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Sludge Batch 6 Acceptance Evaluation: Radionuclide Concentrations in Tank 51 SB6 Qualification Sample Prepared at SRNL

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PREFACE OR ACKNOWLEDGEMENTS

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EXECUTIVE SUMMARY

Presented in this report are radionuclide concentrations required as part of the program of qualifying Sludge Batch Six (SB6) for processing in the Defense Waste Processing Facility (DWPF). The SB6 material is currently in Tank 51 being washed and prepared for transfer to Tank 40. The acceptance evaluation needs to be completed prior to the transfer of the material in Tank 51 to Tank 40. The sludge slurry in Tank 40 has already been qualified for DWPF and is currently being processed as SB5ⁱ. The radionuclide concentrations were measured or estimated in the Tank 51 SB6 Qualification Sample prepared at Savannah River National Laboratory (SRNL). This sample was prepared from the three liter sample of Tank 51 sludge slurry (HTF-51-09-110) taken on October 8, 2009. The sample was delivered to SRNL where it was initially characterized in the Shielded Cells.ⁱⁱ Under the direction of the Liquid Waste Organization it was then modified by eight washes, nine decants, an addition of Pu from Canyon Tank 16.3, and an addition of NaNO₂. This final slurry now has a composition expected to be similar to that of the slurry in Tank 51 after final preparations have been made for transfer of that slurry to Tank 40.

Determining the radionuclide concentrations in this Tank 51 SB6 Qualification Sample is part of the work requested in Technical Task Request (TTR) No. HLW-DWPF-TTR-2009-0014.ⁱⁱⁱ The work with this qualification sample is covered by a Task Technical and Quality Assurance Plan^{iv} and an Analytical Study Plan.^v The radionuclides included in this report are needed for the DWPF Radiological Program Evaluation, the DWPF Waste Acceptance Criteria (TSR/WAC) Evaluation, and the DWPF Solid Waste Characterization Program (TTR Task I.2). Radionuclides required to meet the Waste Acceptance Product Specifications (TTR Task II.2.) will be measured at a later date after the slurry from Tank 51 has been transferred to Tank 40. Then a sample of the as-processed SB6 will be taken and transferred to SRNL for measurement of these radionuclides.

The results presented in this report are those necessary for DWPF to assess if the Tank 51 SB6 sample prepared at SRNL meets the requirements for the DWPF Radiological Program Evaluation, the DWPF Waste Acceptance Criteria evaluation, and the DWPF Solid Waste Characterization Program. The sample is the same as that on which the chemical composition was reported.^{vi} Concentrations are given for thirty-four radionuclides along with total alpha and beta activity. Values for total gamma and total gamma plus beta activities are also calculated.

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- i. Pareizs, J. M., Bannochie, C. J., Click, D. R., Lambert, D. P., Stone, M. E., Pickenheim, B. R., Billings, A. L., and Bibler, N. E., *Sludge Washing and Demonstration of the DWPF Flowsheet in the SRNL Shielded Cells for Sludge Batch 5 Qualification*, SRNS-STI-2008-00111, Savannah River Site, Aiken, SC 29808 (2008).
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 - v. Bannochie, C. J. and Pareizs, J. M., *Qualification of DWPF Sludge Batch 6 and Characterization of Tank 4 Samples in the SRNL Shielded Cells: Analytical Study Plan*, SRNL-RP-2009-00474, Savannah River Site, Aiken SC 29808 (2009).
 - vi. Bannochie, C. J., Pareizs, J. M., and Click, D. R. *Tank 51 SB6 Qualification SRAT Receipt Characterization*, SRNL-L3100-2010-00027, Savannah River Site, Aiken, SC 29808 (2010).

Results also indicate that 99% of the Tc-99 and at least 90% of the I-129 that could have been in this sludge batch have been removed by chemical processing steps in the SRS Canyons or Tank Farm.

