

Research Article

A Critical Heat Generation for Safe Nuclear Fuels after a LOCA

**Jae-Yong Kim,¹ Dong-Bock Kim,² Hee-Jeong Cho,³
Soon-Bum Kwon,⁴ and Young-Doo Kwon⁴**

¹ Korea Atomic Energy Research Institute, No. 111 Daedeokdaero 989 Road, Daejeon 305-353, Republic of Korea

² Department of Global Marketing, Modeun Elevator Co., Ltd., No. 241 Ilbansaneop-gil Geochang-gun, Kyungsangnam-do, Namsang-myeon 670-881, Republic of Korea

³ Graduate School of Mechanical Engineering, Kyungpook National University, No. 80 Daehakro, Daegu 702-701, Republic of Korea

⁴ School of Mechanical Engineering & IEDT, Kyungpook National University, No. 80 Daehakro, Daegu 702-701, Republic of Korea

Correspondence should be addressed to Young-Doo Kwon; ydkwon@knu.ac.kr

Received 17 January 2014; Revised 28 March 2014; Accepted 24 April 2014; Published 18 May 2014

Academic Editor: Inn Seock Kim

Copyright © 2014 Jae-Yong Kim et al. This is an open access article distributed under the Creative Commons Attribution License, which permits unrestricted use, distribution, and reproduction in any medium, provided the original work is properly cited.

This study applies a thermo-elasto-plastic-creep finite element procedure to the analysis of an accidental behavior of nuclear fuel as well as normal behavior. The result will be used as basic data for the robust design of nuclear power plant and fuels. We extended the range of mechanical strain from small or medium to large adopting the Hencky logarithmic strain measure in addition to the Green-Lagrange strain and Almansi strain measures, for the possible large strain situation in accidental environments. We found that there is a critical heat generation after LOCA without ECCS (event category 5), under which the cladding of fuel sustains the internal pressure and temperature for the time being for the rescue of the power plant. With the heat generation above the critical value caused by malfunctioning of the control rods, the stiffness of cladding becomes zero due to the softening by high temperature. The weak position of cladding along the length continuously bulges radially to burst and to discharge radioactive substances. This kind of cases should be avoided by any means.

1. Introduction

This study applies a thermo-elasto-plastic-creep finite element procedure to the analysis of an accidental behavior of nuclear fuels as well as normal behavior. The result will be used as a basic data for the robust design of nuclear power plant and fuels. We extended the range of mechanical strain from small or medium to large adopting the Hencky logarithmic strain measure in addition to the Green-Lagrange strain and Almansi strain measures for the possible large strain situation in accidental environments.

The accidental situation is one of the most severe cases where LOCA is involved simultaneously. In most LOCA simulation, the power is generated 100% at the initial state and assumed to be shut down suddenly through the reactor trip at the beginning of the accident. However, the case assumes an accident unusual to the normal LOCA without ECCS (event category 5) [1–3], and the power is not completely but partly shut down. Therefore, we may call it a kind of RIA compared with the nominal full shutdown. The coolant behaves also

most severely to exhibit an instant loss of pressure and level as one might expect an explosive failure of reactor shell.

2. Finite Element Analysis of Nuclear Fuels

Finite element analysis procedures of nuclear fuels have been developed by many researchers [5–7] and mostly applied to basic designs of them. The nominal operation of the nuclear power plant is known to exhibit the highest pellet temperature between 1100–1400°C and the clad temperature 400–600°C [8]. In this study, we introduce the thermo-elasto-plastic-creep analysis procedure based on Green-Lagrange strain measure first. For the analysis of the fuels under accidental situation like LOCA, we extended the procedure to large strain capability adopting logarithmic strain measure and additive decomposition of plastic strain. Through the adoption of additive decomposition method, we could provide a continuation between small-medium strain analysis and large strain analysis including creep phenomena.

2.1. Thermo-Elasto-Plastic-Creep Analysis. Thermo-elasto-plastic-creep analysis procedure has been developed by many researchers [4, 6, 9–12]. Most of them adopted Green-Lagrange or Almansi strain measures adequate for small-medium strain. It is briefly introduced here. For the thermo-elasto-plastic-creep analysis of nuclear fuel, it is necessary to obtain time-dependent temperatures solving the following well-known transient equation of heat transfer:

$$\begin{aligned} k(T_{,xx} + T_{,yy}) + Q - c\rho\dot{T} &= 0, \\ q &= f_B \quad \text{on } \Gamma_f, \\ q &= h(T - T_\infty) \quad \text{on } \Gamma_h, \end{aligned} \quad (1)$$

where T = temperature, k = coefficient of the thermal conductivity, ρ = density, c = specific heat, q = heat flow rate, f_B = heat flux, and h = convective heat transfer coefficient.

The partial differential equation of heat transfer (1) is expressed in other forms of functional to be minimized as in (2) [13].

Consider

$$\begin{aligned} \Pi = \int \left(\frac{1}{2} \{\mathbf{T}_\partial\}^T [\kappa] \{\mathbf{T}_\partial\} - QT + c\rho T\dot{T} \right) dV \\ + \int \left(f_B T + hT_\infty T - \frac{1}{2} hT^2 \right) dS, \end{aligned} \quad (2)$$

where $[\kappa]$ = conductivity matrix and $\{\mathbf{T}_\partial\}$ = temperature gradient.

