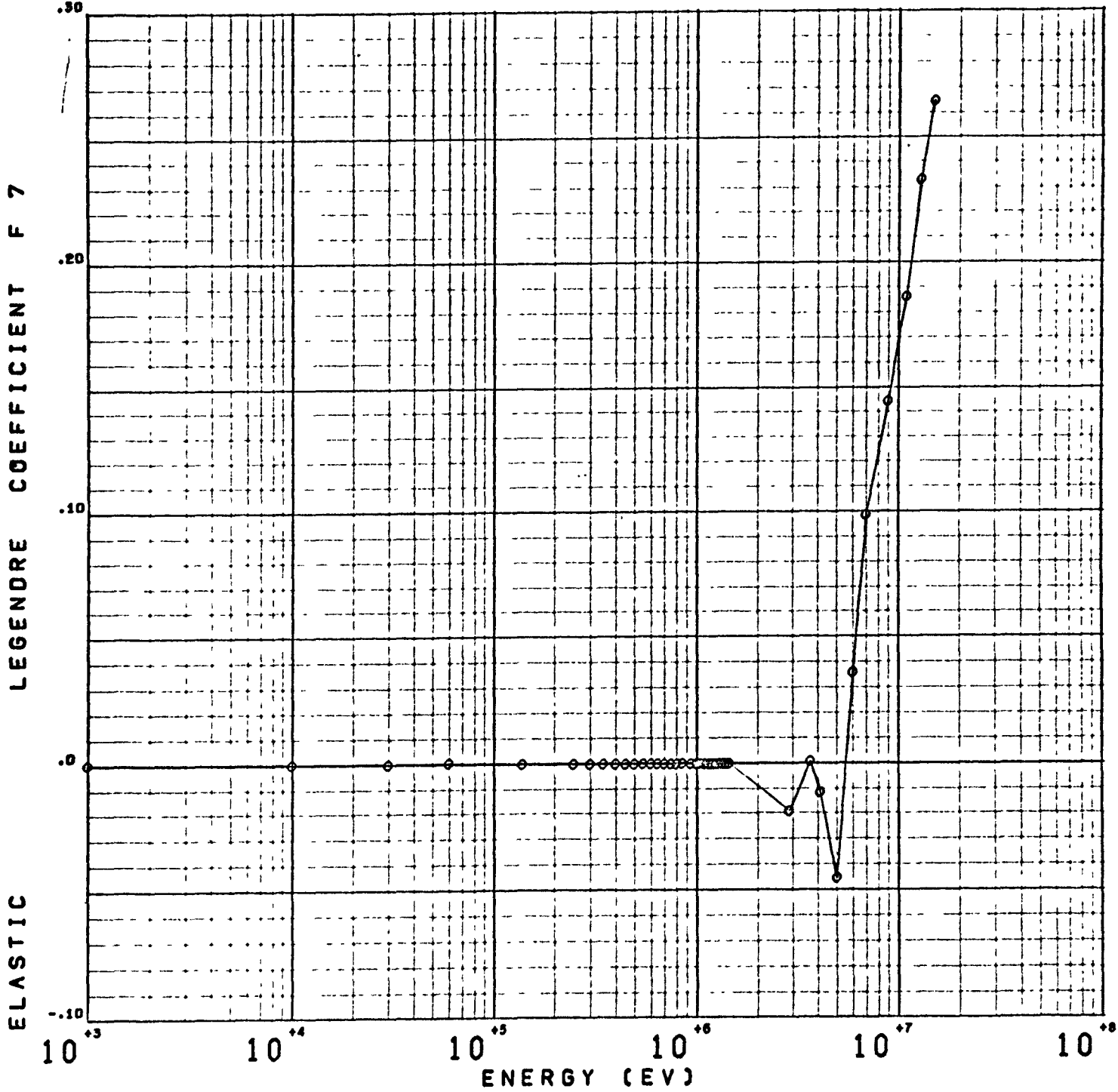
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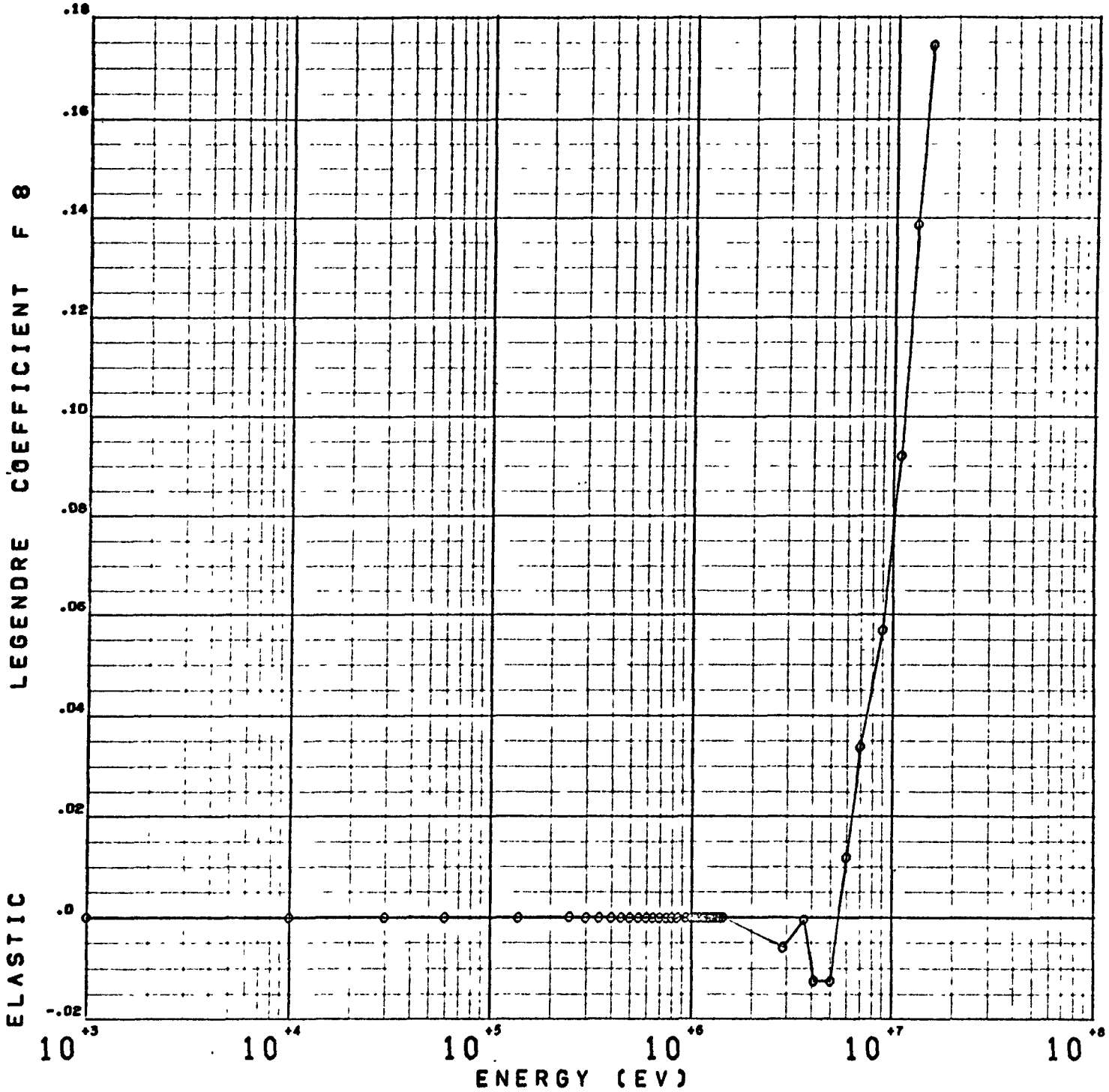
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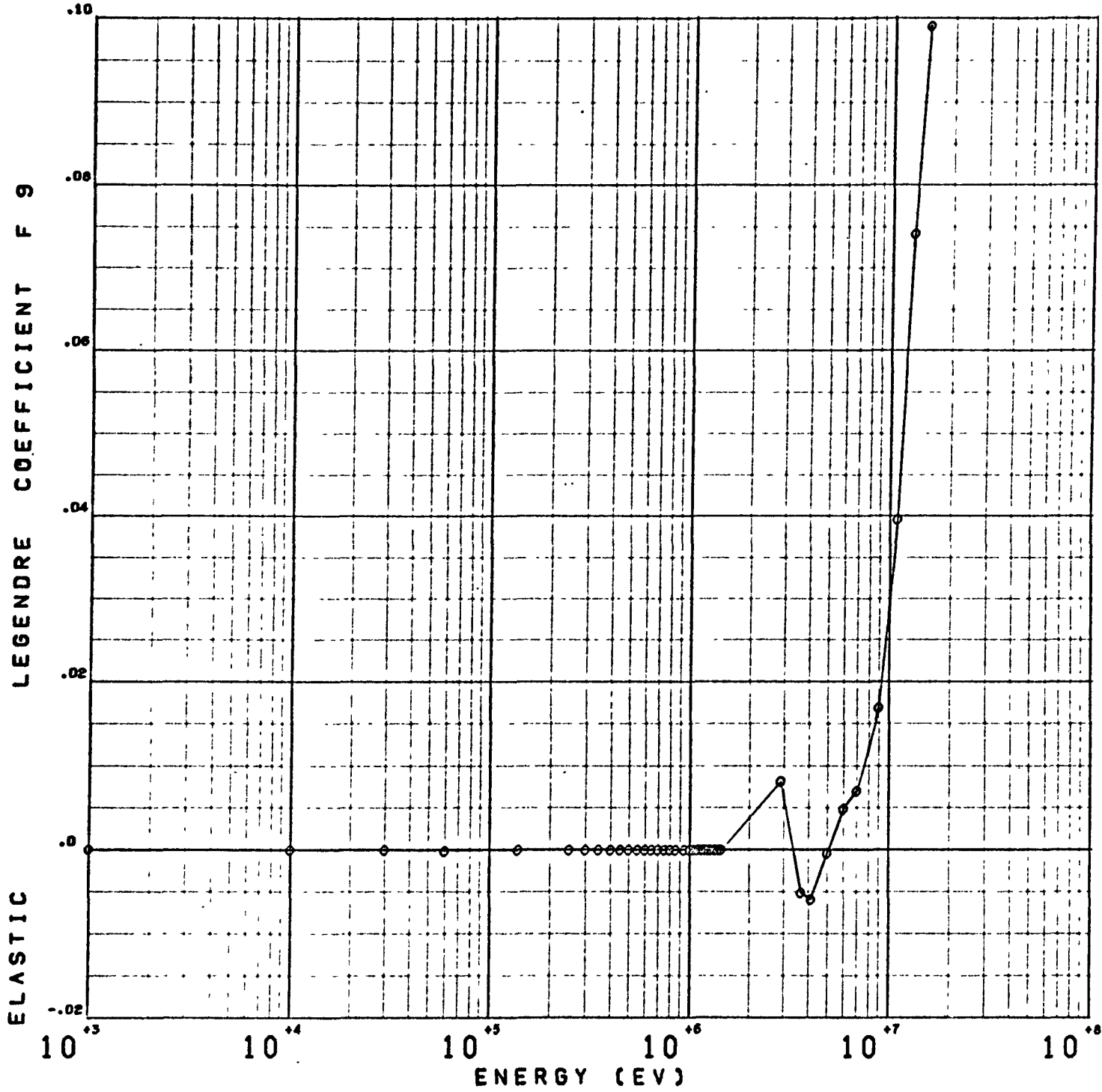