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LIST OF ABBREVIATIONS

AD	Analytical Development
ASP	Analytical Study Plan
DOE	Department of Energy
dpm	disintegrations per minute
DWPF	Defense Waste Processing Facility
g	gram
HTO	tritiated water
ICP-MS	Inductively Coupled Plasma – Mass Spectrometry
L	liter
μCi	micro-Curies
QA	Quality Assurance
SB2	Sludge Batch 2
SB3	Sludge Batch 3
SB4	Sludge Batch 4
SB5	Sludge Batch 5
SB6	Sludge Batch 6
SpA	Specific Activity (Ci/g)
SNRL	Savannah River National Laboratory
SRS	Savannah River Site
$t_{1/2}$	half-life
TTQAP	Task Technical and Quality Assurance Plan
TTR	Task Technical Request
WAPS	Waste Acceptance Product Specifications
WCS	Waste Characterization System

1.0 Introduction

Presented in this report are radionuclide concentrations required as part of the program of qualifying Sludge Batch Six (SB6) for processing in the Defense Waste Processing Facility (DWPF). The SB6 material is currently in Tank 51 being washed and prepared for transfer to Tank 40. The acceptance evaluation needs to be completed prior to the transfer of the material in Tank 51 to Tank 40. The sludge slurry in Tank 40 has already been qualified for DWPF and is currently being processed as SB5¹. The radionuclide concentrations were measured or estimated in the Tank 51 SB6 Qualification Sample prepared at Savannah River National Laboratory (SRNL). This sample was prepared from the three liter sample of Tank 51 sludge slurry (HTF-51-09-110) taken on October 8, 2009. The sample was delivered to SRNL where it was initially characterized in the Shielded Cells.² Under the direction of the Liquid Waste Organization it was then modified by eight washes, nine decants, an addition of Pu from Canyon Tank 16.3, and an addition of NaNO₂. This final slurry now has a composition expected to be similar to that of the slurry in Tank 51 after final preparations have been made for transfer of that slurry to Tank 40.

Determining the radionuclide concentrations in this Tank 51 SB6 Qualification Sample is part of the work requested in Technical Task Request (TTR) No. HLW-DWPF-TTR-2009-0014.³ The work with this qualification sample is covered by a Task Technical and Quality Assurance Plan⁴ and an Analytical Study Plan.⁵ The radionuclides included in this report are needed for the DWPF Radiological Program Evaluation, the DWPF Waste Acceptance Criteria (TSR/WAC) Evaluation, and the DWPF Solid Waste Characterization Program (TTR Task I.2). The sample is the same as that on which the chemical composition was reported.⁶ Radionuclides required to meet the Waste Acceptance Product Specifications (TTR Task II.2.) will be measured at a later date after the slurry from Tank 51 has been transferred to Tank 40. Then a sample of the as-processed SB6 will be taken and transferred to SRNL for measurement of these radionuclides.

2.0 Experimental Procedure

Data presented in this report represents the measured or estimated radionuclide concentrations obtained from several standard analytical methods performed by Analytical Development (AD) personnel within SRNL. These methods were performed on solutions resulting from the dissolutions of the slurry samples. The dissolution methods were an alkali fusion⁷ and an aqua regia digestion⁸. Two additional preparation schemes were performed to obtain the Am/Cm data and the I-129 data: these methods have been described previously.⁹

After the nine decants/eight washes, Pu stream addition, and NaNO₂ addition, an aliquot of the 3-L Tank 51 sample was taken for analyses. The ~130 mL aliquot taken for the radionuclide measurements and other characterization was called Wash I slurry or SC-9 SRAT Receipt Material and is the SB6 Qualification Sample prepared at SRNL. At that time, 1.18 liters of the Tank 51 Qualification Sample remained.

Table 2-1 presents the weight percent solids and density measurements for the washed qualification sample prepared at SRNL, both sludge slurry (henceforth referred to as “slurry”) and supernate.

