



The Technology of Fusion Energy (TOFE 2018)  
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# Advancements in the Helium Cooled Pebble Bed Breeding Blanket for the EU DEMO: Holistic Design Approach and Lessons Learned

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Ivo Moscato (*Università di Palermo*)



Breeding Blanket Project



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- 1 Introduction: Holistic design approach
- 2 Lessons learned
- 3 Enhanced HCPB with “fuel-breeder” pins
- 4 Performances
- 5 PHTS integration
- 6 Summary and Outlook

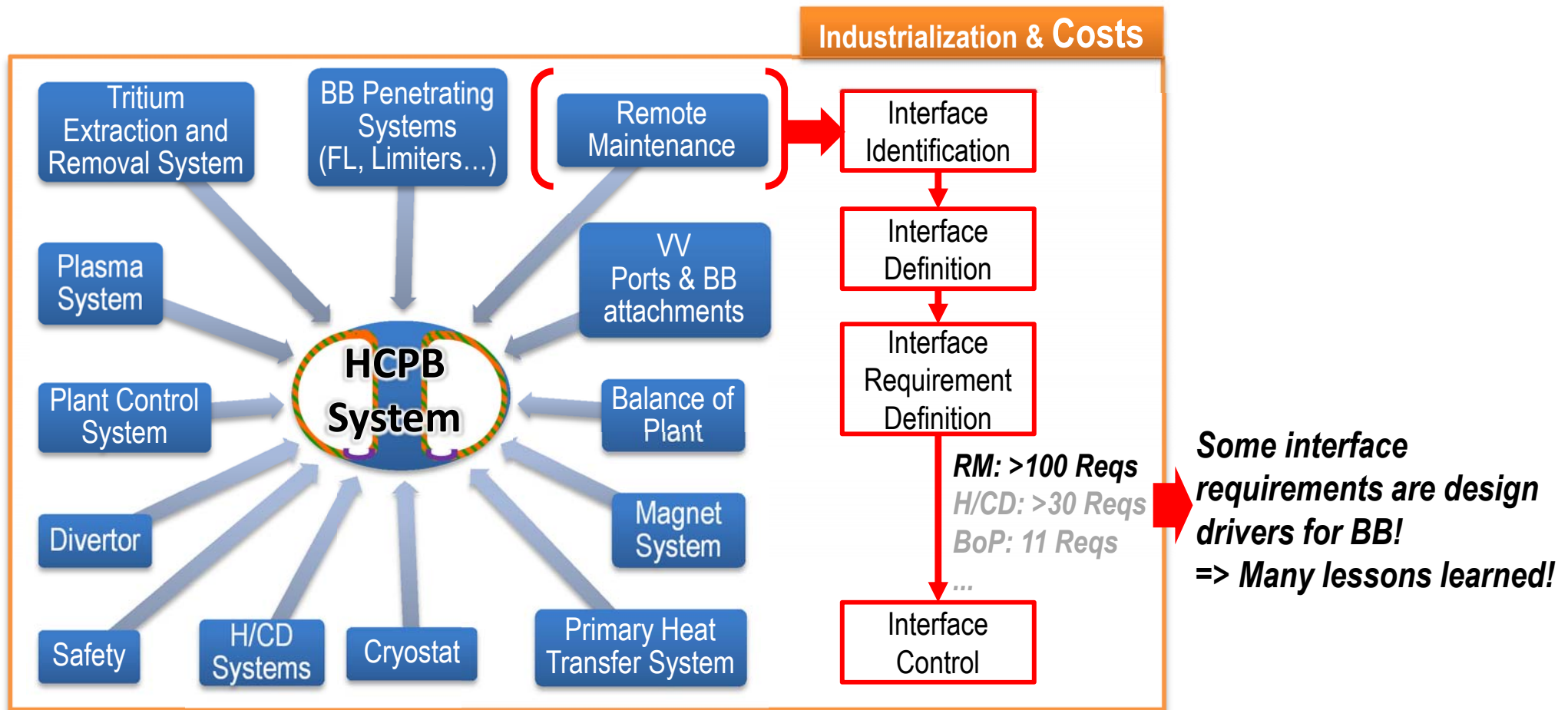


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# 1. Introduction: Holistic design approach

- Current EU DEMO pre-Conceptual Design: focus on Systems Engineering
  - DEMO: >40 systems identified @level 1 PBS => huge number of interfaces
  - Case of BB System:





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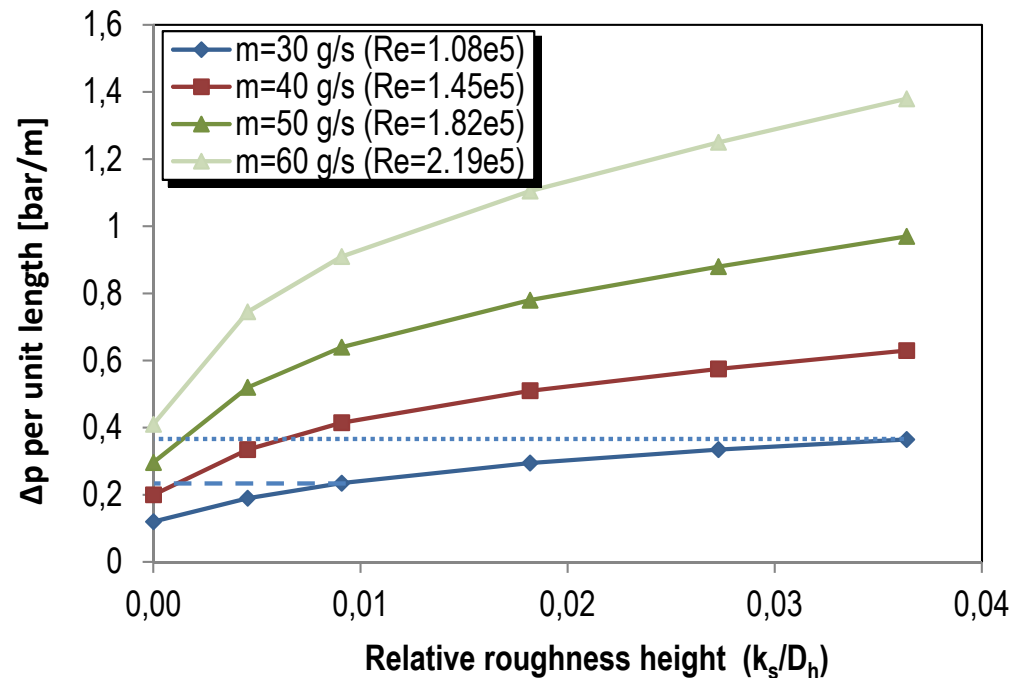
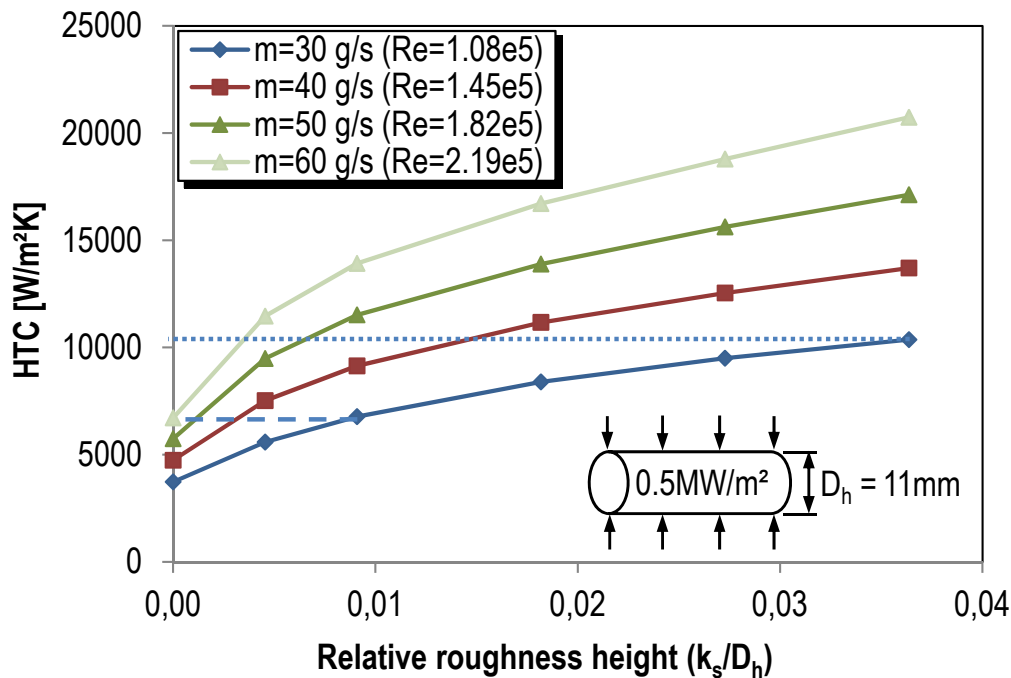
# 2. Lessons learned

## INT On plant integration

- EU DEMO BoP requirement: maximization of resulting BoP System TRL
- Redundant cooling scheme in BB too complex for BoP
- Systems penetrating BB is unavoidable: modularization of BZ
- DEMO/BB large: minimize piping and weight of segments to ease RM (plant availability)

## TH On efficient thermo-hydraulics

- He circulating power quickly escalates with  $\Delta p$ : minimization of  $\Delta p$  at each level
  - DEMO/BB large: maximize „core“  $\Delta T$  to reduce plant circulating mass flow
  - If HTC need: max. turbulence (friction), min. flow speed / If no HTC need: min. both
- Common problems in GCR program!  
What did we learn?*





## 2. Lessons learned

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### SIMP On design simplification, industrialization and costs

- DEMO/BB large => need to simplify manufacturing for mass production and good RAMI
- DEMO/BB large => mass production and costs of functional materials, especially Be NMM

### SAFE On safety

- DEMO/BB large and „core“  $\Delta T$  not large: He inventory quickly escalates => impact on VVPSS
- Be: 40% T retention @ 600°C => few kg of T inventory after 20dpa
- Be: reactivity with steam and air and high swelling

Reducing size of DEMO (e.g. using HTS or less ambitious  $P_{fus}$ ) would mitigate many key problems  
If „HTS“ path: more challenging T breeding (and power exhaust), but HCPB may offer enough margin



