

HGF-Activities for liquid metal fast reactor systems

M. Daubner, S. Gordeev, M. Haselbauer, W. Hering, W. Jäger, A. Onea, J. Pacio, S. Perez, A. Ponomarev, A. Rineiski, V. Sanchez, Th. Wetzel - presented by **R. Stieglitz**
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Institute for Neutron Physics and Reactor Technology (INR)



KIT – The Research University in the Helmholtz Association



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Content

- **Fast Reactor R&D in the Helmholtz-Programm PoF-III**
- **Advanced measurement techniques**
- **Infrastructures**
- **Thermalhydraulics**
- **Safety assessment**

- **PoF-III Funding Period 2015-2019**
- **FR –Research and Development embedded in Waste Management Strategies (subtopic 1.2)**
 - Scenario studies
 - Partitioning
 - **Transmutation and safety assessment of transmuted reactors (spent nuclear fuels-SNF)**
 - Waste conditioning by ceramic matrices
 - Nuclear legacy waste and decommissioning
- ➔ **Substantial reduction compared to previous funding period**
 - declining core design activities (e.g. abdication of KAPROS)
 - reduced effort in safety analysis
 - reorientation of experimental program towards neighbouring R&D fields
 - enhanced engagement in EU-Programs (**only** to be complemented by nat. funds)
- **Contributing Centers**
 - Helmholtz Center Dresden Rossendorf (HZDR)
 - Karlsruhe Institute of Technology (KIT)

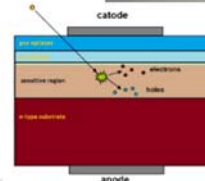
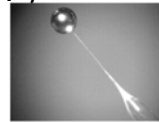
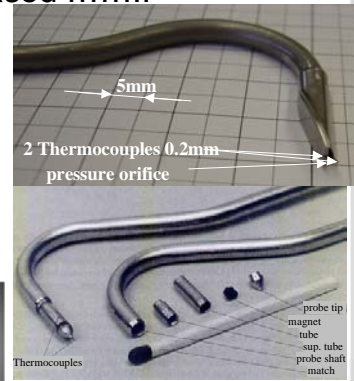
General capabilities

- Flow rate
- Velocity measurement
- Flow visualization -2 phase flow
- Flow field reconstruction

ADVANCED INSTRUMENTATION TECHNIQUES

How to measure in liquid metals ?

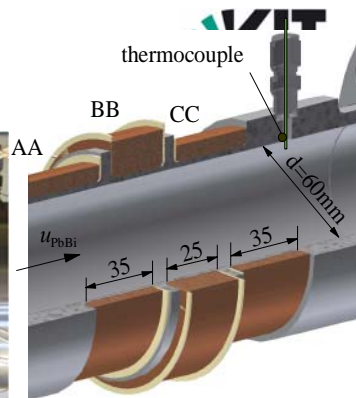
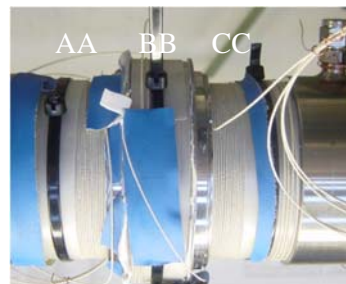
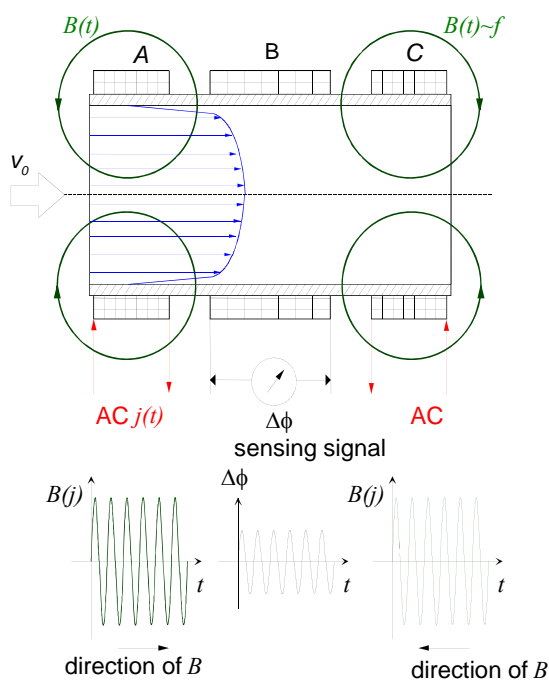
- **Flow rate** – electro-magnetic, D_p , UTT, momentum based
- **Visualization techniques**
 - direct – X-Ray tomography
 - indirect – CIFT, Ultra-sound-transient time (UTT),....
- **Velocity**
 - direct – Pitot-Tube (Δp)
– magnetic potential probes (MPP)
– fibre-mechanics
 - Non-intrusive – Ultra-sound doppler velocimetry (UDV), multi units → mapping
- **Surfaces /2-phase**
 - direct – resistance probes
 - indirect – X-ray, UTT
– optic means for surfaces
- **Neutronic core monitoring**
 - fission chambers
 - semi-conductors- SiC based- diode (SPND) (neutron-generator available)



5 GIF 14th SFR Safety&Operation PMB Meeting, 15th-18th March 2016, KIT, Germany | R. Stieglitz

Flow rate

Electro-magnetic frequency flow meter (EMFM)



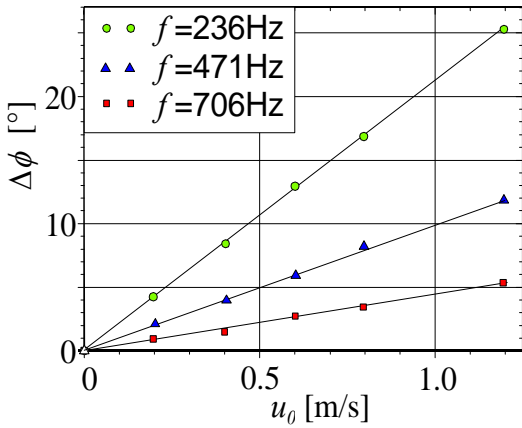
Measurement principle

- Dragging of magnetic fields lines by the flow (RMS-Value $\sim Q$)
- flow direction given by sign of signal
- time delay between Emitter-Sensor (or Phase Angle) $\Delta t \sim Q$
- ➔ **2 independent gross-output quantities for Q**

$$Re = \frac{u_0 \cdot d}{\left(\frac{1}{\mu\sigma} \right)}$$

Th. Schulenberg, R. Stieglitz, NED 2010.

Flow rate-EMFM



Conds. : PbBi tube flow, $T_0=200^\circ\text{C}$,
 $Pr=0.02$, $d=60\text{mm}$, $I_0=410\text{mA}$

Design wishes

- High penetration depth δ of field B into duct (\rightarrow low f f = frequency AC current supply)
- High magnetic field strength (high $\Delta\Phi_{\text{RMS}}$)
- Large amount of windings ($\sim n$ n =wire turns)

Counter arguments

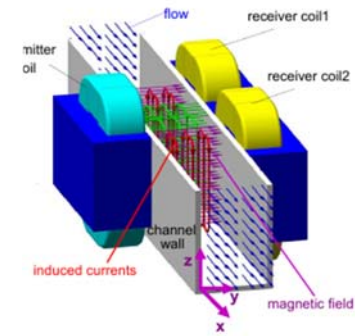
- low f yield high sensitivity to ambient stray signals
- high B modifies the flow Hartmann number $Ha \ll 1$ ($Ha = (\text{EM-forces}/\text{viscous forces})$)

$$Ha = d \cdot B \sqrt{\frac{\sigma}{\rho\nu}}$$

- too large f yield skin-effect $f d^2 \mu \sigma \ll 1$

Other designs

- clamp on systems (validation in liquid Al !!!)



7

Local velocity – miniaturized EMFM

Goal: sensor downstream rod bundle to measure local flowrate

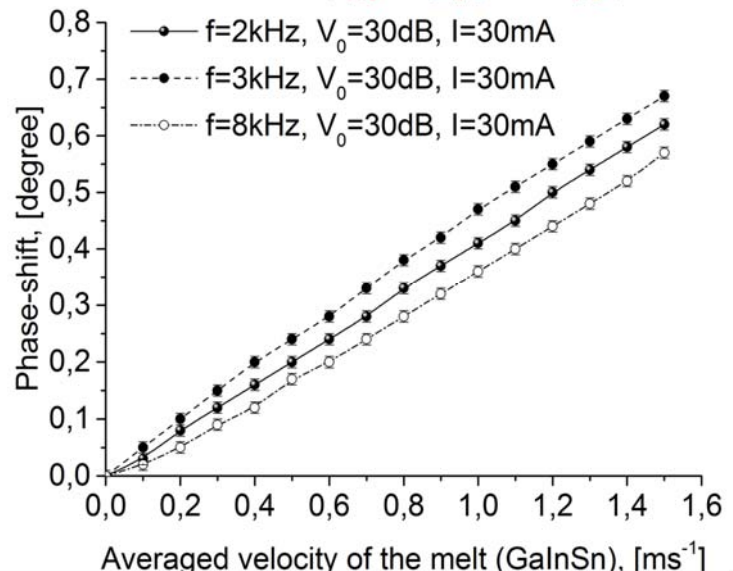
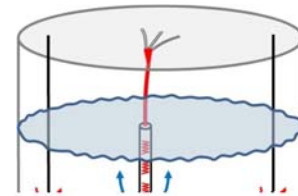
Requirements:

- velocity range: 0...4 m/s
- resolution: ± 0.1 m/s

Sensors $\phi=6, 12\text{mm}$ tested in GalnSn flow



near sensor flow

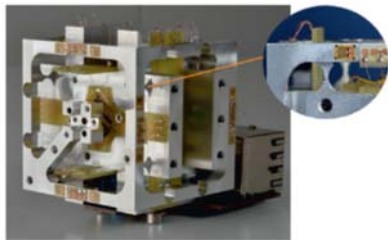
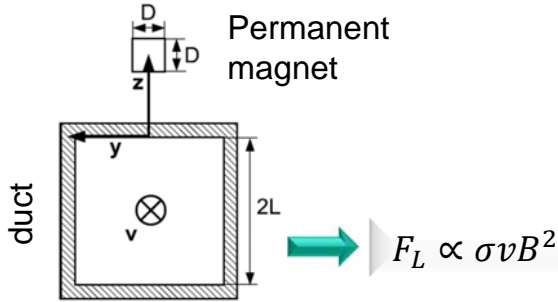