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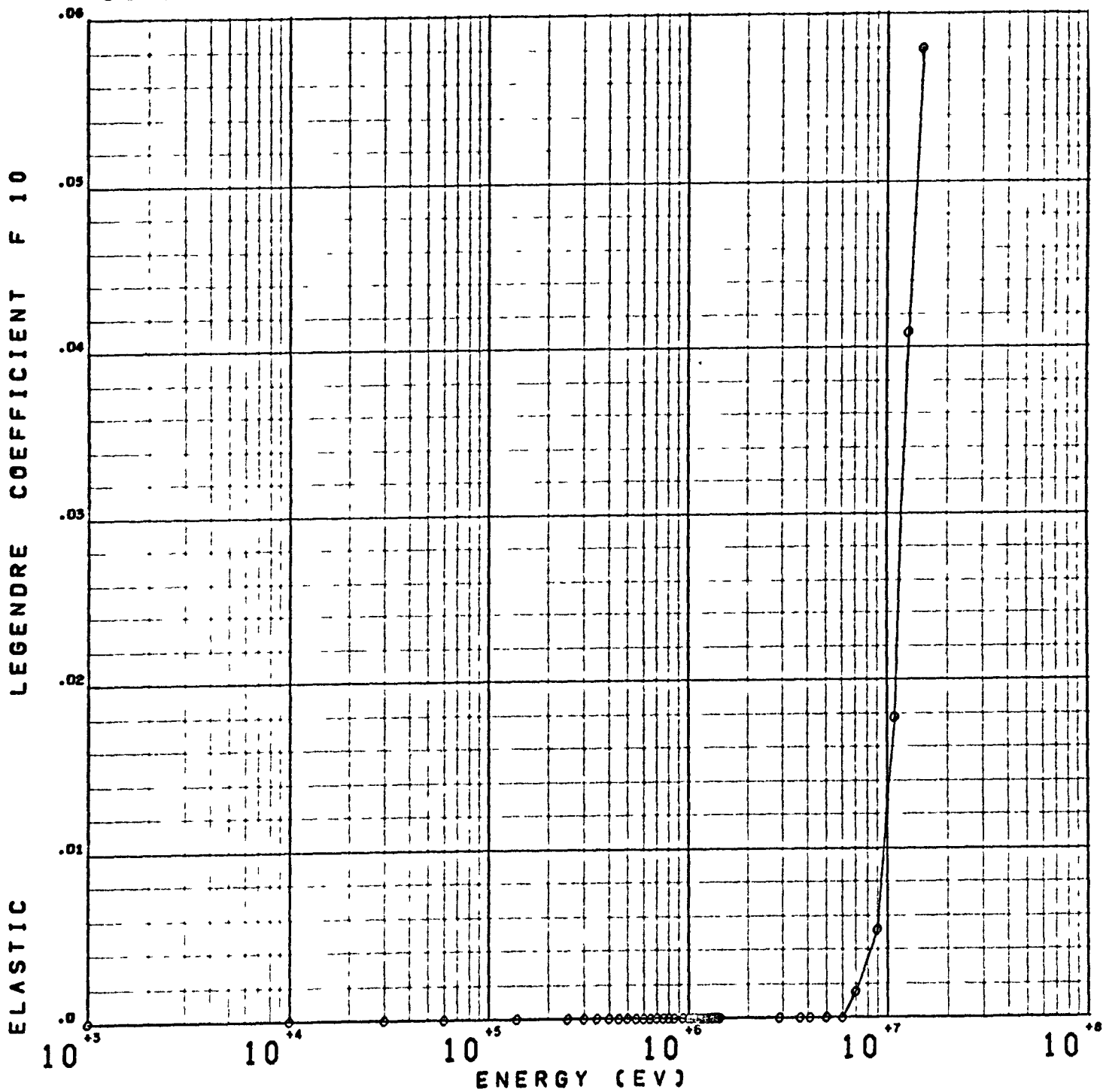
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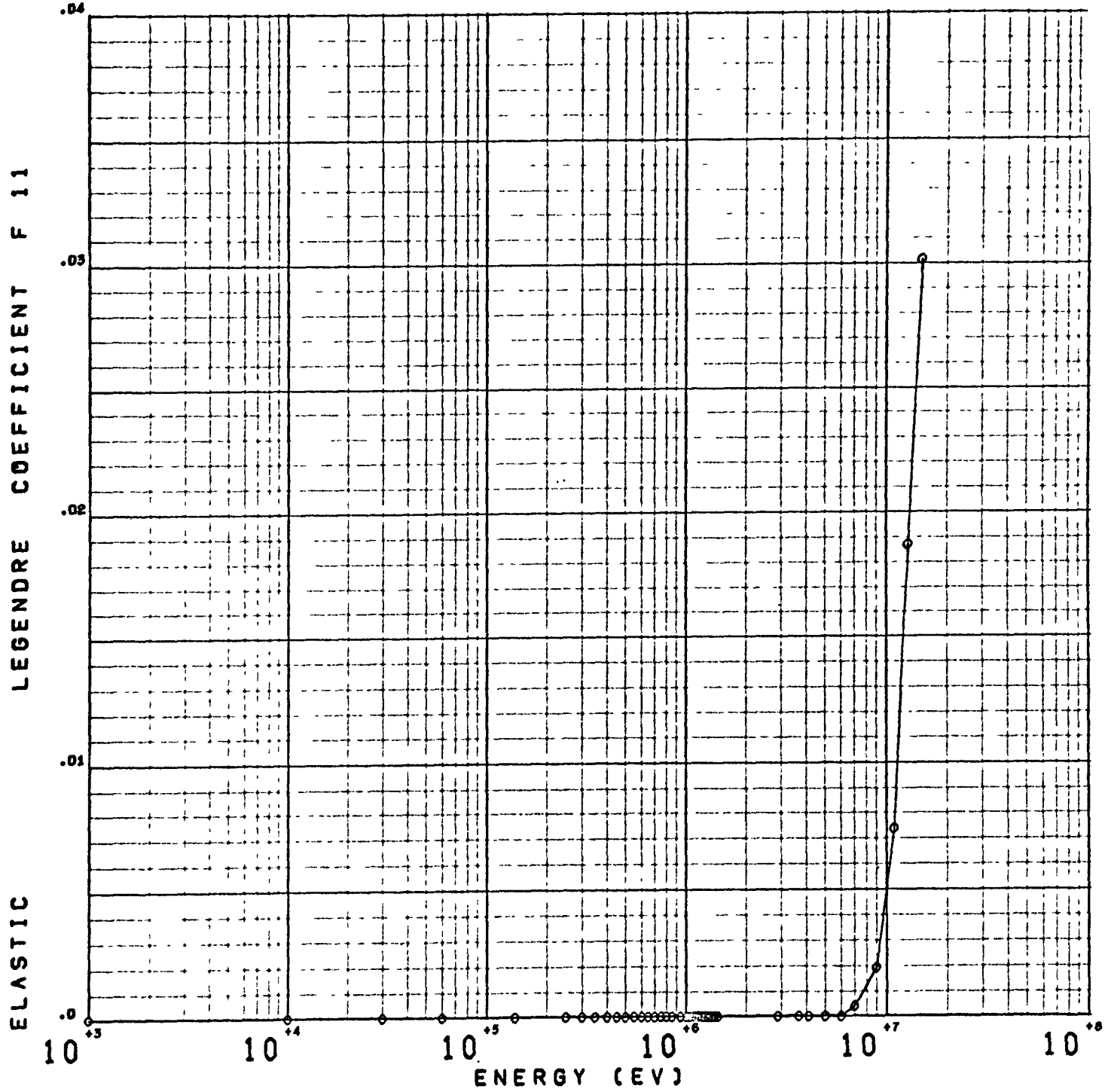


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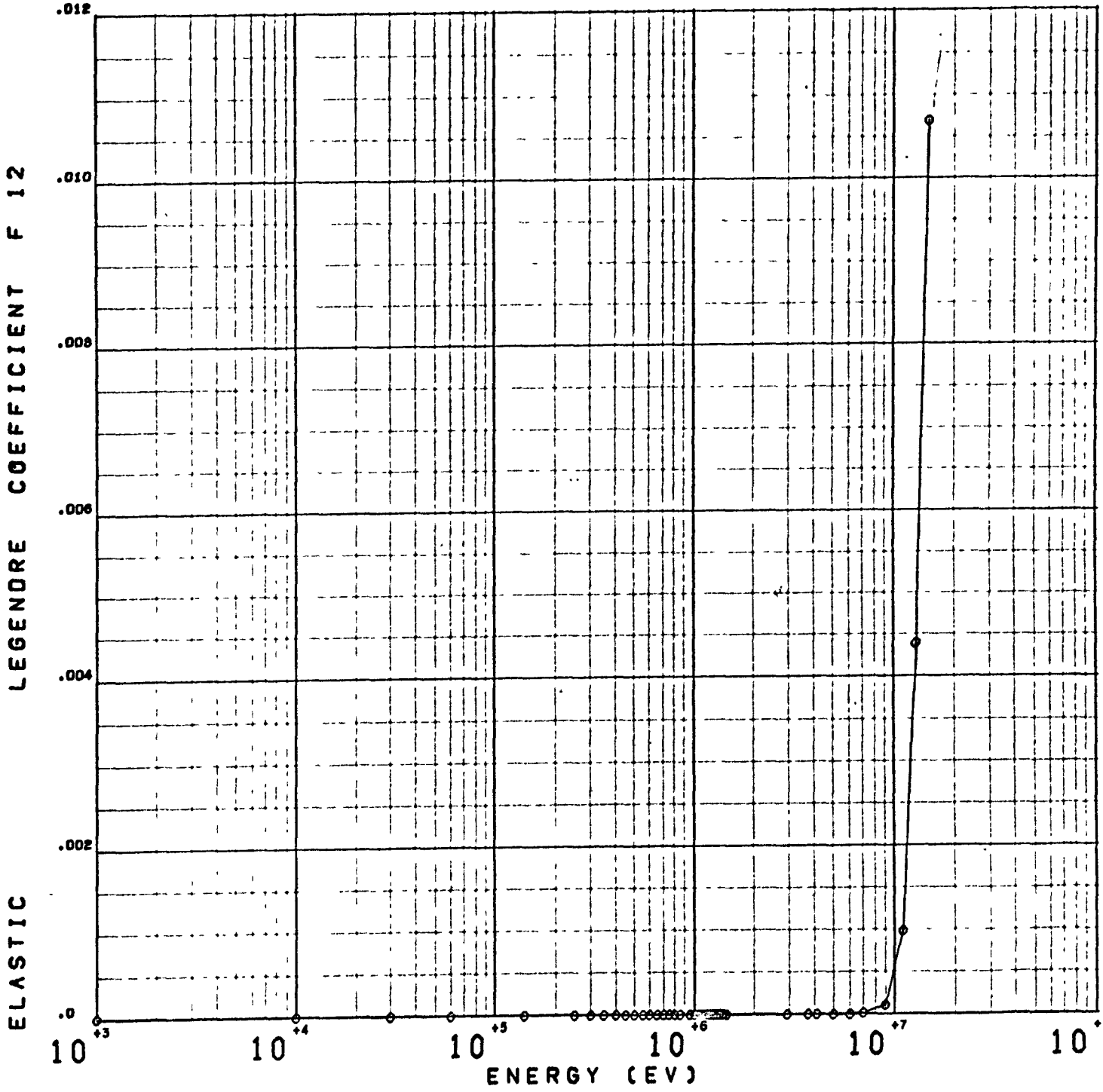