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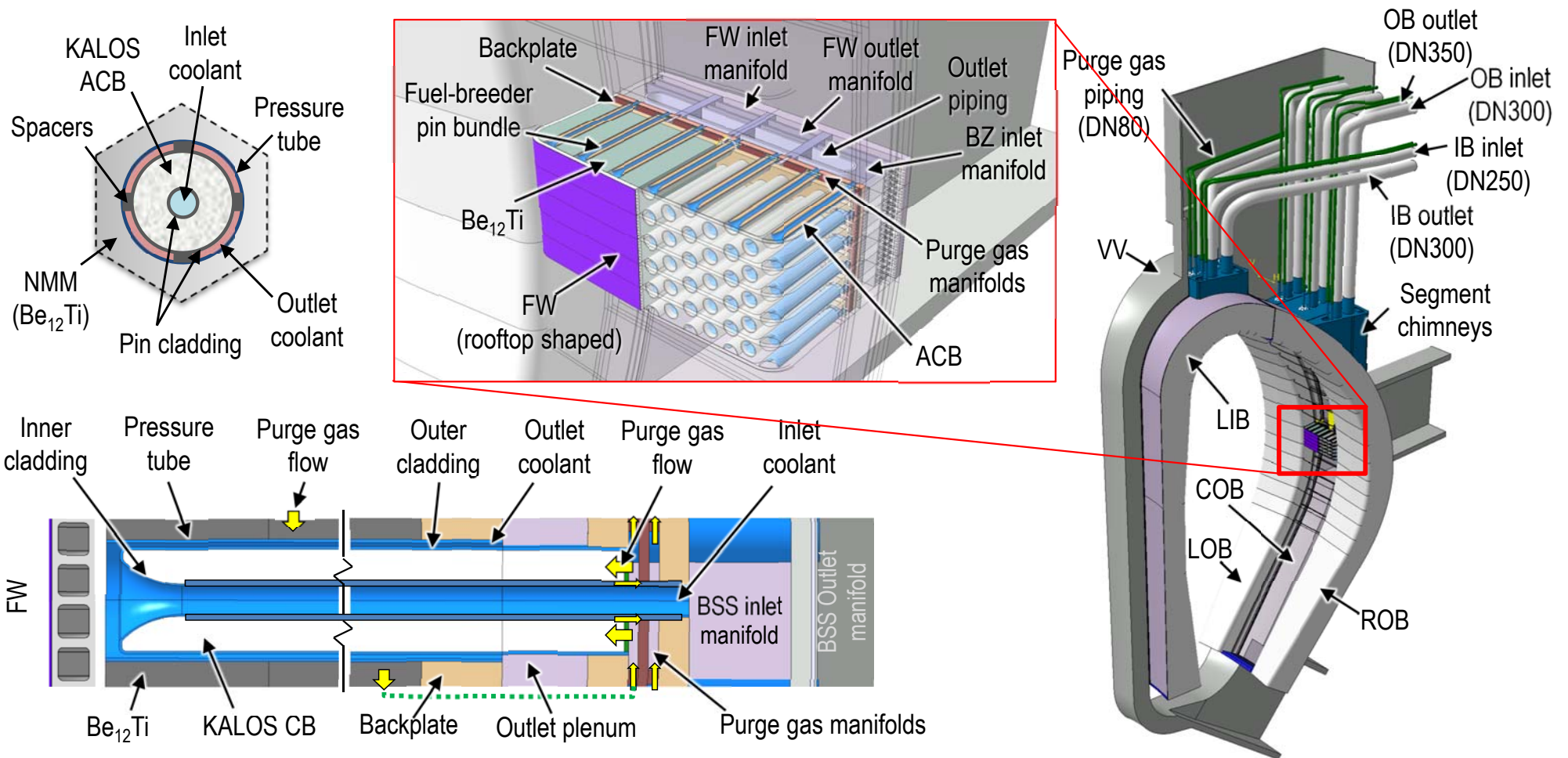


# 3. Enhanced HCPB with „fuel-breeder“ pins



## ■ Design features:

- Coolant redundancy eliminated: BZ flexibility! INT
- Fission-like fuel-breeder pins: simple TH & manufacturing, larger area, low  $v \Rightarrow$  low  $\Delta p$  TH SIMP INT
- Rooftop shaped FW and Single Module Segment architecture INT
- Structural steel: EUROFER97
  - BZ: KALOS ( $\text{Li}_4\text{SiO}_4 + \text{Li}_2\text{TiO}_3$ ) + beryllides



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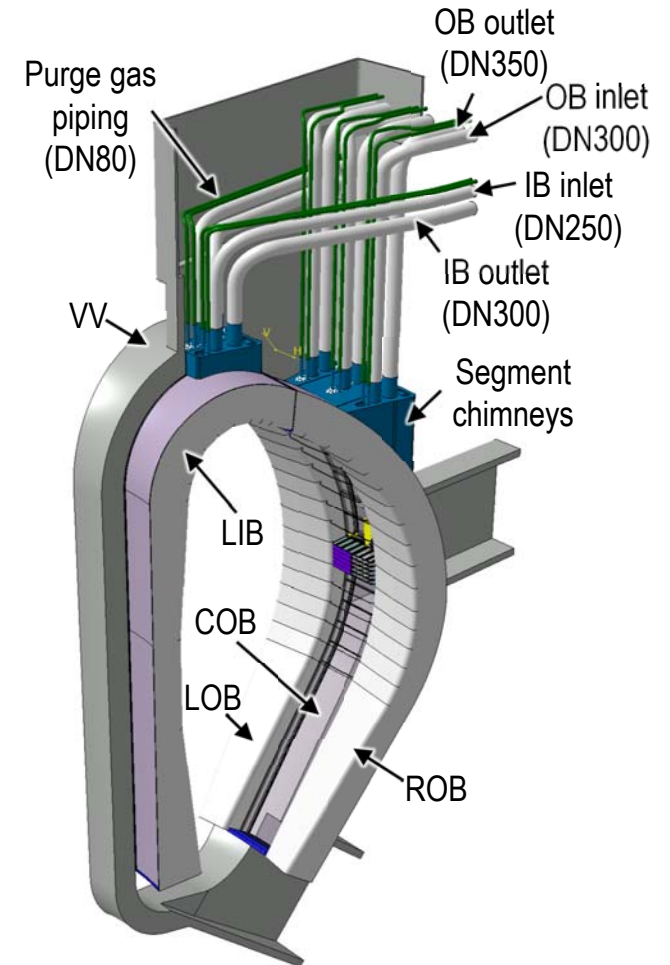
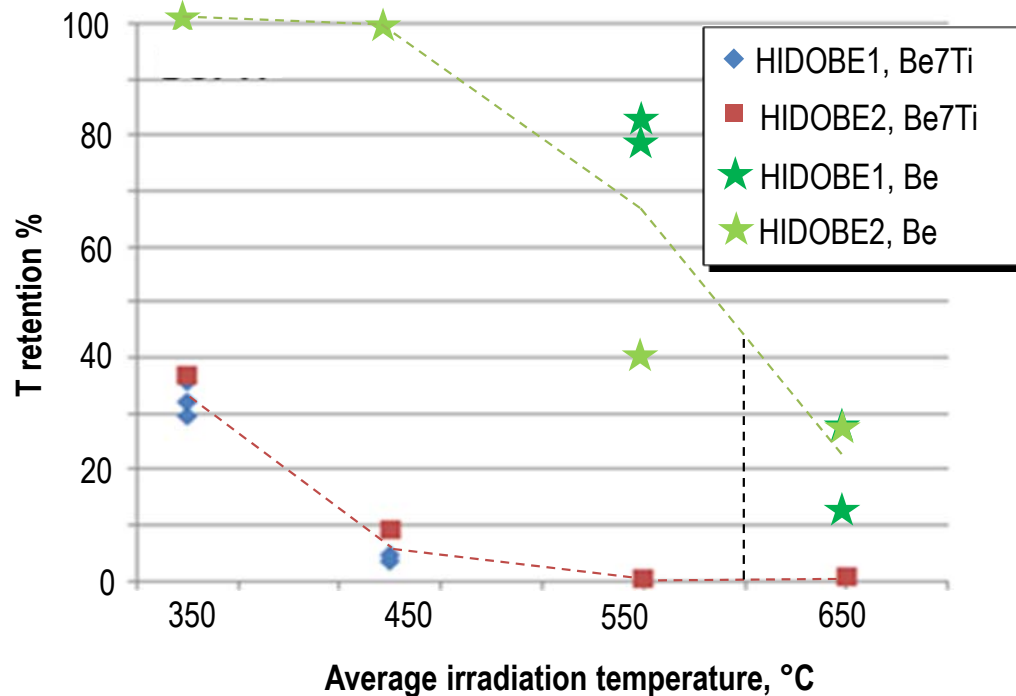


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## ■ Rationale for switching to beryllides

- T retention  $\approx 0\%$  @600°C ( $\approx 40\%$  for Be) SAFE



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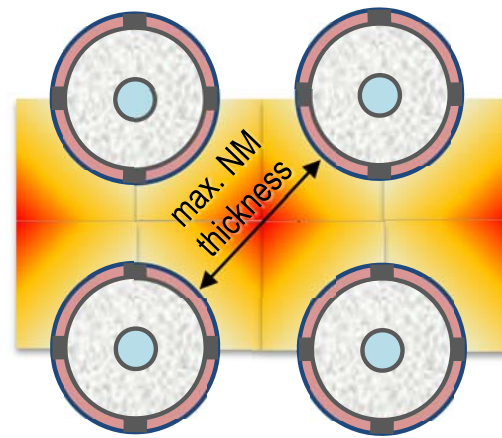
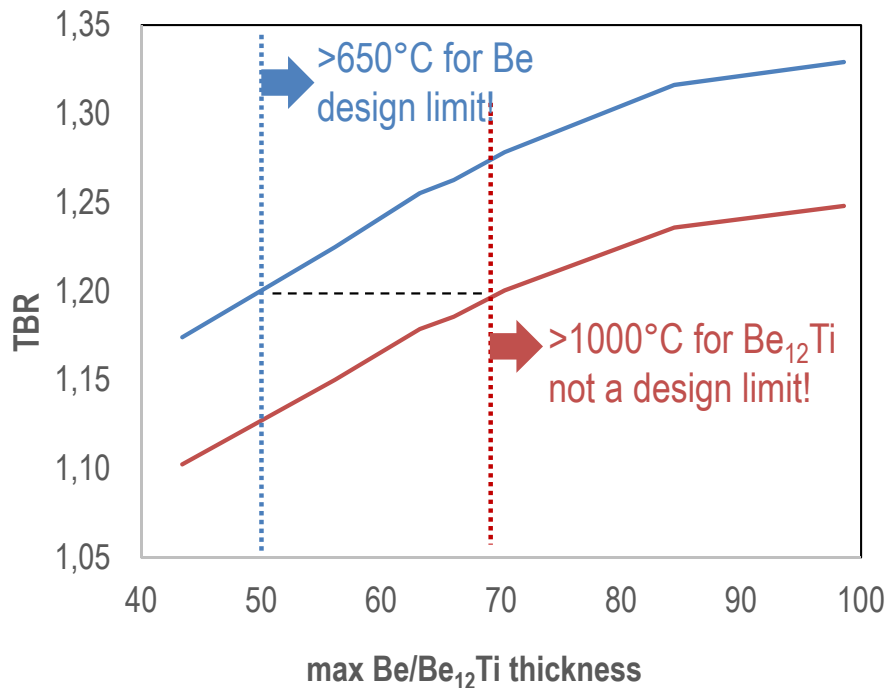