Table 2-1. Weight Percent Solids and Density for Washed Tank 51 SB6 Samples

Sample Name	Treatability Sample ID	Wt% Total Solids [%RSD]	Wt% Soluble Solids [%RSD]	Wt% Insoluble Solids [%RSD]	Density (g/mL) [%RSD]
Wash I Slurry	TS144-09-A-103710	15.12 [0.7]	5.23 [0.1]*	9.89 [0.2]*	1.13 [1.5]
Wash I Supernate	TS144-09-A-103709	NA	5.80 [1.2]	NA	1.06 [0.6]

* %RSD here is more correctly defined as percent standard error for these calculated values

The experimental procedures for specific radionuclides will now be discussed. The concentration of tritium in the supernate of the sludge slurry was determined by using a sample of the supernate that had been separated from the insoluble solids by filtration. The tritiated water (HTO) and H₂O were then distilled from four aliquots of the supernate. The H-3 was determined by counting the beta particles from the HTO in the distillate of each aliquot. The H-3 concentration in the slurry was then calculated based on the density of the slurry, density of the supernate, and weight percent insoluble solids in the slurry.

The concentrations and upper limits reported are based on three or four replicate samples. Concentrations of H-3, Sr-90, and Pu-241 along with total beta activity are based on analyses by beta counting techniques. The concentrations of Co-60, Sb-125, Eu-154, Eu-155, and Am-241 were measured by Cs removed gamma counting. The concentration of Cs-137 was measured by gamma counting. The results for Tc-99, U-233, U-234, U-235, Np-237, U-238, Pu-239, Pu-240, and Cm-246 were determined by Inductively Coupled Plasma Mass Spectroscopy (ICP-MS) that measures the concentration of these radionuclides based on their masses rather than their radioactivity. These concentrations were converted to µC/g using their specific activities¹⁰. The results for Pu-238 and Cm-244 along with the total alpha activity were measured by alpha counting. The concentrations of radionuclides Ru-106, Rh-106, Cs-134, Ce-144, and Pr-144 are reported as method detection limits due to their concentrations being too low to be detected because of their short half-lives and the age of the sludge. These limits were determined by gamma counting.

The radionuclide Pm-147 is a short lived U-235 fission product that was measured. The methodology used to obtain the concentration of Pm-147 has been described previously⁹. A detection limit for the method is reported due to the low concentration in the aliquots measured.

The concentration reported for C-14 is estimated from its measured concentrations in Sludge Batch 2 (SB2), the projected concentrations in SB2 given in the Washington Savannah River Company Concentrate, Transfer, and Storage Characterization Database (WCS), and the projected concentrations given in WCS for the C-14 in the washed Tank 51 slurry prior to transferring it to Tank 40. This is the method that was agreed upon in the TT & QA plan for SB6 Qualification.⁴

The radionuclide I-129 is a long-lived beta emitting fission product ($t_{1/2} = 1.6E+07$ years) that is in SRS wastes. Four aliquots of wet sludge slurry were successfully spiked with a known amount of stable KI to act as an iodine tracer/carrier. The samples were digested with 8M nitric acid. The iodate/iodine in the samples was reduced with sodium sulfite to minimize losses of iodine in the Shielded Cells I-129 procedure. The Fe in the dissolutions was reduced to Fe (II) using ascorbic acid to ensure it would not interfere with subsequent decontamination steps designed to extract Y-90, the lanthanides, and the actinides from the KI traced dissolutions. The dissolutions were then treated with resins (Bio-Rad AMP-1, Eichrom Sr, and Actinide resins) to reduce levels of Sr-90, Cs-137, Y-90, the lanthanides, and the actinides. The resulting samples were then treated a second time with a sodium sulfite reduction and

filtered. The samples were decontaminated a final time with a resin treatment (Eichrom Sr and RE resin cartridges) to ensure Sr-90/Y-90 levels were reduced low enough to allow for sample removal from the Shielded Cells and to remove Cs-137 and the actinide elements. The samples were then rendered caustic and filtered prior to removal from the Shielded Cells. The iodide remained soluble and was removed with the caustic filtrate. In AD the solution was treated with AgNO_3 in order to precipitate the iodide ion as AgI. The precipitate was analyzed with a low energy semi planar high purity germanium detector. The 39 keV gamma ray and 29 and 34 keV X-rays characteristic to I-129 were evident even though there was background interference from low energy activities still present from Cs-137. Concentrations of I-129 were qualified as upper bounds due to background interferences from other isotopes still present in the sample mount. Final quantification was achieved by neutron activation of the iodine in the precipitate, via a Cf-252 neutron source at SRNL, to measure the total iodine present to calculate the recovery of total iodine from the separations process and determine the bounding concentration of I-129.

