10

11 12

13

14 15

17

18

19

20 21

22 23

24

25

# Probabilistic Risk Assessment of Station Blackouts in Nuclear Power Plants

Hindolo George-Williams , Min Lee, and Edoardo Patelli

Abstract—Adequate ac power is required for decay heat removal in nuclear power plants. Station blackout (SBO) accidents, therefore, are a very critical phenomenon to their safety. Though designed to cope with these incidents, nuclear power plants can only do so for a limited time, without risking core damage and possible catastrophe. Their impact on a plant's safety are determined by their frequency and duration, which quantities, currently, are computed via a static fault tree analysis that deteriorates in applicability with increasing system size and complexity. This paper proposes a novel alternative framework based on a hybrid of Monte Carlo methods, multistate modeling, and network theory. The intuitive framework, which is applicable to a variety of SBOs problems, can provide a complete insight into their risks. Most importantly, its underlying modeling principles are generic, and, therefore, applicable to non-nuclear system reliability problems, as well. When applied to the Maanshan nuclear power plant in Taiwan, the results validate the framework as a rational decision-support tool in the mitigation and prevention of SBOs.

Index Terms—Accident recovery, Monte Carlo simulation (MCS), nuclear power plant, risk assessment, station blackout (SBO).

26	$\min{(\mathbf{B})}$	Least element of set/vector <b>B</b> .
27	$\min\{\mathbf{B},\mathbf{Q}\}$	Least element of $\mathbf{B} \cup \mathbf{Q}$ .
28	$(\mathbf{B},i)$	<i>i</i> th element of set/vector <b>B</b> .
29		ABBREVIATIONS
30	AC	Alternating Current.
31	DC	Direct Current.
32	C	Node capacity.
33	CCF	Common-cause failure.
34	CCG	Common-cause group.
35	CS	Cold standby state.
36	F	Failed state.
37	LOOP	Loss of offsite power.
38	MCS	Monte Carlo simulation.

Manuscript received October 6, 2017; revised December 7, 2017; accepted April 3, 2018. This work was supported by the EPSRC and ESRC Centre for Doctoral Training on Quantification and Management of Risk and Uncertainty in Complex Systems and Environments under Grant:EP/L015927/1. Associate Editor: W.-T. K. Chien. (Corresponding author: Edoardo Patelli.)

The authors are with the Institute for Risk and Uncertainty Engineering, University of Liverpool, Liverpool L69 3BX U.K., and also with the Institute of Nuclear Engineering and Science, National Tsing Hua University, Hsinchu City 300, Taiwan (e-mail: H.George-Williams@liverpool.ac.uk; mlee@ess.nthu.edu.tw; Edoardo.Patelli@liverpool.ac.uk).

Color versions of one or more of the figures in this paper are available online at http://ieeexplore.ieee.org.

Digital Object Identifier 10.1109/TR.2018.2824620

S	Shutdown state.	39
SBO	Station blackout.	40
SU	Start-up state.	41
TM	Test/preventive maintenance state.	42
W	Working state.	43
	Nomenclature	44
$\mathbf{A}$	System adjacency matrix.	45
$\mathbf{C}$	Component capacity vector.	46
$c_x^{\{i\}}$	Capacity of component $i$ in state $x$ .	47
$\{c_x^{\{i\}}\}_{M\times 1}$	Set of current capacities of all components.	48
$\dot{\mathbf{E}}_i$	Set of attributes of component $i$ .	49
e	System edge matrix.	50
$f_l$	LOOP frequency.	51
$f_s$	SBO frequency.	52
$f_{xy}\left(t\right)$	Probability density function for transition from	53
	state $x$ to $y$ .	54
$\mathbf{G}$	System graph object.	55
k	Number of edges/links in system graph.	56
lb	Set of minimum flow through edges/links.	57
M	Number of system nodes.	58
m	Number of safety buses/trains.	59
N	Number of Monte-Carlo samples.	60
$n_1$	Number of trains a generator can supply.	61
$p_n$	SBO probability given the $(n-1)$ th SBO.	62
ub	Set of maximum flow through edges/links.	63
r	Number of components affected by a CCF.	64
$\mathbf{r}_n(t)$	Non-recovery probability from the $n$ th SBO.	65
$\mathbf{S}$	Register indicating SBO occurrence.	66
S	Set of source nodes.	67
$s_j$	SBO indicator for the $j$ th simulation sample.	68
$\mathbf{T}$	Component transition matrix.	69
$\mathbf{t}$	ID of virtual output node.	70
$U_{ m tm}$	Unavailability due to test or maintenance.	71
u	Proportion of train demand generator satisfies.	72
$\mathbf{V}$	Set of nodes in the system graph.	73
$x_0$	Initial component state.	74
$X_{ij}$	Flow from node $i$ to $j$ .	75
$X_{\mathrm{out}}$	Flow into the virtual output node.	76
Y	Set containing flows through all the nodes.	77
Θ	System inequality constraint matrix.	78
Γ	System incidence matrix.	79
Φ	System equality constraint matrix.	80
$\Omega_{ij}$	Maximum flow from node $i$ to $j$ .	81
3	Number of intermediate nodes	00

Number of intermediate nodes.

System flow objective function.

82

ð

84	ho	Set of components making up CCG.
85	$\delta$	Number of components in CCG.
86	$oldsymbol{ heta}$	Set of CCF probabilities.
87	$\beta_1$	Common failure mode for CCG.
88	$eta_2$	State rendering CCG vulnerable to CCF.
89	au	Vector of next node transition times.
90	$oldsymbol{\mu}_{ m old}$	Vector of node capacities at last system jump.

91

92

93

94

95

96

97

98

99

100

101

102

103

104

105

106

107

108

109

110

111

113

114

115

116

117

118

119

120

121

122

123

124

125

126

127

128

129

130

131

133

134

135

136

#### I. INTRODUCTION

UCLEAR power is produced by harnessing the heat generated from a fission reaction chain in a reactor vessel. The reactor vessel is placed in a concrete containment to shield the environment from the potential release of radioactive materials. Core damage ensues when the core temperature exceeds a certain threshold or the nuclear fuel elements in the vessel are uncovered. This event may trigger containment breach, inflicting huge environmental and economic catastrophe.

Severe accident mitigation is achieved in part by ensuring a reliable cooling water circulation in the reactor vessel. This objective, during normal plant operation, is achieved through heat exchange between the primary and secondary loops of the plant's main cooling system. The process, however, ceases on plant shut down and backup cooling systems are required to sustain decay heat removal. Like the main cooling system, the backup cooling systems rely on ac power provided by sources outside the plant (offsite power). When these sources fail (loss of offsite power—LOOP), emergency sources onsite are started, to drive the plant's safety systems. If the emergency sources are also unavailable or unable to function as required, the plant is said to be in a station blackout (SBO). The backup cooling systems, however, are equipped with alternative turbine or diesel-driven pumps to help the plant cope with this incident. These systems, on the downside, require for monitoring and control, dc power from dc power banks. Their sustainability, therefore, regardless of their inherent reliability, is limited by the dc battery depletion time. This time, and the boil-off rate of reactor coolant, define the maximum acceptable ac power recovery duration [1].

SBO accidents are the largest contributor to nuclear power plant risk, accounting for over 70% of the core damage frequency at some plants [1], [2]. LOOP events, which initiate these accidents, are classified on the basis of their origin. A grid-centred LOOP is due to the failure of the transmission network outside the plant, switchyard-centred LOOP arises from failures in the switchyard on the plant premises, plant-centered LOOP is triggered by the operational dynamics of the plant itself, while weather-related LOOP is attributed to failures induced by severe and extreme weather, excluding lightning [1], [2]. The effective SBO risk is the sum of the core damage frequencies induced by the various LOOP types.

### A. Review of Existing Models

SBO risk quantification starts with a LOOP event tree analysis [3], where the emergency power system availability is checked in the first heading. This event failure, frequency of which defines the SBO frequency, transfers the analysis to the SBO event

tree [1]. In the latter, the successes of the various mitigating actions, including offsite power and the recovery of the emergency diesel generators (EDGs) at specific times are also checked. These times, however, vary across plants and depend on the status of a plant's mitigating systems. At the Maanshan nuclear power plant, for instance, power recovery is checked at 1, 2, 4, and 10 h into SBO. Each top event probability in the SBO event tree requires one or more static fault trees [4]–[6] for its quantification.

Static fault tree analysis employs an analytical approach, as such, it carries the important advantage of being computationally efficient. For this reason, its sensitivity, importance, and uncertainty analysis capabilities are outstanding. These attributes explain its wide use for risk analysis in the nuclear, aviation [7], and chemical process industries [8]. Unfortunately, fault trees become intractable with large systems or moderate systems with complex interactions [8]. They often require a detailed knowledge of the system being modeled, making them both difficult to apply and error-prone. Their static nature also limits their applicability in many ways. For instance:

- 1) Implementing certain types of interdependencies is either tedious or completely impossible.
- 2) The analyst has to assume that SBO is coincident with LOOP and that all power recovery efforts start simultaneously **after** SBO sets in. As a consequence:
  - a) The SBO frequency and nonrecovery probability are overestimated in most cases, since the repair of a failed element is normally initiated immediately.
  - b) For plants with multiple emergency power systems, it is impossible to determine which sequence of response minimizes the SBO frequency and maximizes the recovery probability simultaneously.
  - c) It is also difficult to investigate the effects of external factors like logistic problems, extreme environmental events, and human resource constraints on the recovery process.
- 3) The analyst is forced to assume the nonoccurrence of a second SBO after power recovery. This assumption, however, loses its validity if the emergency sources are recovered first. In this case, a second failure could initiate another SBO sequence before offsite power recovery.
- 4) Finally, there is the problem of inconvenience due to repetitive modeling. Since the nonrecovery probability is normally required for multiple instances, each would require a dedicated fault tree.

There are numerous instances of remarkable attempts at extending the applicability of fault trees to systems with interdependencies and various forms of dynamic interactions [6], [9]. Kaiser *et al.* [10], for instance, introduced a state/event fault tree approach that translates fault-trees to deterministic and stochastic petri nets. Similarly, Zhou and Zhang [11], quite recently, proposed an approach that converts static fault trees to dynamic uncertain causality graphs in order to tackle the dynamic and uncertainty attributes of practical engineering systems. However, like Kaiser's approach [10], Zhou's [11] is restricted to binary-state components and systems. Even though the performance of most components could be partitioned into two levels, the

151

157

159

161

166

167

168

169

170

172

173

178

181

182

183

existence of multiple failure modes makes binary-state models inadequate. Also, from a modeling perspective, there are occasions when the analyst would need to model a binary-state element as a multistate one in order to fully define its behavior. Such flexibility requires a framework supporting multistate modeling. Bobbio *et al.*'s fault tree to Bayesian Network mapping procedure [12] effectively solve this problem. However, like Kaiser's and Zhou's approaches, Bobbio's mapping procedure is also susceptible to deficiencies (3) and (4) outlined above.