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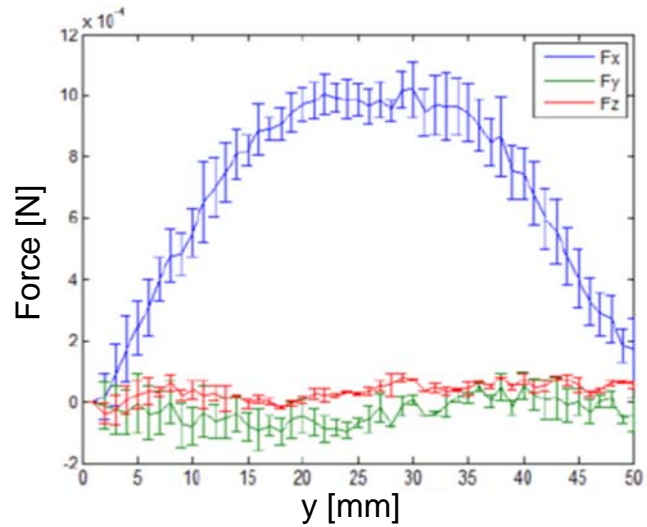
Local velocity - Lorentz Force Flowmeter (L2F2)



- Measurement of force/torque via permanent magnet close to the wall
- Force equal to Lorentz force F_L in flow
- Force/torque depended on near wall velocity
- New Multi-degree of freedom sensor: all 3 components of torque and force



multi degree of freedom sensor



Proof of principle @ GALINKA facility (TUI)

Local flow velocity - UDV



Ultra-Sound Doppler Velocimeter (UDV)

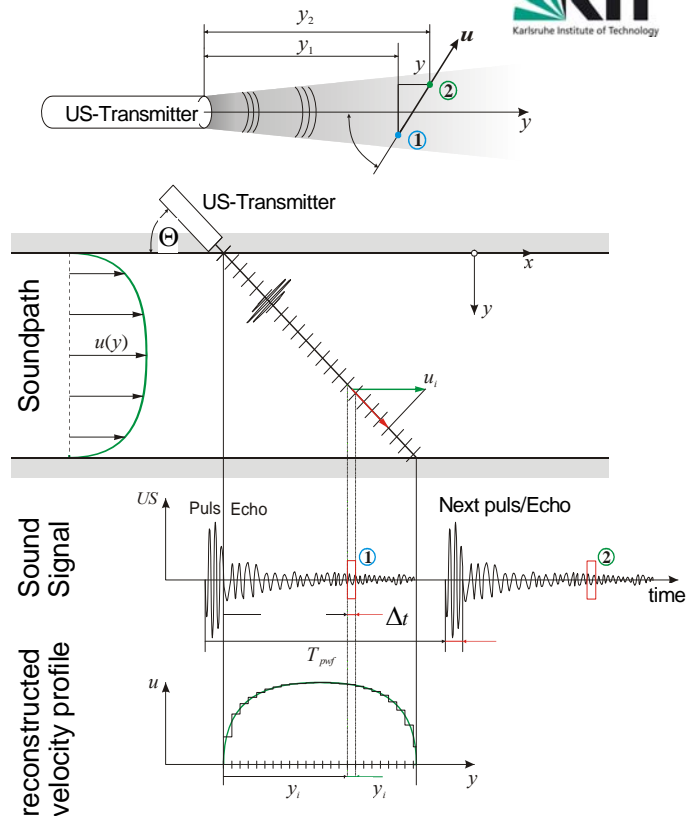
Principle (particle tracking)

- Distance change from sensor due to motion from 1→2 between two pulses.
- Determination of the time difference from the phase shift between received echoes

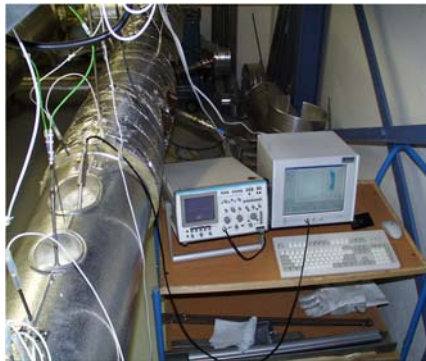
⇒ Velocity at a discrete distance

Profile

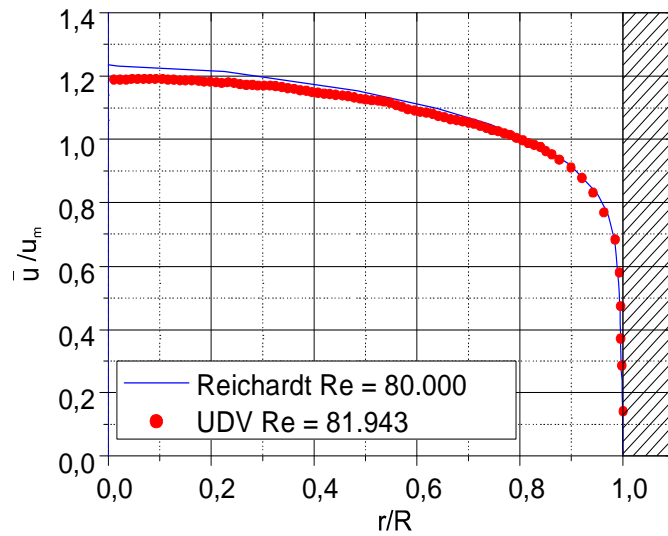
- Separation of sound path in time intervals (gates Δt) allows recording of a velocity profile. Therefore,
 - Coupling of a time t_j with a measurement position
 - Determination of the local velocity u_i in the interval i



Local flow velocity

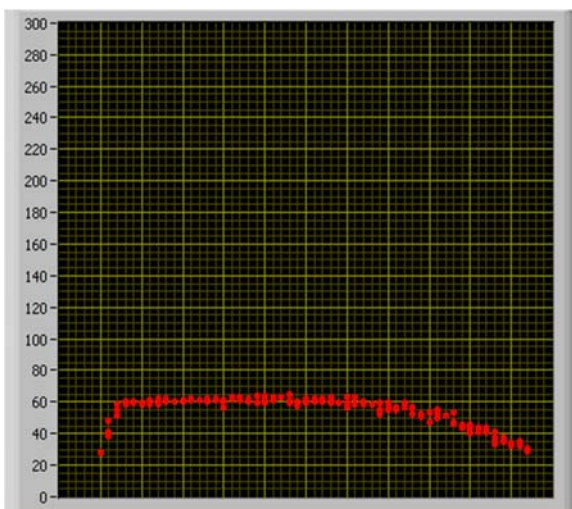


Ultra-Sound Doppler Velocimeter (UDV)-Validation



- good agreement between measurement and literature profile
- detailed resolution of the velocity profile
- deviation literature profile for $r/R > 0.6$ less than 0.5% (Schulenberg&Stieglitz, NED, 2010)

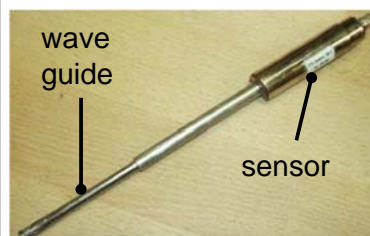
Local flow velocity



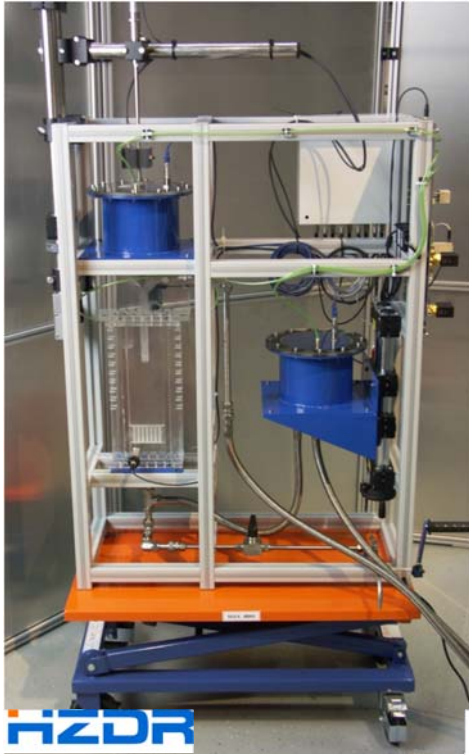
Transient start-up behaviour of EM pump in THESYS Loop

Ultra-Sound Doppler Velocimeter (UDV)

- Fluid temperature: 400°C
- Temperatur compensation durch (Wave Guide)
- Inclination angle: 45°
- Tube diameter: 60 mm



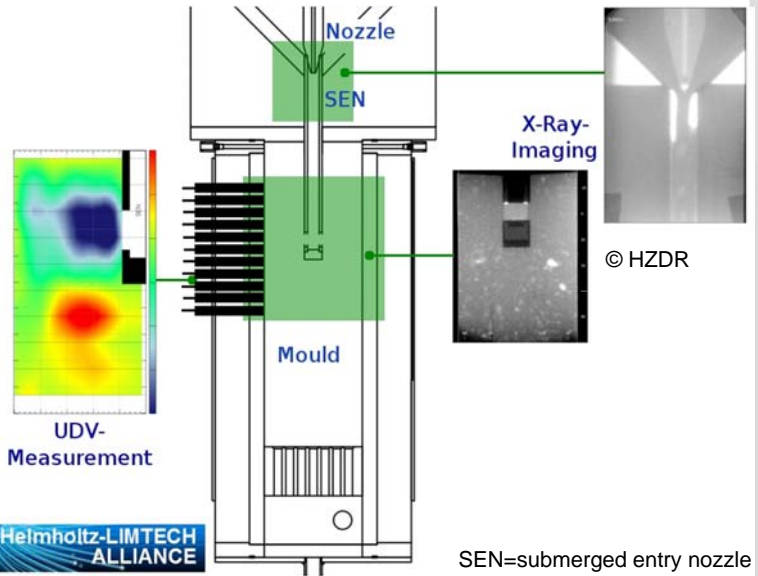
Flow visualization- 2 phase-flow –X-Ray



Main feature:

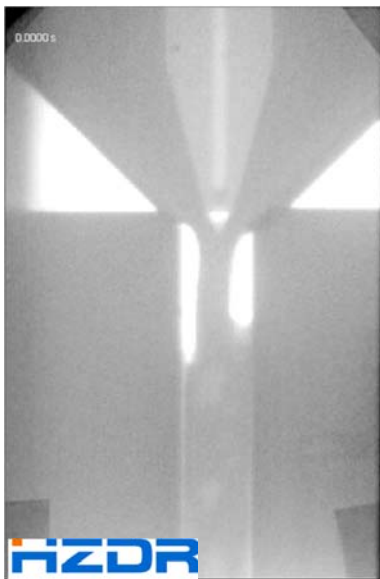
- X-ray visualization of two-phase flows
- Restriction of the mold size in beam direction

