

## $Z_{\text{eff}}$ -PROFILES IN DIFFERENT CONFINEMENT- AND HEATING REGIMES OF ASDEX

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### Introduction

$Z_{\text{eff}}$  is an important quantity in fusion research. It should ideally be as small as possible to minimize both fuel dilution and radiation losses. Further, radial profiles of  $Z_{\text{eff}}$ , together with  $T_e(r)$ , are needed to derive  $j(r)$  used in stability analysis. We report measurements of  $Z_{\text{eff}}$  profiles for a variety of ASDEX plasmas: ohmic (H and D; gas and pellet fuelling), and co and counter neutral beam heated plasmas.

### $Z_{\text{eff}}$ measurements

$Z_{\text{eff}}$  profiles are measured from the intensity of plasma radiation in the near infrared. In this wavelength region recombination and line radiation are generally unimportant and the radiation is mainly bremsstrahlung. The intensity profiles are measured with the ASDEX 16 channel YAG laser Thomson scattering system (Fig. 1) /Ref. 1/. Each of the 16 detection boxes contains 3 broadband interference filters set at wavelengths of 850, 950 and 1000 nm. This provides a means of checking the  $\lambda^{-2}$  scaling of bremsstrahlung intensity and gives an approximate measure of the absence of line radiation. Up to the silicon avalanche diodes used as detectors, the system is identical to that used for Thomson scattering. For the  $Z_{\text{eff}}$  measurements, however, a DC-coupled circuit with a bandwidth of 50 kHz is used. The sample rate is typically 200 Hz. Comparison of the Abel-inverted radiation with hydrogen bremsstrahlung determined from simultaneously measured  $n_e$  and  $T_e$  profiles results in  $Z_{\text{eff}}$  profiles from +20 cm to -38.5 cm with a spatial resolution of about 3 cm. Under some conditions comparisons have been made with radially averaged  $Z_{\text{eff}}$  values from plasma resistivity and from sawtooth analysis. The latter method is based on the electron power balance in ohmic discharges, in the plasma centre, during the period of recovery after the sawtooth crash /ref. 2/. In addition,  $Z_{\text{eff}}$  has been estimated from impurity densities derived by two methods. Firstly, absolutely measured VUV line intensities of the four dominant impurities (Fe, Cu, O, C) are compared with those calculated by a time dependent impurity transport code using measured  $n_e$  and  $T_e$  profiles.

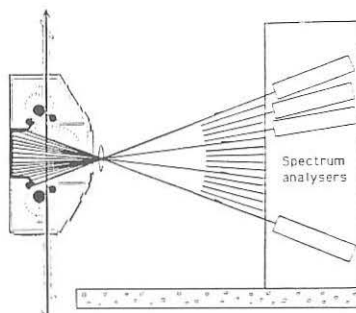


Fig. 1: Bremsstrahlung  $Z_{\text{eff}}$  setup

From this comparison an independent density profile is obtained for each impurity species, allowing its  $Z_{\text{eff}}$  contribution to be calculated. Secondly, the densities of the two light impurity species (O, C) can be also derived from charge exchange recombination spectroscopy (CXRS) resulting in an independent measurement of their  $Z_{\text{eff}}$  contribution. These spectroscopic  $Z_{\text{eff}}$  are generally less accurate than bremsstrahlung  $Z_{\text{eff}}$  but the resulting impurity densities yield considerable insight into the  $Z_{\text{eff}}$  behaviour reported in this paper.

### Results and Discussion

Axial values of  $Z_{\text{eff}}$  as a function of  $n_e$  for steady state ohmic and co-NI heated discharges are shown in Figs. 2 and 3. Normally,  $Z_{\text{eff}}$  is found to decrease with increasing  $n_e$ , to be higher in  $D_+$  than in  $H_+$  plasmas, and higher in NI than in ohmic plasmas. The VUV  $Z_{\text{eff}}$  are in good agreement with the bremsstrahlung  $Z_{\text{eff}}$ , as are the sawtooth  $Z_{\text{eff}}$  for OH discharges in  $H_+$ . The CXRS  $Z_{\text{eff}}$  must be normalized to one point from the bremsstrahlung measurements because optical components of the CXRS diagnostic, inside the vacuum chamber, have been inaccessible to direct calibration. The CXRS  $Z_{\text{eff}}$  then show the same trends with  $n_e$ .