After we introduce finite elements with proper shape functions and minimize the functional, the following finite element equation is derived:

$${}^{t+\Delta t} \mathbf{C}^{(i)} {}^{t+\Delta t} \dot{\mathbf{T}}^{(i)} + \left({}^{t+\Delta t} \mathbf{K}^K + {}^{t+\Delta t} \mathbf{K}^C \right) \Delta \mathbf{T}^{(i)} \quad (3)$$

$$= {}^{t+\Delta t} \mathbf{Q} + {}^{t+\Delta t} \mathbf{Q}^{C(i-1)} - {}^{t+\Delta t} \mathbf{Q}^{K(i-1)},$$

$${}^{t+\Delta t} \mathbf{T}^{(i)} = {}^{t+\Delta t} \mathbf{T}^{(i-1)} + \Delta \mathbf{T}^{(i)}. \quad (4)$$

Superscript i indicates the quantity of iterative state i at time $t + \Delta t$; therefore, ${}^{t+\Delta t} \mathbf{C}$ is the heat capacity matrix at time $t + \Delta t$, ${}^{t+\Delta t} \dot{\mathbf{T}}$ is the rate of temperature at time $t + \Delta t$, ${}^{t+\Delta t} \mathbf{K}^K$ is the conductivity matrix at time $t + \Delta t$, and ${}^{t+\Delta t} \mathbf{K}^C$ is the convection matrix at time $t + \Delta t$. $\Delta \mathbf{T}$ is the incremental nodal point temperature, ${}^{t+\Delta t} \mathbf{Q}$ is the nodal point heat input vector, ${}^{t+\Delta t} \mathbf{Q}^C$ is the nodal point heat contribution due to convection, and ${}^{t+\Delta t} \mathbf{Q}^K$ is the equivalent nodal point heat contribution. Accordingly, to solve the unknown nodal point temperature, ${}^{t+\Delta t} \mathbf{T}$, the equilibrium iteration will be performed repeatedly until the difference in the right hand terms of (3) is smaller than the convergence tolerance.

After obtaining the temperature distributions of fuel, the incremental equilibrium equation for the quasi-static motion of the fuel can be solved to obtain the incremental displacements:

$${}^{t+\Delta t} \mathbf{K}^{(i-1)} \Delta {}^{t+\Delta t} \mathbf{U} = {}^{t+\Delta t} \mathbf{P} - {}^{t+\Delta t} \mathbf{R}^{(i-1)}, \quad (5)$$

where ${}^{t+\Delta t} \mathbf{K}^{(i-1)}$: stiffness matrix, $\Delta \mathbf{U}^{(i)}$: incremental displacement, $\Delta {}^{t+\Delta t} \mathbf{U}^{(i)} = {}^{t+\Delta t} \mathbf{U}^{(i-1)} + \Delta {}^{t+\Delta t} \mathbf{U}^{(i)}$, ${}^{t+\Delta t} \mathbf{P}$: external load, ${}^{t+\Delta t} \mathbf{R}^{(i-1)}$: equivalent load, and all quantities ${}^{t+\Delta t} \mathbf{X}^{(0)} = {}^t \mathbf{X}$.

The stiffness ${}^{t+\Delta t} \mathbf{K}^{(i-1)}$ has linear term and geometrically nonlinear term. The temperatures are included in the external load resulting in the thermal deformation, and the thermal strain derived from the thermal deformation is considered in the evaluation of mechanical strain to compute the mechanical stresses.

The deviatoric stress is computed from the deviatoric elastic strain disregarding plastic and creep strain as

$${}^{t+\Delta t} \mathbf{S} = \frac{{}^{t+\Delta t} E}{1 + {}^{t+\Delta t} \nu} \left({}^{t+\Delta t} \mathbf{e}' - {}^{t+\Delta t} \mathbf{e}^p - {}^{t+\Delta t} \mathbf{e}^c \right). \quad (6)$$

The mean stress is computed from the mechanical mean strain as in (5).

Consider

$${}^{t+\Delta t} \sigma_m = \frac{{}^{t+\Delta t} E}{1 - 2 {}^{t+\Delta t} \nu} \left({}^{t+\Delta t} e_m - {}^{t+\Delta t} e_{th} \right), \quad (7)$$

where ${}^{t+\Delta t} \mathbf{S}$ = deviatoric stress tensor, ${}^{t+\Delta t} \mathbf{e}'$ = deviatoric strain tensor, ${}^{t+\Delta t} \mathbf{e}^p$ = plastic strain tensor, ${}^{t+\Delta t} \mathbf{e}^c$ = creep strain tensor, ${}^{t+\Delta t} \sigma_m$ = mean stress, ${}^{t+\Delta t} e_m$ = mean strain, ${}^{t+\Delta t} e_{th}$ = thermal strain, ${}^{t+\Delta t} E$ = Young's modulus, and ${}^{t+\Delta t} \nu$ = Poisson's ratio.

If we split the plastic and creep strains at $t + dt$ into the terms at time t and incremental terms, (7) becomes

$${}^{t+\Delta t} \mathbf{S} = \frac{E}{1 + \nu} \left({}^{t+\Delta t} \mathbf{e}'' - \Delta \mathbf{e}^p - \Delta \mathbf{e}^c \right), \quad (8)$$

where

$${}^{t+\Delta t} \mathbf{e}'' = {}^{t+\Delta t} \mathbf{e}' - {}^t \mathbf{e}^p - {}^t \mathbf{e}^c. \quad (9)$$

Creep strain may be represented by various creep laws, such as power creep law or exponential creep law. To confirm plastic stress based on the given displacement, ${}^{t+\Delta t} \mathbf{u}^{(i)}$, it must satisfy one of the yield criteria. Adopting the von Mises yield criterion,

$$f_y = {}^{t+\Delta t} \bar{\sigma}^2 - {}^{t+\Delta t} \sigma_y^2 = 0, \quad (10)$$

where

$${}^{t+\Delta t} \bar{\sigma} = \sqrt{1.5 {}^{t+\Delta t} \mathbf{S} \cdot {}^{t+\Delta t} \mathbf{S}}: \text{ von Mises equivalent stress,}$$

$${}^{t+\Delta t} \sigma_y = \text{yield stress.}$$

(11)

We have to solve the equation to obtain the deviatoric stress tensor. By applying the following flow rules of plastic and creep deformations,

$$\Delta \mathbf{e}^p = \Delta \lambda {}^{t+\Delta t} \mathbf{S},$$

$$\Delta \mathbf{e}^c = \Delta t {}^\tau \gamma {}^\tau \mathbf{S},$$

$$\tau = (1 - \alpha)t + \alpha(t + \Delta t): \text{relaxed time}$$

$$\alpha = \text{relaxation parameter,}$$

$$\Delta \lambda = \frac{1.5 \Delta \bar{e}^p}{{}^{t+\Delta t} \bar{\sigma}}: \quad (12)$$

incremental parameter for equivalent

plastic strain and equivalent stress at $t + \Delta t$,

$$\gamma = \frac{1.5 \Delta \bar{e}^c}{{}^\tau \bar{\sigma}}:$$

parameter for relating incremental equivalent creep strain and equivalent creep at $t + \Delta t$.

