

McSAFE – High Performance Monte Carlo Methods for SAFETY Demonstration

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From Proof of Concept to Industry- like Applications

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nuclear Generation II & III R&D

Content



- ❑ Scientific gaps
- ❑ McSAFE Technical goals
- ❑ McSAFE Main Concept
- ❑ McSAFE WPs, Partners and Structure
- ❑ McSAFE Tools

Scientific Gaps



❑ Core analysis relies mainly on deterministic neutronic codes (daily work)

- Diffusion codes include multiple approximations (energy, angle, homogenized geometry)
- Pin power approximately reconstructed from 2D lattice calculations
- SP_3 and S_N solvers are still under development.
 - *Currently very time and memory expensive*
 - *Parallel versions under development*

❑ Experimental data at pin level is scarce and not easy to be measured (pin power)

❑ Alternative option:

- Use of MC codes capable of simulating the neutron transport without approximations
- Potential use taking advantage of HPC and parallelization

Innovative solutions needed to pave the way for industry-like applications

McSAFE: Technical Goals



- ❑ **McSAFE is based on innovative ideas developed within the EU 7. FP HPMC Project (High Performance Monte Carlo Core Analysis: 2011-2014)**
 - Optimal MC/TH coupling, stable MC-based depletion, **dynamic MC**
 - Many more ideas to simulate whole cores using HPC: UFS, Wieland Shift, Stochastic implicit Euler, ...

(Proof of concept)



- ❑ **Goal: Move MC methods towards industrial applications**
 - **Generalize N/TH coupling to provided reference solutions**
 - Optimize depletion simulations (stability, CPU)
 - **Analysis of transients such as RIA and others (Safety)**
 - Solve whole cores making use of HPC (improve statistics, reduced CPU)
 - **Validate MC tools using experimental data**

(Industry-like applications → McSAFE)

McSAFE: Time Frame



- Phase 1: 2011 (10)-2014 (9), 7 FP EU HPMC project

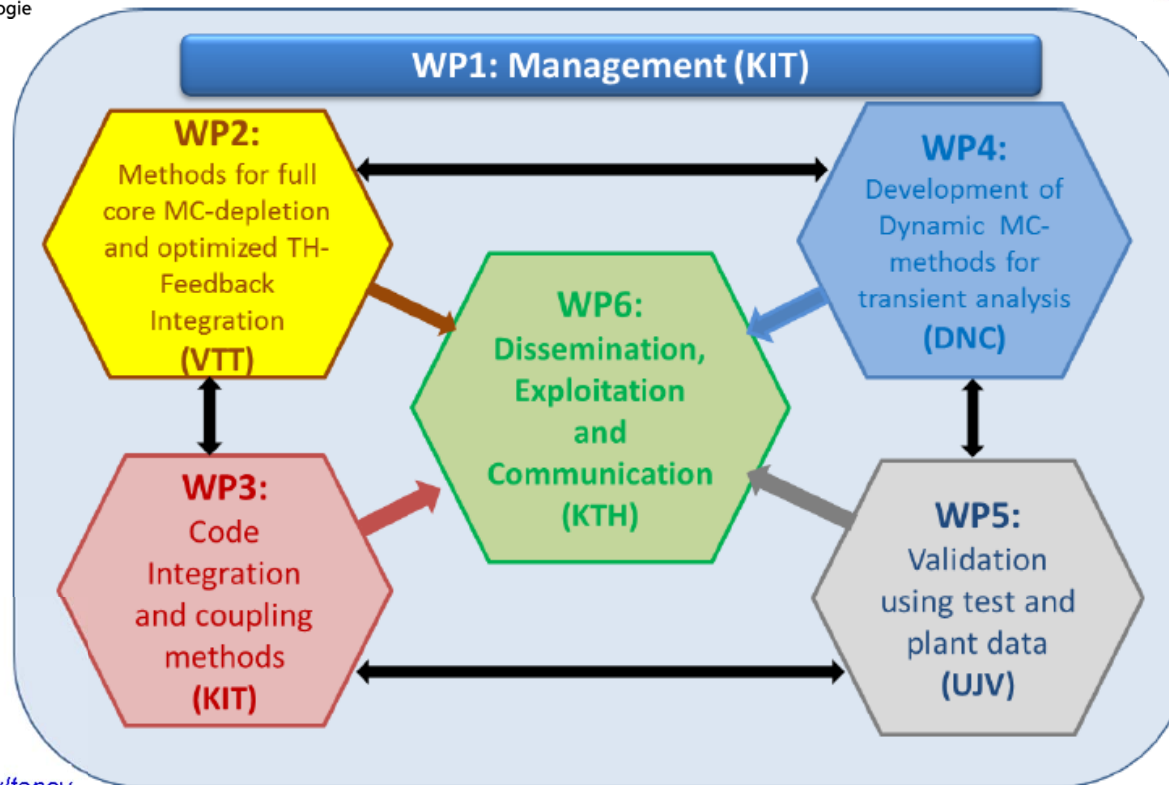
- Phase 2: 2017 (9)-2020 (8) → McSAFE

- Consolidation of the methods for MC-TH coupling
- Advance validation using exp. Data
- Consolidate dynamic MC under HPC