The above behaviour can be understood from the spectroscopic results. In ohmically and NI heated discharges O and C are the dominant light impurities with O giving the largest contribution to  $Z_{\text{eff}}$ . The VUV results for a transition from OH + co-NI heated plasma are shown in Fig. 4 for  $H_+$  and Fig. 5 for  $D_+$ . The dominant heavy impurity in the  $H_+$  discharge is Cu. It originates from the divertor plates and is more important at lower densities and in NI than in OH discharges. The densities of C and O (measured by VUV and CXRS) do not change strongly with  $n_e$  so it is this fact, together with the decrease of Cu, that explains the decrease in  $Z_{\text{eff}}$  with increasing  $n_e$ . Fe is unimportant in  $H_+$  discharges due to carbonization, but gives a significant contribution to  $Z_{\text{eff}}$  in  $D_+$  discharges where the walls were not carbonized (Figs. 3 and 5).

Particularly interesting are the  $Z_{\text{eff}}$  results obtained in three rather different scenarios in which improved energy confinement is found in ASDEX. These are OH discharges in  $D_+$  with reduced gaspuff (Fig. 2) /Ref. 3/, pellet fuelled OH discharges (Fig. 6) /Ref. 4/, and counter-NI heated discharges (Fig. 7) /Ref. 5/.

In these cases there is a reversal of the trend of  $Z_{\text{eff}}$  decreasing with increasing  $n_e$  which shows that the improved energy confinement is accompanied by improved particle confinement. The strong accumulation of light impurities in counter NI discharges can be seen from the  $O^{8+}$  (Fig. 7) and  $C^{6+}$  densities. The relatively slow initial increase of  $n_{O^{8+}}$  and  $n_{C^{6+}}$  leads to a decrease of  $Z_{\text{eff}}$  as in the normal co-NI case. However, the impurity density increase is much steeper later on, so that  $Z_{\text{eff}}$  increases during the phase when the global energy confinement is also increasing. For counter-NI the neoclassical  $Z_{\text{eff}}$  is in good agreement with the bremsstrahlung  $Z_{\text{eff}}$  and this is also true for the OH pellet case (Fig. 6). In the improved confinement OH regime  $Z_{\text{eff}}(r)$  shows slight peaking at the plasma centre at later times (Fig.8) but this is not seen in the pellet (Fig. 9) or counter-NI discharges.

### Summary

The present measurements have shown a wide variety of behaviour of  $Z_{\text{eff}}$  in ASDEX plasmas and  $Z_{\text{eff}}$  values ranging from 1.2 to  $\sim 5$  have been found. In particular, for three different improved energy confinement regimes, the  $Z_{\text{eff}}$  results indicate correspondingly improved particle confinement.

Ref. 1 H. Röhr, K.-H. Steuer et al., Nucl. Fusion 22 (1982), 1099

Ref. 2 A. Eberhagen, Internal Report IPP III/124, 1987

Ref. 3 E.R. Müller et al., this conference

Ref. 4 O. Gruber et al, this conference

Ref. 5 O. Gehre et al, Phys. Rev. Lett., to be published

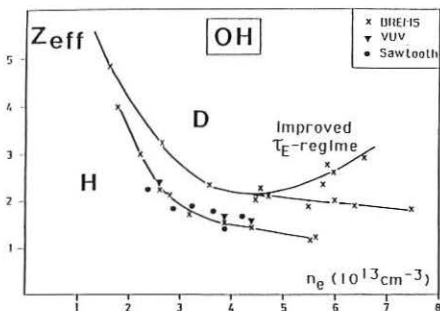


Fig. 2:  $Z_{\text{eff}}$  as a function of  $n_e$  for OH-plasmas in hydrogen (H) and deuterium (D).

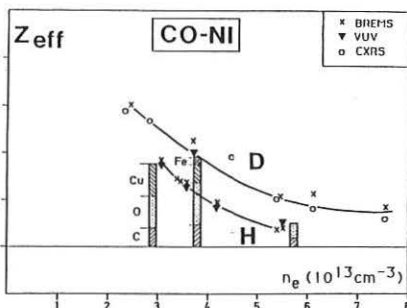


Fig. 3:  $Z_{\text{eff}}$  as a function of  $n_e$  for co-NI heated plasmas in H and D.