We obtain

$${}^{t+\Delta t} \mathbf{S} = \frac{1}{{}^{t+\Delta t} a_E + \alpha \Delta t {}^\tau \gamma + \Delta \lambda} \left[{}^{t+\Delta t} (\mathbf{e}'' - (1 - \alpha) \Delta t {}^\tau \gamma {}^t \mathbf{S}) \right], \quad (13)$$

where

$$a_E = \frac{(1 + \nu)}{E}. \quad (14)$$

Taking product of both sides of the equation, we obtain the equation

$$f_g = a^2 {}^{t+\Delta t} \bar{\sigma}^2 + b {}^\tau \gamma - c^2 {}^\tau \gamma^2 - d^2 = 0, \quad (15)$$

where

$$\begin{aligned} a &= {}^{t+\Delta t} a_E + \alpha \Delta t {}^\tau \gamma + \Delta \lambda = \frac{1}{2G'}, \\ b &= 3(1 - \alpha) \Delta t {}^{t+\Delta t} \mathbf{e}'' \cdot {}^t \mathbf{S}, \\ c &= (1 - \alpha) \Delta t {}^t \bar{\sigma}, \\ d^2 &= 1.5 {}^{t+\Delta t} \mathbf{e}'' \cdot {}^{t+\Delta t} \mathbf{e}'' . \end{aligned} \quad (16)$$

The function, left side of (15), is known as the effective stress function and has to satisfy the condition of zero. Because $\bar{\sigma}$ is function of plastic strain and $\Delta \lambda$ is also a function of plastic strain and yield stress, the equation has to be solved simultaneously for $\bar{\sigma}$ and $\Delta \lambda$ using a root finding algorithm. We tried both Newton method and bisection method. The latter method is very reliable without being affected by the derivative of function. For the assumed ${}^{t+\Delta t} \bar{\sigma} = {}^{t+\Delta t} \sigma_y$, we compute $\Delta \lambda$ first and check whether the equation is satisfied.

If not, follow the usual root finding algorithm to find the ${}^{t+\Delta t} \bar{\sigma} = {}^{t+\Delta t} \sigma_y$ and $\Delta \lambda$ simultaneously that satisfy the chosen type of yield criterion.

Here, we propose a first stage constitutive modulus as

$$C'' = \frac{1}{{}^{t+\Delta t} a_E + \lambda + \Delta t {}^\tau \gamma} = 2G''; \quad (17)$$

see Figure 2.

The tangent constitutive matrix \mathbf{C}^{EPC} can be obtained using the perturbation method [4, 14]. Each column of the matrix \mathbf{C}^{EPC} is the column of perturbed stress tensor divided by the magnitude of perturbed element of corresponding strain. However, at the first stage of interaction, one can use quasi-elastic modulus of Figure 1(b) [4], where C'' is more close to the tangent modulus than C' . We could find fast convergence with C'' than with C' in several examples.

2.2. Large Strain (Hencky Logarithmic) Measure. For large strain analysis, we need some basic relations that are briefly introduced here [9, 10, 15–18]. Deformation gradient is represented by

$${}^t_0 \mathbf{X} = \frac{\partial {}^t \mathbf{x}}{\partial {}^0 \mathbf{x}} = {}^t \mathbf{J}^T ({}^0 \mathbf{J}^{-1})^T, \quad (18)$$

where \mathbf{J} is the transformation between the cartesian coordinate and natural coordinate systems. It can be decomposed as

$${}^t_0 \mathbf{X} = {}^t_0 \mathbf{R} {}^t_0 \mathbf{U} \quad (19)$$

into rotation tensor ${}^t_0 \mathbf{R}$ and right stretch tensor ${}^t_0 \mathbf{U}$. Right Cauchy-Green gradients are expressed using the deformation gradient

$${}^t_0 \mathbf{C} = {}^t_0 \mathbf{X}^T {}^t_0 \mathbf{X}. \quad (20)$$

Solving the following equivalent problem,

$${}^t_0 \mathbf{C} {}^t_0 \mathbf{P} = {}^t_0 \boldsymbol{\Lambda}^2 {}^t_0 \mathbf{P}. \quad (21)$$

We obtain a diagonal matrix ${}^t_0 \boldsymbol{\Lambda}$ of principal stretches and their direction matrix. The right stretch matrix is obtained as

$${}^t_0 \mathbf{U} = {}^t_0 \mathbf{P} {}^t_0 \boldsymbol{\Lambda} {}^t_0 \mathbf{P}^T. \quad (22)$$

Therefore, the rotation matrix is given by

$${}^t_0 \mathbf{R} = {}^t_0 \mathbf{X} {}^t_0 \mathbf{U}^{-1}. \quad (23)$$

Then, the Hencky logarithmic strain tensor is given by

$${}^t_0 \mathbf{E}_H = \frac{1}{2} {}^t_0 \mathbf{P} {}^t_0 \mathbf{H} {}^t_0 \mathbf{P}^T, \quad (24)$$