3.0 Results and Discussion

Table 3-1 presents the measured or estimated concentrations for thirty four individual radionuclides, the measured values for the total beta and total alpha activity along with the calculated values for total gamma and total beta plus gamma activities. The concentrations of those radionuclides that could not be measured due to their low concentrations have been estimated from minimum detection limits based on the analytical method used. For all the radionuclides except tritium (H-3), the concentrations are based on the total dried solids from dissolution of four replicates of the Tank 51 sludge slurry. Column 2 gives the concentrations in units of microcuries (μCi) per gram of total solids in the dried sludge slurry. Column 3 presents the relative percent standard deviations (%RSD). Finally, Column 4 presents the concentrations of the radionuclides in curies (Ci) per gallon of slurry calculated based on the measured weight percent total solids in the slurry (15.12 wt%), the density of the slurry (1.13 g/mL), and a conversion factor of 3785 mL/gal to convert to liquid gallons. Specific radionuclides will now be discussed.

Essentially all of the tritium in the Tank 51 slurry is present as tritiated water (HTO). Consequently, its concentration in the dried solids could not be determined because the HTO was evaporated during the drying of the slurry. The lowest detection limit for the supernate measurements was used to specify the concentration of H-3.

The radionuclides Y-90, Rh-106, Te-125m, Ba-137m, and Pr-144 are in secular equilibrium with their respective parent radionuclides. Thus the activities of Y-90, Rh-106, Te-125m, and Pr-144 are equal to that of their parents. Approximately 4% of the Cs-137 decays directly to stable Ba-137; thus the activity of Ba-137m is 96% of the activity of the Cs-137.

The measured concentration for C-14 in SB2 was $8.10 \times 10^{-3} \mu\text{Ci/g}$.¹¹ The projected concentration in SB2 was $1.40 \times 10^{-6} \text{ Ci/gallon}$.¹² The WCS projection for the total curies of C-14 in Tank 51 when the slurry is ready to be transferred to Tank 40 is $1.88 \times 10^{-1} \text{ curies}$.¹³ The volume projected to be transferred to Tank 40 is $4.11 \times 10^5 \text{ gallons}$.¹³ The projected concentrations of C-14 is then $4.57 \times 10^{-7} \text{ Ci/gallon}$. Multiplying the ratio of the projected concentrations in Tank 51 to the projected concentrations in SB2 by the measured concentrations in SB2 for each isotope gives $2.65 \times 10^{-3} \mu\text{Ci/g}$ for C-14.

The results for Tc-99 in the SB6 qualification sample are interesting in that they indicate that most of the Tc-99 that could have been in SB6 had been removed by the processing steps in the Canyons and Tank Farm of SRS. As shown below, it is estimated that 99% of the Tc-99 that could have been in SB6 had been removed to the salt tanks or volatilized as a result of SRS canyon processing. This estimate is determined by comparing the maximum amount of Tc-99 that can be predicted to be in SB6 with the measured amount. The measured concentration in the total dried solids of the SB6 slurry was 5.35×10^{-4}

wt% as determined by ICP-MS analysis. This concentration times the specific activity of Tc-99 in Ci/g gives the activity concentration shown in Table 3-1.