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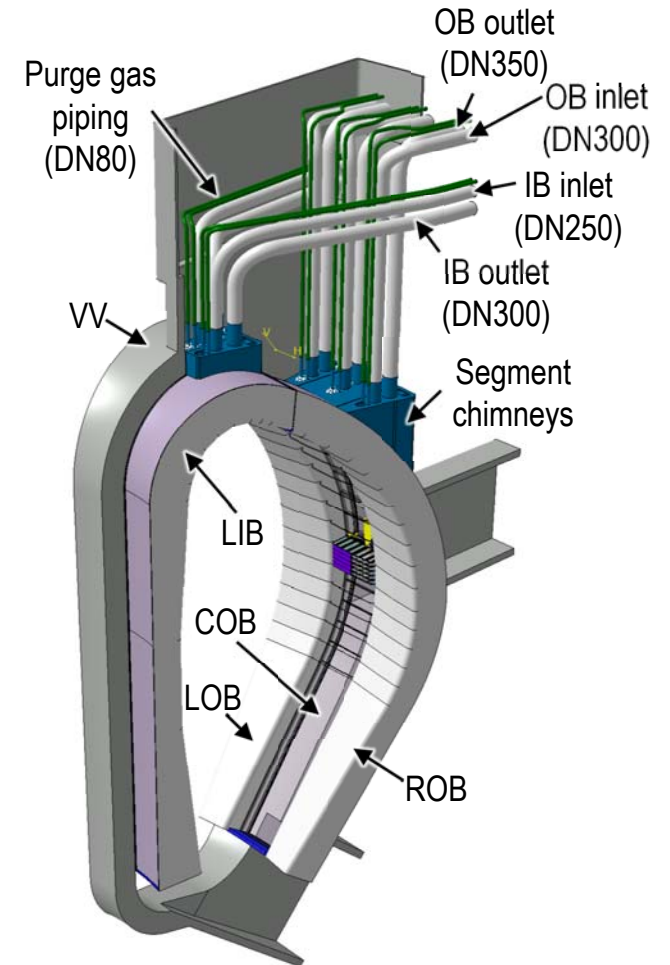
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- Higher temp. limit  $\Rightarrow$  no clear TBR advantage of Be over  $\text{Be}_{12}\text{Ti}$



Results obtained with a former, squared pin arrangement



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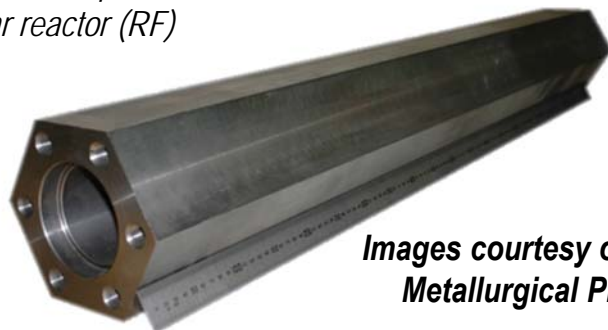
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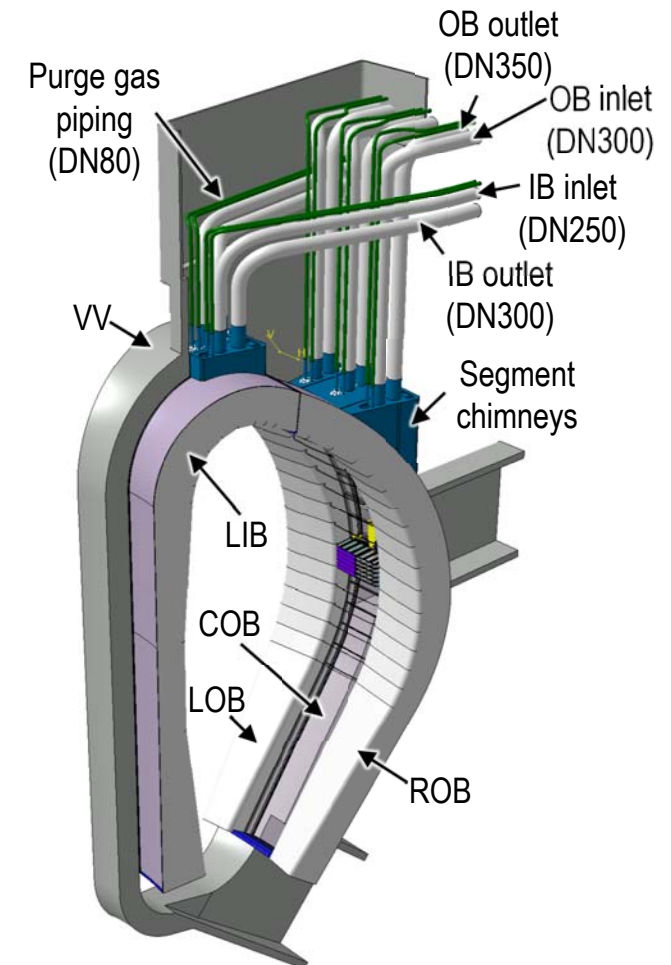
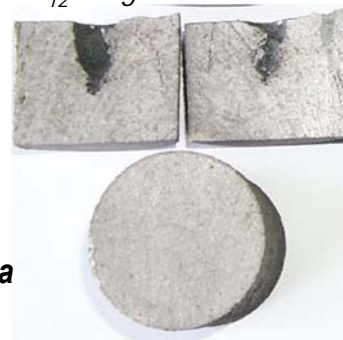
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- Higher temp. limit  $\Rightarrow$  no clear TBR advantage of Be over  $\text{Be}_{12}\text{Ti}$
- Better T release and lower swelling  $\Rightarrow$  no need for pebbles!  $\Rightarrow$  use fission-like  $\text{Be}_{12}\text{Ti}$  as prismatic blocks SIMP

Moderator Be prismatic block for MIR nuclear reactor (RF)



Images courtesy of Ulba Metallurgical Plant

$\text{Be}_{12}\text{Ti}$  ingots



- R&D 2019-2020: quick demonstration industrial production of  $\text{Be}_{12}\text{Ti}$  prismatic blocks and consolidate material properties

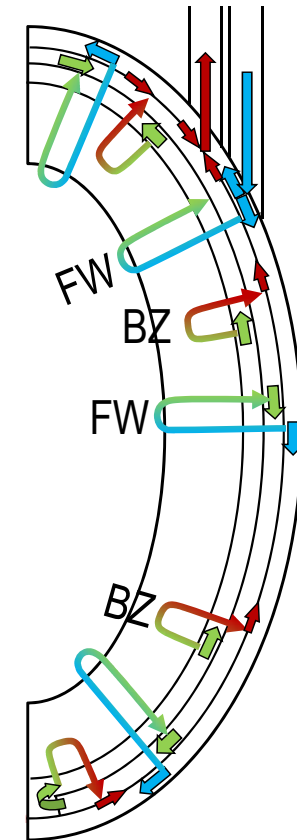
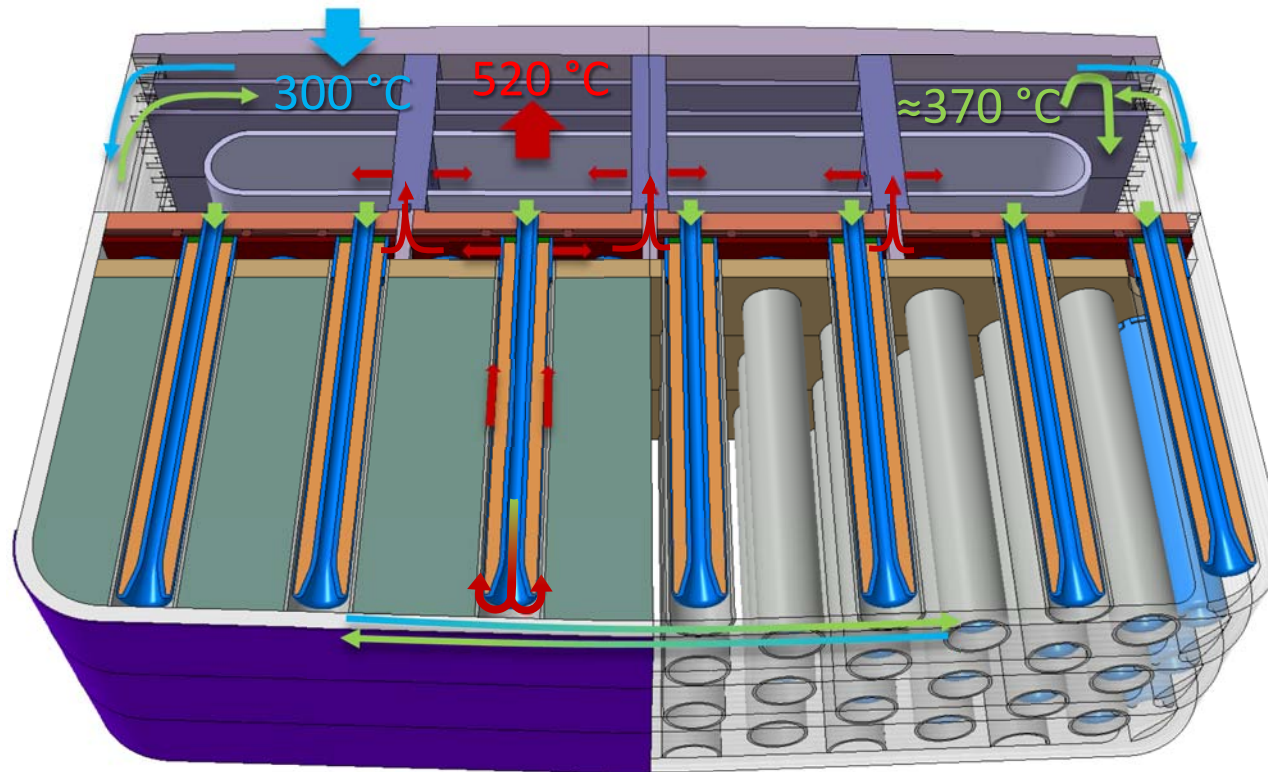