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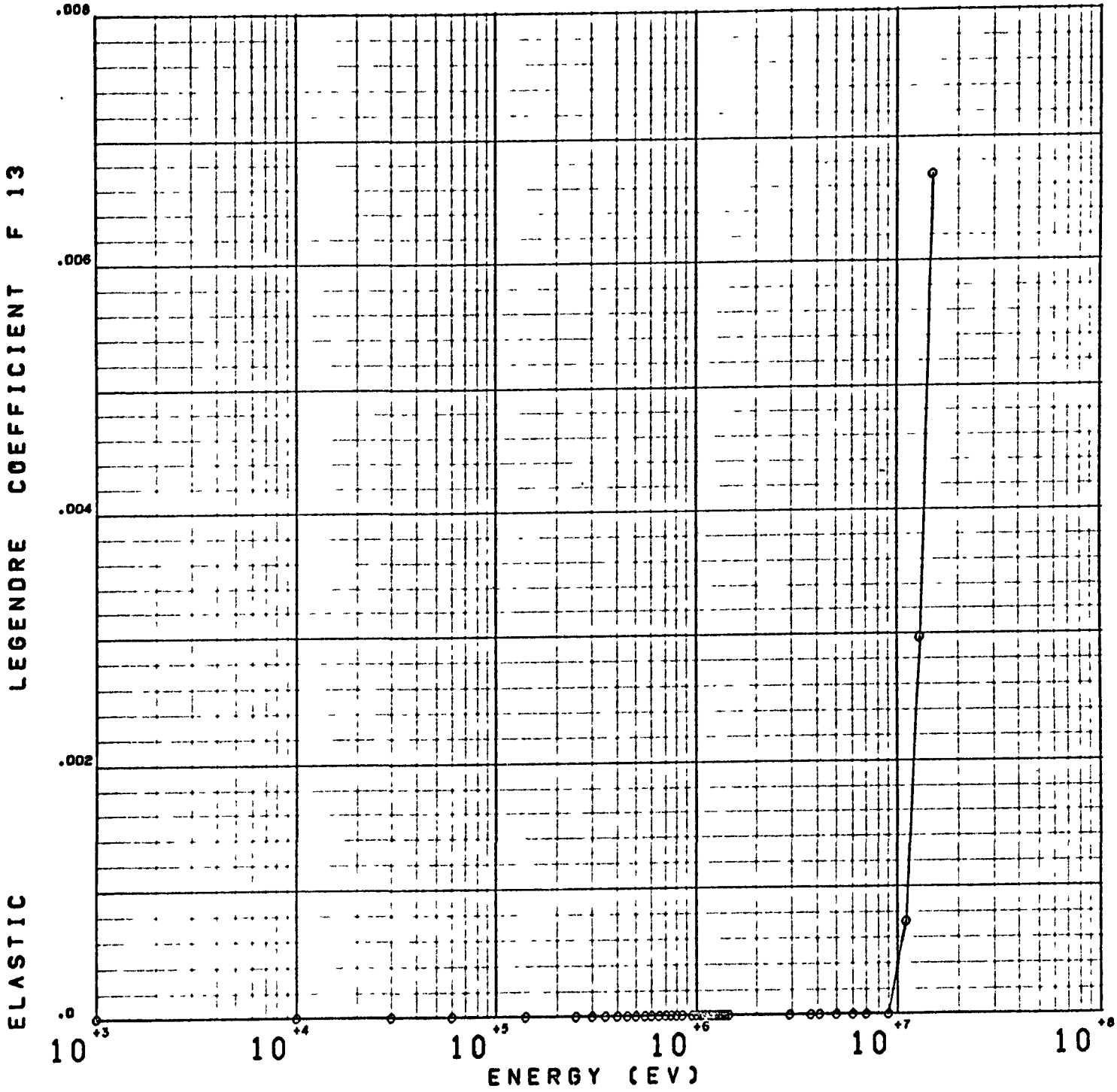


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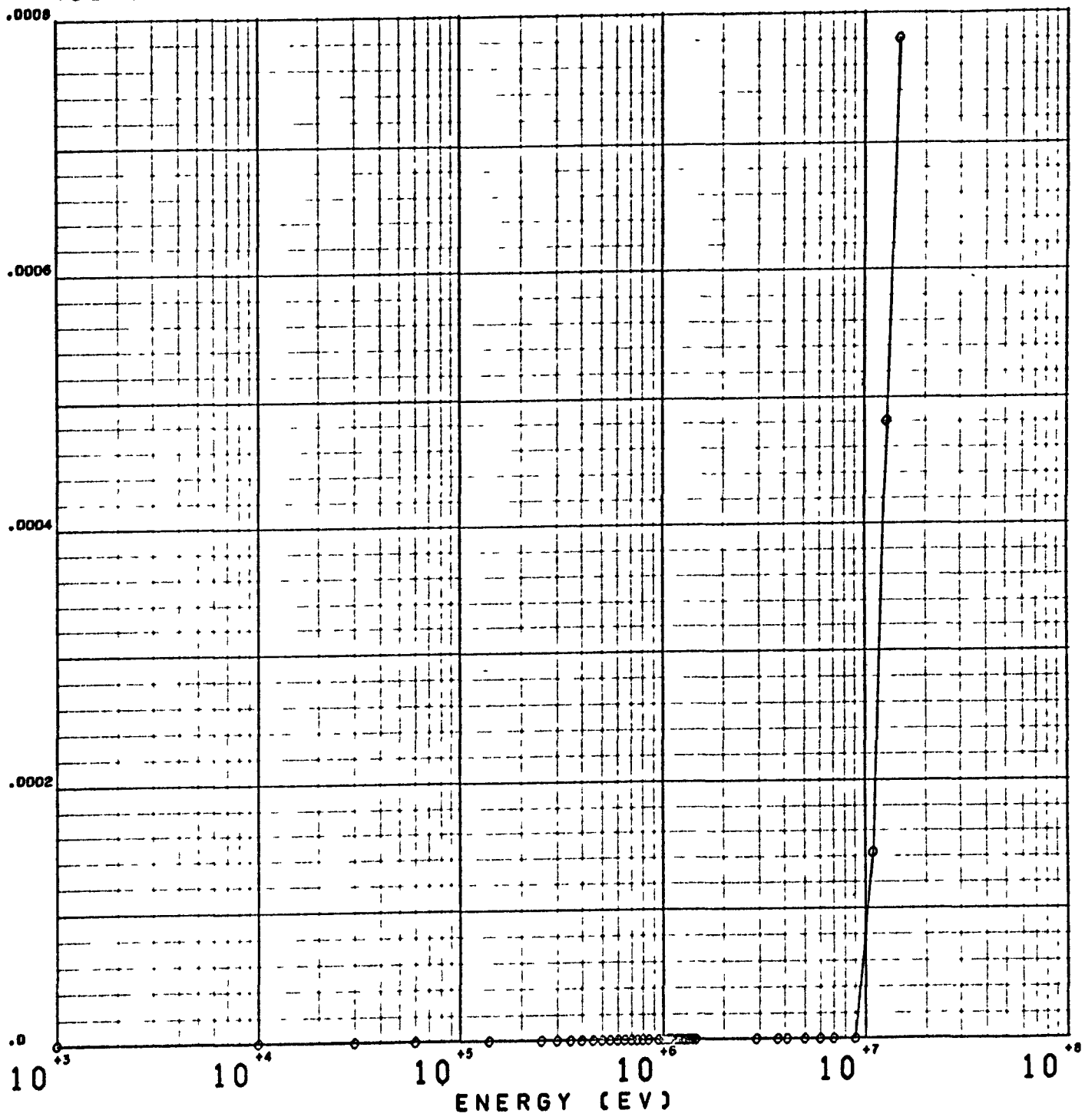
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