The fission products in SB6 were formed in the SRS reactors by thermal neutron fission of U-235. Based on many experimental studies the relative yields of the fission products are known based on the number of atoms of each fission product isotope formed as a result of fission of 100 atoms of U-235.¹⁴ For a particular sludge batch, values for a fission yield scaling factor (FYSF) can be calculated from each measured concentration of a fission product. The FYSF is simply a proportionality factor that relates the concentration in wt% of an isotope in the dried solids of sludge slurry to its fission yield and mass. For those isotopes that have six important properties, values for the FYSF will be equal for a sludge batch. These properties have been previously discussed.¹⁵ The following equation applies for each fission product.

$$FYSF_i = \text{wt}\%_i / (FY_i \times am_i)$$

Where FYSF_i ≡ the fission yield scaling factor based on isotope
 wt%_i ≡ the weight percent of isotope i in the HLW total dried solids
 FY_i ≡ the fission yield of isotope i
 am_i ≡ the atomic mass of isotope i

In SB6 there are 11 fission products that have the six important properties. These isotopes are: Ru-101, Ru-102, La-139, Ce-140, Pr-141, Ce-142, Nd-143 through Nd-146, and Sm-147. All of these isotopes were measured by ICP-MS in the total dried solids of the SB6 slurry. The average FYSF for SB6 calculated with these 11 isotopes is 7.93×10^{-5} with a %RSD of 6.1. After rearrangement of the above equation, the concentration of any fission product can be predicted. For Tc-99 with its half-life of 2.13×10^5 years and fission yield of 6.1%,¹⁴ the predicted wt% for Tc-99 is 4.79×10^{-2} wt%. The measured value was 5.35×10^{-3} wt%, indicating that only 1.1% of the Tc-99 that could have been in this sludge batch was actually still in the sludge slurry. One of the main properties that a fission product must have to be retained in the sludge solids is that it is insoluble in caustic. If it were soluble in caustic it would not precipitate in the sludge solids and would be decanted or washed to the salt tanks in the Tank Farm. Technetium is known to form a pertechnetate anion that is soluble in caustic.¹⁵ This can explain the low concentration of Tc-99 in SB6. A previous study with SB3 also indicated a low concentration of Tc-99 in the sludge solids.¹⁵

The radionuclide I-129 is a U-235 fission product with a long half life of 1.6×10^7 years. Previously it has only been measured successfully in the Sludge Batch 4 and 5 (SB4, SB5) Waste Acceptance Product Specifications (WAPS) studies^{9,11} and SB5 Acceptance Evaluation¹⁶, where its concentration was reported. Other reported concentrations for previous sludge batches are based on the Waste Characterization System at SRS. For I-129 the lowest detection limit measured from four aliquots of the sludge slurry was $<1.0 \times 10^{-3}$ μCi/g of total dried solids. Based on the separations and counting results, this appears to be a reliable result. However, it was further checked by calculating the amount of I-129 that could be present using the FYSF. If this predicted concentration were less than the measured, then this would indicate that the measured number was erroneously high. Using the FYSF of 7.93×10^{-5} and the fission yield for I-129 of 0.54%,¹⁴ the predicted concentration of I-129 in SB6 would be 5.5×10^{-3} wt%, assuming no I-129 were lost in processing. This concentration is at least 10x higher than the measured value supporting the conclusion that the measured detection limit is a reliable value. It also indicates that at least 90% of the I-129 was removed from SB6 by processing steps in the Canyons or Tank Farm. Likely steps are volatilization of iodine species during acid dissolution of irradiated targets or fuel assemblies in the Canyons or solubility of iodine species in caustic.