- Phase 3: 2020-2023 (9): McSAFE II NOIP

- Benchmarking of MC with high-fidelity deterministic core solvers for REFERENCE SOLUTIONS
- Benchmarking of dynamic Monte Carlo solutions with transient deterministic codes for SAFETY CASES (pin level, steady-state and transient)
- Uncertainty quantifications
- Extension to Gen-IV reactors, research reactors

McSAFE: Work Packages

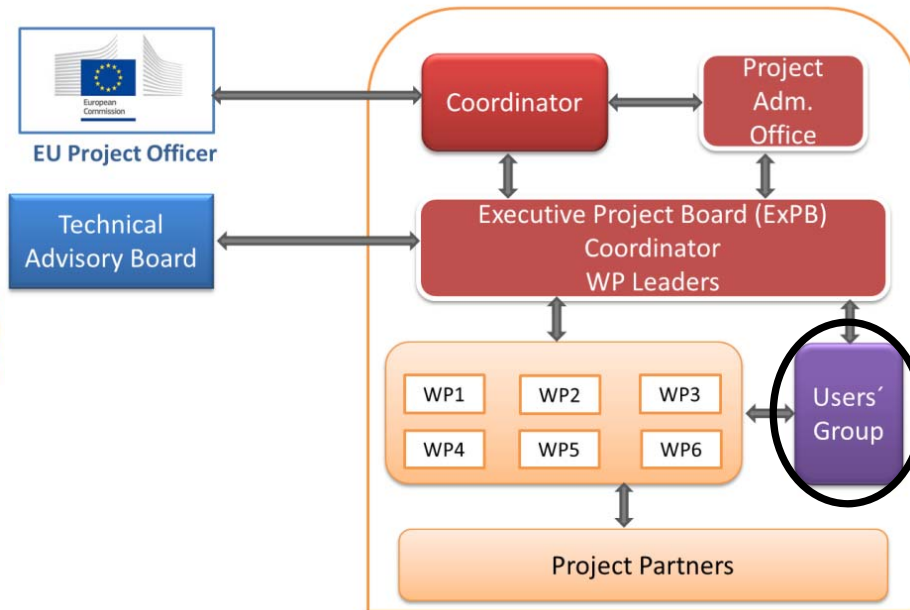


Delft Nuclear Consultancy



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McSAFE: Management Structure and Partners



A Users Group will be created

Confirmed institutions:

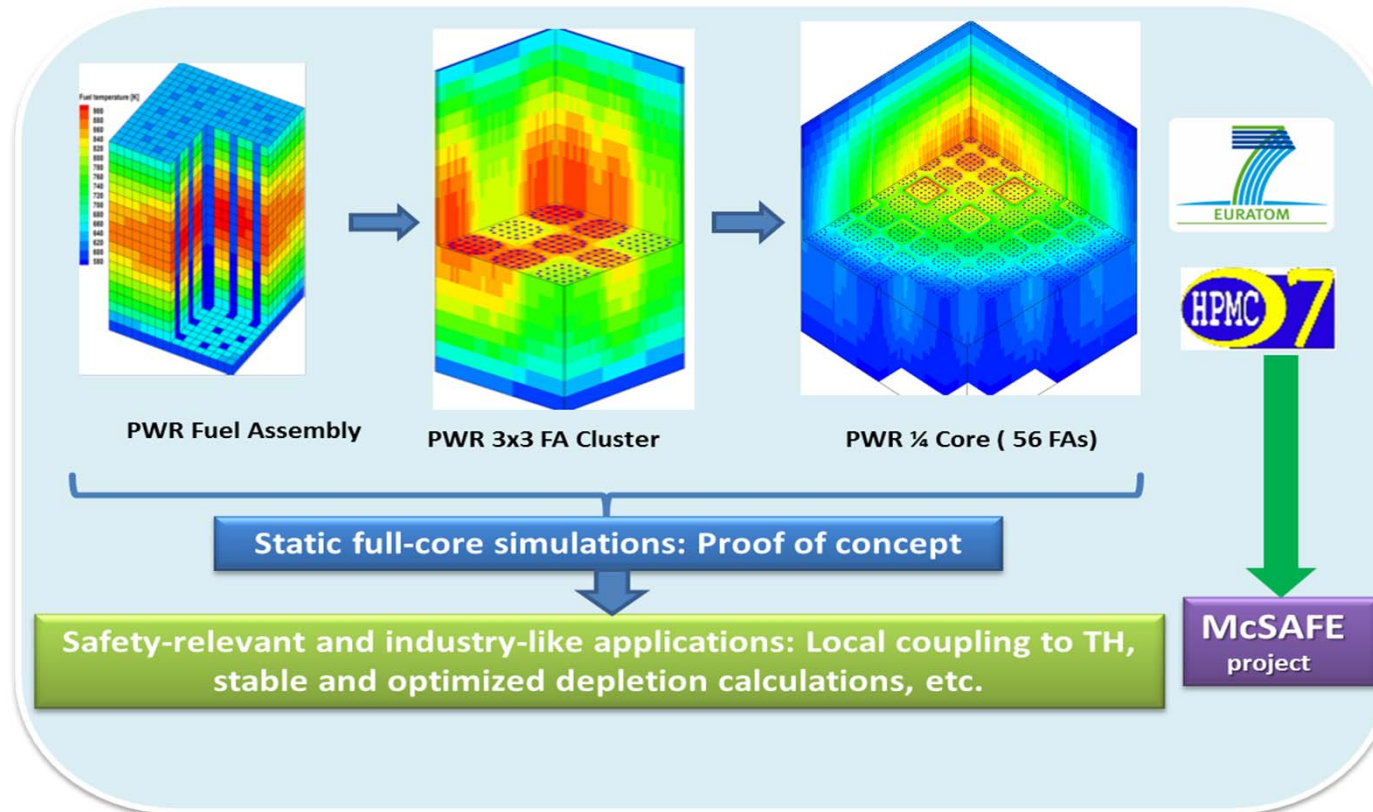
- North Carolina State University (USA)
- National Institute for Nuclear Research (Mexico)
- University of Michigan, (USA)
- University of Illinois, (USA)
- Argonne National Laboratory (USA)
- Canadian Nuclear Laboratories, (Canada)
- Idaho National Laboratory (USA)
- Westinghouse Electric (Sweden)
- SCK-CEN (Belgium)
- IRSN (France)
- POLIMI (Italy)

R E S E A R C H

I N D U S T R Y

McSAFE: Main Concept (1)