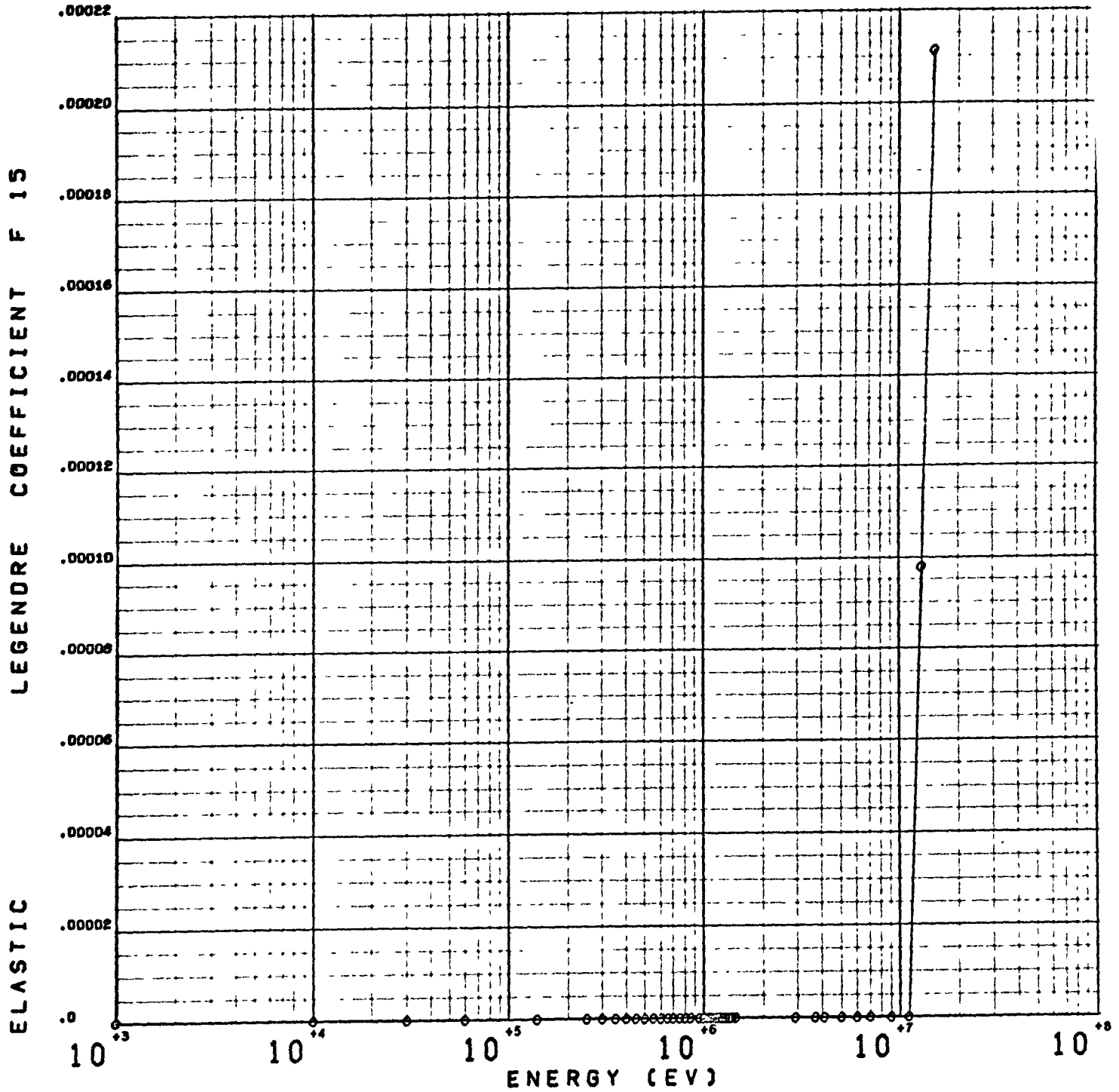
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ELASTIC LEGENDRE COEFFICIENT F 14



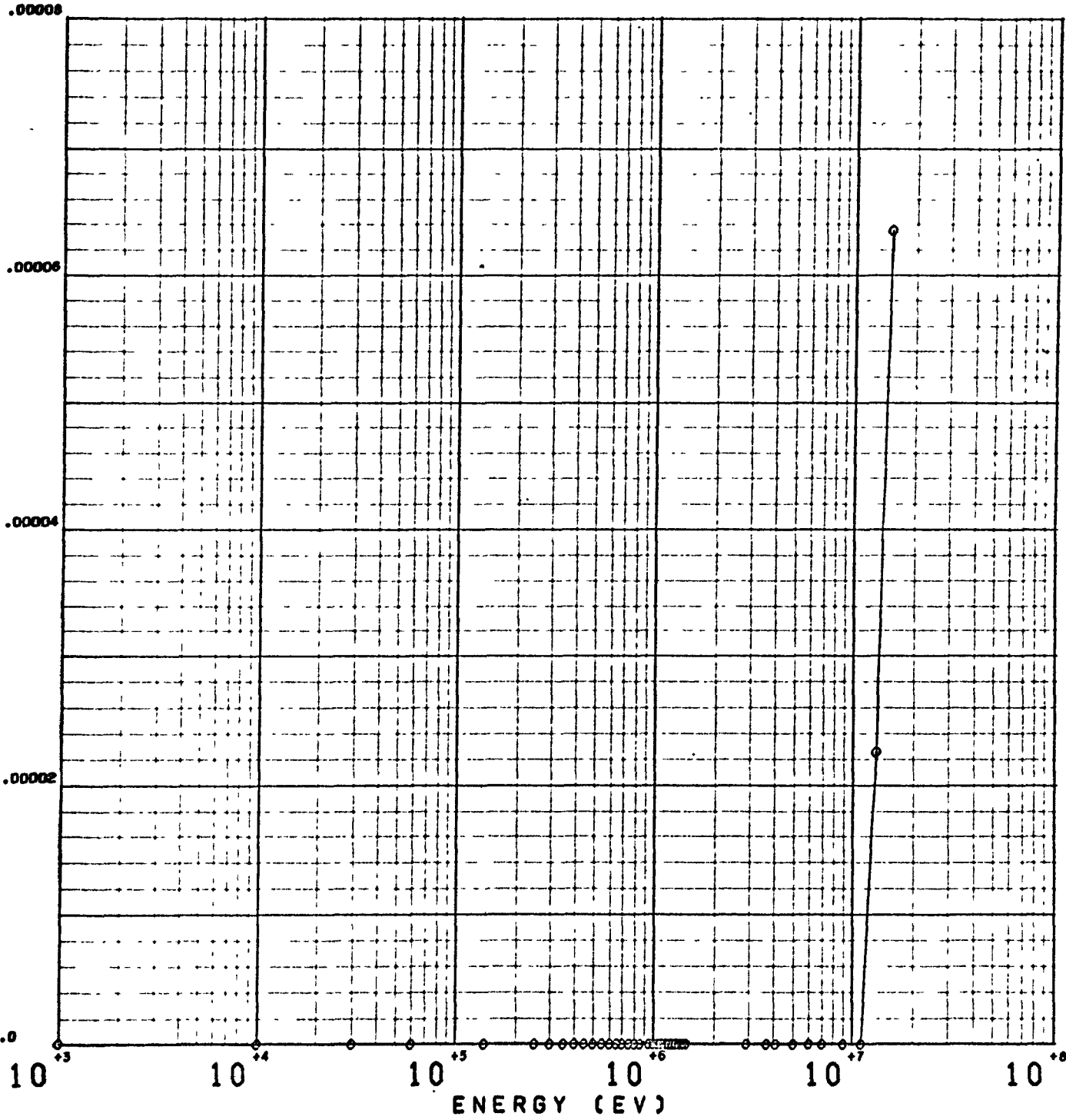
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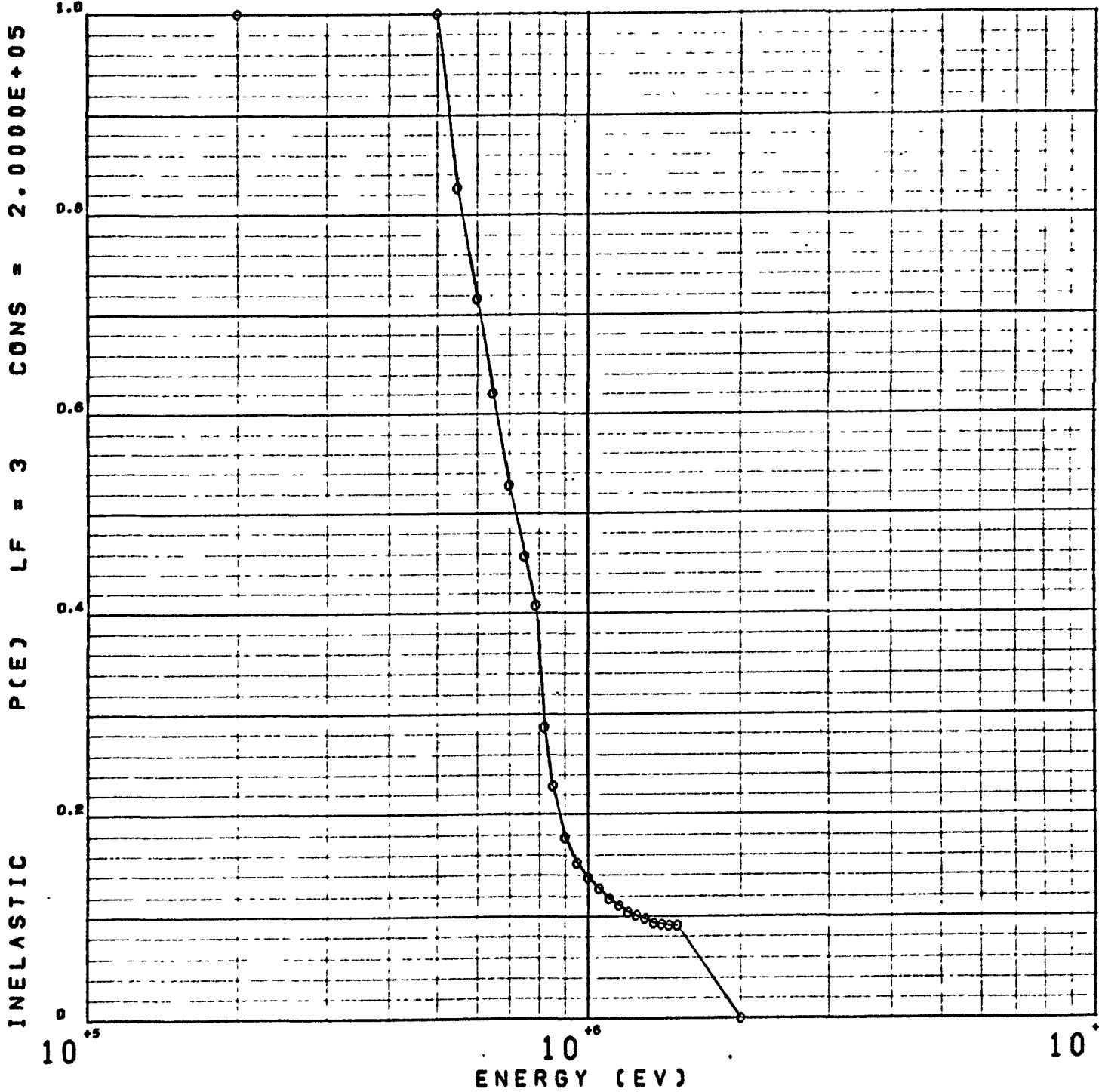