where

$${}^t_0 \mathbf{H} = \ln \boldsymbol{\Lambda}^2. \quad (25)$$

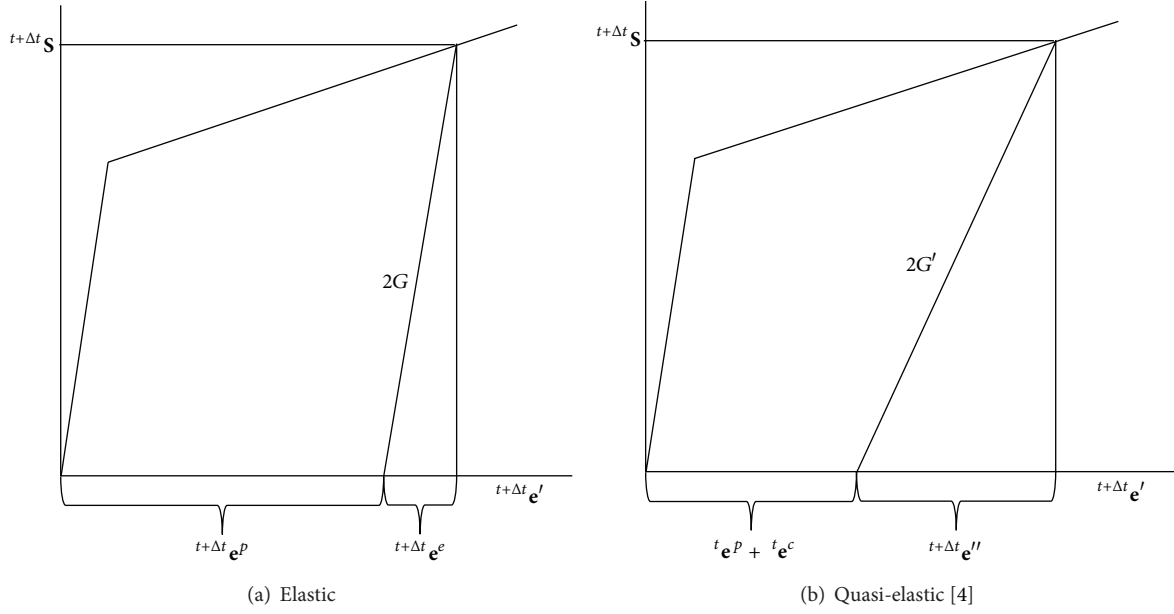


FIGURE 1: First stage constitutive modulus.

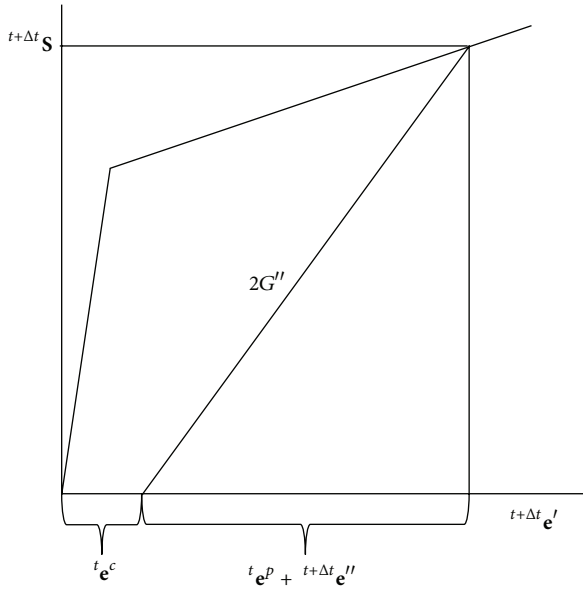


FIGURE 2: Proposed first stage quasi-elastic modulus.

We can also represent the Green-Lagrange strain tensor as

$${}^t_0\mathbf{E}_G = \frac{1}{2} ({}^t_0\mathbf{C} - \mathbf{I}) \quad (26)$$

and the Almansi strain tensor as

$${}^t_0\mathbf{E}_A = \frac{1}{2} (\mathbf{I} - {}^0_t\mathbf{C}). \quad (27)$$

The Hencky logarithmic strain tensor can also be written as

$${}^t_0\mathbf{E}_H = \ln {}^t_0\mathbf{U} = \frac{1}{2} \ln {}^t_0\mathbf{C}. \quad (28)$$

We computed the above equations and compared them (three kinds of marks) with theoretical (29) in Figure 3:

$$\begin{aligned} {}^t_0e_{11} &= \frac{1}{2} ({}^t_0\lambda^2 - 1) : \text{Green-Lagrange strain,} \\ {}^t_0e_{11} &= {}^t_0\lambda : \text{Small strain,} \\ {}^t_0E_{11}^{(H)} &= \ln {}^t_0\lambda : \text{Logarithmic strain,} \\ {}^t_0e_{11}^A &= \frac{1}{2} (1 - {}^t_0\lambda^{-2}) : \text{Almansi strain.} \end{aligned} \quad (29)$$

Using the above procedure of thermo-elasto-plastic-creep with logarithmic strain measure with additive decomposition of plastic strain, our additive decomposition method follows the procedures in [9, 10, 15] except that we do not decompose plastic deformation gradient multiplicatively as

$$\mathbf{X}_A^E = {}^t_0\mathbf{X} ({}^t_0\mathbf{X}^p)^{-1} \quad (30)$$

at the trial elastic state first but considered it additively as in (6) and (8). Therefore, we do not lose continuation between the later large strain procedure and the former small strain procedure including creep deformation.

The case without creep deformation in the following example compares the two procedures of large strain problems, additive decomposition and multiplicative decomposition. The first one is uniaxial isochoric extension with the following condition [15]:

$E = 200$ MPa: Young's modulus;

$\nu = 0.3$: Poisson's ratio;

$\sigma_y = 0.75$ MPa: yield stress;

$H = 2$ MPa: hardening modulus.

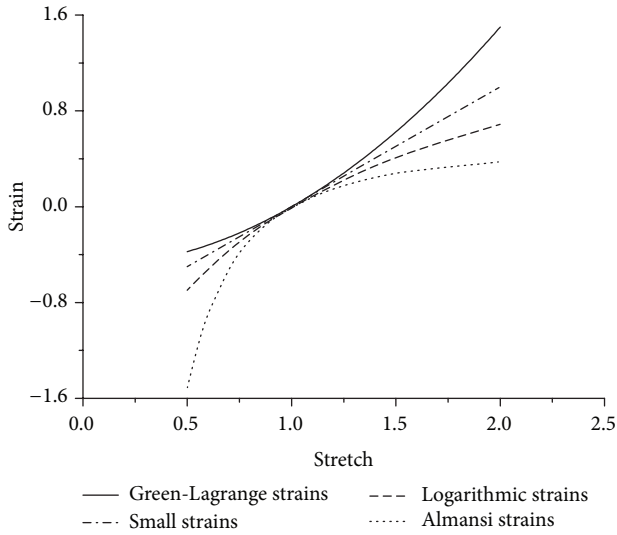


FIGURE 3: Strain-stretch relations for various strain measures.