Example : LIMMCAST @ HZDR

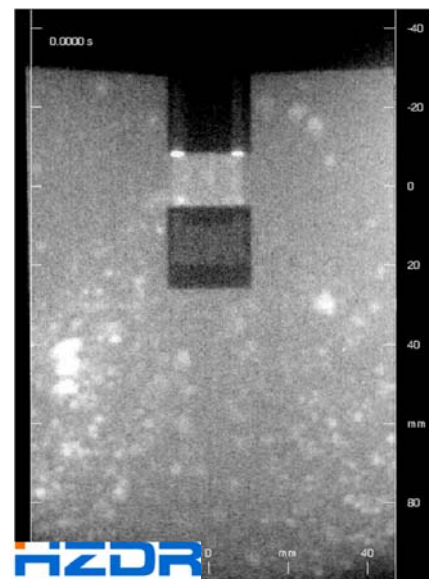
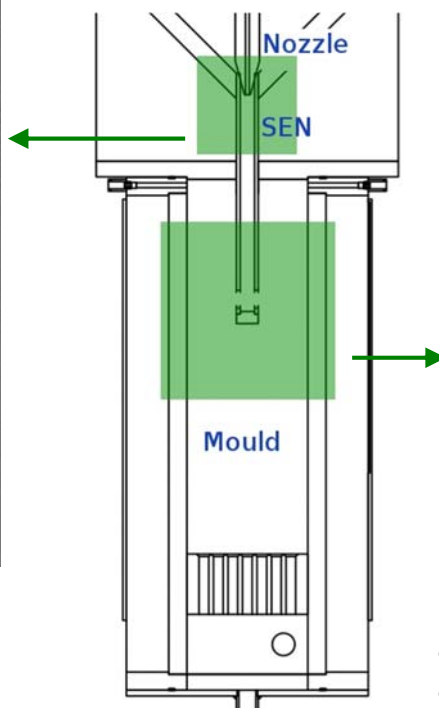


SEN=submerged entry nozzle

Flow visualization- 2 phase-flow –X-Ray



Complex flow regimes



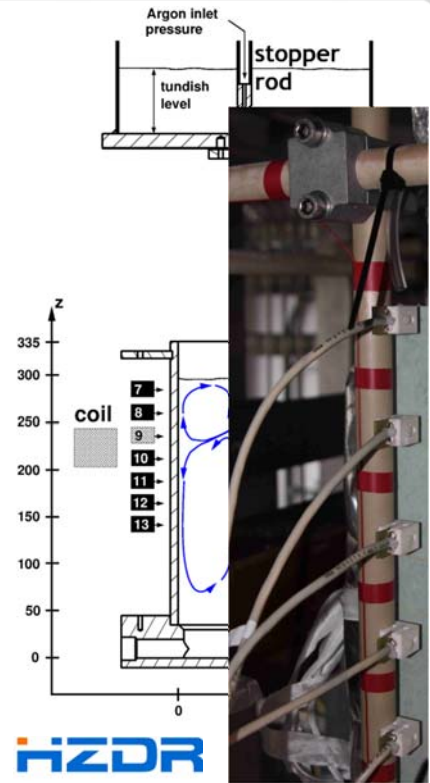
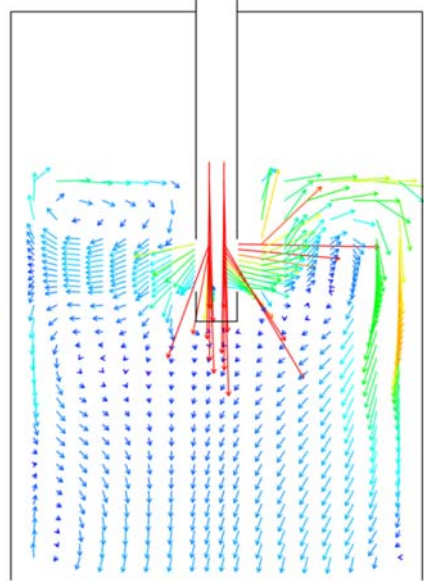
Flow rates:

- Ar: 1,7 cm³/s
- Liquid metal: 120-130 ml/s

Flow field reconstruction - CIFT

Contactless Inductive Flow Tomography (CIFT)

- flow field \mathbf{v} modifies externally applied magnetic field \mathbf{B}
- Measured magnetic field outside melt contains information on flow field
- Reconstruction of velocity field by measured induced magnetic fields \mathbf{b}



$$\mathbf{b}(\mathbf{r}) = \frac{\mu_0 \sigma}{4\pi} \int_V \frac{(\mathbf{v}(\mathbf{r}') \times \mathbf{B}(\mathbf{r}') - \nabla \varphi(\mathbf{r}'))(\mathbf{r} - \mathbf{r}')}{|\mathbf{r} - \mathbf{r}'|^3} dV'$$

F. Stefani et al., Phys. Rev. E (2004) 70, 056306
 T. Wondrak et al, Meas. Sci. Technol. (2010) 21, 045402

➔ higher resolution by use of externally applied AC magnetic fields \mathbf{B}

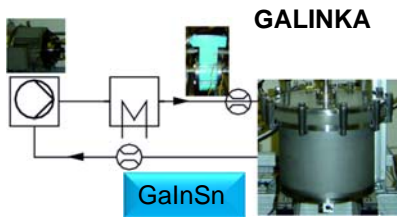
Technology systems

- loop facilities
- material and qualification stands

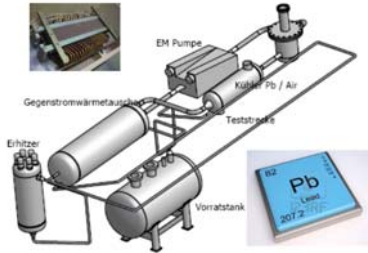
INFRASTRUCTURES

Infrastructures –loop facilities

Table-top/proof of principle



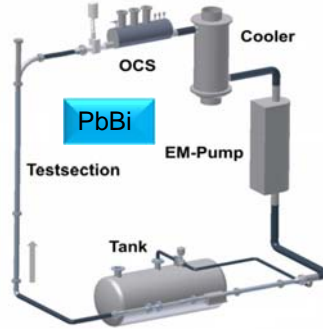
TELEMAT



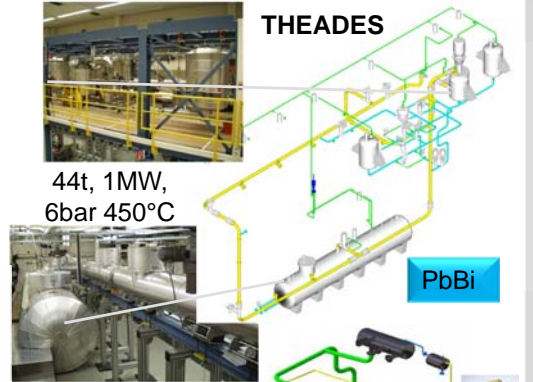
Available liquids

- Lead, PbBi-eutectics, tin
- Gallium-Indium, tin
- Sodium, NaK,

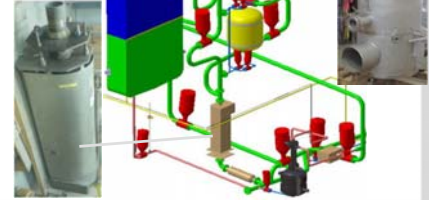
Laboratory scale



Pilot-scale (proto-typical)



KASOLA
7t, 2MW,
6 bar, 550°C



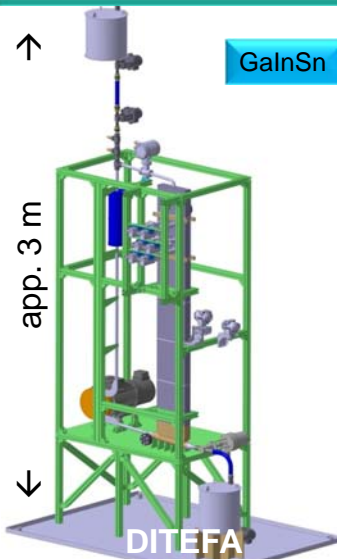
Infrastructures –technology development

Phase transition (boiling)



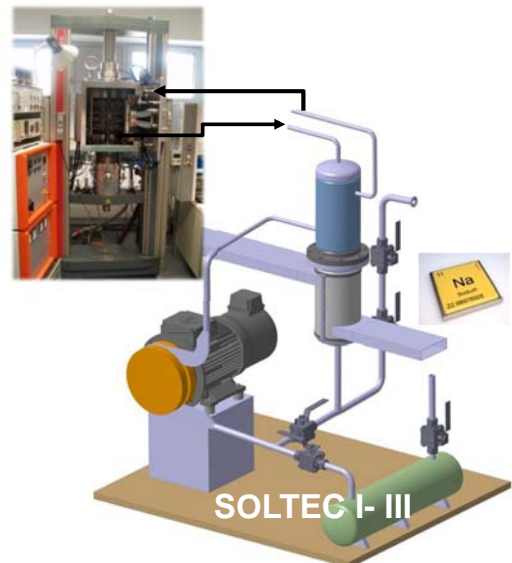
- Sodium boiling (thermal-electric energy conversion)
- Instrumentation qualification

Flow phenomena



- transitional flow evaluation
- proof of principle instrument.
- qualification CFD and systems codes (TRACE, ASTEC-Na*)
- education & training

Materials in flowing LM



- steel corrosion qualification* up to 1000K
- creep tests
- stress corrosion cracking

Infrastructures –material qualification

Material corrosion



COSTA = **C**orrosion test stand for **S**Tagnant liquid lead **A**lloys

- Operative since : 1997
- Pb or Pb-Bi
- Equipped with O₂-control
- Influence of protection layers and coatings on corrosion

Coolant control



KOSIMA: **K**arlsruhe **O**xxygen **S**ensor **I**n **M**olten **A**lloys.