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ELASTIC
LEGENDRE COEFFICIENT F 16



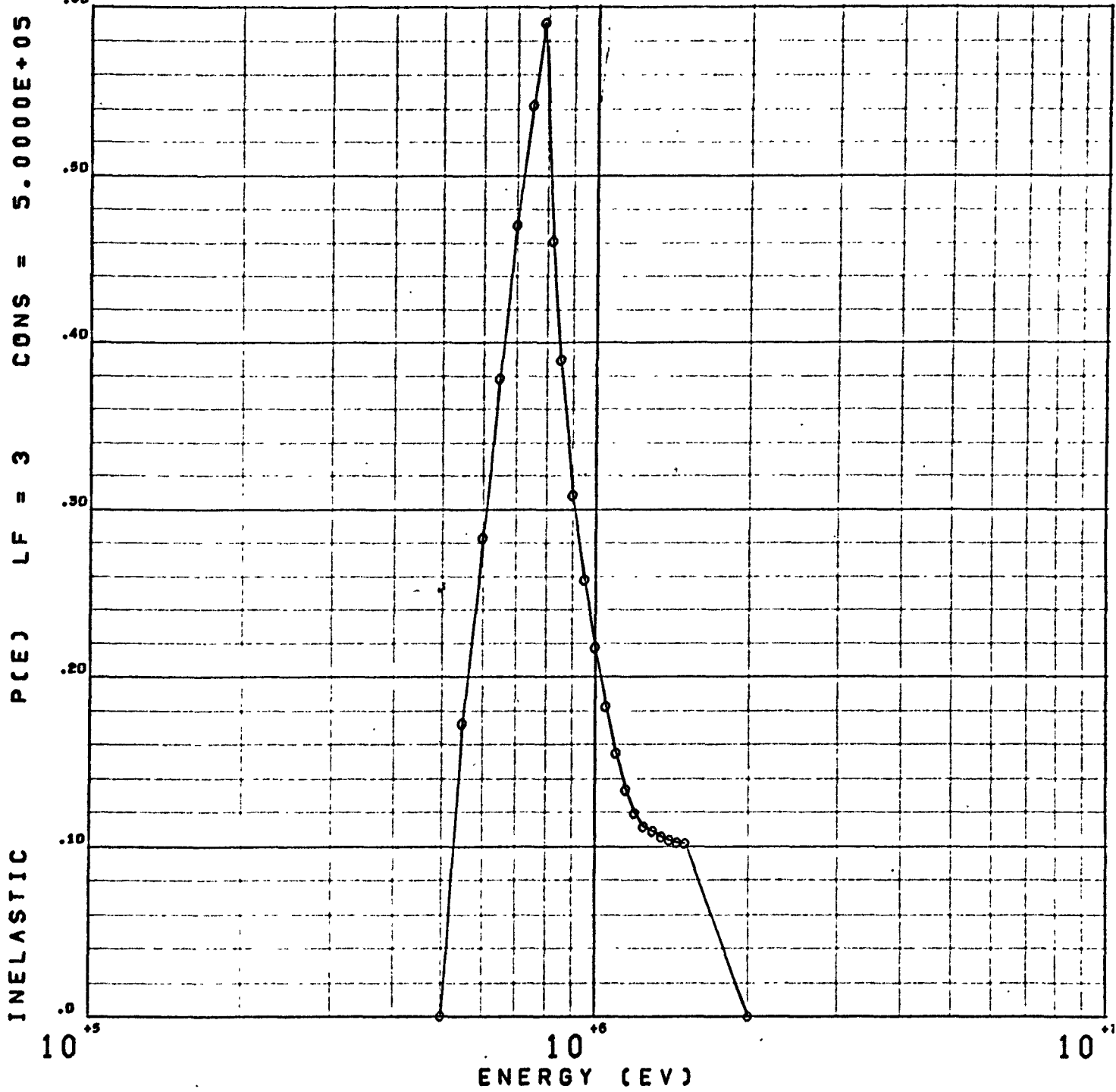
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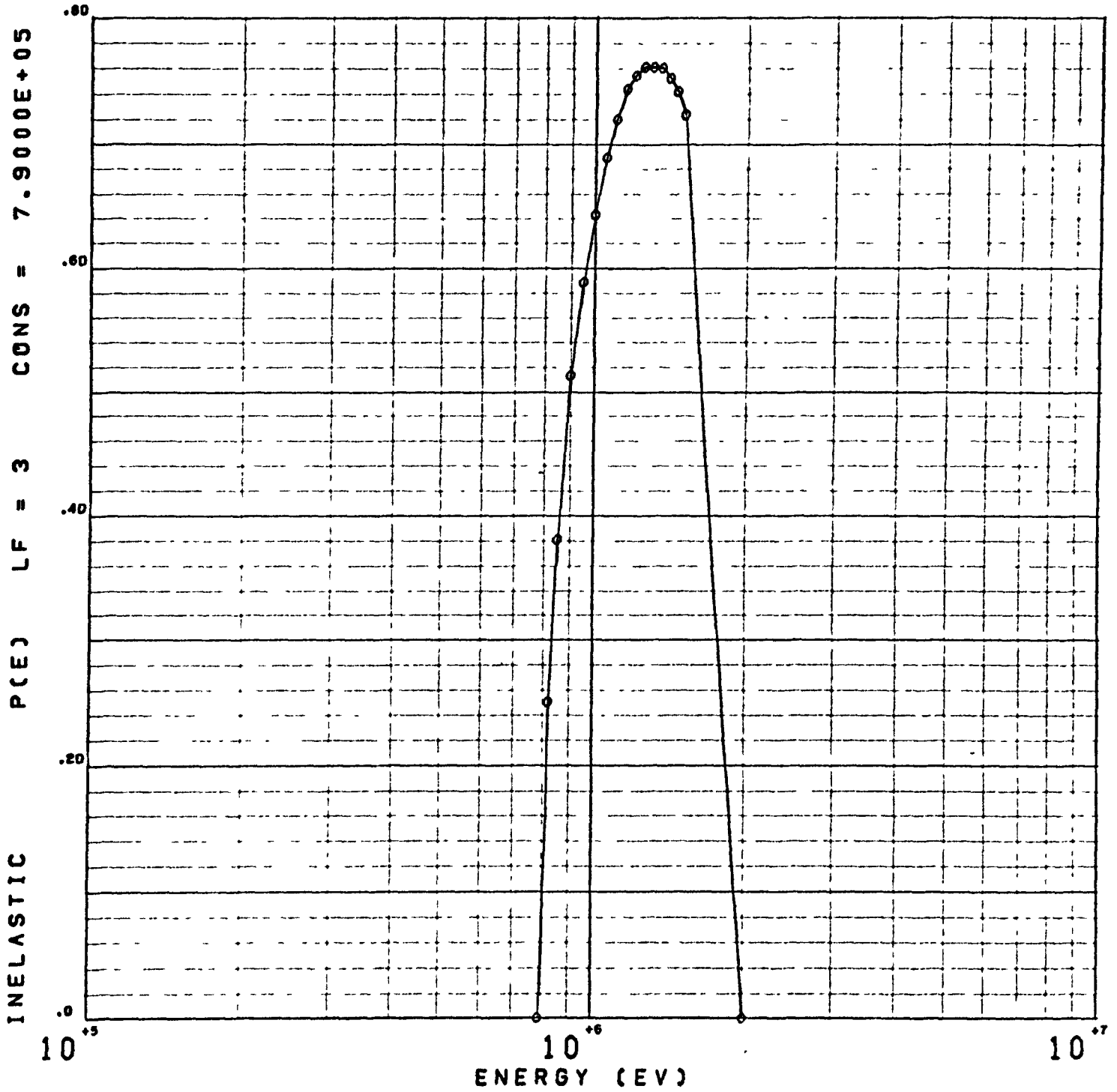