**Table 3-1. Concentrations of Radionuclides in the Tank 51 SB6 Qualification
Sample Prepared at SRNL**

Radionuclide	Wt% in Total Dried Solids	μCi/g in Total Dried Solids	%RSD	Ci/gal in Sludge Slurry (a)
H-3	(b)	(b)	N/A	<1.9E-06
C-14	5.95E-08	2.65E-03	N/A	1.72E-06
Co-60	1.36E-07	1.54E+00	9.1	1.00E-03
Sr-90	1.26E-02	1.72E+04	14	1.11E+01
Y-90	3.15E-06	1.72E+04	14	1.11E+01
Tc-99	5.35E-04	9.06E-02	5.3	5.87E-05
Ru-106	<1.1E-08	<3.7E-01	N/A	<2.4E-04
Rh-106	<1.0E-14	<3.7E-01	N/A	<2.4E-04
Sb-125	1.38E-08	1.42E-01	8.2	9.22E-05
Te-125m	7.90E-10	1.42E-01	8.2	9.22E-05
I-129	<5.7E-04	<1.0E-03	N/A	<6.6E-07
Cs-134	<8.4E-08	<1.1E+00	N/A	<7.0E-04
Cs-137	4.28E-04	3.72E+02	3.8	2.41E-01
Ba-137m	6.64E-11	3.57E+02	3.8	2.31E-01
Ce-144	<1.9E-08	<6.1E-01	N/A	<3.9E-04
Pr-144	<8.0E-13	<6.1E-01	N/A	<3.9E-04
Pm-147	<2.2E-05	<2.0E+02	N/A	<1.3E-01
Eu-154	6.90E-06	1.86E+01	5.1	1.21E-02
Eu-155	5.94E-07	2.76E+00	6.3	1.79E-03
Th-232	2.98E+00	3.27E-03	1.6	2.12E-06
U-233	1.42E-03	1.37E-01	20	8.91E-05
U-234	9.79E-04	6.12E-02	4.7	3.97E-05
U-235	1.98E-02	4.28E-04	1.6	2.77E-07
Np-237	2.26E-03	1.59E-02	2.5	1.03E-05
U-238	2.31E+00	7.76E-03	2.2	5.03E-06
Pu-238	1.70E-03	2.91E+02	3.8	1.89E-01
Pu-239	2.18E-02	1.35E+01	1.6	8.77E-03
Pu-240	2.37E-03	5.40E+00	1.8	3.50E-03
Pu-241	<8.0E-05	<8.2E+01	N/A	<5.3E-02
Am-241	8.25E-04	2.83E+01	5.9	1.83E-02
Am-242m	1.45E-07	1.41E-02	50	9.12E-06
Am-243	1.34E-04	2.68E-01	11	1.73E-04
Cm-244	1.02E-05	8.27E+00	5.6	5.36E-03
Cm-245	6.96E-07	1.19E-03	3.2	7.74E-07
Cm-246	7.94E-07	2.44E-03	7.8	1.58E-06
Total alpha	N/A	<3.9E+02	N/A	<2.5E-01
Total beta	N/A	3.25E+04	1.8	2.11E+01
Total gamma(c)	N/A	4.13E+02	N/A	2.68E-01
Total beta-gamma	N/A	3.29E+04	N/A	2.14E+01

N/A = Not Applicable

(a) This was calculated for all the radionuclides except H-3 using the weight percent total solids in the slurry of 15.12 and a slurry density of 1.13 g/mL.

(b) Most of the H-3 in the slurry is present as HTO; thus, drying the slurry sample would drive off most of the H-3. The concentration of H-3 was measured in four samples of the supernate of the Tank 51 SB6 sample slurry. The result was <5.25E-04 μCi/mL. This was converted to Ci/gal slurry using 9.89 wt% insoluble solids in the slurry and the slurry and supernate densities.

(c) This is the total activity of the gamma emitters: Co-60, Ru-106, Rh-106, Sb-125, Te-125m, Cs-134, Ba-137m, Ce-144, Pr-144, Eu-152, Eu-154, Eu-155, and Am-241.

4.0 Conclusions

The results presented in this report are those necessary for DWPF to assess if the Tank 51 SB6 sample prepared at SRNL meets the requirements for the DWPF Radiological Program Evaluation, the DWPF Waste Acceptance Criteria evaluation, and the DWPF Solid Waste Characterization Program. Concentrations are given for thirty-four radionuclides along with total alpha and beta activity. Values for total gamma and total gamma plus beta activities are also calculated.

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5.0 References

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