- Operative since: 1998
- Pb or Pb-Bi
- Equipped with O₂-control
- Development and calibration of oxygen sensors

Coolant conditioning



KOCOS: **K**inetics of **O**xxygen **C**ontrol **S**ystems

- Operative since: 1999
- Pb or Pb-Bi
- Equipped with O₂-control
- Diffusion coefficient and mass exchange rates for oxygen

Features

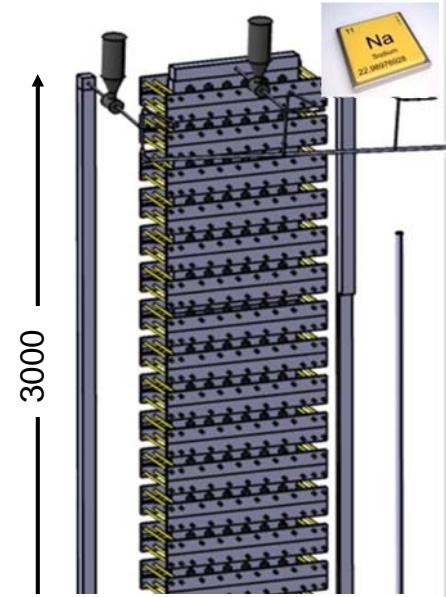
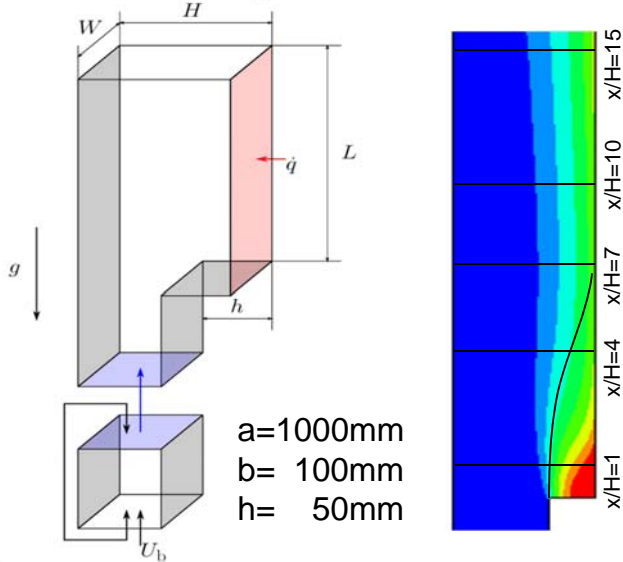
- Generic phenomena (micro-scale)
- FR –FA flow experiments
- System tool qualification

THERMAL-HYDRAULICS

Thermal-hydraulics –Generic science

Vertical Backwards Facing Step (BFS)

- Identification of the **transition regimes**
- **Development**, improvement of **anisotropic heat flux** and momentum **models** (DNS, LES and RANS)
- Enhanced **engineering correlations**



Sodium ($Re=5 \cdot 10^3$)

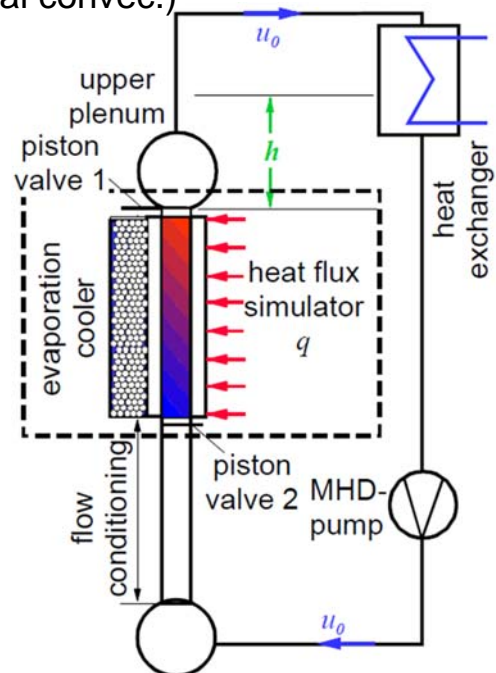
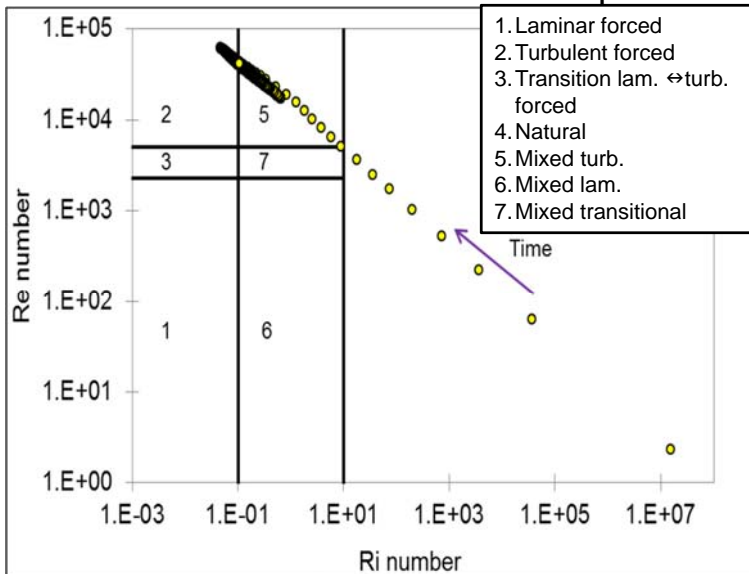
- T_{wall} in recirculation zone too low
- Temperature prediction in developing region adequate



Thermal-hydraulics –Generic science

Transitional Flows

- during start-up, shut-down or LOFA
- flow regimes altered (forced \rightarrow mixed \rightarrow natural convec.)
- instabilities in the heat removal
- excess of material sustainable temperature



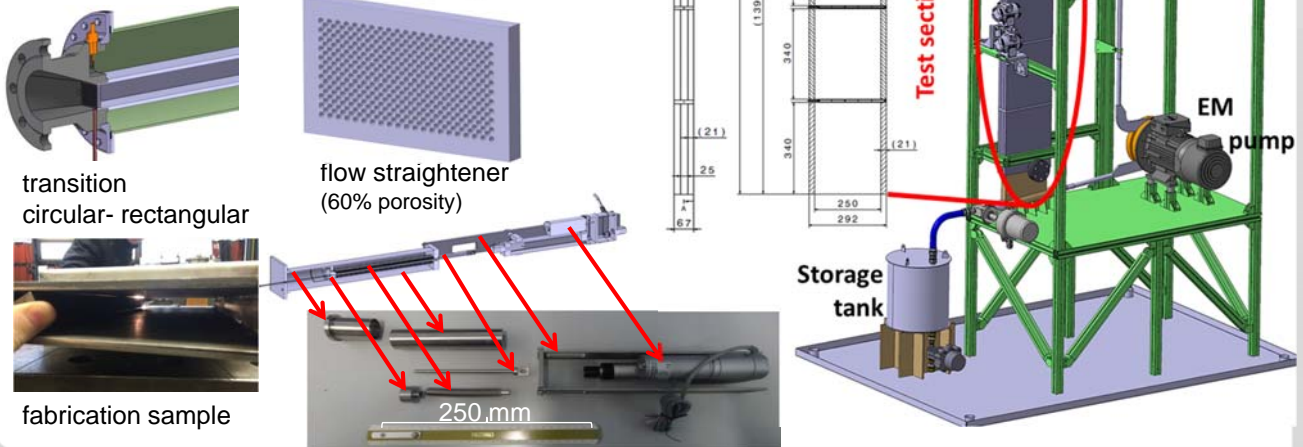
Thermal-hydraulics –Generic science

Transitional flows

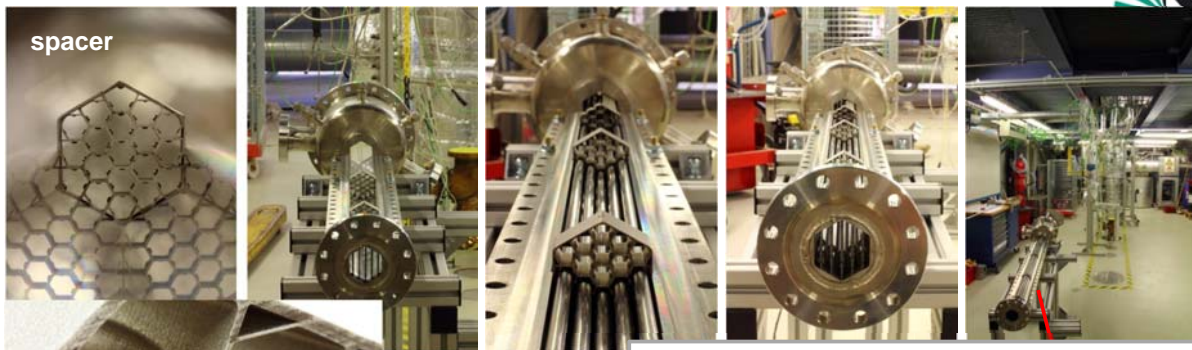
- InGaSn-Loop
- volume of ~ 16 l
- Counter-Current Water-InGaSn HEX
- Test section: 22 x 220 x 2000 (DxWxH,mm)
- Electro-magnetic pump

Goals:

- Identification of transitional states
- Qualification of measurement devices
- Asset for education and training



Thermal-hydraulics – FA experiments



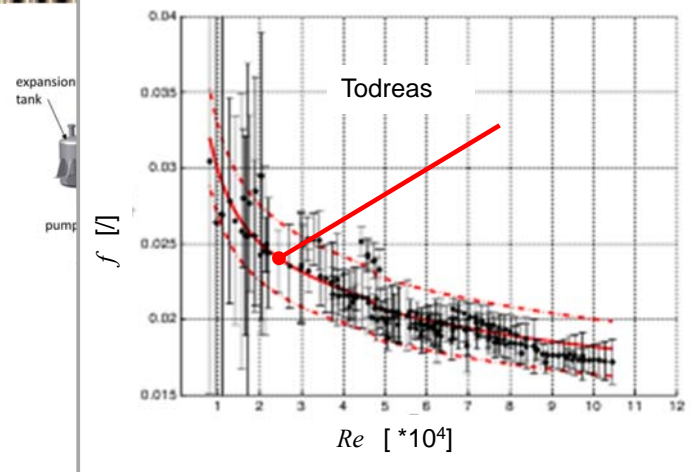
Hexagonal FA with spacers

$P/D= 1.4$, $D = 8.2\text{mm}$, $h=870\text{mm}$
 19 Pins, $Q_{el}=440\text{kW}$, $q''=1\text{MW/m}^2$

Results

- Nu -Numbers 20% higher than Miktiyuk

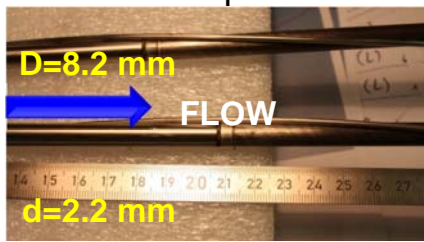
J. Pacio et al. / Nuclear Engineering and Design 273 (2014) 33–46