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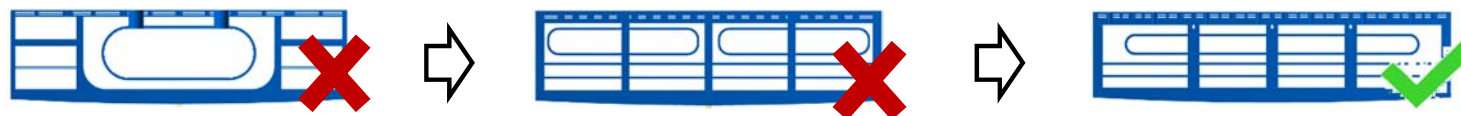


## HCPB internals: thermohydraulic scheme

- FW and BZ in series
- Better temperature control in BZ



- Manifold design: result of a design iteration



- Coolant: He, 8 MPa,  $T_{in} = 300^{\circ}\text{C}$ ,  $T_{out} = 520^{\circ}\text{C} \Rightarrow +20^{\circ}\text{C}$  (due to better thermal management of BZ with pins)  $\Rightarrow$  -10% plant mass flow (w.r.t. former designs)  $\Rightarrow$  **key advantage for PHTS and BoP**



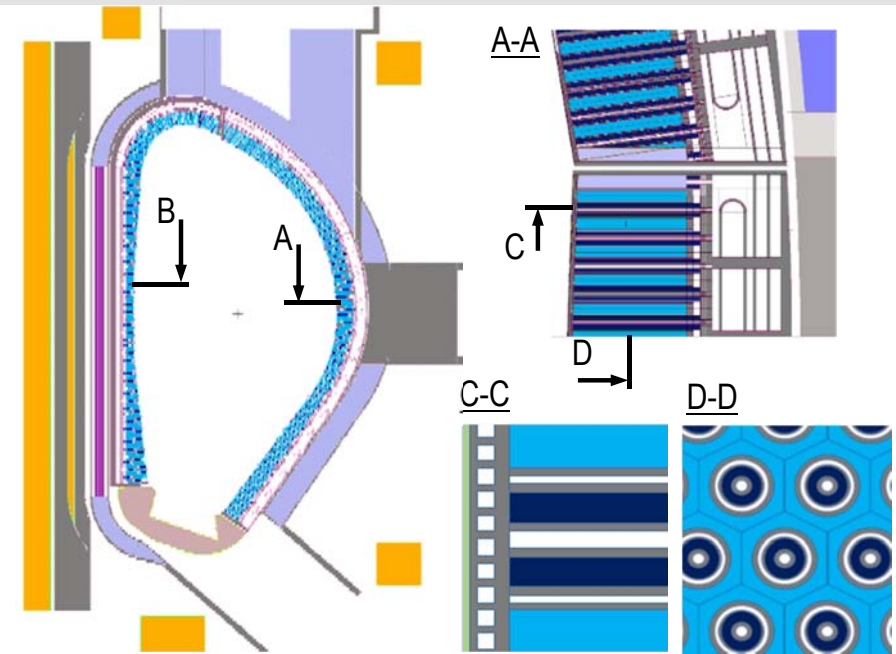
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# 4. Performances: Neutronics



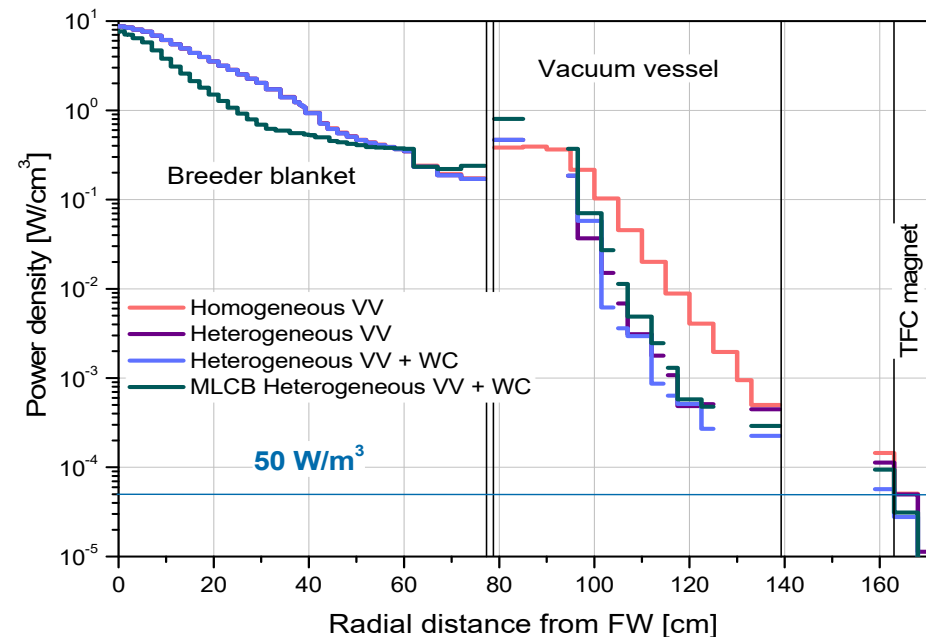
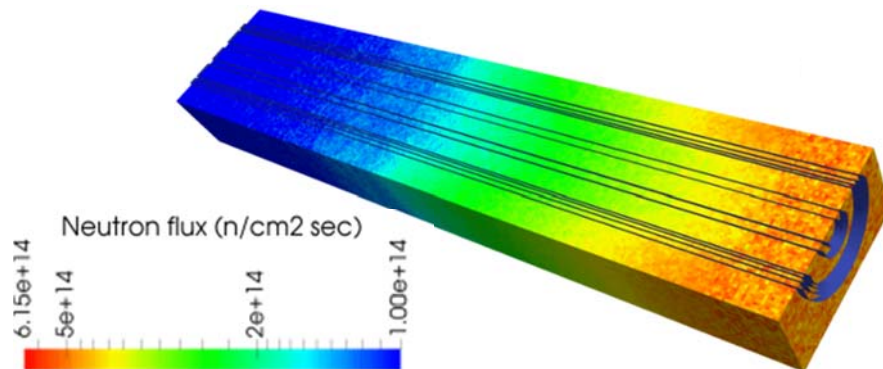
## ■ Tritium breeding performance

- Fully heterogeneous MCNP model (key for reliability)
- Be<sub>12</sub>Ti pebble bed (<sup>6</sup>Li 60%): **TBR ≈ 1.16**
- Be<sub>12</sub>Ti prismatic blocks (<sup>6</sup>Li 60%): **TBR ≈ 1.20**
- **High TBR in very compact configuration: OB = 1m!**
  - Allows very compact BB for small tokamak configurations
  - Allows large coverage reduction for e.g. DN, penetrations...



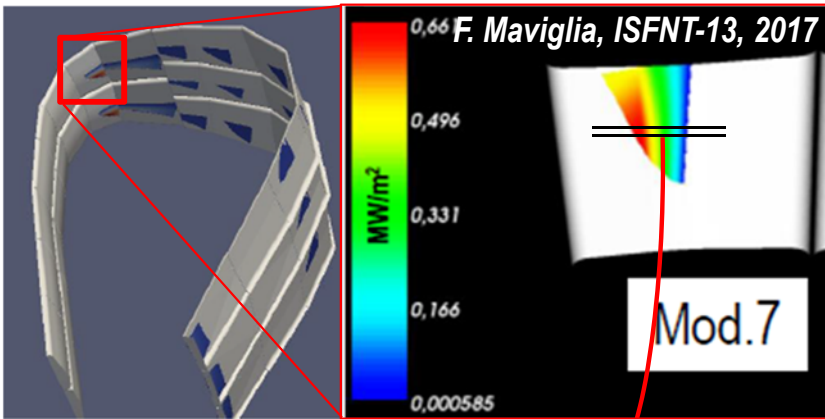
## ■ Shielding

- Streaming in BZ ok despite radial channels
- Limit 50 W/m<sup>3</sup> in TFC ok, yet low margin
- WC inserts in VV can reduce PD ≈50%
- Future focus on shielding improvement keeping compact configuration





# 4. Performances: FW thermo-hydraulics



FW DEMO HHF knowledge vastly improved

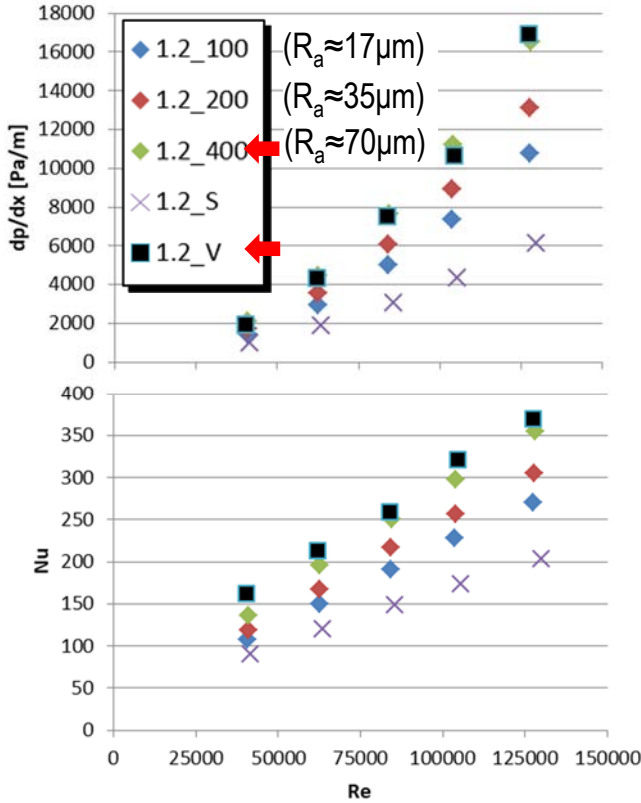
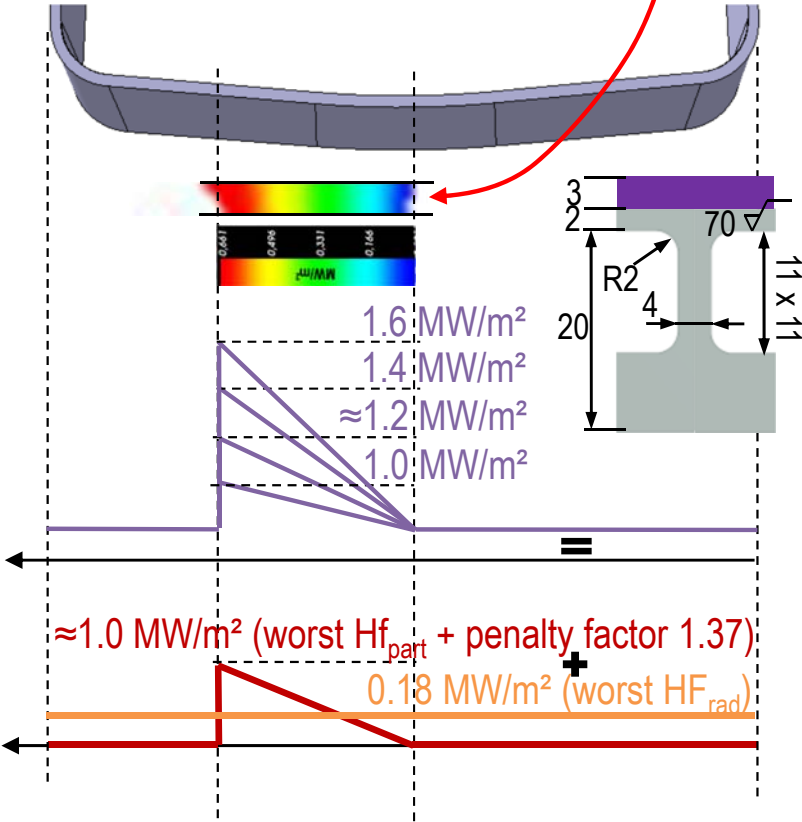
- $HF_{tot} = HF_{rad} + HF_{part}$ , non-homogeneous HF loads