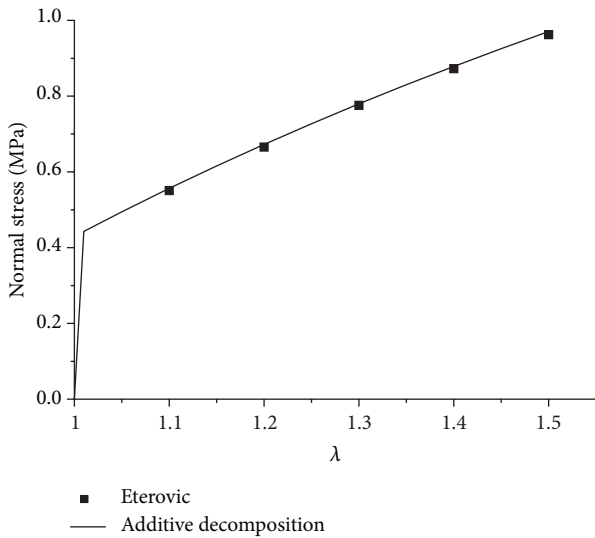


FIGURE 4: Normal stress σ_x along stretch λ .

The results are compared in Figure 4 showing a good agreement.

The second example is shear deformation; the result is compared with the result of the literature (solid square) obtained by the multiplicative decomposition in Figure 5.

We also compared the following example (Figure 6) under shear load in Figure 7, which tests not only the stress integration process but also the equilibrium iteration procedure too.

3. Behavior of Nuclear Fuels after Loss of Cooling Accidents

In the usual case of LOCA, the control rods are inserted into the fuel rods to shut down the reactor. The power is reduced successfully. However, such as the Fukushima accident, the

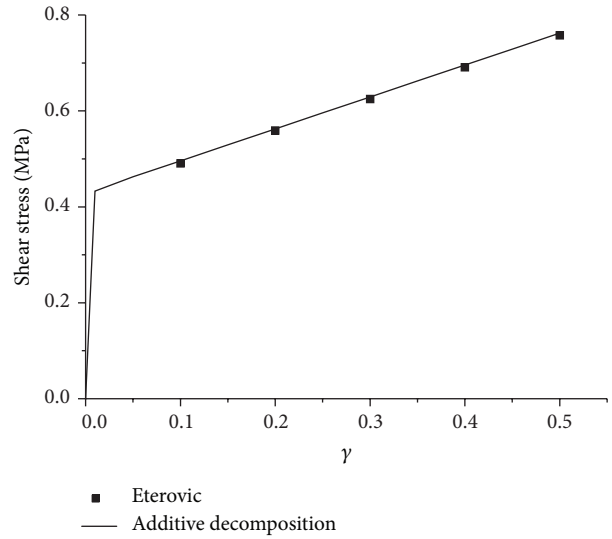


FIGURE 5: Shear stress τ_{12} along shear strain γ .

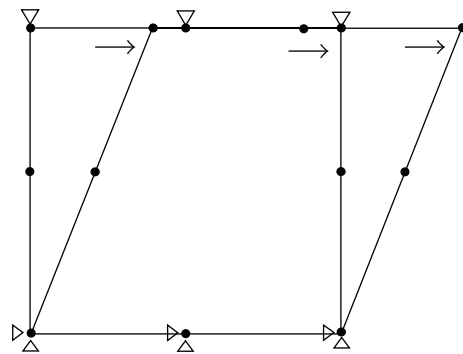


FIGURE 6: FEM model for shear load analyses of multiplicative and additive decomposition methods.

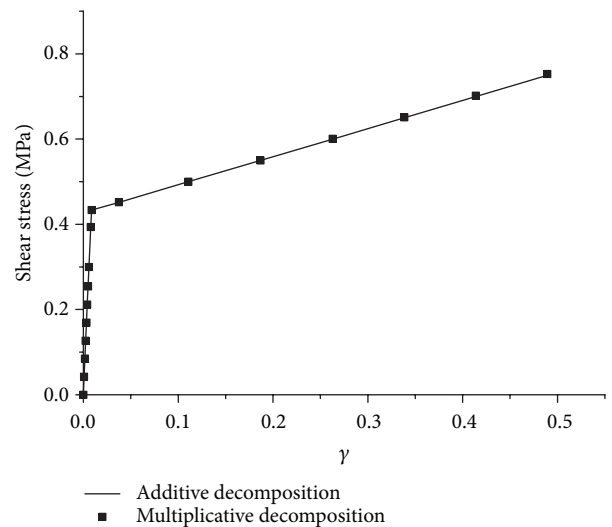


FIGURE 7: Shear stresses under shear loads for different decompositions of plastic strain.

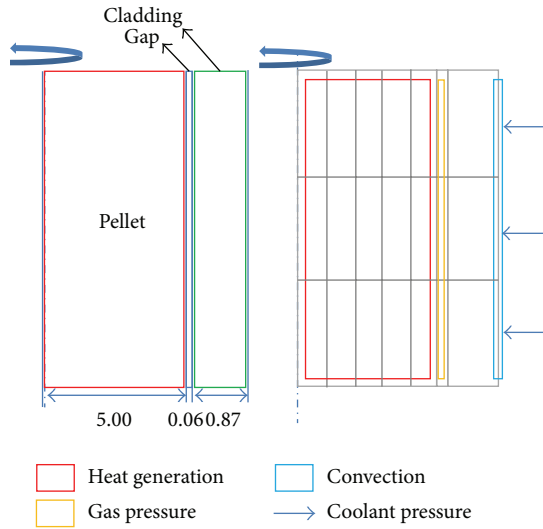


FIGURE 8: Solid nuclear fuel model and boundary conditions.