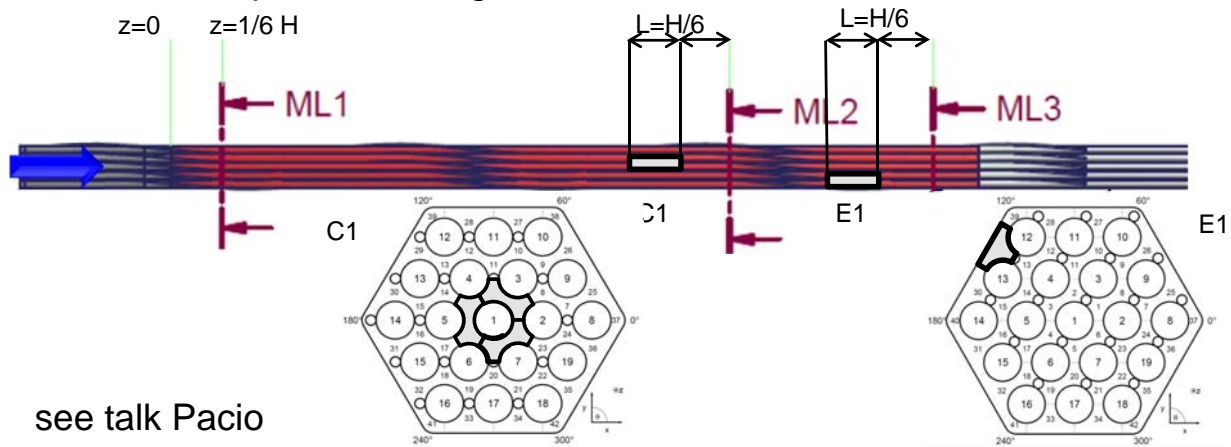
Thermal-hydraulics –FA experiments

Consecutive further elaboration

- FA- with wire wraps $P/D=1.29$



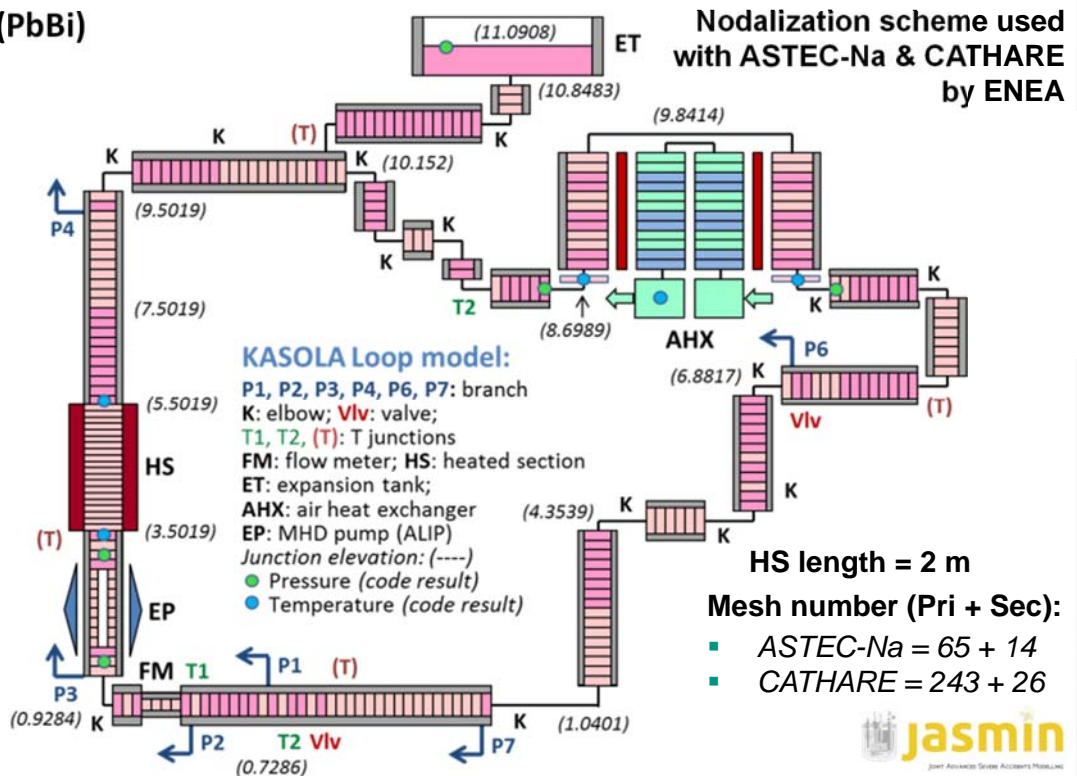
- Now FA with partial blockage



- see talk Pacio

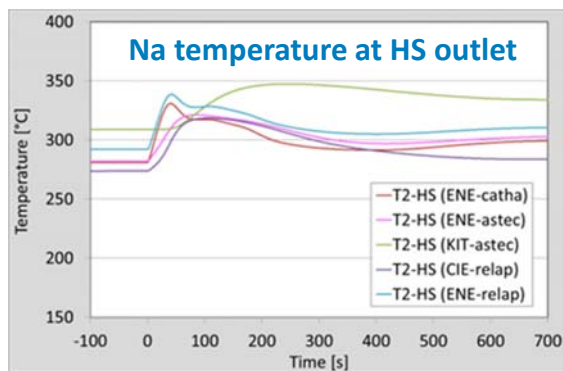
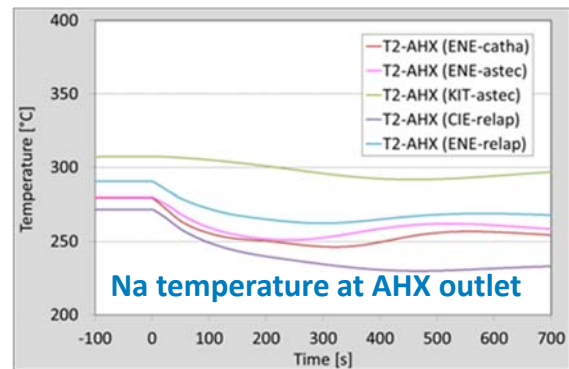
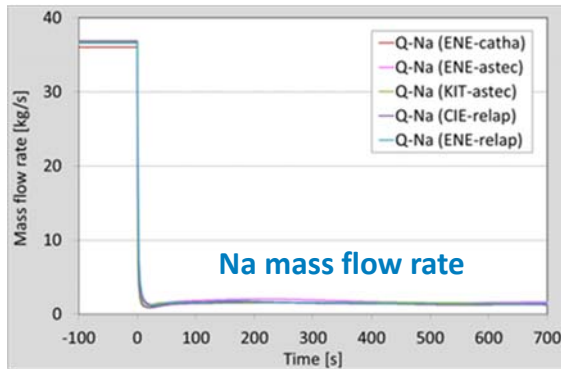
Thermal-hydraulics –loop simulation

- More reliable loop system description in Na (counterpart to LACANES (PbBi))



Thermal-hydraulics –loop simulation

- Anticipated ULOF starting at 80 kW power



- All codes predict NC primary mass flow rate around 4% of nominal one
- Large margin against primary temperature safety limit ($T = 550$ °C) and the risk of sodium freezing ($T_{sol} \sim 100$ °C)

Expertise fields

- Design basis events (from core → system)
- Accident analysis (from failure → core degradation)
- Containment behavior

SAFETY ASSESSMENT

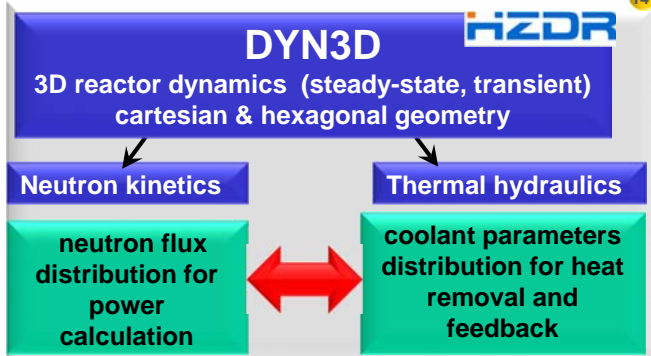
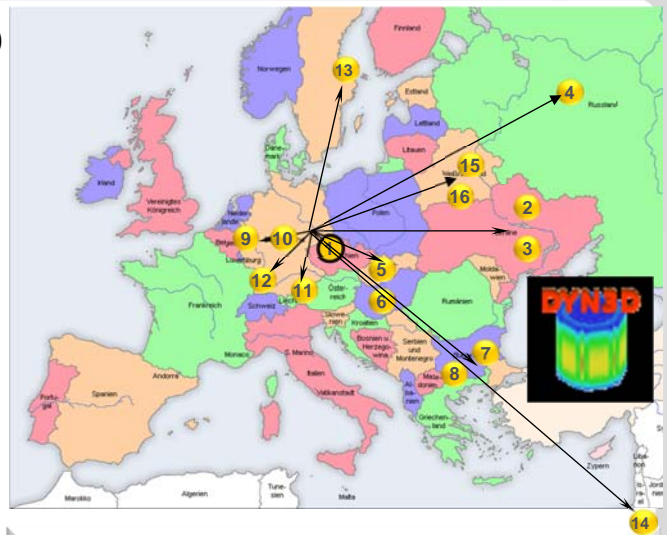
Design basis events – DYN3D

Coupled N-TH simulation by DYN3D

- developed at HZDR for safety analysis of LWR cores
- validated code versions for Western PWR's, BWR's and Russian VVER reactors

Extension of DYN3D to FR's systems focussing on:

- XS generation methodology
 - Validation of DYN3D by fast reactor experiments
- models for thermal expansion effects
 - Axial thermal expansion of the fuel rods
 - Radial thermal expansion of the diagrid



Design basis events – DYN3D

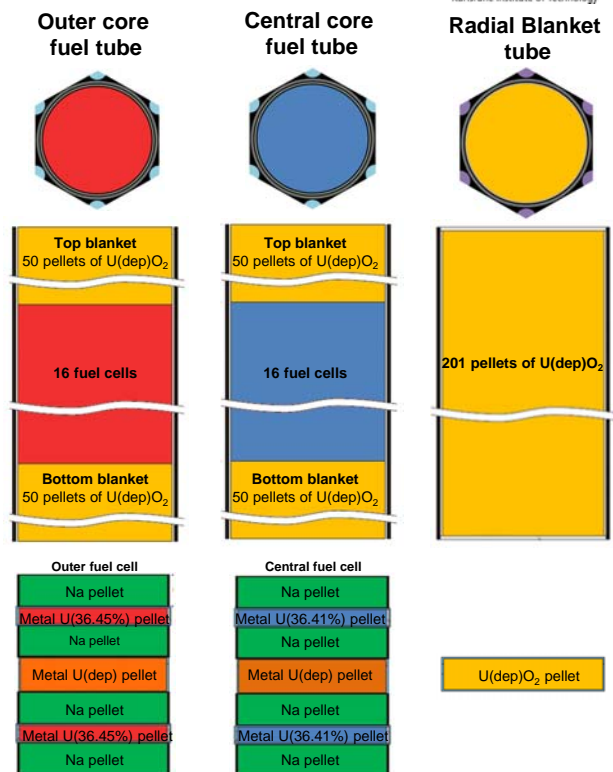
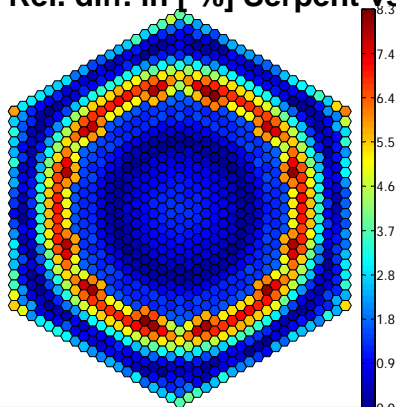
Neutronic analysis of the BFS-73-1 critical assembly using DYN3D and Serpent