□ MCNP/SUBCHANFLOW Pin-wise Core Simulations



PWR Core Problem

Size: $193 \times 265 \times 30 = 1\,534\,350$ cells

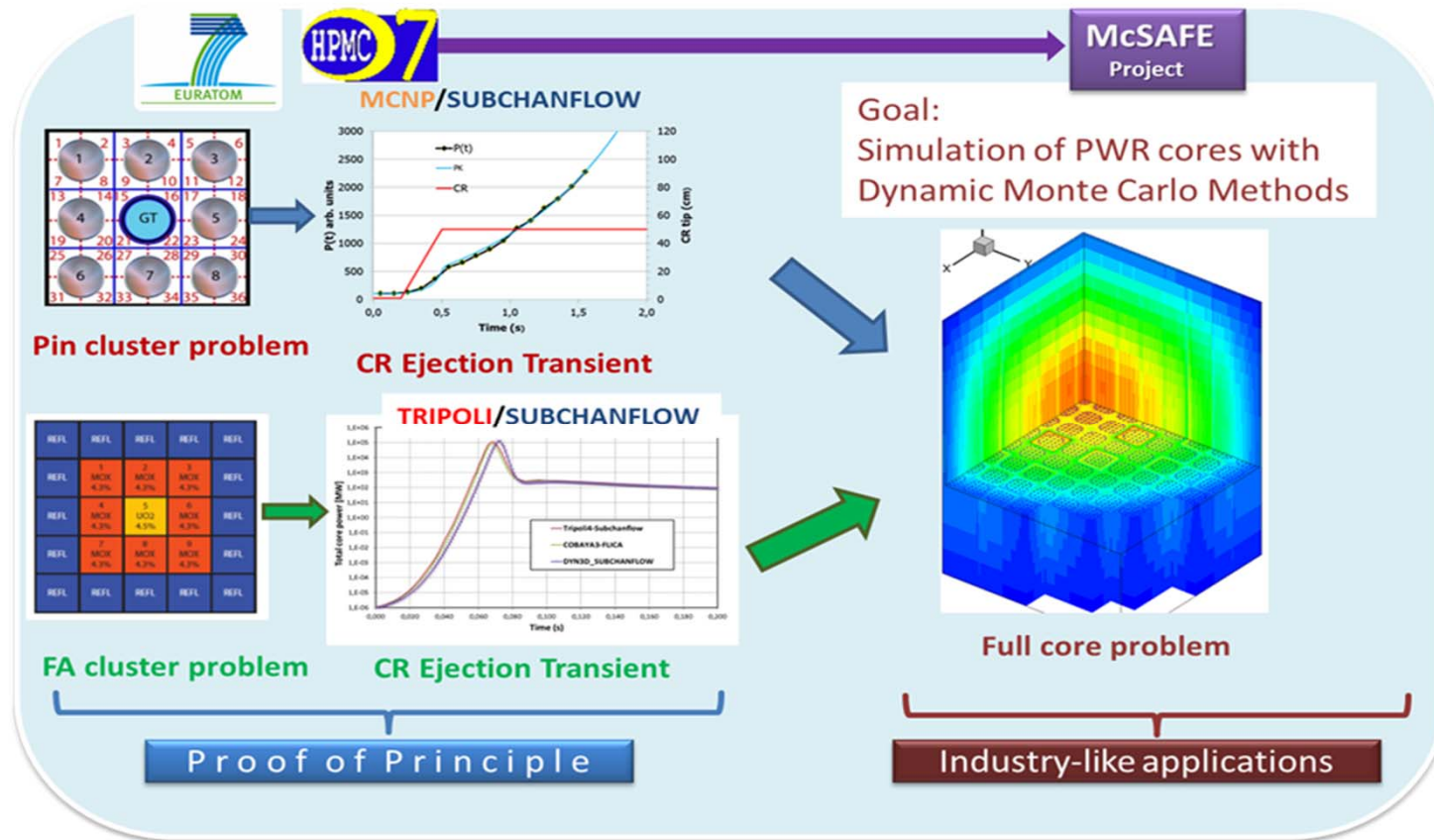
MC/TH simulation: $1.3 \text{ E}9$ histories, **240 cores**, Data Storage: 2.5 GB

Input preparation: < 5 min

Convergence: 20 N/TH iterations, 5600 min (**3.88 days**)

McSAFE: Main Concept (2)