TABLE I: Applied condition.

Coolant pressure	15.5 MPa
Gap pressure	3.45 MPa
Gap conductivity	0.5 mJ/s·mm·°K
Heat generation	450 mJ/mm ³ ·s
Heat transfer coefficient	75 mJ/s·mm ² ·°K
T_{∞}	600°K

control rods could not be inserted to the rods because of the malfunction of inserting system located at the lower part of the reactor. Therefore, we assumed that the heat generation of the fuels is partly reduced and partly working. In the literature concerning LOCA scenario, we saw a 100% power generation adopted in the analysis.

Before we analyze the behavior of nuclear fuels after LOCA (loss of coolant accident), we analyzed normal operation first using the following condition. The material properties of fuel components are the same as in the literature [6]. Mesh generation of the fuel rod was conducted using the quadrilateral 8-node elements, and the boundary conditions were applied as in Figure 8.

The boundary conditions include the application of heat generation to the pellet and the application of coolant pressure and convection resulting from the coolant to cladding. By applying gas pressure to the gap between the pellet and cladding, the overall environment when actual nuclear fuel rods are activated is shown in Table 1.

Figure 9 shows the temperature changes at pellet center and outer surface of the cladding along time, and temperatures go into steady state before 100 sec.

Suppose that there is a LOCA (loss of coolant pressure and relative heat transfer coefficient changed from 7500 mJ/s·mm²·°K to 75 mJ/s·mm²·°K) at time 70 sec. Moreover, there is an additional situation of the malfunctioning of control rod.

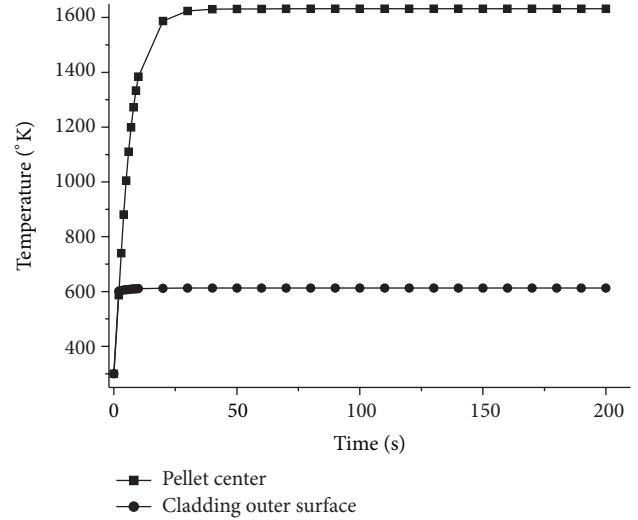


FIGURE 9: Temperatures changes at pellet center and cladding outer surface along the time.

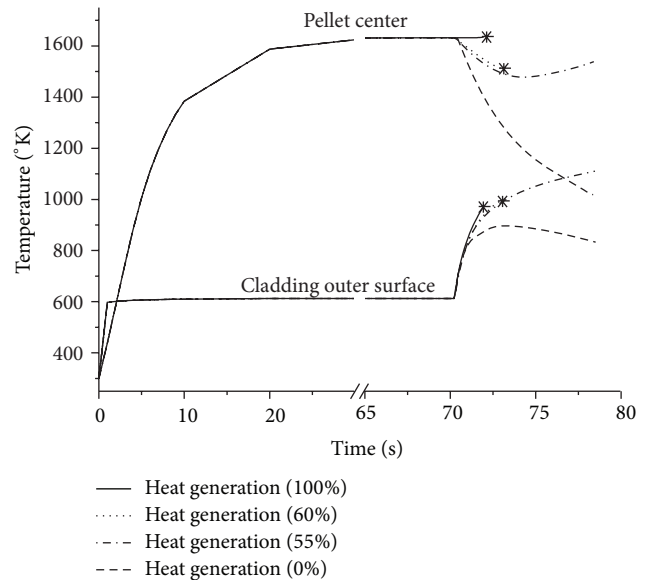


FIGURE 10: Temperatures changes at pellet center and cladding outer surface after LOCA at 70 sec.

We assumed that one of four conditions of heat generation (complete shutdown = 0%, 55%, and 60% and full heat generation = 100%) is maintained after the LOCA. Figure 10 shows temperature changes at the two positions. As one may expect, shutdown makes the rapid drops of temperature at both positions of fuel. For the intermediate heat generations, the temperatures of pellet drop first and then increase afterwards. The temperature of clad increases fast and slowly increases later on. Finally, sharp increase of temperature is detected at the clad for full heat generation. The asterisk marks denote that the analyses have been stopped due to the instability of clad stiffness caused by the softening by temperature increase, surpassing the hardening by strain. The load applied to the clad due to the rod internal pressure is

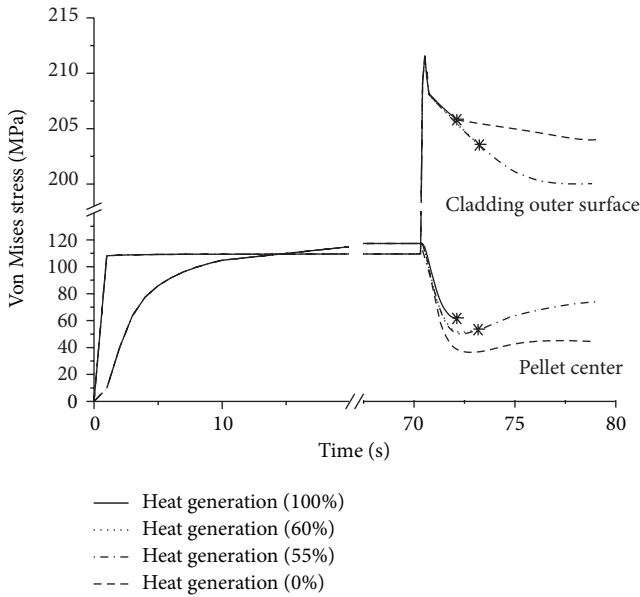


FIGURE 11: Von Mises stresses changes at pellet center and cladding outer surface after LOCA at 70 sec.

