



Use of Mesh based Variance Reduction Technique for Shielding Calculations of the Stellarator Power Reactor HELIAS

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Motivation and Objective

- Helical-Axis Advanced Stellarator (HELIAS) is a demonstration power reactor with 3000 MW D-T fusion power.
- First thorough neutronic investigation of HELIAS with DAG-MCNP (DAG = Direct Accelerated Geometry) approach.
- Shielding capability needs to be assessed for the stellarator by applying mesh based weight window variance reduction method.
- Nuclear responses in a critical area: high neutron wall load (~1.4 MW/m²) and reduced material thickness (~103 cm).

Evaluation of the obtained results according to the EU DEMO tokamak recommended radiation design limits.

HELIAS Geometry



Blanket modules incl. support structure and shield Vacuum vessel –	Vacuum v inner she	Vessel – H HELIAS CAD Mon-planar field coils including material layers
shield		and plasma distribution
Plasma : Last Closed		Vacuum vessel –
ux Surface outer shell		
Radial Structure	Thickness [cm]	Material Composition
Tungsten Armor	0.2	100% Tungsten
First Wall	2.5	70% Eurofer, 30% Helium
Breeder Zone	50	HCPB with 60% Li-6 enrichment
Back Support Structure (BSS)	~10 - 40	75% Eurofer, 25% Helium
Vacuum Vessel (VV) inner shell	6	100% Steel (SS-316)
VV shield	20	60% Steel (SS-316), 40% Water

Two different mesh setups: uniform with 20x20x20 cm³ (optimized for neutrons) and nonuniform with 5x5x5 cm³ (optimized for neutrons) and photons) in target region (blue box).

- Relative statistical error determined inside the target region, with a mesh tally, for neutrons and photons.
- Statistical error is significantly decreased \rightarrow non-uniform WW mesh is used.



VV outer shell

6

100% Steel (SS-316)

radial distance from first wall [cm]

Computation and Results

- Radial profiles of nuclear responses in critical area, evaluated against radiation design requirements specified for EU DEMO tokamak.
- Nuclear responses of interest:
 - "Maximum neutron fluence to epoxy insulator" \rightarrow target: **10⁹ cm⁻²s⁻¹** to coils.
 - "Peak nuclear heating in winding pack" \rightarrow limit: 50 W/m³ to coils.
 - "Lifetime criteria in order to ensure that the fracture toughness is reduced by no more than 30%" \rightarrow limit: **2.75 dpa/lifetime** to VV.
- Shielding requirements for superconducting magnets not met in critical area
 - \rightarrow shielding performance need to be improved.
- Displacement damage at VV_inn: ~0.11 dpa/fpy \rightarrow lifetime of 25 years guaranteed to reach EU DEMO design limit.



Conclusion and Outlook

- Variance reduction: Mesh based weight window method suitable for HELIAS.
- Calculations: Statistical reliable radial profiles of relevant nuclear responses from first wall to magnetic field coil.
- Shielding performance: Requirements for superconducting magnets not fulfilled in critical area.
- Recommended design improvements: Larger shielding layer and/or more efficient shielding materials.



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