Transient analysis with MC/Sub-channel Codes



DynMCNP: 3x3 Rod problem

DynMCNP Simulation

1E06 particles

DynMCNP run

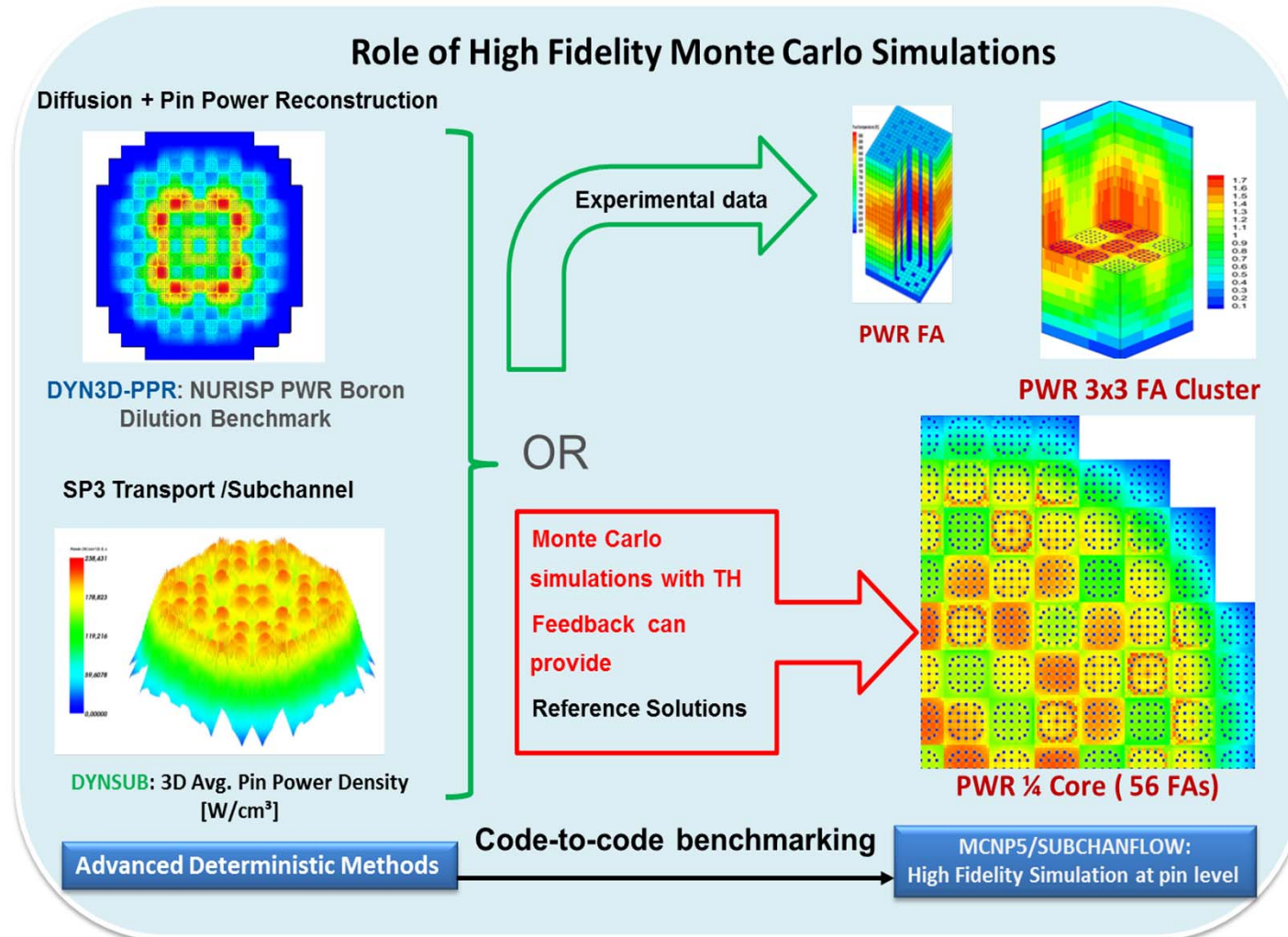
- 35 cores (MPI/openMP)
- 2.5 days

TRIPOLI/SCF : 3x3 Rod problem

Small cluster (28 cores): one week

McSAFE: Validation

- High-fidelity coupled MC-TH simulations as reference solutions



McSAFE: Numerical Tools and Coupling approaches



- Focus on the **further development of European MC, TH and TM codes and on the coupled systems** already developed during the 7FP EU project HPMC to make possible **industry-like simulations of both fuel depletion with stable and accurate depletion methods and LWR transients** (safety-related) with the developed dynamic MC solutions

Methods	Monte Carlo (MC)	Thermal Hydraulic (TH)	Thermo Mechanics (TM)
<i>Static MC-TH</i>	TRIPOLI SERPENT MCNP MONK	SCF, FLICA, TRUST SCF, COSI SCF SCF	
<i>Static MC-TH with depletion</i>	TRIPOLI SERPENT MONK	SCF, FLICA, TRUST SCF, COSI SCF	TU TU, FINIX TU
<i>Dynamic MC-TH</i>	DynTRIPOLI Dyn SERPENT DynMCNP	SCF SCF, COSI SCF	
<i>Dynamic MC-TH-TM</i>	DynTRIPOLI DynSERPENT	SCF, FLICA, TRUST SCF, COSI	TU TU, FINIX

McSAFE: WP5 – Validation Matrix



- **Newly available plant data** in the frame of international benchmarks or in the single initiatives by utilities (e.g. PreussenElektra or CEZ) pave the way **for the validation of the improved capability of multi-physics tools based on Monte Carlo codes** regarding e.g. local safety parameters, depletion, and fuel behaviour

Plant Data	Static MC-TH Problem	Static MC-TH-TM Depletion Problem	Dynamic Problem
<i>VVER-1000</i>	TRIPOLI SERPENT/SCF MCNP/SCF	SERPENT/SCF/TU	
<i>PWR Konvoi</i>	TRIPOLI/SCF SERPENT/SCF MCNP/SCF	SERPENT/SCF/TU	
<i>SPERT III E REA</i>			DynTRIPOLI/SCF DynSERPENT/SCF DynMCNP/SCF

McSAFE: Expected Contribution



- ❑ The project will deliver **improved** and **validated high-fidelity numerical simulation tools** that can be used by different end-users (industry, regulators, research centres, etc.) to provide **reference solutions to deterministic codes for safety demonstration**.
- ❑ The McSAFE **tools** are **essential to design reactor systems with improved safety features keeping sufficient safety margins**. The project will **reinforce the European leadership's nuclear engineering methods** to better assess the safety of NPP and make NPP operation more flexible while keeping high safety standards
- ❑ The **McSAFE calculation schemes** are **applicable to any reactor type**, current or future, **provided the thermal-hydraulics codes are capable of dealing with the specific reactor type**. For future nuclear reactors verification of design calculation methods is even more important as reference data are not available



Thank you for your attention!

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