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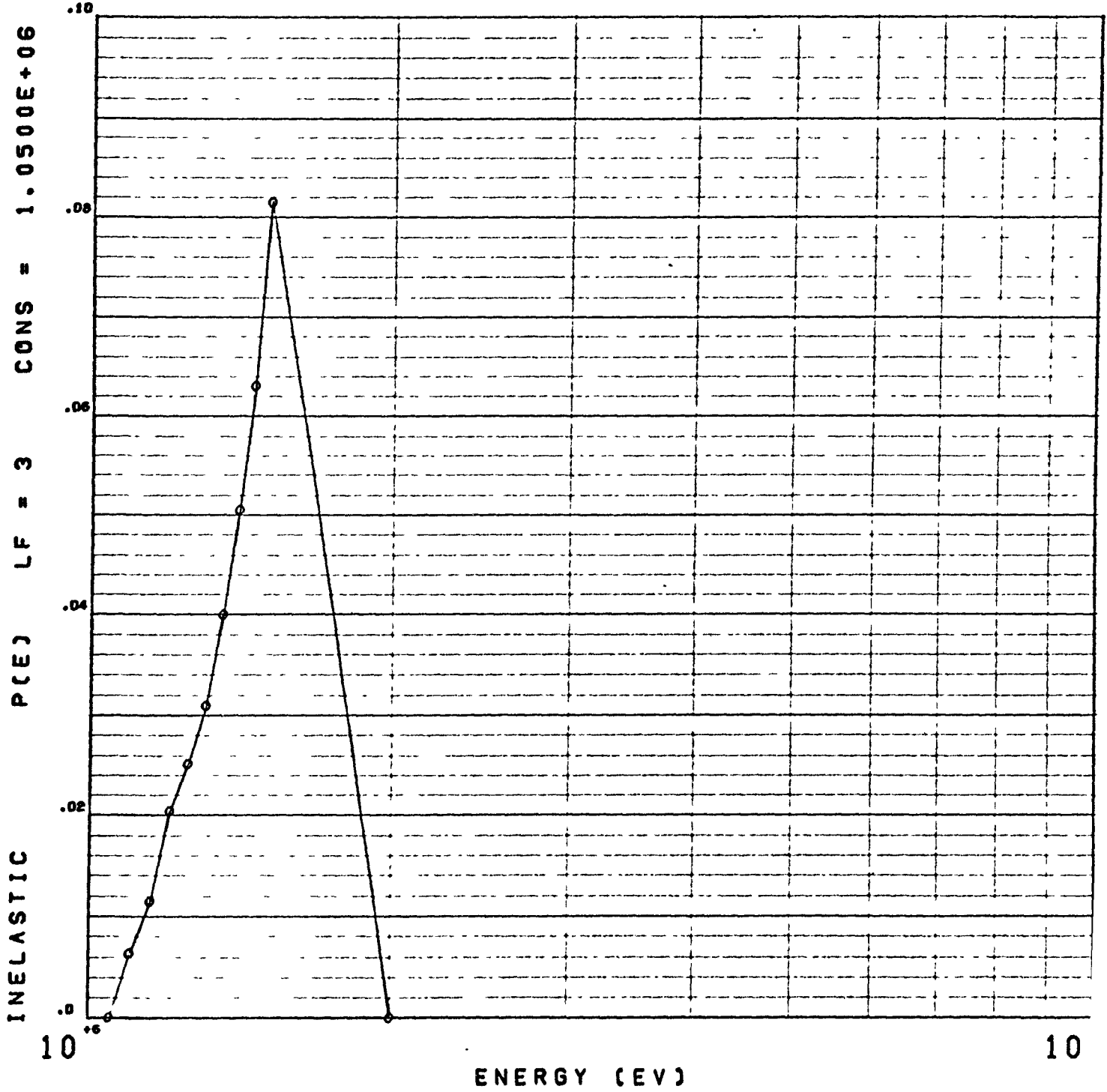
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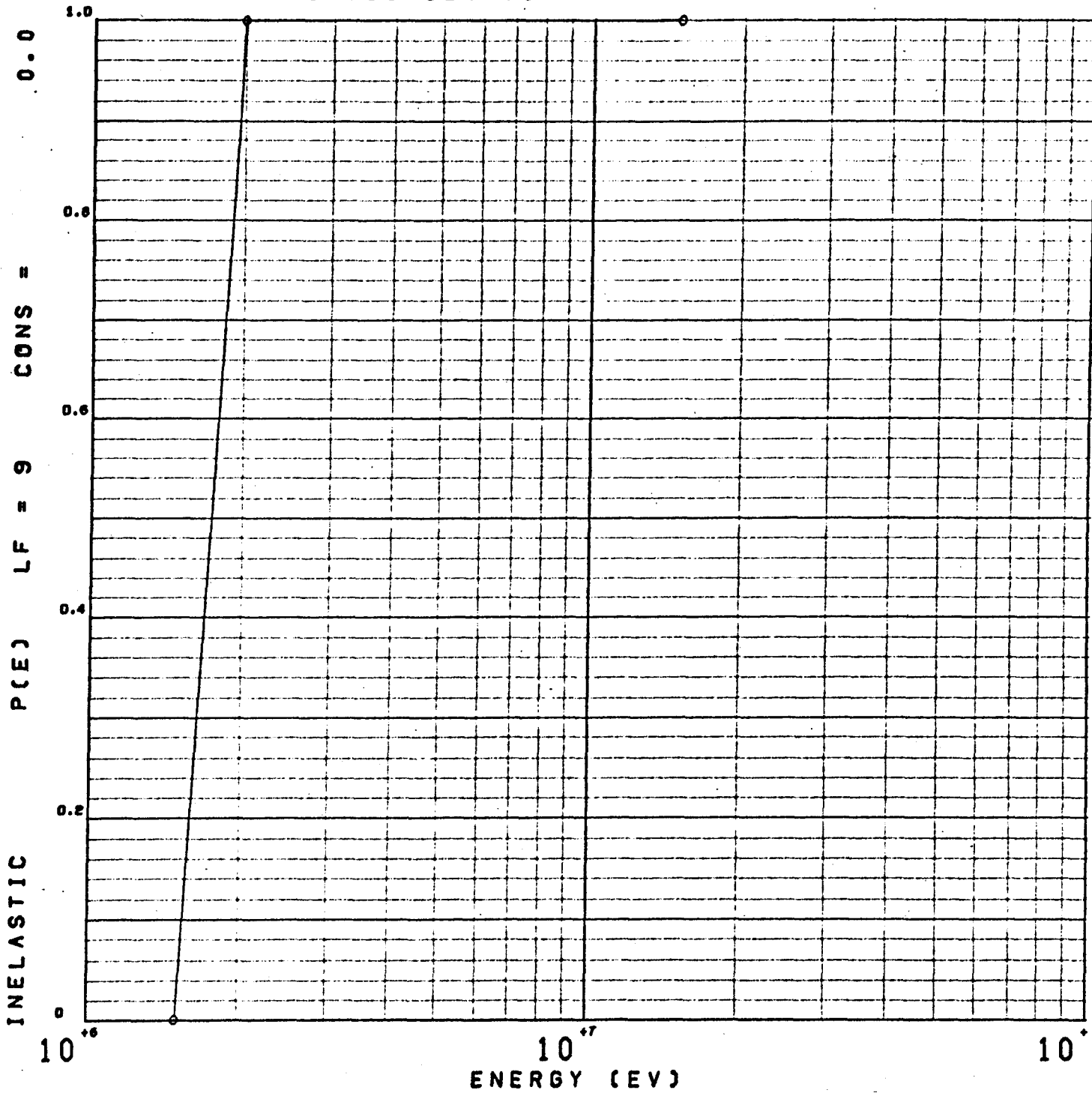
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