- 3D detailed assembly calculation
 - DYN3D: diffusion calculations
 - Serpent: MC calculations
 - Generation of few-group XS for DYN3D
 - Reference solution

Assembly k_{eff} : Difference: -12 pcm

Assembly radial power distribution

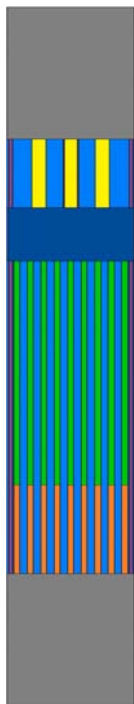
Rel. diff. in [%] Serpent vs. DYN3D



Design basis events – DYN3D

Axial fuel rod expansion model via mixing method:

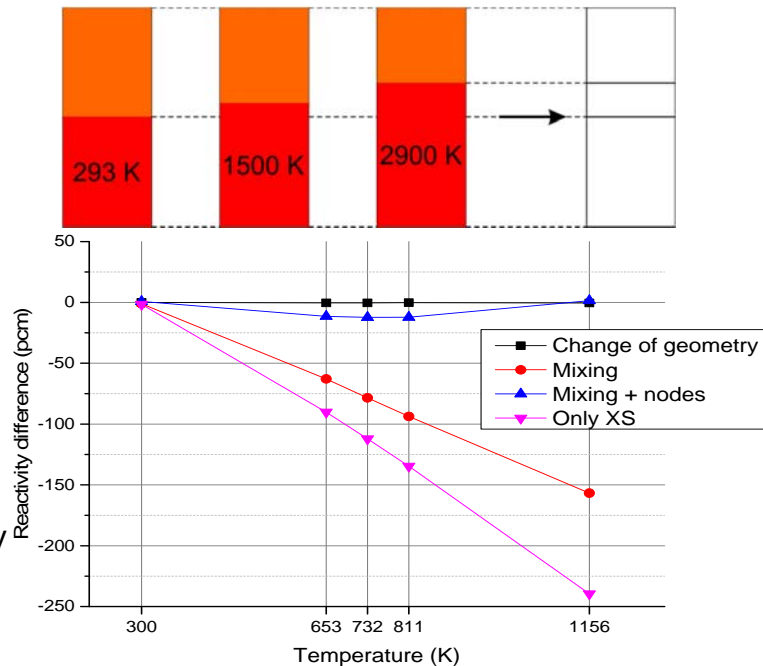
- Approach: insertion of additional node after each expanding material interface



Example:

- Axial fuel rod expansion in one PHENIX assembly

Reasonable agreement in terms of reactivity



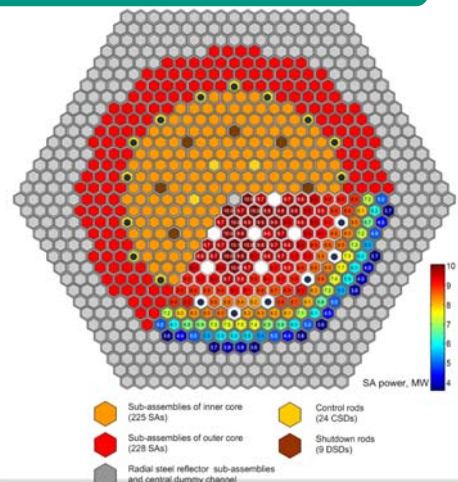
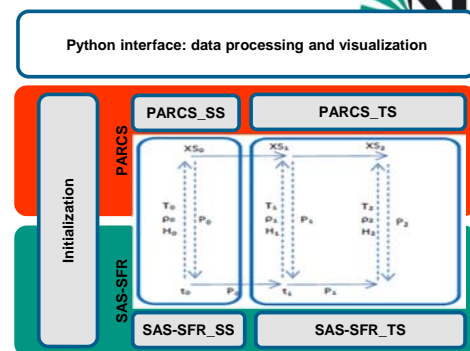
Design basis events - Coupling PARCS/SAS-SFR

Methods applied

- “on-the-fly” XS generation system (“Sigma-0” method, 33 EG, all nuclides)
- data exchange procedure with time clocking
- geometry transfer, expansion model developed (independent axial representation in TH and N models and averaged fissile core height)

Capability to account reactivity feedbacks:

- fuel Doppler
- axial + radial expansion of pin & SA elements
- sodium density and void
- fuel and clad relocation



Validation: ESRF Reference Oxide core, BOL

- models for 2 and 10 channels
- feedbacks for all important nodes of fissile height and upper core regions (UAB, UGP, USP)

PhD Ponomarev

Design basis events - Coupling PARCS/SAS-SFR



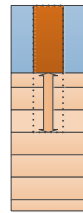
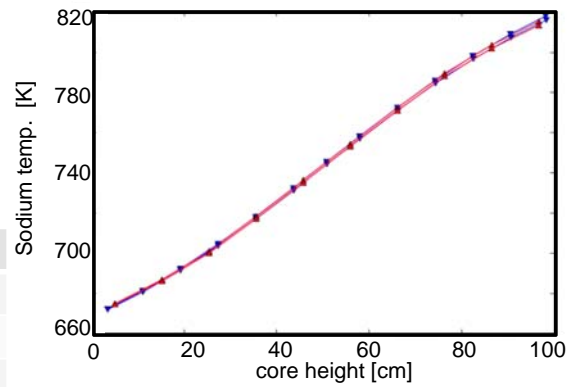
Steady state calculations

- Different independent axial representations
- Comparison: SAS-SFR, PARCS
- Updated node parameters (each iteration):
fuel, clad, sodium masses + temp, axial node height

Comparisons with SAS-SFR/MCNP:

Parameter	SAS-SFR/MCNP	SAS-SFR/PARCS
K-effective (nom.)	1.0124	1.01195
Doppler constant, pcm	-1200	-1198
SVE active core	+1650	+1773
SVE active core + SP	-	+1650 (-123)

➔ steady state adequately depicted



Transient calculations –Validation cases

- CR insertion/ withdrawal (fig./table right)
- “Hypothetical” ULOF transient with 2.5 s pump half-time trip and 200% of nom.power
- Reference ULOF 10s (as in ESFR)

core config. (from top)	insertion/ withdrawal, [cm]	transient dynamic reactivity [\$]		β [pcm]
		positive	negative	
1 mesh	10.17	0.81	-	249.88
2 mesh	20.34	2.10	-1.51	

PhD Ponomarev

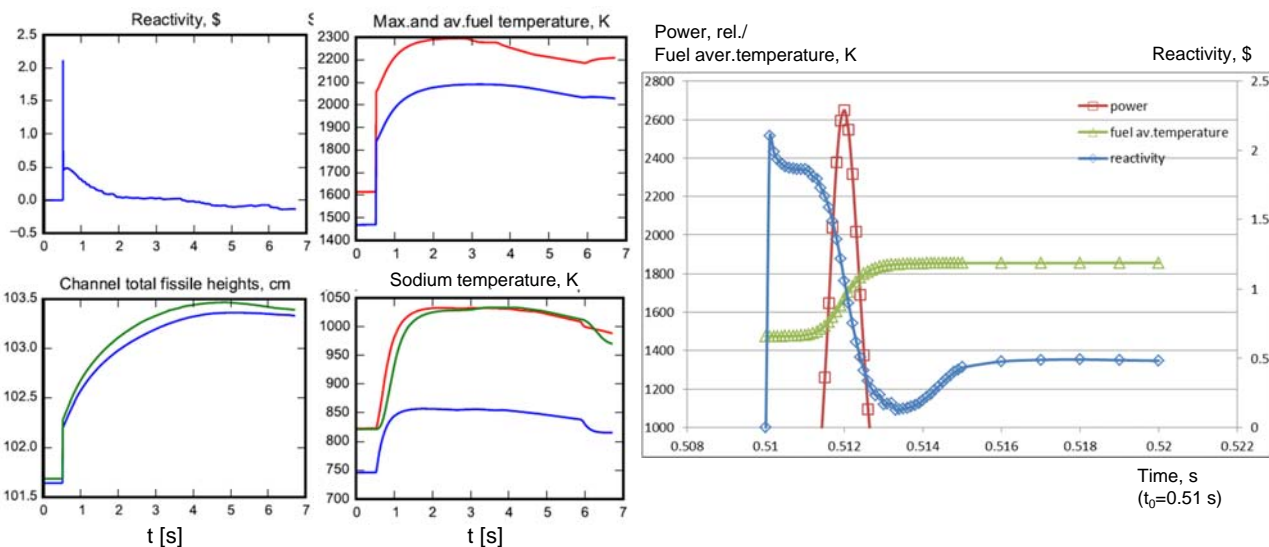
Design basis events - Coupling PARCS/SAS-SFR



Transient simulations results CR movement

Case #3 - outer CRs withdrawal: 2.10\$

- power rapid increase by factor 2.65, radial power redistribution up to 8%
- fuel failure occurred (few nodes in mid.plane): ch#2 at 2.596 s, ch#1 at 4.046 s