Channels with V-ribs: best HTC vs dp/dx



Resource-intensive CFD procedures for full-scale FW and BB CFD analyses of V-ribs (LES):

- V-ribs vs. augmented surface roughness



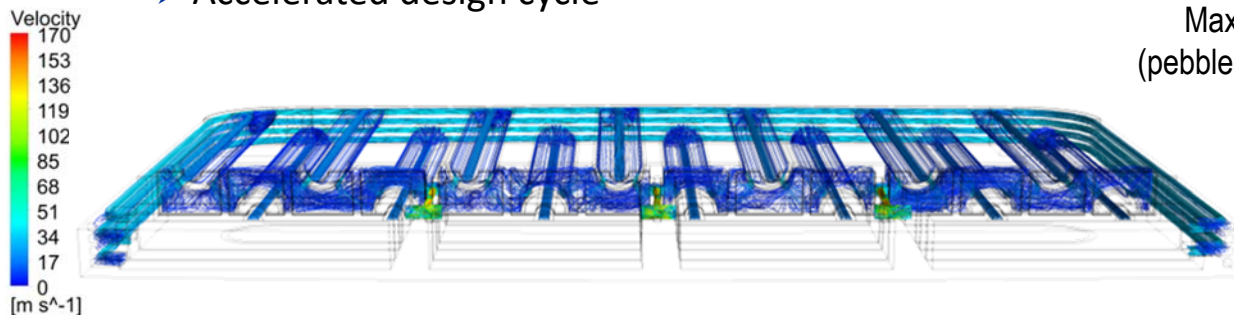
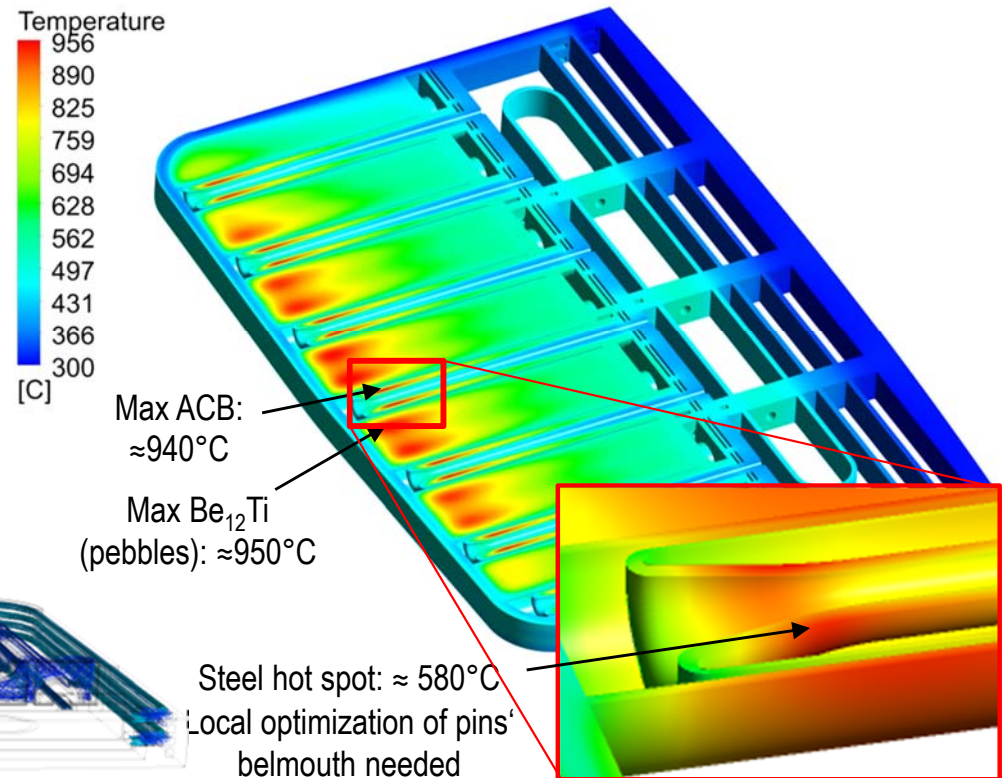
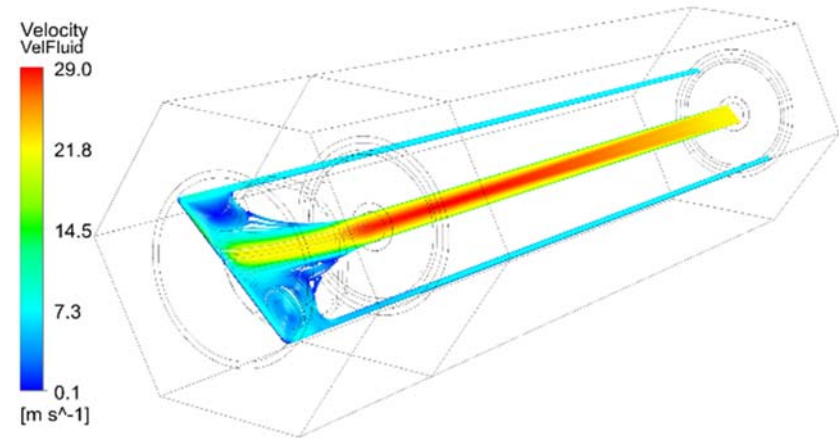
	m [kg/s]	T <sub>peak</sub> [°C]	Δp [bar]	
Peak q" 1 MW/m²	0.03	553	0.37	OB
	0.04	509	0.64	
	0.05	482	0.98	
	0.06	464	1.39	
Peak q" 1.2 MW/m²	0.03	593	0.38	IB
	0.04	542	0.64	
	0.05	511	0.98	
	0.06	491	1.39	
Peak q" 1.4 MW/m²	0.03	634	0.38	
	0.04	576	0.64	
	0.05	541	0.98	
	0.06	518	1.39	
Peak q" 1.6 MW/m²	0.03	675	0.38	
	0.04	610	0.65	
	0.05	571	0.98	
	0.06	545	1.39	





# 4. Performances: BZ thermo-hydraulics

- Fuel-breeder pins design rationale:
  - Maximize size: reduce number pins
  - Large area  $A$  (low speed) + rough walls ( $\epsilon_s/D_h < 0.05$ )
- BZ temperatures and colant  $\Delta p$ :
  - $\Delta p_{\text{fuel-pin}}$  (i.e. BZ)  $< 0.1$  bar ( $\Delta p_{\text{CP, former designs}} \approx 1$  bar)
  - Unit slice CFD: temperature globally under limits
  - $T_{\text{out}}$  increased to  $520^\circ\text{C}$
- BBS colant pressure drops:
  - to be updated and optimized
    - $\Delta p_{\text{BBS,IB}} \approx 0.91$  bar
    - $\Delta p_{\text{BBS,OB}} \approx 0.66$  bar
  - Future approach: design optimization with TH system codes (RELAP5)
    - First benchmarks CFD – RELAP5
    - Accelerated design cycle

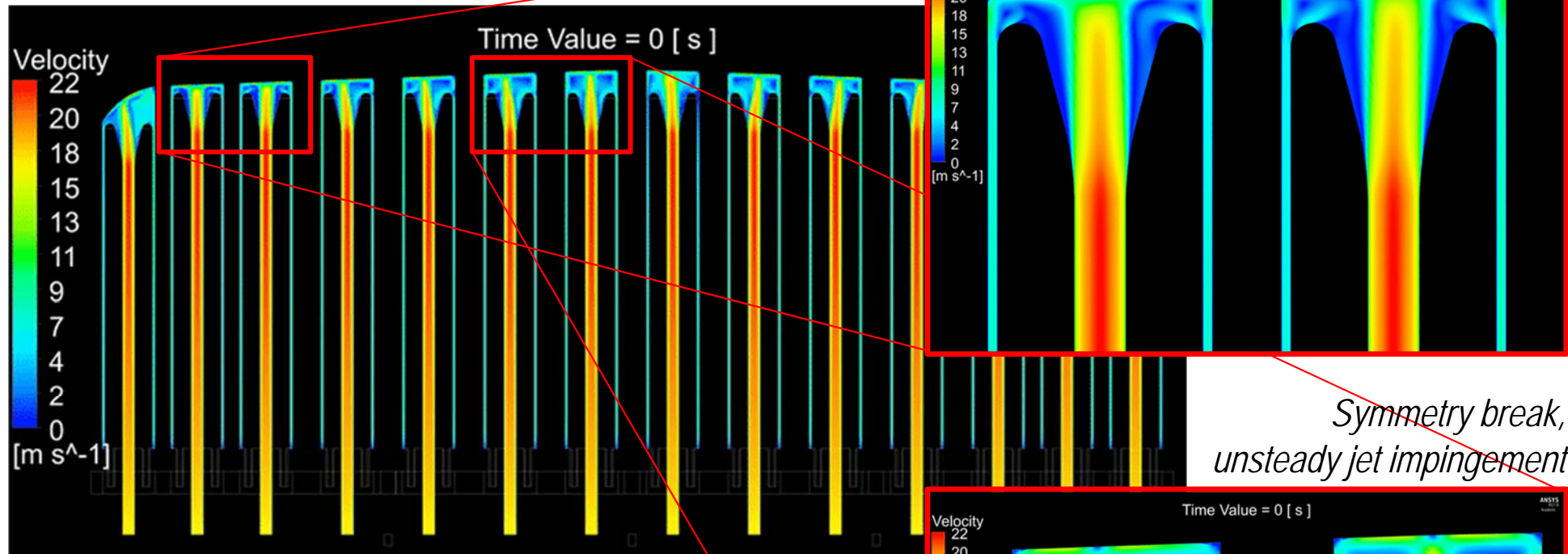




# 4. Performances: BZ thermo-hydraulics

- Detailed transient analysis of a unit slice:

*Symmetry,  $\approx$  steady jet impingement*



*Symmetry break, unsteady jet impingement*

- Preparation of 2 experimental campaigns:

- 1. Understand design space for onset of symmetry break of jet impingement region
- 2. Validation of heat transfer correlations for transitional and fully rough regimes

G. Zhou, TOFE 2018



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# 6. Primary Heat Transfer System integration



- HCPB BoP = PHTS (He) + IHTS (MS) + PCS
- Goal BoP: maximize TRL for PHTS
  - PHTS TRL in HCPB mainly limited by He circulator technology currently proven for <6MW/unit

<b>Former PHTS BL2015</b>		
Former HCPB design		
$P_{BB,th} \approx 2100 \text{ MW}$		
$T_{in}/T_{out} \text{ He } [^{\circ}\text{C}]$	500/292.5	
$\Delta p \text{ [bar]}$		
	IB	OB
In-VV	2.14	1.74
Piping	0.62	0.57
IHX		
<i>S&amp;T U-tube</i>	0.88	0.85
$\Delta T_{log} = 28^{\circ}\text{C}$		
Total	3.64	3.16
$P_{tot,el} \text{ [MW]}$		
<b>130.4</b>	$(\eta_{el}=0.90)$	



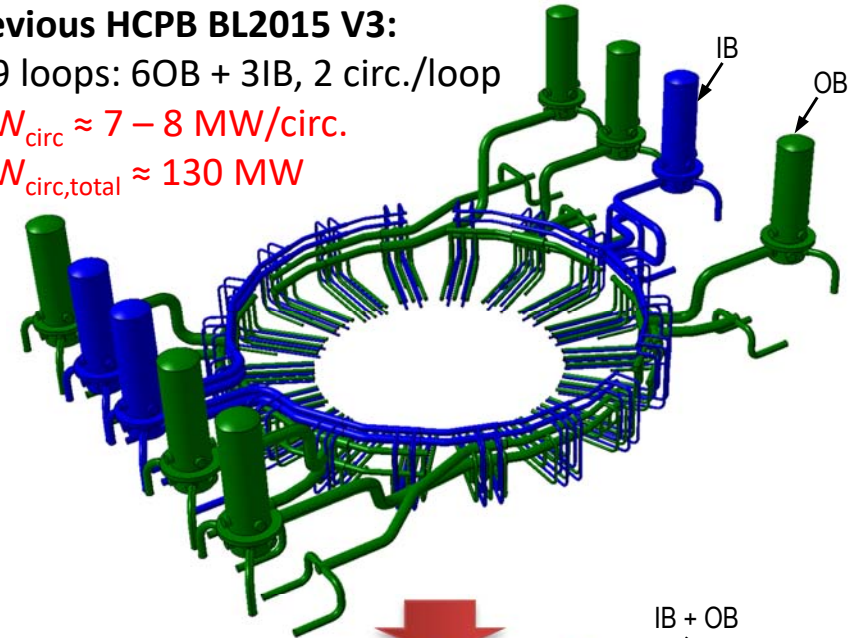
<b>Proposed PHTS BL2017</b>		
HCPB pin design		
$P_{BB,th} \approx 2100 \text{ MW}$		
$T_{in}/T_{out} \text{ He } [^{\circ}\text{C}]$	520/292.3	
$\Delta p \text{ [bar]}$		
	IB	OB
In-VV	1.56	1.07
Piping	0.45	0.94
IHX		
<i>CWHE</i>	0.34	
$\Delta T_{log} = 36^{\circ}\text{C}$		
Total	2.35	
$P_{tot,el} \text{ [MW]}$		
<b>83.6</b>	$(\eta_{el}=0.90)$	

Source: I. Moscato (Uni. Palermo)

- Target: 60 – 70 MW
  - Key component to optimize now: manifold

## Previous HCPB BL2015 V3:

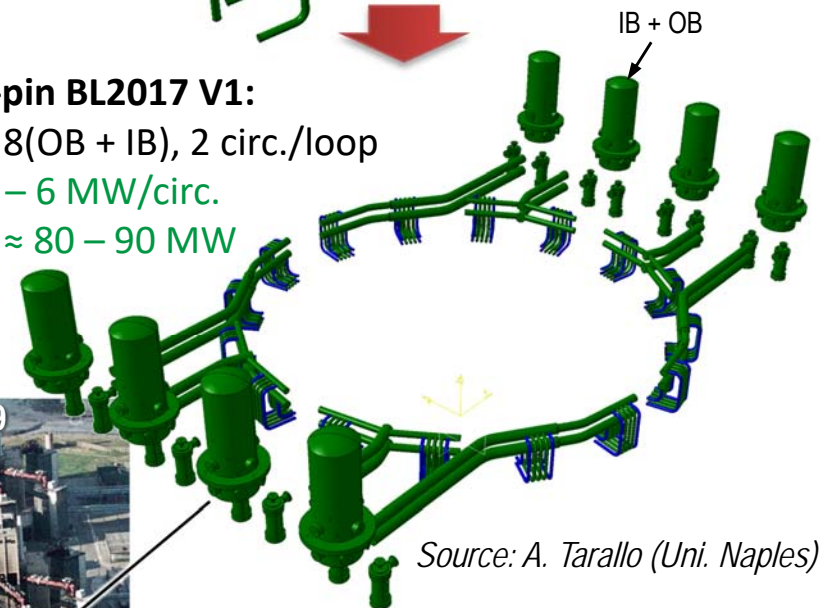
- 9 loops: 6OB + 3IB, 2 circ./loop
- $W_{circ} \approx 7 - 8 \text{ MW/circ.}$
- $W_{circ,total} \approx 130 \text{ MW}$



## HCPB fuel-pin BL2017 V1:

- 8 loops: 8(OB + IB), 2 circ./loop
- $W_{circ} \approx 5 - 6 \text{ MW/circ.}$
- $W_{circ,total} \approx 80 - 90 \text{ MW}$

Berkley NPP, GCR (UK)



Source: A. Tarallo (Uni. Naples)



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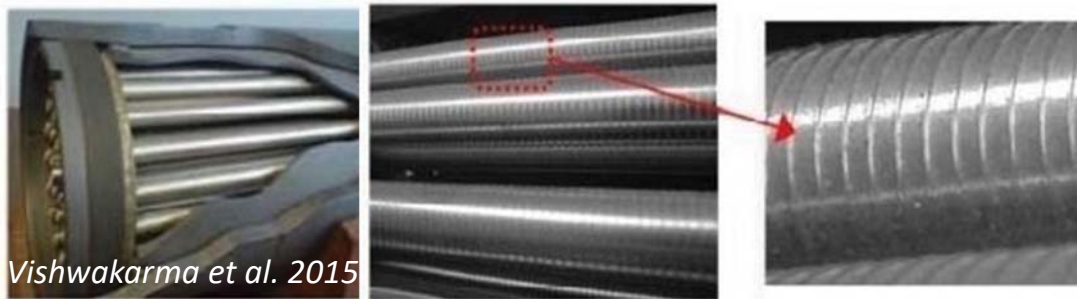




# 7. Summary and outlook

- EU DEMO development strategy: holistic (systems engineering) design
  - Many interfaces, requirements, some drive design => lessons learned => enhanced HCPB, fuel-pins
- Maximize/exploit commonalities of solid BB (fusion) with solid core (fission)
  - Lesson learned for heat transfer enhancement with low  $\Delta p$  with common approach to GCR program

*AGR fuel element*

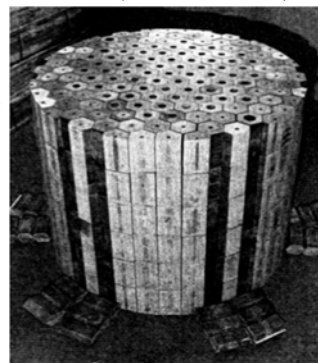


- Multiplier ( $\text{Be}_{12}\text{Ti}$ ) prismatic blocks: common configuration to other Be-moderated reactors

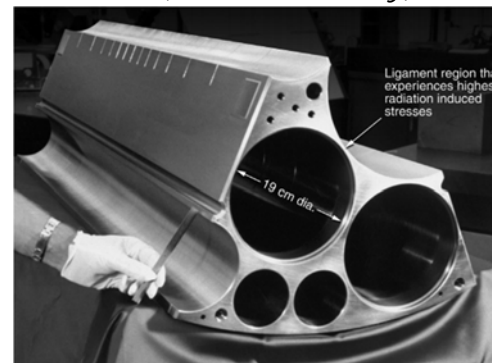
*MIR (RF, 1967-today)*



*ARE (US, 1940s)*



*ATR (US, 1967-today)*



*BR2 (BE, 1962-today)*





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- Maximize/exploit commonalities of solid BB (fusion) with solid core (fission)
  - Lesson learned for heat transfer enhancement with low  $\Delta p$  with common approach to GCR program
  - Multiplier/moderator ( $\text{Be}_{12}\text{Ti}$ ) prismatic blocks : inspiration from Be-moderated reactors
- Design research led to the HCPB fuel-breeder pin design
  - Milestone: record low reactor circulating power (80-90MW)! Aim at 60-70MW
  - AGR-like PHTS, state-of-the-art He-turbomachinery can be used => milestone of high BoP-TRL!
  - Simpler internals, manufacturing, functional materials => cost reduction and RAMI improvement
- Main R&D needs for near future
  - Near term: validation of fuel-breeder pins thermohydraulics with 2 tests in HELOKA
    - 1. Determination of design space range for onset on symmetry break of jet-impingement region
    - 2. Validation of heat transfer correlations for transitional and fully rough regime in FW and BZ (pins)
  - Mid term: multiple-effects experiment with fuel-pin bundle in HELOKA
  - Functional materials: proof of industrial scale and irradiation campaign