Dynamic fault trees [13]-[16] are perhaps the closest researchers have come to solving the limitations of static fault trees. Various approaches have been proposed for their solution but Markov analysis [14], [15], [17] remains the most popular. Markov modeling, however, like static fault tree analysis, becomes intractable with large systems and is only applicable to exponentially distributed transitions. Nevertheless, state explosion is no longer an issue, with the introduction of intuitive dynamic fault tree software [18], [19]. Even with these developments, most of the dynamic fault tree solution approaches are susceptible to deficiencies (3) and (4) outlined above. These deficiencies can only be addressed by approaches offering the flexibility to replicate the exact behavior of the system. Such an approach, however, was put forward by Rao et al. [16], which they used to model the power supply system of a nuclear power plant. The approach simulates a system's dynamic fault tree and addresses most of the limitations of static fault trees. However, like the majority of system reliability models, Rao's work is only applicable to binary-state components. The development of a more universal simulation framework, therefore, is desirable.

#### B. Proposed Approach and Scope

As evidenced in Rao *et al.*'s [16], Rocha *et al.*'s [20], and Lei *et al.*'s [21] works, Monte Carlo simulation (MCS) is flexible enough to model any system attribute. Its problem, however, is that most of the existing MCS algorithms are system-specific and require either the structure function, cut sets, or path sets of the system. An intuitive event-driven MCS procedure, offering multistate component modeling opportunities has recently been proposed [22]. This procedure is general and does not require the definition of the system's path and cut sets or structure function, thanks to its embedded graph model.

In this work, the graph and multistate models proposed in [22] are adopted. The graph model is used to model the topology of the system and allow the performance of the system to be directly computed from the performance of the components. This attribute eliminates the need for an explicit association of component failure combinations to the state of the system. The multistate model, on the other hand, is used to model the behavior of the components, overcoming the assumption of a perfectly binary behavior of components. It is particularly useful to the multiple failure mode and dynamic attribute representation of the emergency power systems. This model, for instance, could be exploited to investigate the effects of limited maintenance teams or the unavailability of spares on the emergency power systems recovery [23]. We extend the original model to incorporate interdependencies by means of a dependency matrix and an

efficient recursive algorithm to propagate the effects of failures across the system. Completing the framework, we propose a simple MCS algorithm that induces LOOP in the system, replicate the ensuing sequence of events, and monitor the availability of power at the various safety buses. The number of available safety buses, as a function of time, is computed after each system event. From the simulation history, any SBO index can be computed, thereby providing an opportunity for more insights into SBO risks. The multistate component model, together with the dependency matrix, adequately captures and represents the redundancies in the emergency power system of the plant. Consequently, the explicit modeling of these redundancies, which poses a significant challenge, is eliminated.

1) Merits and Novelty of the Proposed Approach: The framework, for now, is limited to grid and switchyard induced LOOP, given their dominance [2]. Its preliminary results were first presented at the 13th Probabilistic Safety Assessment and Management conference [24]. However, this paper proposes several improvements. First, an extensive review of the suitability of fault trees and their derivatives, to SBO analysis has been included. We have also considered the effects of common-cause failures (CCF), unavailability due to test or maintenance, and human error on the SBO frequency and recovery probability. We also show how the results obtained from the framework can be absorbed in the existing model. Finally, we extend the number of computable SBO indices and consider the effects of system configuration and the sequence of operator response on system recovery.

This paper is the first documented application of load-flow simulation to a complete SBO risk assessment. With respect to the existing models discussed in Section I-A, the proposed framework exhibits the following advantages:

- Adequacy and Flexibility: It models realistic attributes of the plant's power recovery and provides more insights into SBO risks. For instance, it enhances the investigation of the possibility of a second SBO after the first.
- Convenience and Generality: It is convenient in the sense
  that the modeler does not need to deduce the combination
  of component failure leading to system failure. They also
  do not need to explicitly model component redundancies,
  as these are implicitly captured by the modeling framework. The modeling framework, in addition, is applicable
  to many system reliability problems.
- 2) Solution Sequence: The proposed approach is applied as summarized by the following chronological steps:
  - Identify the key elements of the system, define its topology, and derive its flow equation parameters.
  - Develop the multistate model for each system element.
  - Model the interdependencies between the elements.
  - Force a LOOP event and simulate the behavior of the standby power systems.
  - Compute the SBO indices from the simulation history.

#### II. SBO MODELING

A nuclear power plant's power system consists of the grid, the switchyard, the emergency power systems, alternative emergency power system, and the safety buses. The alternative

emergency power systems are additional emergency sources [such as gas turbine generators (GTGs)] available at some plants to boost their LOOP/SBO recovery capability. In this section, we show how the plant's power system is accurately modeled and analyzed, in line with the solution sequence outlined in Section I-B2. 312

#### A. System Topology 313

4

307

308

309

310

311

314

315

316

317

318

319

320

321

322

323

324

326

328

329

330

331

332

333

334

335

336

337

We represent the topology of the plant's power system by graph nodes of which depict the components of the system. Connecting the nodes are perfectly reliable links portraying the direction of power flow. Flows from all the safety buses are terminated on a virtual node, introduced to represent the total available power. This virtual node would later be used to compute the nonrecovery probability of ac power.

Let the nodes of the system be numbered from 1 to M and represented by the set  $V = \{1, 2, \dots, M\}$ . Since the links are perfectly reliable, the adjacency matrix, A, of the system is defined as

$$\mathbf{A} = \{a_{ij}\}_{M \times M} \mid a_{ij} = \begin{cases} 1 & \text{If flow is } i \to j \\ 0 & \text{Otherwise.} \end{cases}$$
 (1)

The topology of the system, therefore, can be defined by G $G = (\mathbf{V}, \mathbf{A})$ . Using the parameters of G only, the flow equations of the system can be derived [22]. These equations can then be used in synergy with the current state properties of the system nodes to deduce the performance of the system. For this, a linear programing algorithm is employed, given the possibility of flow redirection and the need to satisfy the capacity constraints of the nodes and their links. The objective is to find the flow across each link of the system that maximizes the flow into the virtual node. If  $X_{ij}$  is the flow across the link between nodes i and j and given there are k such links for all  $(i, j) \in \mathbf{e}$ , where  $\mathbf{e}$ is the edge matrix of the system as defined in [22], the linear programing problem is formulated by (2), (5), (7), and (8)

$$\Theta\{X_{ij}\}_{k\times 1} \le \{c_x^{\{i\}}\}_{M\times 1} \mid (i,j) \in \mathbf{e} \quad \forall i \in \mathbf{V}.$$
 (2)

Equation (2) expresses the inequality constraints to be satisfied, where  $c_x^{\{i\}}$  denotes the capacity of node i when residing in state  $x. \{c_x^{\{i\}}\}_{M \times 1}$ , therefore, is the vector of current capacities of all the nodes of the system. The inequality matrix,  $\Theta$ , is related to the incidence matrix,  $\Gamma$ , as follows:

$$\Theta = \{\theta_{iq}\}_{M \times k} \mid \theta_{iq} = \begin{cases} 1, & \gamma_{iq} \neq 0 \\ 0, & \text{otherwise} \end{cases}$$
 (3)

$$\Gamma = \{\gamma_{pq}\}_{M \times k} \mid \gamma_{pq} = \begin{cases} 1, & p = i \\ -1, & p = j \\ 0, & \text{otherwise.} \end{cases}$$
 (4)

 $\Gamma$  is related to  $\mathbf{A}$  by (4), where  $q = 1, 2, \dots, k$  (the edge number) is the index of the edge between nodes i and j in **e** and p = $1, 2, \ldots, M$ 

$$\mathbf{\Phi}\{X_{ij}\}_{k\times 1} = \{0\}_{\eth\times 1} \quad \forall (i,j) \in \mathbf{e}. \tag{5}$$

Equation (5) expresses the equality constraint to be satisfied, where  $\Phi$  and  $\Gamma$  are related, thus

$$\Phi = {\phi_{\lambda q}}_{\eth \times k} \mid \phi_{\lambda q} = \gamma_{pq}$$

$$\lambda = 1, 2, \dots, \eth \mid \eth < M \quad f : \lambda \to p \quad \forall p \in (\mathbf{s} \cup \mathbf{t})'. \quad (6)$$

347

349

350

351

352

353

354

355

356

360

364

366

367

369

370

371

373

374

377

378

380

381

d is the number of intermediate nodes, s is the set of source nodes, which comprises the grid and standby power systems, while t is the virtual node representing the total output of the system. If the intermediate nodes of the system (i.e., nodes not in s and t) are arranged in ascending order of their ID, (6) suggests the  $\lambda$ th row of  $\Phi$  is identical to the pth row of  $\Gamma$ , where p is the λth element of the ordered set of intermediate nodes. In other words,  $\Phi$  is a submatrix of  $\Gamma$ , containing all the rows of the latter corresponding to intermediate nodes

$$\mathbf{lb} = \{0\}_{k \times 1}, \quad \mathbf{ub} = \{\Omega_{ij}\}_{k \times 1}$$

$$\Omega_{ij} = \min\{c_{\max}^{\{i\}}, c_{\max}^{\{j\}}\} \quad \forall (i, j) \in \mathbf{e}.$$
(7)

Equation (7) defines the lower and upper bound vectors, lb and **ub**, of the flow through the links, where  $c_{\text{max}}^{\{i\}}$  is the maximum capacity of node i. Finally, the objective function of the linear programing problem is expressed as

$$\Psi = -\{\psi_q\}_{1 \times k} \{X_{ij}\}_{k \times 1} \mid \psi_q = \sum_{i \in s} \gamma_{iq}.$$
 (8)

Following the termination of the linear programing algorithm, the vector of flow, Y, through the nodes of the system is given by  $\Theta_{M\times k}\{X_{ij}\}_{k\times 1}$ . The total output, therefore, is given by the tth element, (Y, t), of Y. Interestingly, all the parameters, but  $\{c_x^{\{i\}}\}_{M\times 1}$ , required to compute Y remain static during system simulation. The main task, therefore, is to update  $\{c_x^{\{i\}}\}_{M\times 1}$  after each system event. The derivation of (2) to (8) is outside the scope of this paper, interested readers are referred to [22]. However, an illustrative example of the linear programing problem formulation is provided in the Appendix of this paper.

#### B. System Components

Each component is defined by a multistate model that takes into account the various parameters that characterize its operation. Let  $E_i$  denote component i, then

$$E_i = (\mathbf{T}, \mathbf{C}, x_0) \tag{9}$$

$$\mathbf{T} = \{T_{xy}\}_{n \times n} \mid x \neq y \quad (x, y) \in \{1, 2, \dots, n\},\$$

$$T_{xy} = \begin{cases} \infty, & \text{If } x \to y \text{ is a forced transition} \\ 0, & \text{If no transition between states x and y} \\ f_{xy}(t), & \text{Otherwise} \end{cases}$$
(10)

where T is the transition matrix of the component;  $C \mid C =$  $\{c_x\}_{1\times n}$  is the capacity vector;  $x_0$  is the initial state;  $c_x$  is the capacity in state x; n is the number of states; and  $f_{xy}(t)$  is the probability density function characterizing the transition from state x to y. T contains the density function objects for all the transitions depicted in the multistate model of the component and C defines the capacity of the component in each state.

443

444

446

450

451

453

455

457

458

461

463

464

465

466

467

468

469

470

471

Each state capacity is expressed as a nondimensional number defining the proportion of total system output the node can supply or transmit while residing in that state. If m is the total number of power trains at the plant,  $n_1$ , the number of power trains the node simultaneously supplies, u, the proportion of power train demand it can satisfy, then, its capacity when working perfectly is,  $n_1um^{-1}$ . It expresses the total system output as a fraction of the number of power trains/safety buses present at the plant. On this note, the grid and switchyard nodes are each assigned unity capacity when available and 0, otherwise. The virtual output node has a fixed capacity of 1 and each safety bus, a fixed capacity of  $m^{-1}$ .