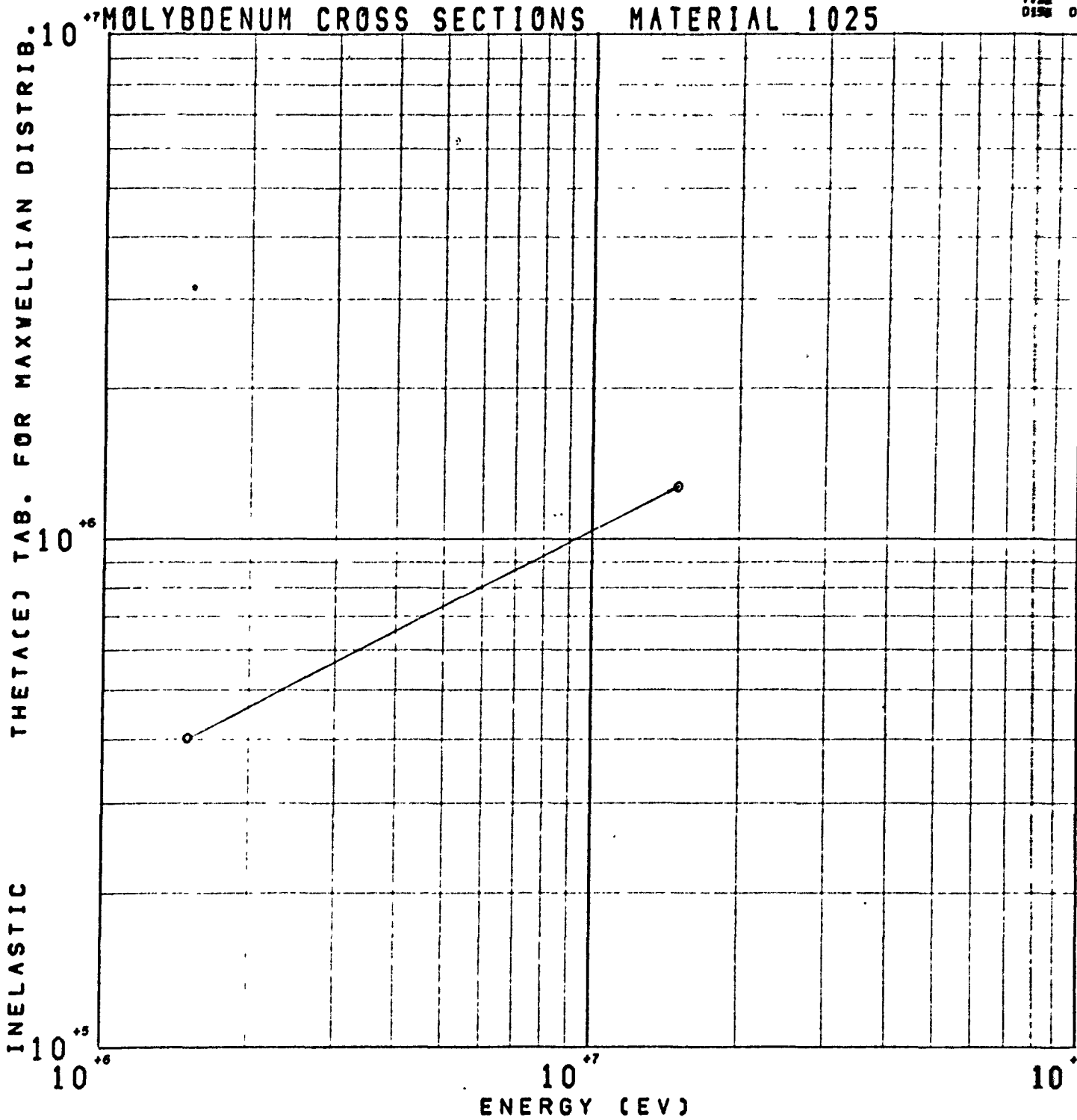


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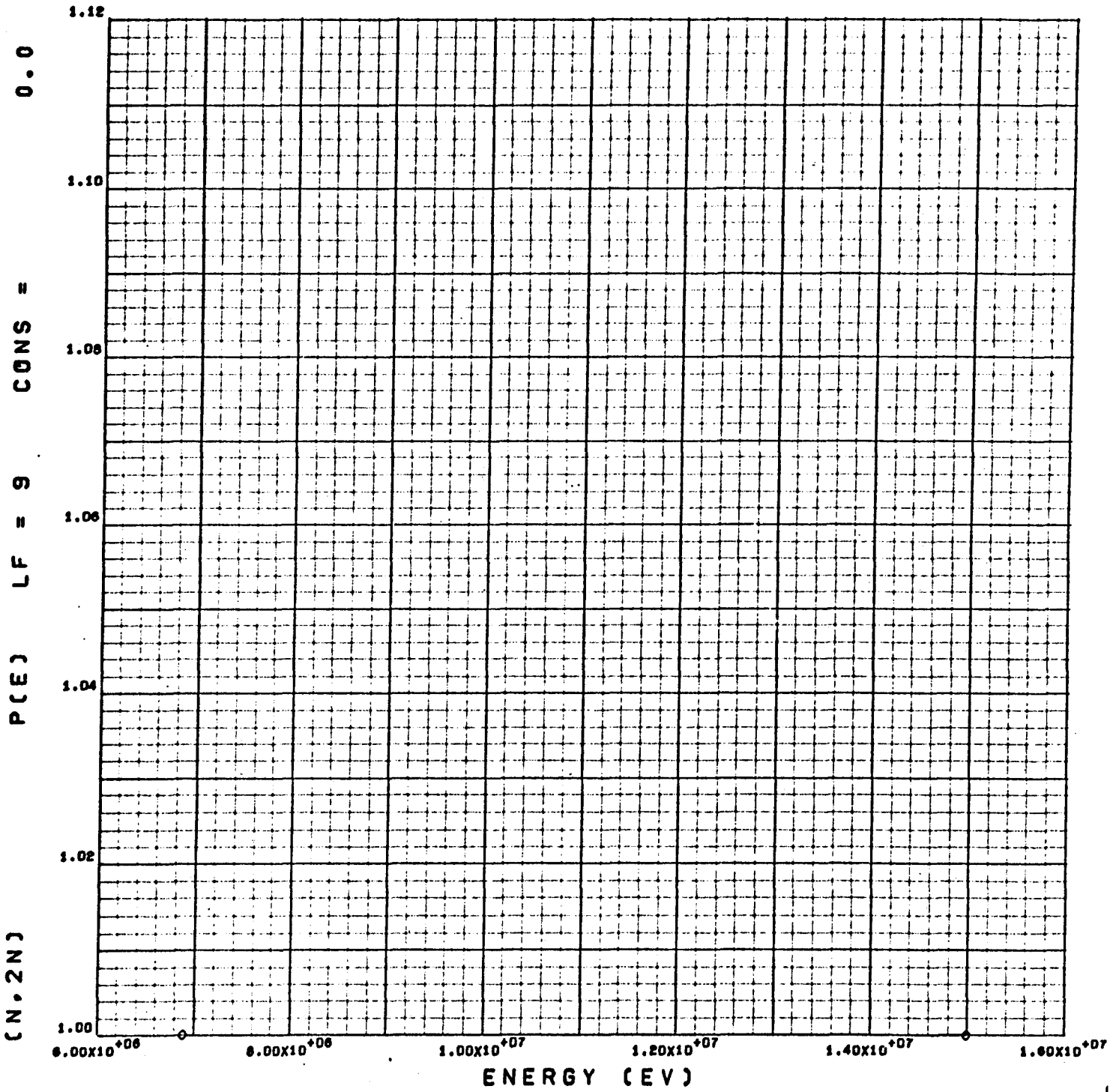


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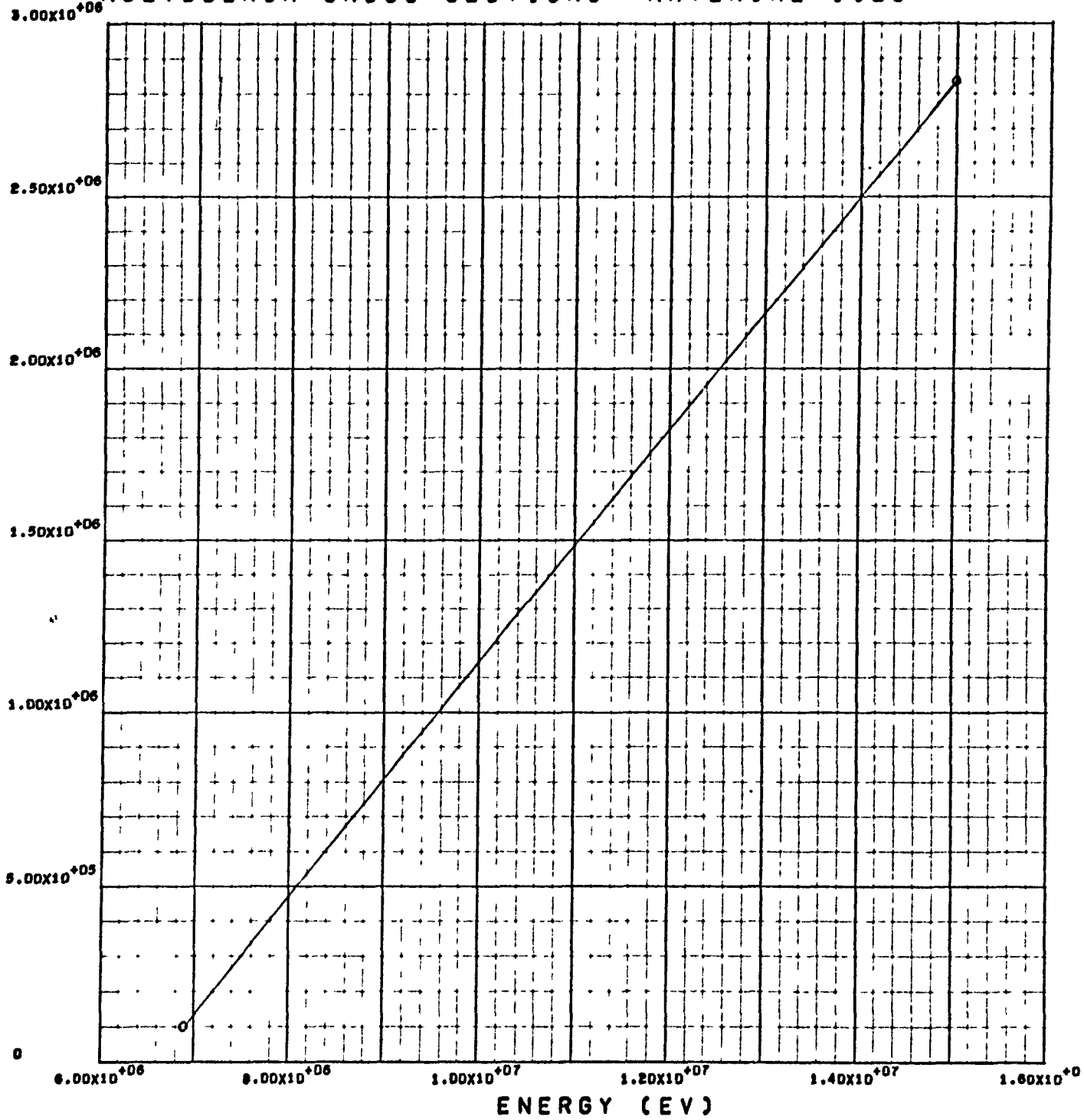