# Back-up slides

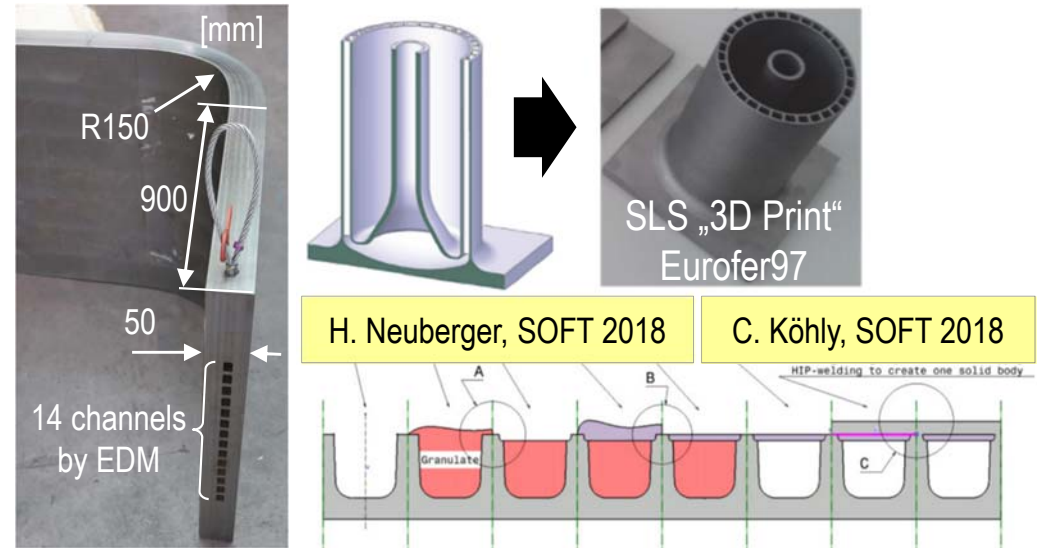




# 5. Manufacturing and costs

## ■ Manufacturing and costs:

- Fuel-pins: conventional fabrication
- FW former enabler technology: EDM + forming, but costs increase rapidly with length of EDM
- New approaches: „Metal Powder Application“ (MPA) or „fail-safe“ (Commin, 2013),
  - Less limitations, cost reduction  $\approx 50\%$  w.r.t EDM
- Alternative: SLS, but not in code (e.g. RCC-MRx)



## ■ RAMI:

- „Main Challenge of Fusion“ (D. Maisonnier, 2017); „Achilles Heel for Fusion“ (M. Abdou):
  - Imperative to include RAMI relevant aspects into design from beginning
- Initial scoping RAMI studies:
  - Design seems more robust against degraded operation due to higher modularization
  - General improvement on failure modes related to welds scaling with length
  - Large improvement on failure mode related to channels (clogging)

	(1) Reference HCPB	(2) Enhanced HCPB	Type of weld (1) vs (2)	Ratio (2)/(1)	Failure mode	Predicted Yearly Fail Rate Ratio (2)/(1)
Cooling channels/ small pipes	1461 km	300 km	-	<b>-79.4%</b>	Clogging	<b>-70%</b>
Welds as seals for in- BB leak	167 km	94 km	rectang. vs. orbital	<b>-43.6%</b>	In-BB coolant leak	<b>-51%*</b> / <b>+159%**</b>
Welds as seals for in- VV leak	23 km	10 km	linear vs. linear	<b>-54.2%</b>	In-VV coolant leak	<b>-57%</b>

\*Estimation considering number AND unit length of welds

\*\*Conservative estimation considering ONLY no. of welds

HIP welds not included / Reliability differences linear vs. orbital welds not included

# 3. HCPB performance highlights: Thermo-mechanics

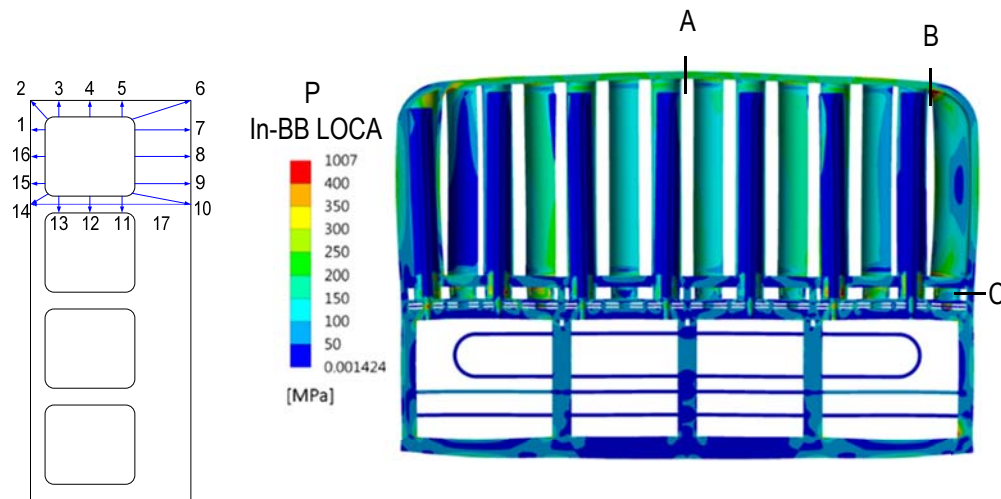


## Accidental scenario

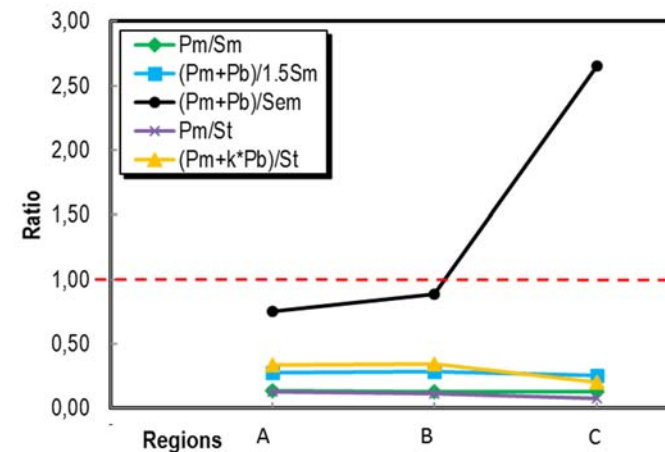
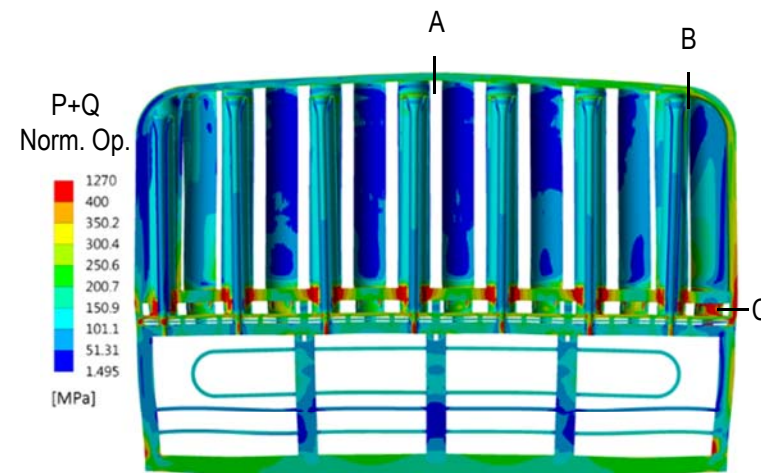
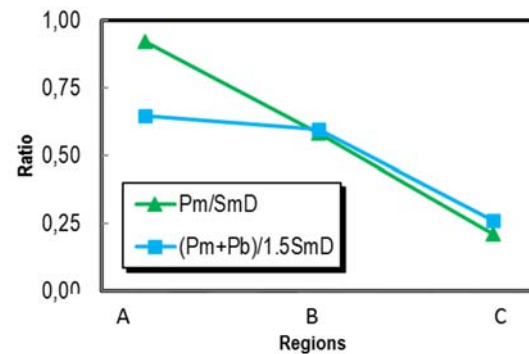
- In-box LOCA: level D, globally ok

## Normal operation

- Monotonic modes: level A, globally ok
  - Design optimization needed for local peak stresses
  - Revision of the IPFL mode: overly-conservative for EUROFER97



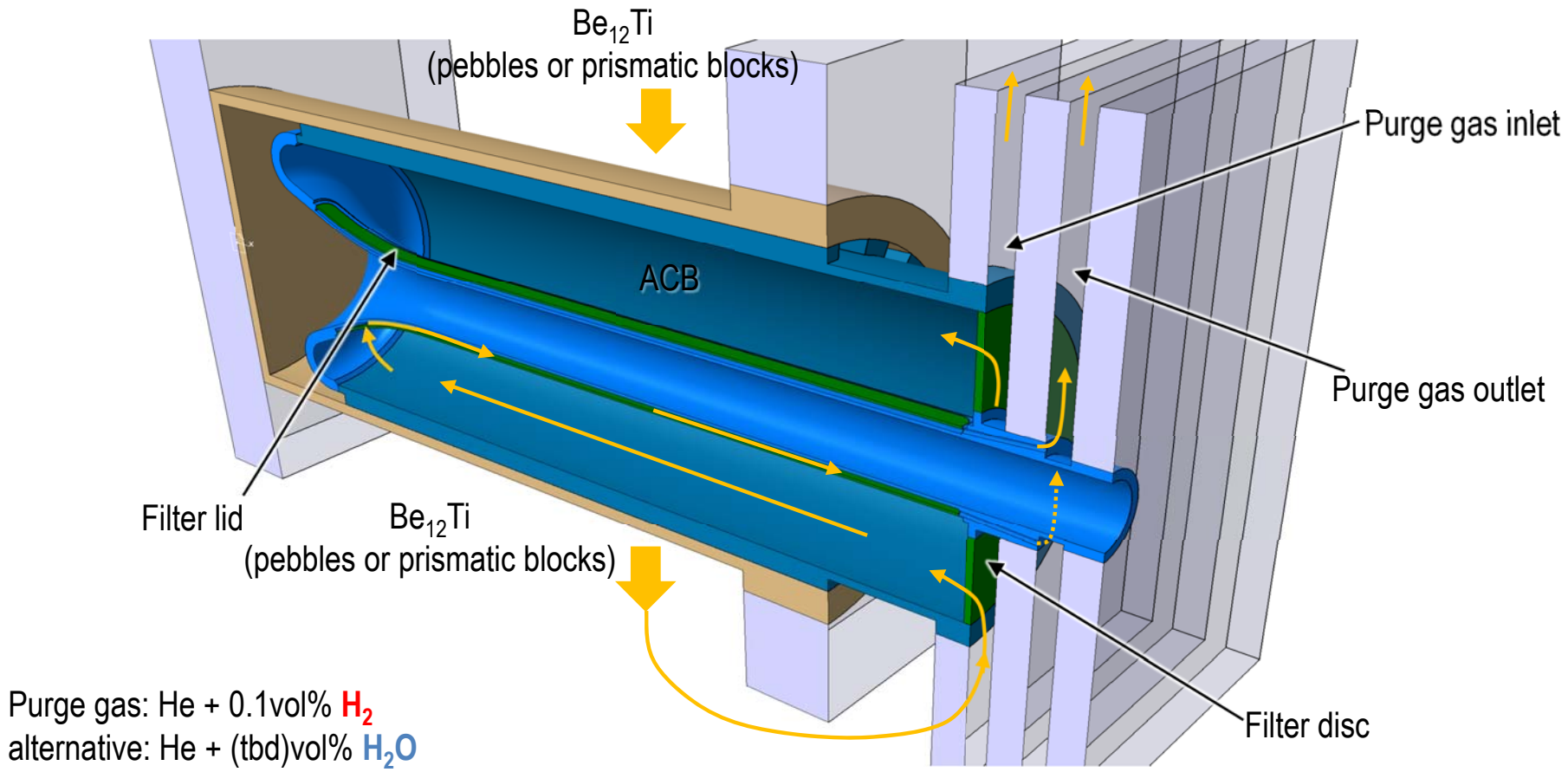
Display of paths for stress linearization