382

383

384

386

388

390 391

392

393

394

395

396

397

398

399

401

402

403

404

405

406

407

408

409

410

411 412

413

414

415

416

417

418

419

420

421

422

423

424

425

426

427

428

429

430

431

432

433

434

435

436

437 438

1) Modeling the Grid and Switchyard: The grid is modeled as a two-state node: "Working," when available and "Failed," otherwise. Though grid failures are mostly random, we model them as forced transitions [23], since they already are incorporated in the LOOP frequency. Most often, plants tap their ac power from multiple offsite sources, and grid failure is defined as the failure of all of these sources. The repair of at least one of the failed sources, however, is sufficient to achieve grid recovery. For this reason, the transition from "Failed" to "Working" is defined by the upper bound of the envelope around the cumulative density functions (cdf) of the individual source repair distributions. Given this, sampling the grid recovery time entails generating a uniform random number and reading off its corresponding time from the envelope cdf, interpolating where necessary. An important point to note is that this approach slightly underestimates the grid recovery probability, as it assumes the individual source repair actions are initiated concurrently. In practice, the sources do not necessarily fail simultaneously and their recovery actions may commence at different times. This implies, by the time the last source fails, the restoration of already failed sources would have begun. The actual grid recovery time, therefore, is less than that given by the envelope cdf. This, however, is acceptable, as the goal in risk management is to ensure risk levels are acceptable, even in worst case scenarios.

Similarly, normal switchyard operation is defined by a twostate node. In cases where the plant is enhanced with multiple switchyards, switchyard recovery is treated as in the case of multiple grid sources. Fig. 1 shows the multistate model for the grid and switchyard.

- 2) Modeling the Standby Power Systems: The emergency power system is constituted by the EDGs, and in this work, GTGs constitute the alternative emergency power system. In this section, we model only the multistate behavior of the standby power systems, and the effects of redundancies on their operation is considered in a latter section. We make the following assumptions in developing these models.
  - The initiation of test/maintenance is coincident with LOOP, and at any instance, there is not more than one source in test or maintenance.
  - 2) Sources in test or maintenance remain unavailable through the sequence.
  - 3) Repairs are commenced immediately.
  - 4) A generator just from maintenance cannot fail to start. This implies a perfect maintenance scenario.

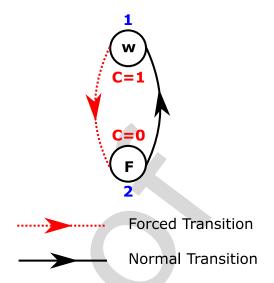


Fig. 1. Multistate model for grid and switchyard nodes.

The alternative emergency power system recovery is assumed offsite power recovery in [24]. This assumption is on the premise that their failure is included in the LOOP frequency. However, the assumption is impractical, given they are mostly a standby source. We, therefore, modify their multistate model to include running failures, rendering them an onsite source.

We consider failure-to-start and failure-to-run as the only failure modes an EDG is susceptible to. Failure-to-start refers to the EDG failure to start from cold-standby and failure-to-run denotes its failure to function for the duration of the LOOP. While the former is defined by a crisp probability, the latter is characterized by a time-to-failure probability density function. However, the standardized plant analysis risk model [1] considers a third EDG failure mode, failure-to-load, defining the case when the EDG starts but cannot power the load. This failure mode is considered failure-to-start, in the proposed framework. We introduce two additional states, "Working" and "TM," as shown in Fig. 2, to account for the perfect operation of the EDG and its unavailability due to test or maintenance, respectively. Except otherwise, the transition from cold standby to working is instantaneous, while the transition from cold standby to failure or TM is also instantaneous but conditional. Conditional transitions are a special type of forced transition depending on a probabilistic event that is external to the component and with a known likelihood [23]. Conditional and forced transitions have the same representation in the transition matrix of the component [see (10)].

The GTGs behave in almost the same way as the EDGs, save for the difference in their start-up and manual alignment times. For this, a start-up state is inserted between their cold-standby and working states, as shown in Fig. 2. While in start-up, they could fail, explaining the transition from start-up to failure.

3) Accounting for Human Error: Human error is very important in the risk assessment of engineering systems. In SBO recovery, human errors mostly manifest themselves as delayed response to certain SBO mitigation action. For instance, the switchyard is forced into a temporary shutdown state during grid

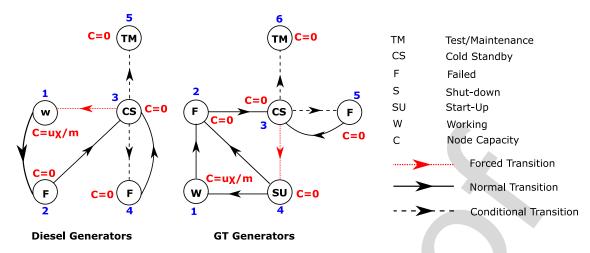


Fig. 2. Multistate models for emergency diesel and GTGs without human error consideration.

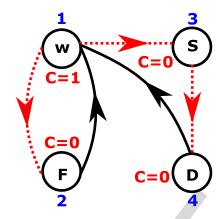


Fig. 3. Multistate model for switchyard with human error consideration.

failures. On grid recovery, the plant personnel manually initiate its restoration, which process is susceptible to human-induced delays. Accounting for these delays, two additional states are introduced in the two-state model discussed in Section II-B1, as shown in Fig. 3. The transitions from "Working" to "Shutdown" and from "Shutdown" to "Delay" (D), are influenced by grid failure and recovery respectively. "Shutdown" denotes grid recovery-in-progress, while "Delay" represents switching-in-progress. The latter determines the difference between the potential and actual bus recovery times. If this difference is negligible or the potential, instead of the actual bus recovery time is required, the model in Fig. 1 is retained.

Similarly, the GTG and some EDGs require manual start-up and alignment, this is the case for shared diesel generators. A generator is said to be shared if it can substitute several units but, however, can only replace one unit at a given instance. Therefore, in the case of sequential multiple unit failures, only the first unit is replaced. For simultaneous failures, any of the units can be replaced, since they normally are identical. Since these replacements are manually executed, they are susceptible to delays, contrary to what most models suggest. Fig. 2, for instance, assumes the transition from cold standby to the fully functional or failure state to be instantaneous. This, by extension, implies,

any maintenance action (if the generator fails to start) is initiated at once. However, with human error, the start-up procedure may be initiated later than scheduled. We, therefore, introduce two states, one each, between cold standby and working and failure and cold standby, as shown in Fig. 4, to account for these delays. We have assumed the plant personnel to be well trained, experienced, and fit to perform their assigned tasks as expected. Consequently, the possibility of inappropriately executed actions is ignored.

Transitions  $6 \to 1$  with  $4 \to 7$  and transition  $7 \to 4$  with  $5 \to 8$ , of Fig. 4, account for human error in the recovery of manually operated emergency diesel and GTGs, respectively. In practical applications, human error is expressed in terms of the probability of not completing a given action within a specified time. If this probability is known for multiple times, a cdf could be fitted through the points. For this, we recommend the Weibull distribution, since it can yield a wide range of distributions. Recall the cdf of a Weibull distribution is  $1 - e^{-(t/a)^b}$ , where a and b are its scale and shape parameters, respectively. Given the human error probabilities are the likelihoods of inaction, they define the complement of the human reaction time cdf. Therefore, the Weibull parameters, a and b, are obtained by fitting the set of probability values to the function  $e^{-(t/a)^b}$ .

#### C. Modeling Component Interdependencies

To ensure resilience, system designers often employ multiple layers of defense, either in the form of redundancies or shared components. This proactive strategy inadvertently introduces interdependencies in the system, resulting in modeling accuracy issues. We define interdependency in a more general sense as the potential for a state change in one element to trigger a state change in another. We propose two models, the CCF and the cascading failure models, to implement these interdependencies.

1) CCF Model: This model is used when the random failure of any member of a group of similar components, performing the same task could cause the failure of one or more of the remaining components [25]. Such a group of components is

n 512 ne 513 ll 514 s. 515 re 516

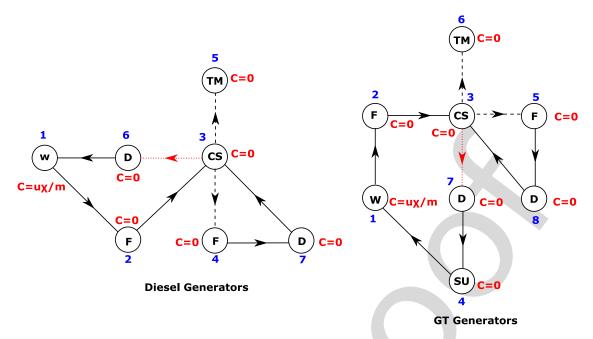


Fig. 4. Multistate models for emergency diesel and GTGs with human error consideration.

called a common-cause group (CCG), and its key attributes are as follows:

- 1) There is a set of probabilities associated with the number of components involved in any random failure event. Let this set of probabilities be defined by  $\theta \mid \theta = \{\theta_r\}^{\delta}$ , where r is the number of components affected by the failure event,  $\delta$ , the total number of components in the group, and  $\sum_{r=1}^{\delta} \theta_r = 1$ .
- 2) All the components in the CCG fail in the same mode. Implying, the CCG for start-up failures cannot influence the CCG for running failures, for instance.

Each CCG, therefore, is defined by the quadruple,  $(\rho, \beta_1, \beta_2, \theta)$ , where  $\rho$  is the set of components in the CCG,  $\beta_1$ , the common failure mode, and  $\beta_2$ , the state the components have to be in to be susceptible to this failure mode. The algorithm for propagating CCF is summarized thus.

- 1) When a component fails, check if its new state matches  $\beta_1$  for its CCG.
- 2) Go to step (v) if there is no match. Else, determine the number of components, r, that will fail.
- 3) Go to step (v) if r = 1. Else, remove from  $\rho$ , the component initiating the failure event. From the remainder, randomly select r 1 components.
- 4) For each component selected in step (iii), check if its current state matches  $\beta_2$  and set this to  $\beta_1$ .
- 5) End procedure.

The procedure above requires  $\theta$  to be in conformity with the  $\alpha$ -Factor model [25]. CCF probabilities expressed in the multiple Greek letter model would need to be converted as in [25].

2) Cascading Failure Model: This model is used for interdependencies not satisfying the CCF criteria. For instance, the redundancies among the standby power systems and the dependence of the latter on the grid and switchyard. An important assumption invoked in this model, however, is that on occurrence of the trigger event, the dependent event occurs immediately.