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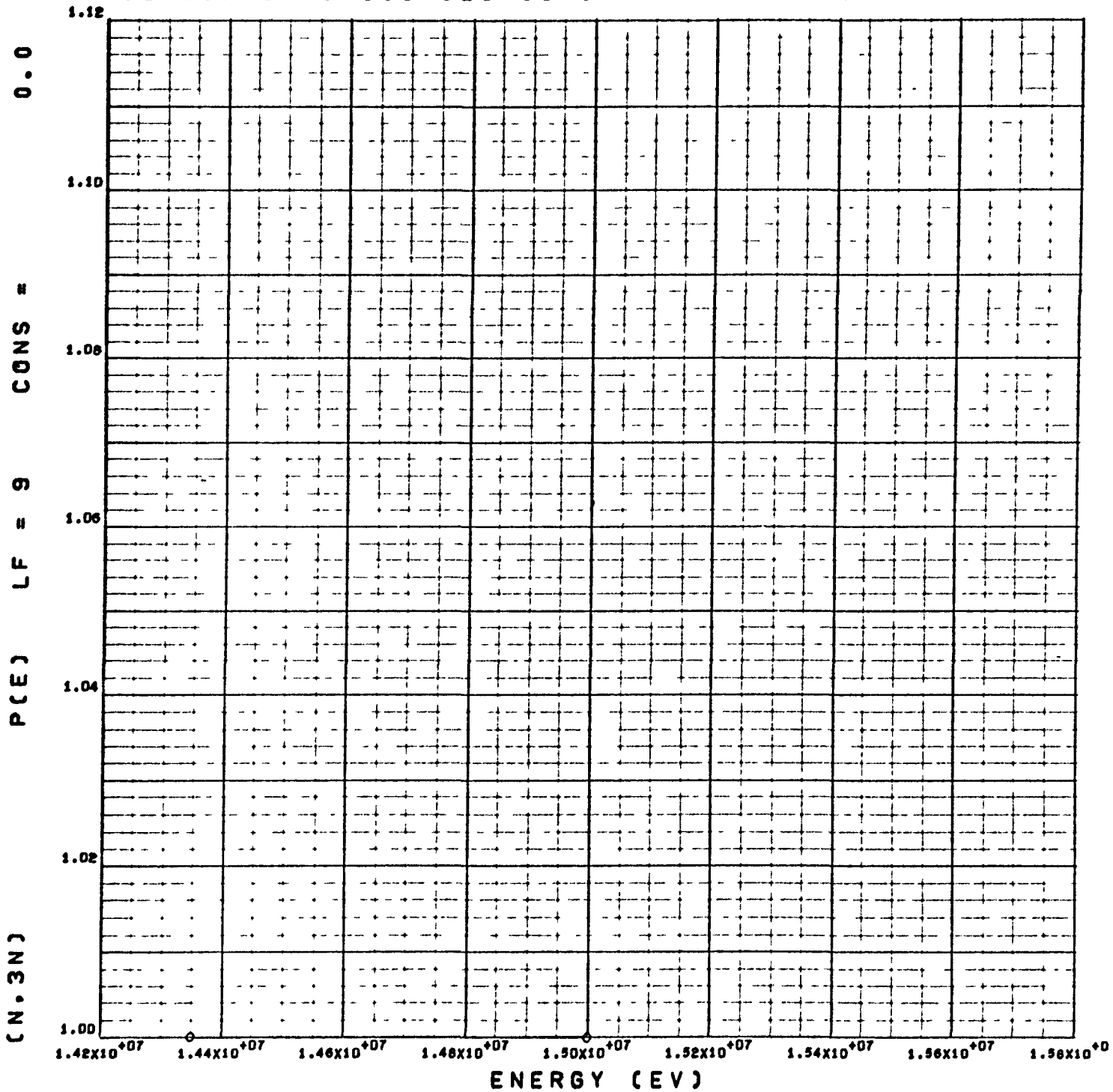


MOLYBDENUM CROSS SECTIONS MATERIAL 1025

THE T(ε) TAB. FOR MAXWELLIAN DISTRIB.
(N, 2N)



MOLYBDENUM CROSS SECTIONS MATERIAL 1025



MOLYBDENUM CROSS SECTIONS MATERIAL 1025