Design basis events - Coupling PARCS/SAS-SFR



Transient simulations results reference ULOF

Case set-up

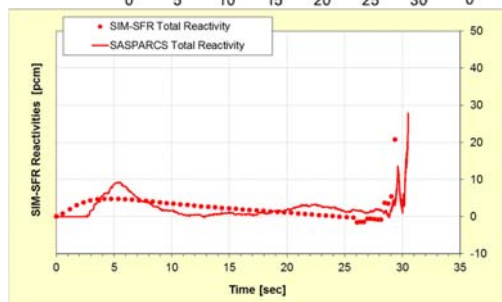
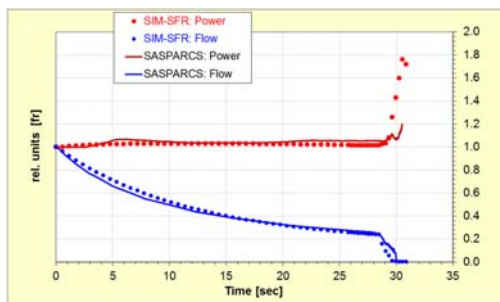
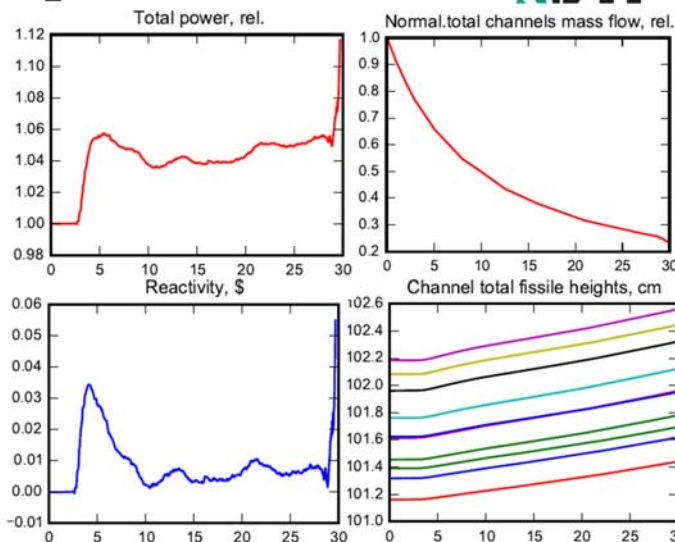
- nom. power 3600 MWth
- pump trip with 10 s half-time

Results

- shortest boiling time 27.35s
- boiling in almost all channels
- radial power shape change ~1.5%

Comparison with SIM-SFR

- good agreement with ESFR comparison (without CR drive lines effect)



PhD Ponomarev

Accidental safety analysis- Frame



FR cores with TRU= core not in most reactive configuration
Re-criticality potential allow disruptive accidents (CDAs)

CDA described by 3 different phases :

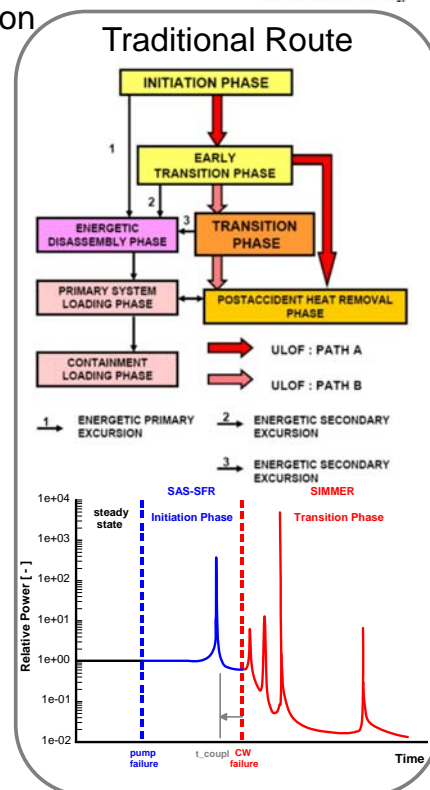
- Initiation Phase (IP) → can-wall failure
- Transition Phase (TP) → creation of molten fuel pool,
- Expansion Phase (EP) → mechanical energy release.
- Multi-physics description mandatory N/TH/SM

Options

- SAS4A/SAS-SFR- FRED (channel approach) for IP
- SIMMER for TP and EP
- SIMMER stand-alone simulation**

Complementary approach

- Model optimization in accidental codes mainly ASTEC**
 - boiling models, gap heat transfer, fission gas behaviour (axial fuel expansion, melting limits), clad mechanics
- SIMMER improvements**
 - thermal expansion reactivity feedbacks (axial/radial)
 - reactivity effects due to fuel-steel mixing
 - fine mesh approach → accountance for coherency effects



Accidental safety analysis- model improvement



Simulation of fundamental boiling experiments

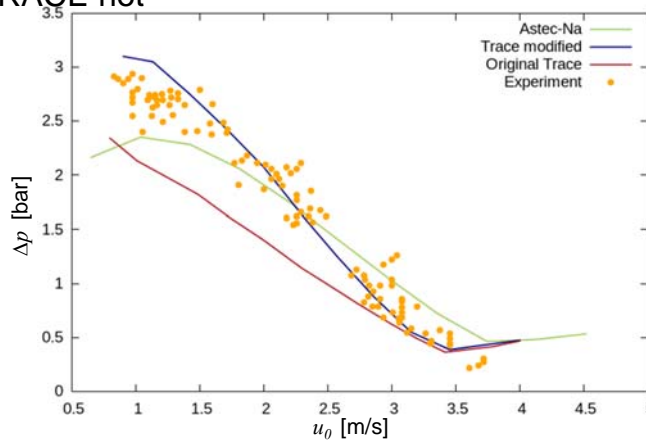
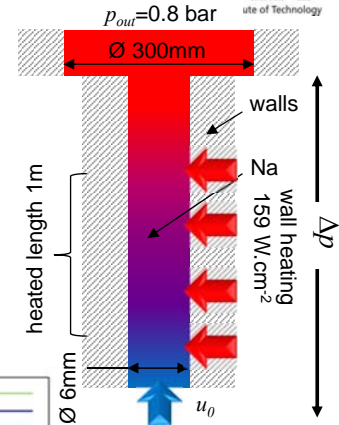
- Ispra Tubular Experiment (ITE)

Approach

- modified Lockhart-Martinelli correlation for pressure regular two-phase pressure
- several flow quality factors investigated (from literature)

Results

- ➔ improved system provides better description
- ➔ original ASTEC, TRACE not conservative



PhD Haselbauer

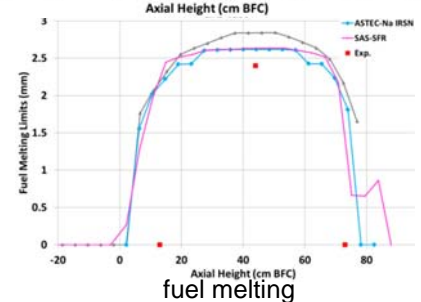
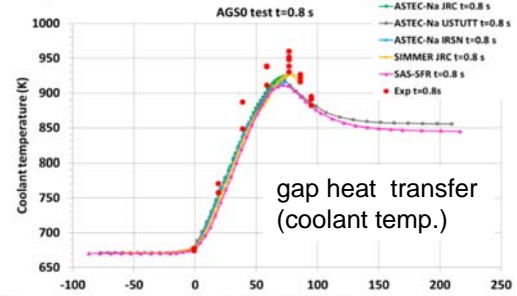
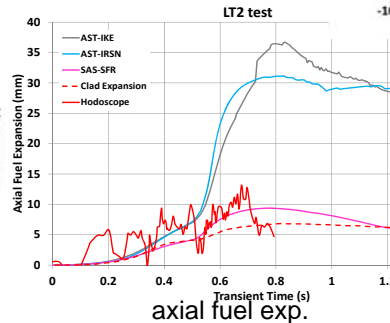
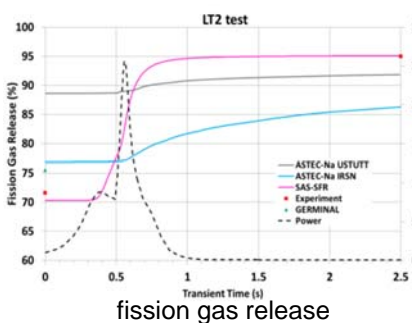


Accidental safety analysis- model improvement



Approach

- identification of modelling deficits
- by code-to-code comparison complemented by experimental data



Some Results

- gap heat transfer model validated
- fission gas model contains many parameters ➔ sensitivity analysis of some parameters
- axial fuel expansion overestimated ➔ visco-plasticity model now in ASTEC-Na V2.0
-