Initially proposed in [26], the model defines interdependencies by a dependency matrix. The dependency matrix,  $\mathbf{D}_i$ , for node i, defines the effects of the node's state transition on other nodes. It takes the form,  $\mathbf{D}_i = \{d_{j1}, d_{j2}, d_{j3}, d_{j4}\}_{v \times 4} \mid j=1,2,\ldots,v-1,v$ , where  $d_{j1}$  is the state of i triggering the event,  $d_{j2}$ , the affected node,  $d_{j3}$ , the state the node has to be in to be vulnerable, and  $d_{j4}$ , its target state after the event. Each row of  $\mathbf{D}_i$  defines the behavior of an affected node, and v, the number of relationships. For example, consider a two-component system, with each component existing in three possible distinct states. When component 1 makes a transition to state 3, component 2 is forced to make a transition to state 2 as well, if and only if the latter is currently residing in state 1. Since component 1 is the trigger component in this case, the interdependency is defined by  $\mathbf{D}_1$  as

$$\mathbf{D}_1 = \begin{pmatrix} 3 & 2 & 1 & 2 \end{pmatrix}. \tag{11}$$

Let a third three-state component be added to the system. In addition to its effect on component 2, let the transition of component 1 also affect component 3, such that the latter is forced to state 1 if it is in state 3 at the time of the trigger event. To represent the overall behavior of component 1,  $\mathbf{D}_1$  is updated as shown in (12), to reflect the new information:

$$\mathbf{D}_1 = \begin{pmatrix} 3 & 2 & 1 & 2 \\ 3 & 3 & 3 & 1 \end{pmatrix}. \tag{12}$$

(12) shows that each row of the dependency matrix represents a possible outcome.

Occasionally, a state change in a node can only affect another node if a third node is in a certain state. This type of dependency is known as a joint dependency, and it is outside the scope of the initial model in [26]. We introduce the joint dependency matrix,

 $\mathbf{D}' = \{d'_{i1}, d'_{i2}, d'_{i3}, d'_{i4}\}_{v \times 4}$ , to resolve this problem. Element  $d'_{i1}$  defines the state the third node must be in to satisfy the joint dependency, while  $d'_{i2}$ ,  $d'_{i3}$ , and  $d'_{i4}$  have the same meaning as  $d_{i2}$ ,  $d_{i3}$ , and  $d_{i4}$ , respectively. Assuming a certain state change in node i only affects, say node x, if node  $\omega$  is in state  $\sigma$ ,  $\mathbf{D}_i$  defines the relationship between nodes i and  $\omega$ , while  $\mathbf{D}'_{\omega}$ defines the relationship between  $\omega$  and x. Nodes i,  $\omega$ , and x are the trigger, intermediate, and target nodes, respectively. The intermediate node does not undergo a state change, meaning its target state is the same as its vulnerable state. Therefore, in  $\mathbf{D}_i$ , the third and fourth elements of the row corresponding to the intermediate node are equal. Given j = 1, for  $\mathbf{D}_i$ ,  $d_{12} = \omega$ ,  $d_{13}=d_{14}=\sigma$  and for  $\mathbf{D}'_{\omega}$ ,  $d'_{11}=\sigma$ ,  $d'_{12}=x$ . The remaining elements retain their meaning, as defined earlier. Let, for illus-trative purposes, the dependency between components 1 and 3 (second row of  $D_1$  in (12)) only hold if component 2 is in state 2:

$$\mathbf{D}_1 = \begin{pmatrix} 3 & 2 & 1 & 2 \\ 3 & 2 & 2 & 2 \end{pmatrix} \quad \mathbf{D}_2' = \begin{pmatrix} 2 & 3 & 3 & 1 \end{pmatrix}. \tag{13}$$

To represent this attribute, the second row of  $\mathbf{D}_1$  is modified to reflect the relationship between components 1 and 2, and the relationship between components 2 and 3, defined by  $\mathbf{D}_2'$  as shown in (13). Notice  $\mathbf{D}_2'$ , instead of  $\mathbf{D}_2$ , has been used, since the relationship between components 2 and 3 is due to a joint dependency with another component.

The dependency and joint dependency matrices, indeed, can be used to represent a wide range of dependencies. However, there are a few instances that may result in large matrices. Such cases require an intuitive manipulation, to keep the matrix size moderate and prevent modeling error. We introduce a negative sign in front of the trigger or vulnerable state to signify that the dependency is satisfied only if the component is **not** in that state. This notation is analogous to the **NOT-gate** in fault trees. For instance, if component 1, in the scenario above, can affect component 3 only if component 2 is in states 2 or 1, it is efficient to exploit the **NOT** notation, instead of inserting an additional row in each of  $\mathbf{D}_1$  and  $\mathbf{D}_2'$ . Recalling that component 2 has 3 states, state 2 **OR** state 1 is logically equivalent to **NOT** state 3. Hence, the dependency matrices,  $\mathbf{D}_1$  and  $\mathbf{D}_2'$ , become

$$\mathbf{D}_1 = \begin{pmatrix} 3 & 2 & 1 & 2 \\ 3 & 2 & -3 & -3 \end{pmatrix} \quad \mathbf{D}_2' = \begin{pmatrix} -3 & 3 & 3 & 1 \end{pmatrix}.$$

We propose a recursive algorithm to implement the dependency matrices. If  $x_i$  denotes the new/current state of node i, the algorithm is summarized thus.

- i) Define a register, **R**, to hold the affected components, their vulnerable, and target states.
- ii) Using  $D_i$  and  $x_i$ , find all components affected by the state change and update R with elements 2 to 4 of the rows representing the components.
- iii) Select the last row of  $\mathbf R$  and check if its last two elements are equal. This row defines the dependency induced in component  $\omega$  by component i.
- iv) If the response to the query in step (iii) is in the affirmative, designate the equal elements,  $\epsilon$ , delete the last row of  $\mathbf{R}$ , and

a) Using  $\omega$ ,  $\mathbf{D}'_{\omega}$ , and  $x_{\omega}$  as inputs, call steps (i) to (vii), noting that a row in  $\mathbf{D}'_{\omega}$  is affected by the state change only if its first element is  $\epsilon$ .

- b) Continue from step (iii). Else, proceed to step (v).
- v) Force the designated transition as determined in step (iii) and delete the last row of **R**. If the affected node is in standby, and its target state, working, delay, or start-up, initiate its start-up procedure.
- vi) If  $\mathbf{D}_{\omega}$  exists, repeat steps (ii) to (vi), replacing  $\mathbf{D}_{i}$  and  $x_{i}$  with  $\mathbf{D}_{\omega}$  and  $x_{\omega}$ , respectively.
- vii) Repeat steps (iii) to (vi) until **R** is empty, and terminate the procedure.

### III. SYSTEM SIMULATION AND ANALYSIS

The system's operation is imitated by generating random failure events of components and their corresponding repairs. For every component transition, the capacity vector,  $\{c_x^{\{i\}}\}_{M\times 1}$ , of the system is updated and used to deduce the flow, (Y,t), through the output node. At time t = 0, the grid and switchyard nodes are in operation, while the emergency power systems and alternative emergency power systems are in cold standby. LOOP is initiated by setting the grid (for grid centred LOOP) or the switchyard (for switchyard centred LOOP) to its failure state. The next transition parameters of the standby systems are sampled, and the simulation is moved to the earliest transition time, t. Components with next transition time equal to t are identified, the required transitions effected, their next transition times sampled, the new system performance computed, and the next simulation time determined. This cycle of events continues until offsite power is recovered.

Let  $\mu_{\rm old}$  hold the node capacities at the previous system transition,  $\tau$ , the vector of next node transition times, N, the number of simulation samples, and  $\mathbf{S} = \{s_j\}^N$ , the register indicating the occurrence of an SBO. The indicator register,  $\mathbf{S}$ , is such that,  $s_j = 1$  if an SBO occurs in the jth sample, and 0, otherwise. The simulation algorithm is summarized thus.

- i) Initialize the register storing the flow through the output node, set N=1,  $\mathbf{S}=\{\}$ , and define the simulation stopping criterion. The stopping criterion could be the number of LOOP, number of SBO, or convergence of the SBO probability.
- ii) Determine which component will be unavailable due to test or maintenance.
- iii) Set  $s_N = 0$  and  $\tau = {\infty}^M$ , where M is the number of nodes in the system.
- iv) Force LOOP as described earlier, accounting for interdependencies according to the procedures described in Sections II-C1 and II-C2. Remember to sample the next transition parameters after every node transition and update  $\tau$ . See [22] for the procedure for sampling the transition parameters of a multistate node.
- v) Define  $\mu$  using the current states of the nodes, that is,  $\mu = \{c_{x_0}^{\{i\}}\}_{M \times 1}$  and set t = 0,  $\mu_{\text{old}} = \mu$ . vi) Determine  $X_{\text{out}} \mid X_{\text{out}} = (\mathbf{Y}, \mathbf{t})$  and save as a function
- vi) Determine  $X_{\text{out}} \mid X_{\text{out}} = (\mathbf{Y}, \mathbf{t})$  and save as a function of time.

- vii) Set  $s_N = s_N + 1$  if  $X_{\text{out}} = 0$  and determine the next simulation time,  $t = \min(\tau)$ .
- viii) Find nodes with next transition time equal to t. For each node, force the required transition, sample its next transition parameters (except for nodes returning to cold standby), and update  $\mu$  and  $\tau$ .
- ix) Restart nodes returning from repairs if  $X_{\rm out}$ , as previously determined, is less than 1.
- x) If  $\mu_{\text{old}} \neq \mu$ ;

- a) Compute  $X_{\text{out}}$  and set  $s_N = s_N + 1$  if  $X_{\text{out}} = 0$ .
- b) Save  $X_{\text{out}}$  if different from the previous.
- c) Temporarily set the capacity of the switchyard node to 1 if it is in "Shutdown" and calculate the new system flow. If this flow is nonzero, set the switchyard to start-up, sample its next transition parameters, and update  $\tau$ .
- xi) Set  $\mu_{\rm old} = \mu$ ,  $t = \min(\tau)$ , and check if offsite power is recovered.
- xii) Repeat steps (viii) to (xi) until offsite power is recovered. Discard history N if  $s_N = 0$  and set N = N + 1.
- xiii) Repeat steps (ii) to (xii) until the simulation stopping criterion is met, and terminate algorithm.
- xiv) Compute the relevant SBO indices

#### 727 A. SBO Indices: Computation and Relevance

The SBO frequency,  $f_s$ , makes the list of the most informative and desired SBO indices. It defines the expected number of times, per year, an SBO occurs at a plant. If  $p_1$  defines the conditional probability of an SBO given a LOOP occurring at frequency,  $f_l$ , per year, then

$$f_s = p_1 f_l$$

$$p_1 = \frac{\sum (\mathbf{S} > 0)}{N - 1}.$$
(14)

The fraction of  $f_s$  occurring at start-up is deduced from the number of SBO at time 0. This index could be used to assess the efficiency of the start-up procedure, as well as the vulnerability of the generators in cold standby.

The nonrecovery probability,  $\mathbf{r}_1$  (t), defines the likelihood of recovery duration from an SBO accident exceeding a given time. It is computed as detailed in [26], and like  $p_1$ , belongs to the set of inputs to the SBO event tree. Given it defines the unavailability of power at the plant,  $\mathbf{r}_1$  (t) can be directly compared with the reliability of the SBO mitigating mechanism. The outcome of such a comparison would help ascertain the adequacy of the mitigating mechanism. In addition,  $f_s \times \mathbf{r}_1$  (t) yields the frequency of exceedance, a measure of the overall SBO risk at the plant. The quantity also presents a means of assessing the relative effectiveness of multiple recovery responses or operational constraints.