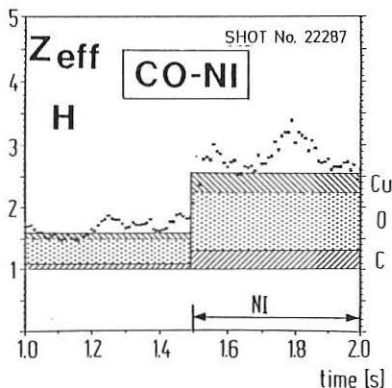


Fig. 4: bremsstrahlung  $Z_{\text{eff}}$  and the VUV imp. contributions during a transition from OH+co-NI in a hydrogen plasma.

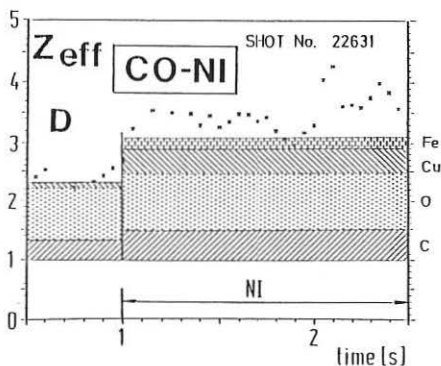


Fig. 5: bremsstrahlung  $Z_{\text{eff}}$  and the VUV imp. contributions during a transition from OH+co-NI in a deuterium plasma.

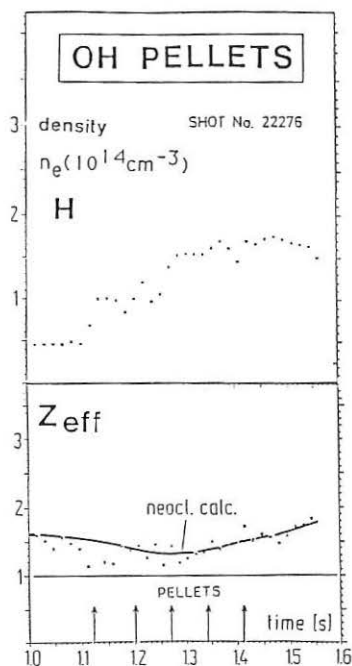


Fig. 6: Time dependence of  $n_e$  (above) and  $Z_{\text{eff}}$  (below) in a OH pellet refuelled discharge in hydrogen.

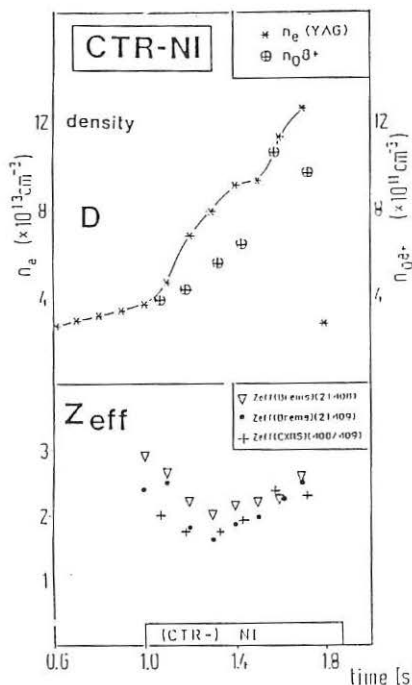


Fig. 7: Time dependence of  $n_e$  and  $n_{0\beta^+}$  (above) and  $Z_{\text{eff}}$  (below) in a counter NI heated discharge in deuterium.

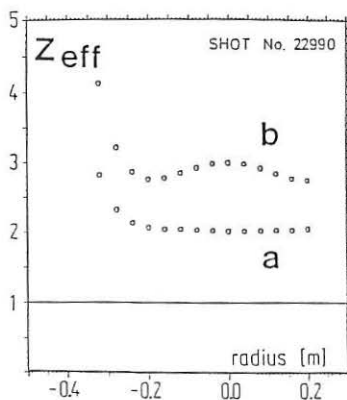


Fig. 8: Radial profiles of  $Z_{\text{eff}}$  for OH discharges in deuterium. (a) "normal" regime, (b) improved confinement regime ( $n_{e0} = 6.6 \times 10^{13} \text{cm}^{-3}$ , see Fig. 2).

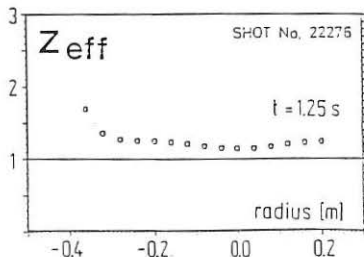


Fig. 9: Radial profile of  $Z_{\text{eff}}$  for a pellet fuelled OH discharge in H (see Fig. 6).