# Purge gas loop in BZ

- Purge gas loop:
  - Sequential: first  $\text{Be}_{12}\text{Ti}$  (top-bottom poloidal flow), then in-pin flow through KALOS CBs

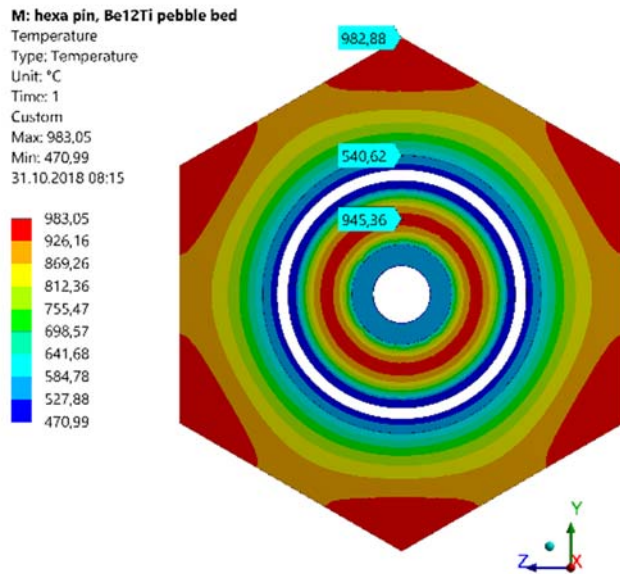


# 3. HCPB design: rationale and performances

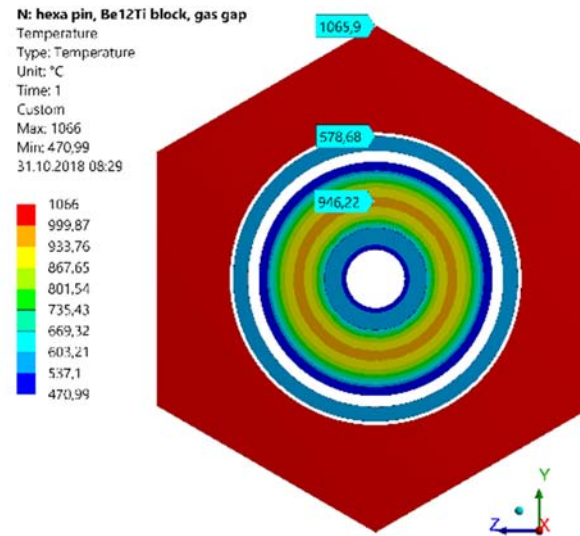


## ■ Sensitivity analysis on thermal conductivity degradation

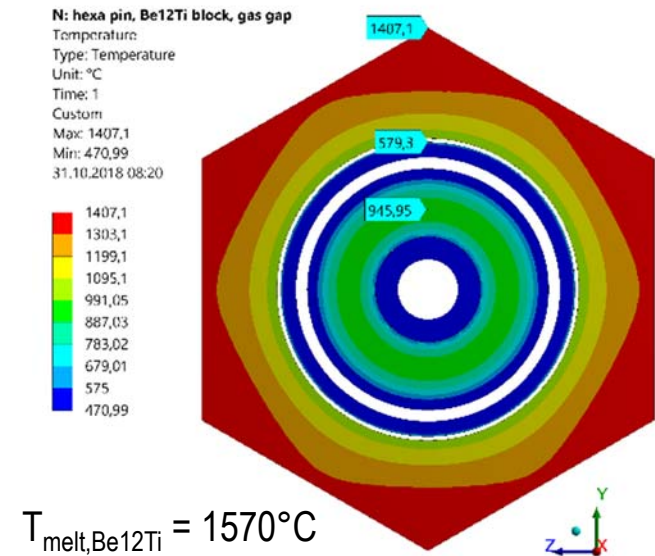
Cross section  $\approx 50\text{mm}$  behind FW  
Be<sub>12</sub>Ti pebble bed



Cross section  $\approx 50\text{mm}$  behind FW  
Be<sub>12</sub>Ti prismatic block, 1mm gas gap  
Conductivity as in Be-Ti HIDOB2  
650°C@37.1 dpa



Cross section  $\approx 50\text{mm}$  behind FW  
Be<sub>12</sub>Ti prismatic block, 1mm gas gap  
Hypothetical EoL, conductivity as a  
pebble bed



## • Conclusions:

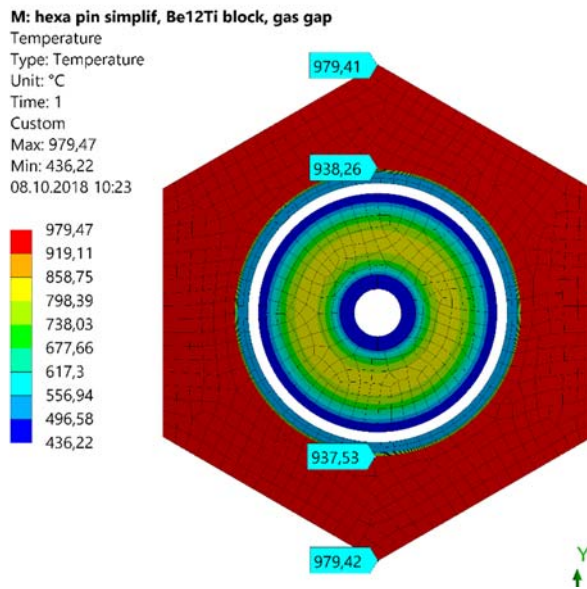
- No melting of beryllide even under hypothetical case of block reduced to a pebble bed



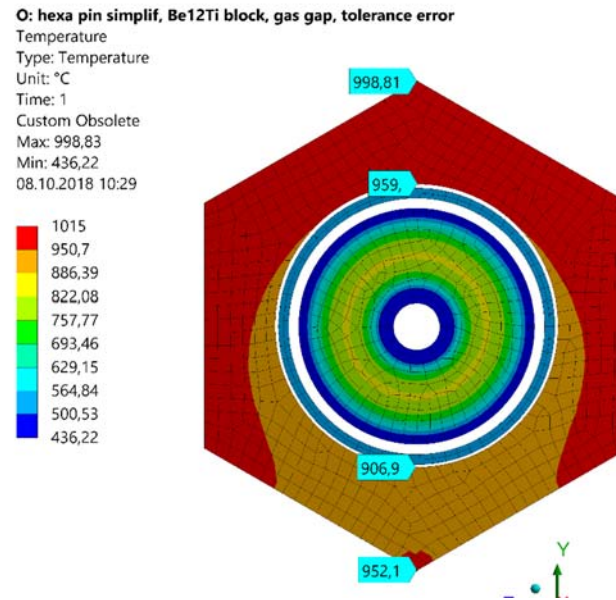
# Outcomes from CMSB simulation



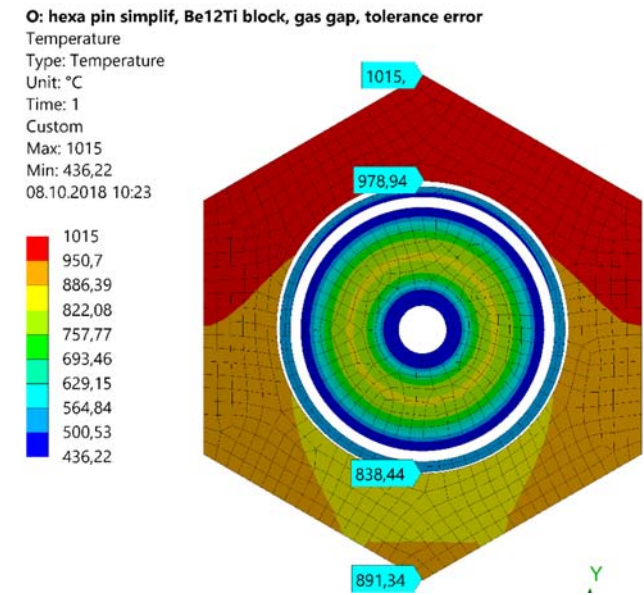
- Sensitivity analysis on concentricity mismatch tolerance error of prismatic  $\text{Be}_{12}\text{Ti}$  blocks with He gas gap



1mm He gas gap



1mm He gas gap, 0.2mm concentricity error



1mm He gas gap, 0.5mm concentricity error

# Toroidal blanket dimension variation: how are the pins at the boundaries?



- The case of the VVER reactor (Russian version of PWR):
  - VVER has also core with hexagonal assemblies
  - Core has a hexagonal matrix, but reactor core is circular, i.e. „toroidal dimension“ also variable
  - => core baffle acts as transition between matrix and core boundary
- => **side walls of the FW** (analog to core baffle in VVER –also for PWR-) can adjust the geometry toroidally