Finally, the conditional probability of a second SBO,  $p_2$ , given an SBO has already occurred is given by

$$p_2 = \frac{\sum (\mathbf{S} > 1)}{\sum (\mathbf{S} > 0)}.$$
 (15)

Knowledge of  $p_2$  may shape the recovery response on the occurrence of a second SBO. For instance, a plant with a large  $p_2$  would require the logistics used in the recovery of the first SBO left in the field and the operations staff kept on high alert. This reduces human error, ensuring a lower nonrecovery probability,  $\mathbf{r}_2(t)$ , of the second SBO.

Generally, the conditional probability,  $p_n$ , of the *n*th SBO given the (n-1)th SBO is expressed as

$$p_n = \frac{\sum (\mathbf{S} > n - 1)}{\sum (\mathbf{S} > n - 2)}.$$
 (16)

If absolute probabilities are required instead, the denominator in (16) is replaced with N-1.

## B. Incorporation Into the Existing Framework

Shown in Fig. 5 is an excerpt from the SBO event tree presented in [1]. Of its 12 headings, only four T(PG), EM, ER1, and ER2 are of relevance to SBO recovery. The first depicts LOOP, and requires the LOOP frequency. The second represents SBO occurrence, and requires the unavailability of the standby power systems. Here, the chain of complicated fault trees in the existing model can be replaced with the conditional SBO probability,  $p_1$ . The last two headings represent offsite and standby power recovery, respectively. These can be merged into one heading, say ac power recovery, and the complicated fault trees replaced with a crisp value read from  $\mathbf{r}_1(t)$ . With these, the core damage frequency induced by the first SBO is computed by solving the event tree, using standard procedure. For the second SBO, the first is regarded the initiating event. The LOOP frequency, therefore, is replaced with  $f_s$ ,  $p_1$  with  $p_2$ , and  $\mathbf{r}_1(t)$ with  $\mathbf{r}_2(t)$ .

# IV. CASE STUDY: AN APPLICATION TO THE MAANSHAN NUCLEAR POWER PLANT IN TAIWAN

The Maanshan plant is a two-unit, 1902 MW, Westinghouse PWR nuclear power plant operated by the Taiwan Power Company. Its offsite power is supplied by six independent sources, four of which are connected to the 345-kV switchyard and the remainder, through the 161-kV switchyard. It is powered through two safety buses, AIE-PB-S01 and BIE-PB-S01, each with a dedicated EDG: DG-A, and DG-B, respectively. A shared EDG, DG-5, connected as shown in Fig. 6 is available as a backup in case any of the dedicated generators is unavailable. In addition to the shared EDGs, are two GTGs, GT1 and GT2, connected via the 161-kV switchyard. These generators form the alternative emergency power system of the plant, each satisfying the demand on both power trains.

During normal plant operation, the safety buses are fed by the main plant generator, G1, via the red lines and the normally closed breakers 19 and 01. On plant shut down, G1 becomes unavailable, and the safety buses are forced to tap power from the 345-kV switchyard (via the blue lines and the normally open breakers 17 and 03) or the 161-kV switchyard (via the green lines and the normally open breakers 15 and 05). When these sources also become unavailable, DG-A and DG-B are automatically started and aligned. DG-5 is manually started and

LOOP	ONSITE POWER FAILURE	REACTOR PROTECTION SYSTEM	RCS	AFW	EMERGENCY DEPRESURI- ZATI			RCP SEAL STAGE 2 INTEGRITY	RCP SEAL STAGE 2 INTEGRITY	OFFSITE POWER RECOVERY	ONSITE POWER RECOVERY
T(PG)	EM	K	Q	L(T)	X(E)	BP1	01	BP2	O2	ER1	ER2

Fig. 5. Excerpt from the SBO event tree showing headings (credit: [1]).

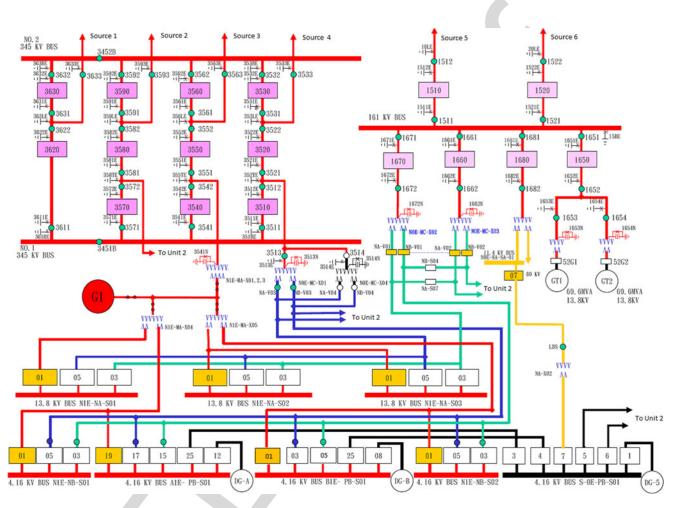


Fig. 6. Layout of the Maanshan nuclear power plant ac distribution system (credit: Dr. S.-K. Chen, NTHU, Taiwan).

aligned by the plant operators on the failure of any of these. The manual start-up and alignment procedure of GT1 and GT2 is initiated when at least 2 out of the 3 EDGs become unavailable. Following their successful start-up, the GTGs take about 30 min to become fully functional.

A probabilistic assessment of the SBO risk of the plant due to grid and switchyard initiated LOOP is required.

#### A. Developing the System and Component Models

Fig. 7 is the simplified schematic of the plant's ac power system, showing all the elements relevant to an SBO. DG-5, though serving only one bus at a time, is assumed connected to both buses in the system's adjacency matrix. This implies, its flow is divided between the buses, contrary to what is obtained in reality. However, since the flows from the two buses are

emptied into the virtual output node, **t**, the total flow from the shared generator is accounted for. As shown, the six grid sources and the two switchyard sources have each been represented by single nodes, as proposed in Section II-B1.

Nodes 1, 7, 8, and 9 are modeled as proposed in Sections II-B and II-B1. The switchyard, on the other hand, is modeled according to Fig. 3, to account for human error during its start-up from shut down. Since DG-A (node 5) and DG-B (node 6) are automatically started following a LOOP, they are not susceptible to human error, and, therefore are modeled as shown in Fig. 8. DG-5, GT1, and GT2, however, require human intervention for their start-up and alignment. Node 10, therefore, is modeled according to Fig. 9 and nodes 3 and 4, according to Fig. 10.

Justifying the values assigned to the state capacities of the generators, recall the system consists of 2 safety buses (m=2) with each of DG-A and DG-B serving only one bus at a time

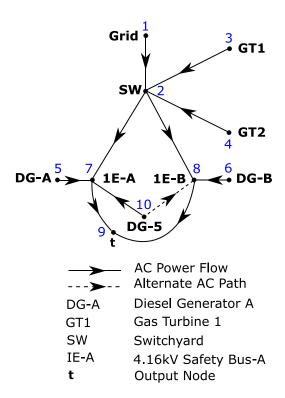


Fig. 7. Simplified schematic of plant's ac distribution system.

 $(n_1=1)$ . Since these generators can, however, fully meet the demand on the bus they serve (u=1), they are assigned a capacity of 0.5 when working, as proposed in Section II-B. The GTGs, on the other hand, can fully serve both buses simultaneously  $(n_1=2)$ , and therefore, have a capacity of 1 when working. From the multistate models, the capacity vector for the main diesel generators, the shared diesel generator, and the GTGs are  $\{0.5,0,0,0,0\}$ ,  $\{0.5,0,0,0,0,0,0\}$ , and  $\{1,0,0,0,0,0,0,0,0\}$ , respectively. Using these parameters in conjunction with Fig. 7, the adjacency matrix of the system is derived as

Given the adjacency matrix, the other parameters of the system flow equations are obtained as described in Section II-A, where  $\mathbf{s} = \{1, 3, 4, 5, 6, 10\}$  and  $\mathbf{t} = 9$ . Fig. 11 is the system's graph model showing the maximum flow along each link, derived from the adjacency matrix and the maximum node capacities.

Component Reliability Data: Though realistic, the data used do not represent the actual data for the Maanshan plant. They were, however, assumed with the view to reflecting the reliability

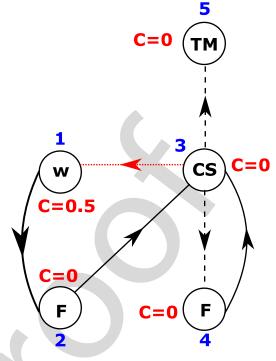


Fig. 8. Multistate model for the main diesel generators (DG-A and DG-B).

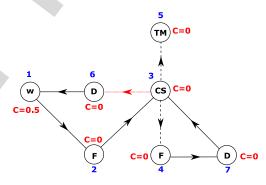
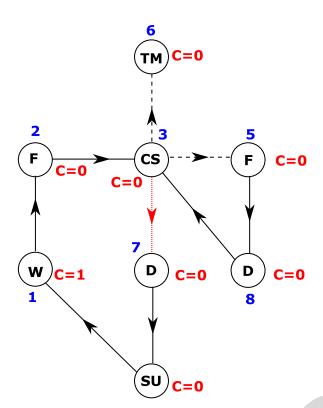


Fig. 9. Multistate model for the shared diesel generator (DG-5).

data used in Volumes 1 and 2 of the NUREG/CR-6890 report 850 (see [1] and [2]). 851

The repair times for the six grid sources are lognormally distributed with means and corresponding standard deviations defined by  $\{8.99, 11.84, 8.24, 10.25, 9.61, 9.15\}$  and  $\{6.71, 4.83, 4.05, 6.61, 1.92, 5\}$ , respectively. Similarly, switchyard repair times are lognormally distributed, with  $\{8, 10.41\}$  and  $\{5.83, 2.5\}$ , respectively, being the sets of means and corresponding standard deviations for the two switchyards. The effective repair distributions for the grid and switchyard nodes are modeled according to the proposal in Section II-B1, as shown in Figs. 12 and 13, respectively.

All five standby generators are assumed to have a start-up failure probability of  $1.756 \times 10^{-2}$ . Also, the human errors associated with the failure to complete the start-up procedures for GT-5 and the switchyard are assumed equal but one-sixth of those for GT1 and GT2. Table I defines the probability of



Multistate model for the GTGs (GT1 and GT2).

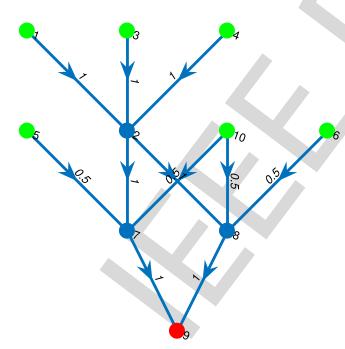


Fig. 11. Full system graph model showing maximum flow along links.

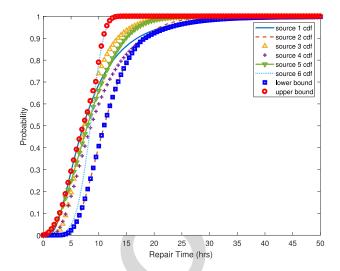
867

868

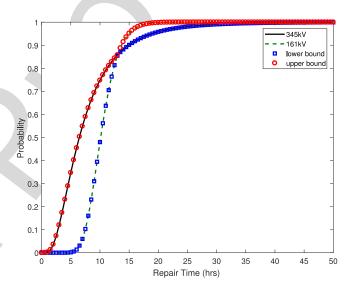
869

871

the operators not completing the start-up of the GTGs within selected times. Using the procedure proposed in Section II-B3, the parameters defining transitions  $7 \rightarrow 4$  and  $5 \rightarrow 8$  of the GTGs were obtained. The same procedure was used to obtain the parameters for transitions  $6 \rightarrow 1$  and  $4 \rightarrow 7$  of DG-5 and transition  $4 \rightarrow 1$  of the switchyard. These and the parameters



Effective repair cdf for multiple grid sources.