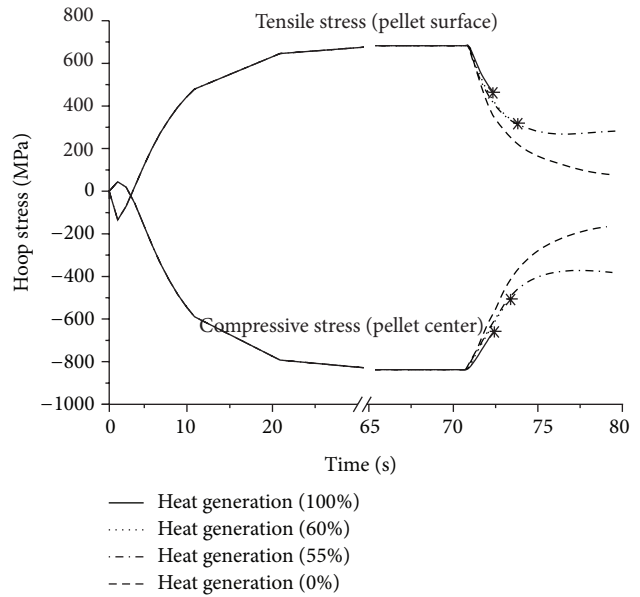


FIGURE 13: Compressive and tensile stresses changes at pellet center and outer surface after LOCA at 70 sec.

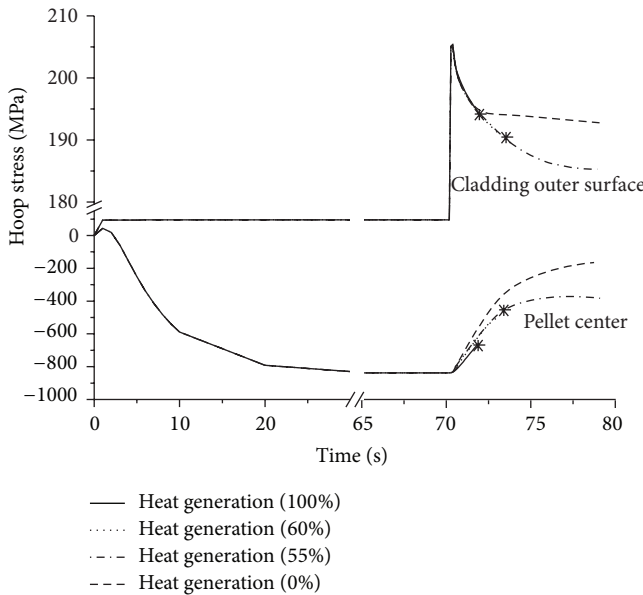


FIGURE 12: Hoop stresses changes at pellet center and cladding outer surface after LOCA at 70 sec.

supported by the strength of clad. However, if the temperature of the clad is too high, Young's modulus and strength of the clad are reduced, and it cannot sustain the applied load. Once it cannot resist the load and starts to deform, the situation is out of control and the clad becomes thinner and thinner causing larger and larger stresses resulting in fuel failure. This phenomenon is a kind of instability like a buckling.

The variations of von Mises stress of clad are shown in Figure 11. The heat generations above 55% induced instability at the clad to cause continuous expansion to burst at the weakest point along the length.

In the early stage, there is a gap between the pellet and clad. After a long time of operation, the coolant pressure compresses the clad to cause a creep phenomenon and the gap is gone. Then the contact stress between pellet and clad raises the compressive stress at the inner part of the pellet. If this stress is above the ultimate strength of the pellet material, pellet may be fractured to pieces and may give some harm to the clad.

Hoop stresses at the center of pellet and clad surface are shown in Figure 12. We may have temporal time interval for the rescue of the plant in accident for the condition of heat generation under 55% of full generation. Over 60%, a catastrophic situation occurs when the clad of zero stiffness expands continuously to bust and discharge radioactive substances, a situation we have to avoid by any means.

Figure 13 exhibits the tendencies of thermal stresses in the pellet. Except the very early stage, outer portion of the pellet experiences tensile hoop stress, and the inner portion of it experiences compressive stresses. Every case including full heat generation shows lowering of thermal stresses in the pellet, which is thought to originate from smaller temperature difference between the extreme positions and insulation effect at the outer surface of the clad due to the loss of coolant. What is ironic is that the LOCA results in the reduction of temperature gradient in the pellet and causes more even distribution of temperature within the pellet to make it safer.

4. Conclusions

We extended the finite element procedure of code TEPC (thermo-elasto-plastic-creep) from small and medium strain to Hencky logarithmic strain measure for the large strain analysis to handle the case that may happen in accidental situation. The TLPC (thermo-large strain-plastic-creep) which is the extended version of the TEPC has been applied to the

analysis of nuclear fuels under LOCA without ECCS (event category 5) as well as normal operation.

A LOCA is concerned with the system pressure decrease, the rod internal pressure, the hoop stress on the cladding, the temperature of the cladding, strain, and so forth. We added an accidental condition of malfunctioning of the control rod system resulting in incomplete shutdown of the fuel power. And we checked a critical remaining heat generation that causes clad failure. The results are summarized as follows.

- (i) The fuels under consideration have reached quasi-steady state condition after 70 sec. The maximum core temperature was 1631°K, and the outer cladding temperature was 612.6°K when there was no contact between the pellet and cladding.
- (ii) We adopted a LOCA without ECCS (event category, 5), and the external pressure of the fuel rod has zero gauge pressure. After the LOCA (lower heat transfer coefficient of $75 \text{ mJ/s} \cdot \text{mm}^2 \cdot \text{°K}$) at time 70 sec with the heat generation of the pellet under the critical value, there was no instability at the cladding for the time being, which means one may have temporal time interval for the rescue of the system by recovery of the cooling system or shutdown of the system letting the heat generation be zero.
- (iii) After the LOCA at time 70 sec, there was an unstable condition that the stiffness of the cladding becomes zero which is followed by a local uncontrollable expansion of clad to burst, with above critical heat generation of the pellet.