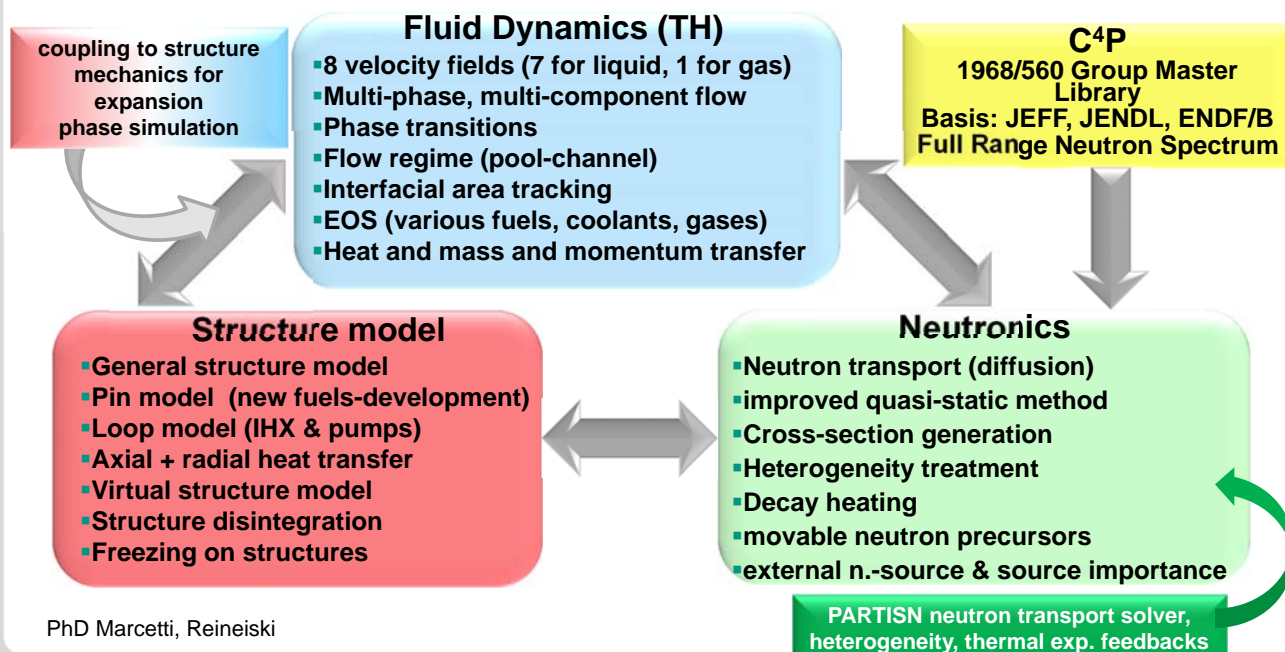


Accidental safety analysis- SIMMER developments



Simmer III, IV advancements

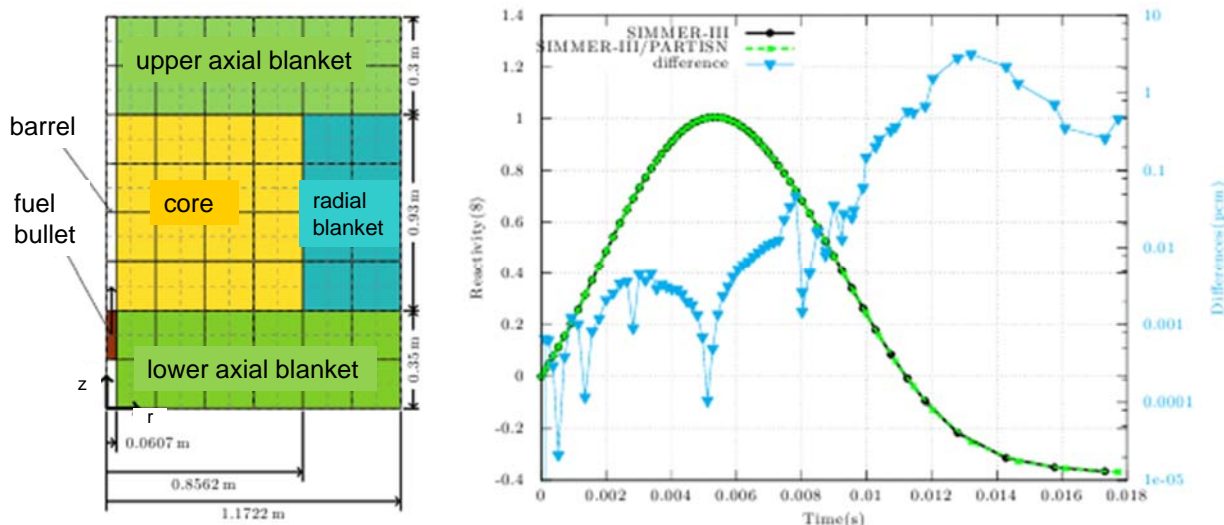
- modifications TH –SM Coupling
- advanced neutron physics description (incl. expansion, feedbacks)



Accidental safety analysis- SIMMER developments



- Performance assessment by Space Time Neutronics (STN) Benchmark



Results

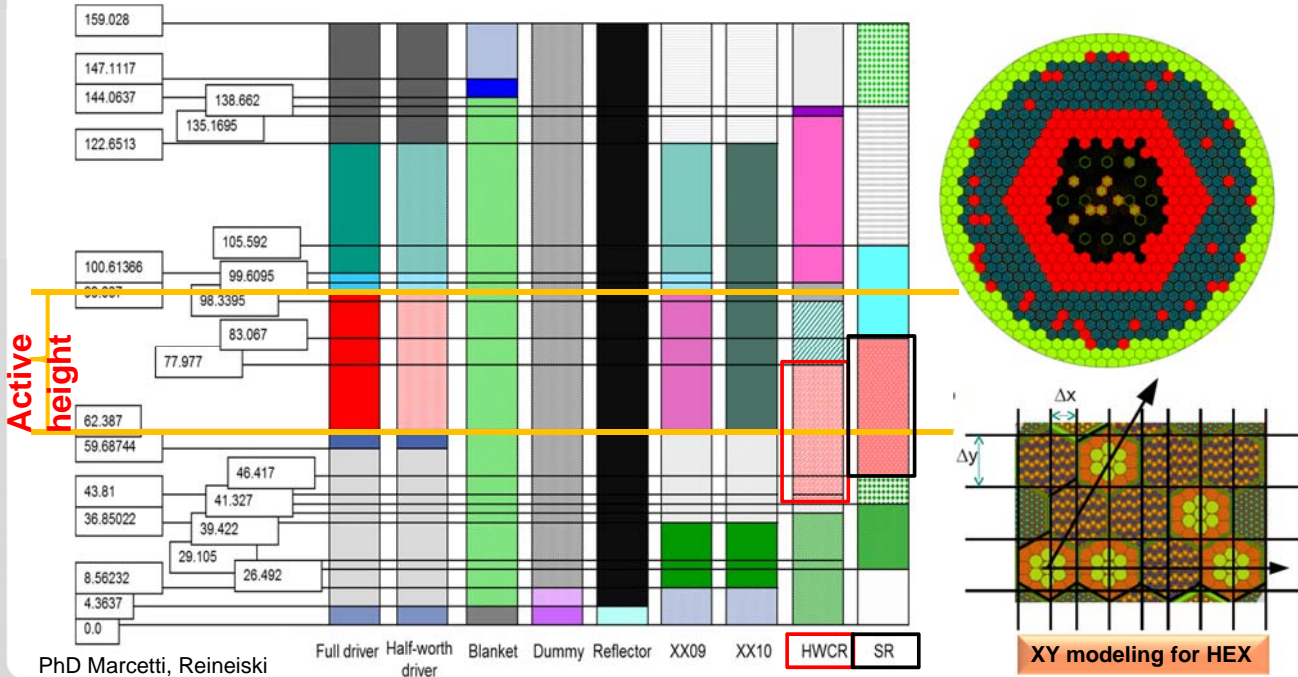
PhD Marcetti, Reineiski

- Excellent agreement in reactivity, flux, amplitude (<1% deviation)
- Similar performance for 2D ULOF of pool type LMFBR
- ➔ space-time-kinetics also with heterogenous approach validated

Accidental safety analysis- SIMMER developments

IAEA Benchmark (EBR-II Shutdown Heat Removal Tests) Results:

- Good agreement with partners **and** experiment, using HEX and XYZ models.
- Excellent performance of PARTISN **XYZ model for criticality, reactivity effects, power profile**



PhD Marcelli, Reineiski

Accidental safety analysis- Containment behaviour

Sodium fires (physical & chemical processes) in severe accidents

Approach: Validation of CONTAIN LMR wrt. single effect phenomena

- atmosphere thermodynamics including condensation and vaporization of sodium;
- reactions between sodium and oxygen or water (sodium spray or pool fires);
- sodium aerosol behaviour.

Reference FAUNA experiments @ KIT

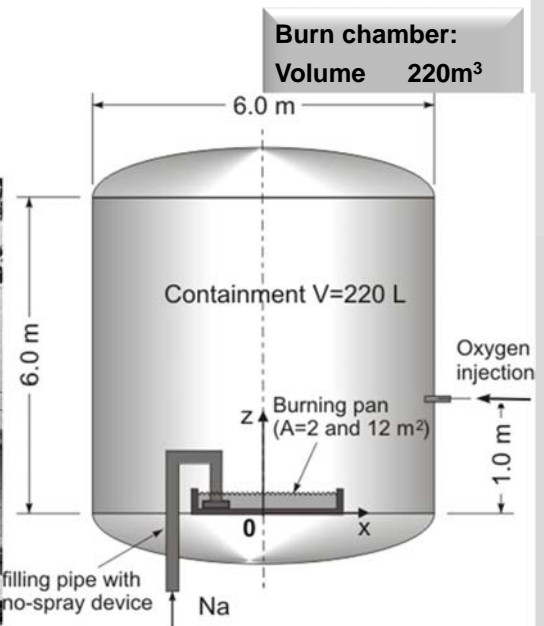
CONTAIN- Features

- oxygen diffusion model (pool-fire), spray fire models
- sodium combustion chemistry
- aerosol description (diffusion, gravitational settling, agglomeration)

experiment No.	F1	F2	F3
pool surface (m ²)	2	2	12
sodium (kg)	150	250	500
pool depth (mm)	90	150	50
c _{O₂} (vol.%)	19-22	17-25	15-25
T _{start} (°C)	550	550	550
T _{Pan} (°C)	250	250	250

More information:

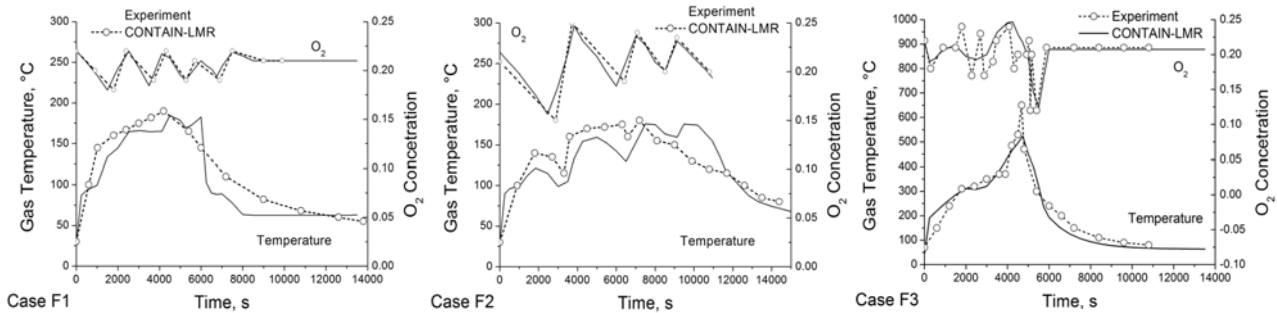
- Gordeev et al. 2014, CONTAIN-LMR simulation of sodium aerosol behaviour during sodium fires, ICAPP 2014.
- Cherdron W, 1985. Thermodynamic Consequences of Sodium Spray Fires in Closed Containments. Part 1. KfK 3829, Juni 1985



Accidental safety analysis- Containment behaviour



Computed vs. measured oxygen concentration and gas temperature in containment



measured vs. calculated average values of burning rates

Experiment	Experiment (kgNa/m ² h)	CONTAIN- LMR (kg Na/m ² h)
F1	29.5	30.
F2	21	28.5
F3	33	30

Results

- For F1 and F2 the max. gas temperature in second phase of pool fire over-predicted + cooling down more abrupt. (Reason: @ large pool depth crusting of the pool surface controls combustion process in second fire phase.
- incomplete combustion of sodium and slow cooling down of hot reaction residuals and unconsummated sodium.

OVERALL

- reasonable description of different sodium fire types
- More accurate prediction requires both experimental and numerical efforts

SUMMARY



- SFR related R&D mainly conducted in frame of international programs (EU programs, bilateral cooperations) – focus reactor safety
- Participation in international context (IAEA, NEA-OECD), in particular for IAEA CRP projects desired – conceived as preservation of competence
- Technology development reduced in nuclear context but also contributions in future by R&D in complementary science fields

R&D focus

- Development of simulation tools to improve analytic capabilities
- Technology development as cornerstone of knowledge
- Safety assessment as mission of provisional R&D in Helmholtz
- Education and training

DRESDYN

A European platform for dynamo experiments and thermohydraulic studies with liquid metals



Internal containment (~ 50% of the lab) for a precession driven dynamo experiment

Building ready
first GaInSn experiments: spring 2016

START-UP 2016



Water pre-experiments : mid 2017
first Na experiments: 2017/2018