Effective repair cdf for multiple switchyard nodes.

for the remaining transitions are presented in Table II. The column,  $U_{\rm tm}$ , defines the unavailability due to test/maintenance of the generators. The CCF parameters are defined by a set in 875 which each element represents the probability of a certain number of components being involved in any failure event initiated by the component. The number of components is determined by the index of the element in the set. For instance, from the 879 table, the probability that the start-up failure of any of the main diesel generators leads to the failure of the other generator is 0.021. This implies a total of two component failures, explaining why the probability value is the second element of the set (see Section II-C1 for details). Transition  $4 \rightarrow 1$  of the GTGs depicts their start-up duration, which as we are told in Section IV, takes 30 min, explaining why it is assigned a deterministic 0.5 h.

# B. Representing Component Interdependencies

The first and easily recognizable form of interdependency in 888 the system is CCF, where the failure of a generator could trigger

 $\begin{tabular}{l} TABLE\ I\\ HUMAN\ ERROR\ PROBABILITIES\ FOR\ GT1\ AND\ GT2\\ \end{tabular}$ 

Time (h)	1	2	3	4	6	7	8	10
Probability	$2.07 \times 10^{-1}$	$2.07 \times 10^{-2}$	$3 \times 10^{-3}$	$3 \times 10^{-4}$	$2 \times 10^{-4}$	$1 \times 10^{-4}$	$1 \times 10^{-5}$	$1 \times 10^{-5}$

TABLE II COMPONENT RELIABILITY DATA

Component	Transition	Distr	ribution	$U_{tm}$	CCF Parameters		
Component	11 ansition	Type	Parameters	$U_{tm}$	Start-up Failure	Running Failure	
	1-2	Weibull	(100,1.24)				
DG-A & DG-B	2-3	Lognormal	(6.42,2)	0.009	$\{0.979, 0.021\}$	$\{0.972, 0.028\}$	
	4-3	Lognormal	(5,1.2)				
	4-1	deterministic	0.5				
	4-2	Weibull	(200,1.5)				
	2-3	Lognormal	(5,2)				
GT1 &GT2	8-3	Lognormal	(7,1.8)	0.0099	$\{0.959, 0.041\}$	$\{0.962, 0.038\}$	
	1-2	Weibull	(100,1.05)				
	7-4	Weibull	(0.2872, 0.8194)				
	5-8	Weibull	(0.2872, 0.8194)				
	1-2	Weibull	(100,1.24)				
	2-3	Lognormal	(6.42,2)				
DG-5	7-3	Lognormal	(5,1.2)				
	6-1	Weibull	(0.197, 0.7467)				
	4-7	Weibull	(0.197, 0.7467)				
Switchyard	4-1	Weibull	(0.197, 0.7467)				
Switchyard	2-1	See Fig. 13					
Grid	2-1	See	Fig. 12				

TABLE III CCG DEFINITION

CCG	Description	Att	ributes
CCG	Description	Designation	Value
		ρ	$\{5,6\}$
1	Emergency Diesel Generator failure to start	$\theta$	$\{0.979, 0.021\}$
1	Emergency Dieser Generator fantire to start	$\beta_1$	4
		$eta_2$	3
		ρ	$\{5,6\}$
2	Emergency Diesel Generator failure to run	$\theta$	$\{0.972, 0.028\}$
		$\beta_1$	2
		$eta_2$	1
		$\rho$	$\{3,4\}$
3	Gas Turbine Generator failure to start	$\theta$	$\{0.959, 0.041\}$
	Gas Turbine Generator failure to start	$\beta_1$	4
		$\beta_2$	3
	Gas Turbine Generator failure to run	ρ	$\{3,4\}$
4		$\theta$	$\{0.962, 0.038\}$
	Gus furome Generator fundre to fun	$\beta_1$	2
		$\beta_2$	$\{1,4\}$

the almost instantaneous failure of another generator. This type of interdependency is modeled according to the CCF model presented in Section II-C1. DG-A and DG-B, as we know, are of the same design and model, different from the make of DG-5. Therefore, while the former are susceptible to CCF, DG-5 is immune. Similarly, GT1 and GT2 are susceptible to CCF, giving rise to four CCGs, as defined in Table III. The table is developed from the CCF parameters in Table II in conjunction with the CCF model proposed in Section II-C1. CCG 1, for instance, represents the CCF due to the start-up failure of any of the main diesel generators. Since these generators are denoted as nodes 5 and 6 in the system,  $\rho$ , the set of components in the CCG is defined as  $\{5,6\}$ . Now, as shown in Fig. 8, the start-up

failure of DG-A or DG-B is denoted by state 4. Also, the other generator could only be affected by this event if it is in cold standby (state 3) at the time of occurrence. This explains why  $\beta_1$  and  $\beta_2$  are assigned the values, 4 and 3, respectively. The parameters for CCG 2 to 4 are derived in a similar fashion.

The other form of interdependency, like the grid failure necessitating the start-up of the standby generators or the failure of GT-5 forcing the start-up of the GTGs, is a little more subtle and difficult to deduce. It requires a good knowledge of the operating principle of the system and cannot be modeled by the CCF model. For this, the cascading failure model proposed in Section II-C2 is invoked. To ensure the reproducibility of the case study, the step-by-step procedure for developing the

916 dependency matrices have been shown by recreating the se-917 quence of events following a LOOP.

1) Let us assume the occurrence of the initiating event (LOOP), due to the failure of the grid (node 1). As already stated at the beginning of Section IV, the main diesel generators, A (node 5) and B (node 6), are restarted from cold standby. This is accounted for by the first two rows of the dependency matrix,  $\mathbf{D}_1$ . However, if the main generators are not in cold standby, maybe

$$\mathbf{D}_{1} = \mathbf{D}_{2} = \begin{pmatrix} 2 & 5 & 3 & 1 \\ 2 & 6 & 3 & 1 \\ 2 & 5 & -3 & -3 \\ 2 & 6 & -3 & -3 \end{pmatrix}$$

$$\mathbf{D}_{5}' = \mathbf{D}_{6}' = \begin{pmatrix} -3 & 10 & 3 & 6 \\ -3 & 10 & -3 & -3 \end{pmatrix}$$

$$\mathbf{D}_{10}' = \begin{pmatrix} -3 & 3 & 3 & 7 \\ -3 & 4 & 3 & 7 \end{pmatrix}$$
(17)

due to test/maintenance or failure, the shared standby generator, DG-5 (node 10), is restarted. Recalling the concept of joint dependency discussed in Section II-C2, the joint dependency between the grid and DG-5 can be deduced. Here, the main generators are the intermediate nodes, since they dictate whether or not to start the shared generator. This behavior is jointly represented by the last two rows of  $D_1$  and the first row of  $D_5'$  in (17). Again, if the shared generator too is unavailable (i.e., it is not in cold standby), the GTGs, GT1 (node 3) and GT2 (node 4), are restarted (see Fig. 10). This attribute is jointly represented by  $\mathbf{D}'_{10}$  and the last row of  $\mathbf{D}'_{5}$ . If, however, the GTGs are not in cold standby on arrival of their start-up signal, no action is taken. This is due to the fact that the signal signifies the unavailability of all the standby sources at the plant.  $\mathbf{D}_5'$  and  $\mathbf{D}_6'$  are equal because nodes 5 and 6 produce the same effect on the shared generator when unavailable for start-up. Similarly,  $D_1$  and  $D_2$  are equal, as the response of the standby systems is the same for grid and switchyard failures

$$\mathbf{D}_5 = \begin{pmatrix} 2 & 6 & 3 & 1 \\ 4 & 6 & 3 & 1 \\ 2 & 6 & -3 & -3 \\ 4 & 6 & -3 & -3 \end{pmatrix}. \tag{18}$$

2) DG-A (node 5) fails to start or starts but fails to run (see Fig. 2). The system will first check if DG-B (node 6) is available for start-up and initiate its start up, if available. This behavior is defined by the first two rows of D<sub>5</sub>, as shown in (18). The effect of the unavailability of DG-B on arrival of its start-up signal has already been defined in scenario 1) (see the last row of D<sub>1</sub>). This representation is adapted to account for the case when DG-A fails to start or run and DG-B is unavailable for start-up, in the last two

rows of  $\mathbf{D}_5$  [see (18)]

$$\mathbf{D}_6 = \begin{pmatrix} 2 & 5 & 3 & 1 \\ 4 & 5 & 3 & 1 \\ 2 & 5 & -3 & -3 \\ 4 & 5 & -3 & -3 \end{pmatrix}. \tag{19}$$

- 3) Similarly, DG-B (node 6) fails to start or starts but fails to run (see Fig. 8). The system will first check if DG-A (node 5) is available, and initiate its start-up. The ensuing sequence of events is similar to that in scenario 2). Hence, the dependency matrix is as obtained in (19).
- 4) DG-5 in cold standby fails to start or starts but fails to run (see Fig. 9). In this case, any repaired EDG is restarted first, otherwise, the GTG are restarted. The ensuing possible sequence of events are already covered by scenarios (1)–(3), and it is, therefore, recommended to not explicitly redefine these in D<sub>10</sub>, for simplicity. It is deducible that the failure of DG-5 induces the same response sequence as grid or switchyard failure. Therefore, recreating a LOOP event accounts for the failure of DG-5. Hence

$$\mathbf{D}_{10} = egin{pmatrix} 2 & 1 & 2 & 2 \ 2 & 2 & 2 & 2 \ 4 & 1 & 2 & 2 \ 4 & 2 & 2 & 2 \end{pmatrix} \quad \mathbf{D}_1' = \mathbf{D}_1 \quad \mathbf{D}_2' = \mathbf{D}_2.$$