For the analysis of this kind of unstable situation, we need certain special analyzing methods. One method is a postbuckling method usually applied to static problems, and it is expected to be improper for this kind of transient problem. The other one is a dynamic analysis, which usually is not necessary for this kind of quasi-static problem; however, it is expected to be the most effective technique which can handle the problem of zero stiffness.

Conflict of Interests

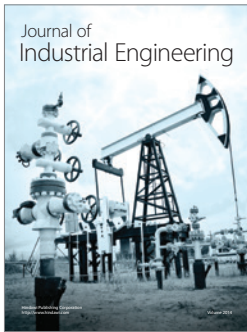
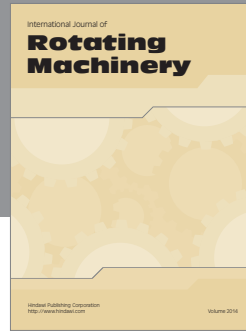
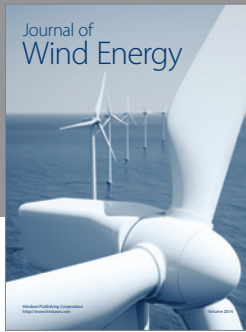
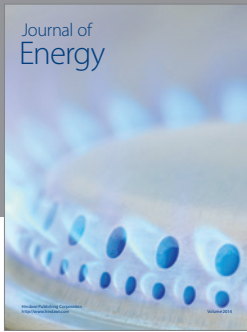
The authors declare that there is no conflict of interests regarding the publication of this paper.

Acknowledgments

This research was supported by the Basic Science Research Program through the National Research Foundation of Korea (NRF) funded by the Ministry of Education (NRF-2012RIA1A2008903). The authors are partially supported by BK21 plus project of the National Research Foundation of Korea.

References

- [1] J. N. Lillington, *Light Water Reactor Safety: The Development of Advanced Models and Codes for Light Water Reactor Safety Analysis*, Elsevier Science, Amsterdam, The Netherlands, 1995.
- [2] Nuclear Energy Agency OECD, "Nuclear fuel behavior in loss-of-coolant accident (LOCA) conditions," State-of-the-Art-Report, 2009.
- [3] IAEA, *Analysis of Severe Accidents in Pressurized Heavy Water Reactors*, IAEA-TECDOC-1594, 2008.
- [4] M. Kojic and K. J. Bathe, "The "Effective-stress-function" algorithm for thermo-elasto-plasticity and creep," *International Journal for Numerical Methods in Engineering*, vol. 24, no. 8, pp. 1509–1532, 1987.
- [5] M. Suzuki and H. Saitou, *Light Water Reactor Fuel Analysis Code FEMAXL-IV(VER.2); Detailed Structure and User's Manual*, Department of Reactor Safety Research Nuclear Safety Research Center, Tokai Research Establishment, Japan Atomic Energy Research Institute Tokai-mura, Ibaraki-Ken, Japan, 1997.
- [6] Y. D. Kwon, S. B. Kwon, H. J. Song, M. S. Kim, and J. S. Kim, *Evaluation of Nuclear Fuel Behaviors Using Finite Element*, KAERI, 2010.
- [7] Y. D. Kwon, Y. S. Yang, J. S. Kim, and S. B. Kwon, "A study on the flow characteristics in an annular type fuel pellet of PWR," *Journal of Mechanical Science and Technology*, vol. 27, no. 1, pp. 257–261, 2013.
- [8] C. B. Lee, Y. S. Yang, D. H. Kim, S. K. Kim, and J. G. Bang, "A new mechanistic and engineering fission gas release model for a uranium dioxide fuel," *Journal of Nuclear Science and Technology*, vol. 45, no. 1, pp. 60–71, 2008.
- [9] K. J. Bathe, *Finite Element Procedures*, vol. 2, no. 3, Prentice Hall, Englewood Cliffs, NJ, USA, 1996.
- [10] M. Kojic and K. J. Bathe, *Inelastic Analysis of Solids and Structures*, Computational Fluid and solid Mechanics, 2004.
- [11] W. E. Haisler and D. R. Sanders, "Elastic-plastic-creep-large strain analysis at elevated temperature by the finite element method," *Computers & Structures*, vol. 10, no. 1-2, pp. 375–381, 1979.
- [12] M. D. Snyder and K. J. Bathe, "A solution procedure for thermo-elastic-plastic and creep problems," *Nuclear Engineering and Design*, vol. 64, no. 1, pp. 49–80, 1981.
- [13] R. D. Cook, D. S. Malkus, M. E. Plesha, and R. J. Witt, *Concepts and Applications of Finite Element Analysis*, John Wiley & Sons, New York, NY, USA, 2002.
- [14] C. Miehe, "Numerical computation of algorithmic (consistent) tangent moduli in large-strain computational inelasticity," *Computer Methods in Applied Mechanics and Engineering*, vol. 134, no. 3, pp. 223–240, 1996.
- [15] A. L. Eterovic and K. Bathe, "A hyperelastic-based large strain elasto-plastic constitutive formulation with combined isotropic-kinematic hardening using the logarithmic stress and strain measures," *International Journal for Numerical Methods in Engineering*, vol. 30, no. 6, pp. 1099–1114, 1990.
- [16] J. C. Simo, "A framework for finite strain elastoplasticity based on maximum plastic dissipation and the multiplicative decomposition: part I. Continuum formulation," *Computer Methods in Applied Mechanics and Engineering*, vol. 66, no. 2, pp. 199–219, 1988.
- [17] F. J. Montáns and K. J. Bathe, "On the stress integration in large strain elastoplasticity," *Computational Fluid and Solid Mechanics*, vol. 2003, pp. 494–497, 2003.
- [18] M. Á. Caminero, F. J. Montáns, and K. Bathe, "Modeling large strain anisotropic elasto-plasticity with logarithmic strain and stress measures," *Computers & Structures*, vol. 89, no. 11-12, pp. 826–843, 2011.



Hindawi

Submit your manuscripts at
<http://www.hindawi.com>