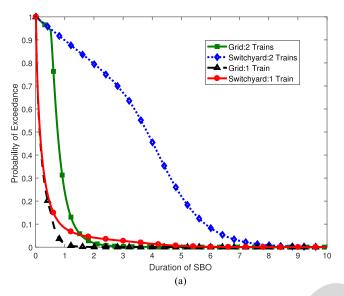
- 5) GT1 (node 3) starts up successfully and enters the start-up state (see Fig. 10). Recall, states 7 and 8 account for the time taken by the operator to initiate the start-up of the generator. However, since both GT1 and GT2 (node 4) are in the same location, they are exposed to equal delays. Hence, the transitions, 7 → 4 and 5 → 8, of GT1 and GT2 are equal. To ensure the satisfaction of this constraint, when GT1 enters state 4, GT2 too is forced to state 4 if it is in state 7 or state 8, if it is in state 5. Similarly, when GT1 enters state 8, GT2 is forced to state 8 if it is in state 5 or state 4 if it is in state 7. This behavior is expressed by the first four rows of D<sub>3</sub>, as shown in (20).
- 6) GT2 (node 4) starts up successfully and enters the start-up state (see Fig. 10). This scenario has the same effect on GT1 (node 3) as scenario (v) has on GT2. Therefore, the ensuing sequence of events is accounted for by the first four rows of D<sub>4</sub>, as shown in the following:

$$\mathbf{D}_{3} = \begin{pmatrix} 8 & 4 & 5 & 8 \\ 8 & 4 & 7 & 4 \\ 4 & 4 & 5 & 8 \\ 4 & 4 & 7 & 4 \\ 2 & 4 & 3 & 7 \\ 2 & 4 & 2 & 2 \\ 2 & 4 & 8 & 8 \\ 2 & 4 & 5 & 5 \\ 2 & 4 & 6 & 6 \end{pmatrix} \quad \mathbf{D}_{4} = \begin{pmatrix} 8 & 3 & 5 & 8 \\ 8 & 3 & 7 & 4 \\ 4 & 3 & 5 & 8 \\ 4 & 3 & 7 & 4 \\ 2 & 3 & 3 & 7 \\ 2 & 3 & 2 & 2 \\ 2 & 3 & 8 & 8 \\ 2 & 3 & 5 & 5 \\ 2 & 3 & 6 & 6 \end{pmatrix}$$

$$\mathbf{D}_{3}' = \mathbf{D}_{4}' = \begin{pmatrix} 2 & 1 & 2 & 2 \\ 5 & 1 & 2 & 2 \\ 6 & 1 & 2 & 2 \\ 8 & 1 & 2 & 2 \end{pmatrix}. \tag{20}$$

TABLE IV
SUMMARY OF THE STATIC SBO INDICES OBTAINED

LOOP Type	$p_1$	$f_s$ (per yr)	$p_2$	% of SBO at Start-Up	Simulation Samples
Grid	0.0033	$6.18 \times 10^{-3}$	0.0022	29.23	$1 \times 10^{8}$
Switchyard	0.0035	$3.65 \times 10^{-3}$	0.0153	27.97	$4.5 \times 10^{7}$



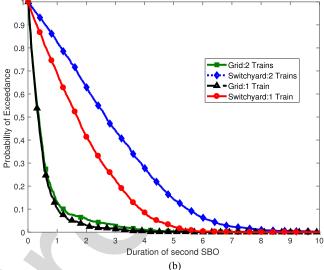


Fig. 14. Probability of SBO duration exceedance.

987

988

989

990

991

992

993

994

995

996

997

998

999

- 7) GT1 fails to run. GT2 is restarted, if it is available for start-up, otherwise the system checks whether or not the failed diesel generators have been repaired. The first case is represented by the fifth row of D<sub>3</sub>, as shown in (20). The sequence of events involved in the second case is similar to the events following a LOOP. Therefore, a LOOP scenario is recreated, as shown in the last four rows of D<sub>3</sub> and D'<sub>4</sub>. States 1, 4, and 7 have been left out of the possible GT2 states to necessitate the second case because, they mean either GT2 is already in operation (state 1), or on the verge of operation (states 4 and 7).
- 8) Similarly, GT2 failure to run produces the same effect on GT1 and the diesel generators, as in scenario (7). The ensuing sequence of events is defined by  $\mathbf{D}_4$  and  $\mathbf{D}_3'$ .

We have not considered the sequence of events following the failure of the GTGs to start because, being the last standby sources to be called into operation, their start-up failure means the unavailability of the other standby sources.

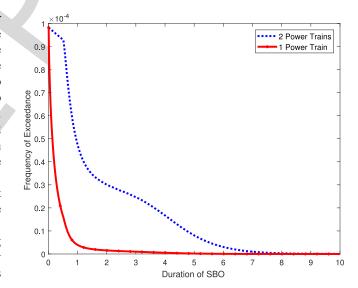


Fig. 15. Composite frequency of first SBO exceedance.

#### 1004 C. Results and Discussions

The proposed framework is implemented in the open source uncertainty quantification toolbox, OpenCOSSAN [27], [28], 1007 and used to quantify the SBO risk at the Maanshan nuclear 1008 power plant. For a grid and switchyard LOOP frequency of 1009  $1.86\times10^{-2}$  and  $1.04\times10^{-2}$  per/year respectively, the case 1010 study was analyzed on a 2.5-GHz, E5-2670 v2 Intel Xeon CPU. 1011 A 5% coefficient of variation was imposed on the conditional 1012 probability of SBO as the simulation convergence criterion. The 1013 analysis took about 3 h, and the results yielded are summarized

in Table IV, Fig. 14, and Fig. 15. The probability of exceedance 1014 gives a measure of the likelihood of nonrecovery from the SBO 1015 within a given time. The composite frequency of exceedance is 1016 the sum of the frequencies of exceedance yielded by the two 1017 LOOP categories. 1018

As shown in Table IV, the probability of an SBO given a 1019 LOOP is almost the same for both LOOP categories. The slight 1020 difference is due to the fact that the GTG are unusable during 1021 switchyard centred LOOP. Their effect, however, is prominent in 1022

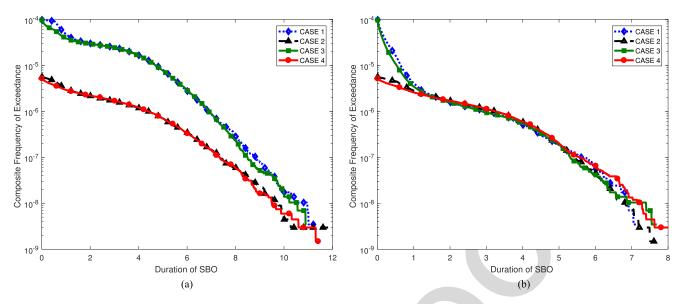


Fig. 16. Comparison of composite frequencies of exceedance. (a) Composite frequencies of exceedance when a minimum of two powertrains are required for power recovery; (b) Composite frequencies of exceedance when one power train is sufficientfor power recovery.

1023 mitigating the second SBO. The nonrecovery probability from 1024 an SBO, as shown in Fig 14, is expressed as the nonrecovery 1025 likelihood as a function of time and number of safety buses. 1026 The overall SBO risk at the plant is defined by the composite 1027 frequency of exceedance, as shown in Fig. 15.

As a way of verifying the convergence of the simulation, the product of  $p_1$  and the fraction of SBO at start-up, should match the probability,  $p_0$ , of the emergency power system being unavailable at time 0. Bear in mind that GT-5 and the GTG have no influence on  $p_0$ , as a result of the delays characterizing their start-up. Therefore, the emergency power system is unavailable at start-up only if DG-A (or DG-B) is unavailable to test/maintenance and DG-B (or DG-A) fails to start or both are not in test/maintenance but fail to start. If  $U_{\rm tm}$  is the unavailability due to test/maintenance of DG-A and DG-B and unavailability due to test/maintenance of DG-A is obtained as

$$p_0 = U_{\text{tm}} (p_s + p_s) + (1 - U_a) p_s^2$$

$$p_0 = 2U_{\text{tm}} p_s + (1 - U_{\text{tm}}) p_s^2.$$
(21)

1039 Substituting the required values in (21), an error of 3.17% is 1040 realized for grid LOOP and 4.7%, for switchyard LOOP. Since 1041 the error in each case is not in excess of 5%, the convergence of 1042 the simulation is verified.

Ensuring an enhanced risk insight, the system was reanalyzed for three additional scenarios as follows.

- 1) Case 2: No delays in the start-up of DG-5. This implies, the effects of human error are removed.
- 2) Case 3: GTG start-up is simultaneous with DG-A and DG-B. The generators, however, are kept in warm standby after start-up.
- 3) Case 4: A combination of Case 2 and Case 3.

1045

1046

1047

1048

1049

1050

Case 1 represents the scenario already analyzed, and the results for the four cases are summarized in Figs. 16 to 18 (please note the composite frequencies in Figs. 16(a) and (b) are

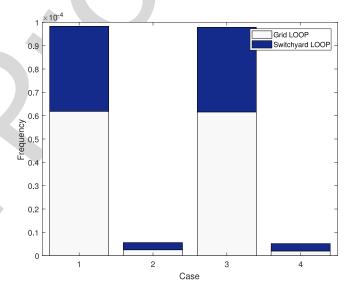
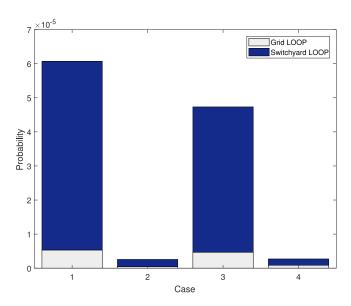


Fig. 17. Comparison of SBO frequencies.

expressed on a log-scale). We have used absolute, instead of 1054 conditional probabilities in Fig. 18, to ensure uniformity.

The following risk insights are inferred by the outcome of the 1056 case study.

- 1) As shown in Fig. 14, SBOs induced by switchyard failures are more difficult to recover from and, therefore, contribute more to the overall SBO risk at the plant. In this 1060
  light, feasible reliability improvement programs should be 1061
  designed to ensure the high reliability of the switchyard. 1062
  Such a reliability program should be complemented by an 1063
  efficient repair policy to keep the nonrecovery probability 1064
  low. 1065
- 2) The GTGs are the only difference between the recovery 1066 durations of grid and switchyard LOOP. These generators, 1067 therefore, are very instrumental to mitigating SBO risks 1068 at the plant, and their availability should be kept high. 1069



Comparison of second SBO probabilities.

1071

1072

1073

1074

1075

1076

1077

1078

1079

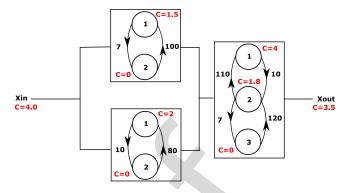
3) Automating the start-up of DG-5 and initiating the startup of the GTG just after LOOP guarantees an improved resilience to SBO, as endorsed by Figs. 16 to 18. However, starting the GTG simultaneously with the EDG brings with it additional costs, borne from fuel consumption and maintenance. This decision, therefore, should be preceded by a robust cost-benefit analysis. In fact, under economic constraints, it is prudent to automate the start-up of DG-5 only, as the difference between the outcomes yielded by Case 2 and Case 4 is only just slight.

In this case study, we have ignored the explicit sensitivity and 1080 importance analyses of the individual components, since these quantities can be achieved even with the existing techniques.

#### V. CONCLUSION 1083

SBO accidents, though a rare occurrence, can have devastat-1084 ing consequences on a nuclear power plant's ability to achieve 1085 and maintain safe shut down. Consequently, the plant's capability to cope and recover from such occurrences makes a key 1087 input to its probabilistic risk assessment model. 1088

In this paper, we have proposed an intuitive simulation frame-1089 work to model a nuclear power plant's recovery from SBO acci-1090 dents. The framework provides a simple means of defining the 1091 complex interdependencies that often characterize the opera-1092 tion of practical engineering systems, and therefore, applicable 1093 without unrealistic assumptions. This attribute, coupled with 1094 its ability to intuitively tolerate the multistate behavior of the 1095 system's building block, distinguishes it from the existing ap-1096 proaches. Its applicability has been demonstrated by modeling 1097 1098 the SBO recovery of a pressurized water reactor, providing an informed insight into its SBO risks. The proposed approach was 1099 able to fully model the dynamic behavior of the power system 1100 and provide valuable insights on the SBO risk at the plant. The nonrecovery probability curve obtained, for instance, can be ab-1103 sorbed into the existing probabilistic risk assessment models,



Structure of a three-component pipe network

getting rid of laborious fault trees. Since this curve also depicts 1104 the unavailability of ac power, it can be directly compared with 1105 the reliability of the plant's SBO coping mechanism, providing 1106 an easier means of determining the need for their reliability improvement. It also helps ascertain the adequacy of the plant's 1108 SBO recovery capability, without revisiting the entire model. 1109 A key desirable feature of the proposed framework is its wide 1110 applicability, even to nonnuclear applications.

In spite of their well-documented limitations relative to the 1112 proposed framework, the existing static fault tree-based mod- 1113 els still possess desirable attributes that give them an edge in 1114 importance, sensitivity, and uncertainty analyses. With this in 1115 mind, the proposed framework has been developed with the 1116 view to complementing their applicability, instead of serving 1117 as an explicit replacement. We have, therefore, included a clear 1118 description of how its output can be incorporated into these 1119 models. The framework, in addition, has been implemented in 1120 the open-source uncertainty quantification toolbox developed 1121 at the Institute for Risk and Uncertainty (see [27] and [28]), 1122 thereby rendering it readily available.

The multistate model and dependency matrices proposed cre- 1124 ate the foundation for the incorporation of additional dynamic 1125 considerations. Such considerations as the optimal number of 1126 maintenance teams on-site, EDG failure during cold standby, 1127 optimal inspection interval, and the availability of spares are 1128 a possibility. Efforts are underway to extend the framework to 1129 these considerations, other LOOP categories, and incorporate 1130 epistemic uncertainties.

#### APPENDIX 1132

1131

This section is introduced with the view to providing a de- 1133 tailed example of how the linear programing problem is formu- 1134 lated, stating the exact values of the relevant parameters. The 1135 goal is to enable readers to grasp, fully, the concept proposed in 1136 this paper, as well as provide a benchmark for validating their 1137 implementation of this concept.

Consider the three-component pipeline shown in Fig. 19, 1139 adapted from [22]. A maximum of four tons of oil could 1140 be pumped from the source,  $X_{\rm in}$ , to the output,  $X_{\rm out}$ , where 1141 the demand is fixed at 3.5 tons. The state-space of each of 1142 the other components is shown, with the number beside each 1143

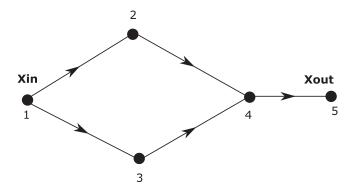


Fig. 20. Network model of pipe network.

1144 state denoting the capacity of the component in that state. The 1145 equivalent graph model of the system is shown in Fig. 20. Notice 1146 the two extra nodes, 1 and 5, representing the source and output, 1147 respectively. The available information is sufficient to formulate 1148 the linear programing problem and derive its parameters. The 1149 first step is to define the adjacency matrix, since all the other 1150 parameters depend on it. From Fig. 20, the adjacency matrix, 1151  $\bf A$ , is obtained as

$$\mathbf{A} = \begin{pmatrix} 0 & 1 & 1 & 0 & 0 \\ 0 & 0 & 0 & 1 & 0 \\ 0 & 0 & 0 & 1 & 0 \\ 0 & 0 & 0 & 0 & 1 \\ 0 & 0 & 0 & 0 & 0 \end{pmatrix}.$$

1152 The next task is to deduce the edge and incidence matrices, e 1153 and  $\Gamma$ , respectively. They are obtained thus

$$\mathbf{e} = \begin{pmatrix} 1 & 2 \\ 1 & 3 \\ 2 & 4 \\ 3 & 4 \\ 4 & 5 \end{pmatrix} \quad \mathbf{\Gamma} = \begin{pmatrix} 1 & 1 & 0 & 0 & 0 \\ -1 & 0 & 1 & 0 & 0 \\ 0 & -1 & 0 & 1 & 0 \\ 0 & 0 & -1 & -1 & 1 \\ 0 & 0 & 0 & 0 & -1 \end{pmatrix}$$

1154 With A, e, and  $\Gamma$  known, the linear programing problem is 1155 formulated as follows.

1) At time 0, all the components are in their best performance state. The inequality constraint, therefore, is expressed as

$$\begin{pmatrix} 1 & 1 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 \\ 0 & 1 & 0 & 0 & 0 \\ 0 & 0 & 1 & 1 & 0 \\ 0 & 0 & 0 & 0 & 1 \end{pmatrix} \begin{pmatrix} X_{12} \\ X_{13} \\ X_{24} \\ X_{34} \\ X_{45} \end{pmatrix} \le \begin{pmatrix} 4.0 \\ 1.5 \\ 2 \\ 4 \\ 3.5 \end{pmatrix}.$$

1158 2) The equality constraint is expressed as

$$\begin{pmatrix} -1 & 0 & 1 & 0 & 0 \\ 0 & -1 & 0 & 1 & 0 \\ 0 & 0 & -1 & -1 & 1 \end{pmatrix} \begin{pmatrix} X_{12} \\ X_{13} \\ X_{24} \\ X_{34} \\ X_{45} \end{pmatrix} = \begin{pmatrix} 0 \\ 0 \\ 0 \end{pmatrix}.$$

3) The bounds on the flow through the edges are

$$\mathbf{lb} = \begin{pmatrix} 0 \\ 0 \\ 0 \\ 0 \\ 0 \end{pmatrix} \qquad \mathbf{ub} = \begin{pmatrix} 1.5 \\ 2 \\ 1.5 \\ 2 \\ 3.5 \end{pmatrix}.$$

4) The objective function is expressed as

$$m{\Psi} = egin{pmatrix} -1 & -1 & 0 & 0 & 0 \end{pmatrix} egin{pmatrix} X_{12} \ X_{13} \ X_{24} \ X_{34} \ X_{45} \end{pmatrix}.$$

#### ACKNOWLEDGMENT

The authors are grateful to Dr. S.-K. Chen and team, of the 1162 National Tsing Hua University in Taiwan, for their invaluable 1163 contribution.

#### REFERENCES 1165

1159

1160

- S. A. Eide, C. D. Gentillon, T. E. Wierman, and D. M. Rasmuson, 1166 "Reevaluation of station blackout risk at nuclear power plants," Tech. Rep. 1167 NUREG/CR-6890, U.S. Nuclear Regulatory Commission, Rockville, 1168 MD, USA, vol. 2, 2005. [Online]. Available: https://www.nrc.gov/ 1169 docs/ML0602/ML060200479.pdf
- [2] S. A. Eide, C. D. Gentillon, T. E. Wierman, and D. M. Rasmuson, 1171
   "Reevaluation of station blackout risk at nuclear power plants," Tech. Rep. 1172
   NUREG/CR-6890, U.S. Nuclear Regulatory Commission, Rockville, 1173
   MD, USA, vol. 1, 2005. [Online]. Available: https://www.nrc.gov/ 1174
   docs/ML0602/ML060200477.pdf
- [3] M. Čepin, "Event Tree Analysis," in Assessment of Power System Reliability: Methods and Applications. London, U.K.: Springer London, 2011, 1177 pp. 89–99.
- [4] W. E. Vesely, F. F. Goldberg, N. H. Roberts, and D. F. Haasl, "Fault 1179 tree handbook," Tech. Rep. NUREG/CR-0492, U.S. Nuclear Regula- 1180 tory Commission, Rockville, MD, USA, 1981. [Online]. Available: 1181 https://www.nrc.gov/docs/ML1007/ML100780465.pdf
- [5] M. Čepin, "Fault tree analysis," in Assessment of Power System Reliability: 1183
   Methods and Applications. London, U.K.: Springer, 2011, pp. 61–87.
   1184
- [6] E. Ruijters and M. Stoelinga, "Fault tree analysis: A survey of the state-of-the-art in modeling, analysis and tools," *Comput. Sci. Rev.*, vol. 15, 1186 pp. 29–62, 2015.
- [7] W. E. Vesely, M. Stamatelatos, J. Dugan, J. Fragola, J. Minarick, 1188 and J. Railsback, "Fault tree handbook with aerospace applica-1189 tions," Version 1.1, NASA Office of Safety and Mission As-1190 surance, Washington, DC, USA, 2002. https://www.hq.nasa.gov/1191 office/codeq/doctree/fthb.pdf
- [8] F. I. Khan and S. Abbasi, "Analytical simulation and {PROFAT} ii: A 1193 new methodology and a computer automated tool for fault tree analysis in 1194 chemical process industries," *J. Hazardous Mater.*, vol. 75, no. 1, pp. 1–27, 1195 2000.
- [9] S. K. Shin and P. H. Seong, "Review of various dynamic modeling methods and development of an intuitive modeling method for dynamic systems," 1198
   Nucl. Eng. Technol., vol. 40, no. 5, pp. 375–386, 2008.
- [10] B. Kaiser, C. Gramlich, and M. Förster, "State/event fault trees: A safety analysis model for software-controlled systems," *Rel. Eng. Syst. Safety*, 1201 vol. 92, no. 11, pp. 1521–1537, 2007.
- Z. Zhou and Q. Zhang, "Model event/fault trees with dynamic uncertain 1203 causality graph for better probabilistic safety assessment," *IEEE Trans.* 1204 *Rel.*, vol. 66, no. 1, pp. 178–188, Mar. 2017.
- [12] A. Bobbio, L. Portinale, M. Minichino, and E. Ciancamerla, "Improving the analysis of dependable systems by mapping fault trees into 1207 Bayesian networks," *Rel. Eng. Syst. Safety*, vol. 71, no. 3, pp. 249–260, 1208 2001.

1279

1293

1294

- 1210 [13] M. Čepin and B. Mavko, "A dynamic fault tree," Rel. Eng. Syst. Safety, 1211 vol. 75, no. 1, pp. 83-91, 2002.
- 1212 [14] J. B. Dugan, S. J. Bavuso, and M. A. Boyd, "Dynamic fault-tree models for fault-tolerant computer systems," IEEE Trans. Rel., vol. 41, no. 3, 1213 1214 pp. 363-377, Sep. 1992.
- 1215 [15] J. B. Dugan, S. J. Bavuso, and M. A. Boyd, "Fault trees and markov 1216 models for reliability analysis of fault-tolerant digital systems," Rel. Eng. Syst. Safety, vol. 39, no. 3, pp. 291-307, 1993. 1217
- 1218 [16] K. D. Rao, V. Gopika, V. S. Rao, H. Kushwaha, A. Verma, and A. Srividya, 1219 "Dynamic fault tree analysis using Monte Carlo simulation in probabilistic safety assessment," Rel. Eng. Syst. Safety, vol. 94, no. 4, pp. 872-883, 1220 1221 2009.
- 1222 L. Meshkat, J. B. Dugan, and J. D. Andrews, "Dependability analysis of 1223 systems with on-demand and active failure modes, using dynamic fault 1224 trees," IEEE Trans. Rel., vol. 51, no. 2, pp. 240-251, Jun. 2002.
- [18] J. B. Dugan, K. J. Sullivan, and D. Coppit, "Developing a low-cost high-1225 quality software tool for dynamic fault-tree analysis," IEEE Trans. Rel., 1226 1227 vol. 49, no. 1, pp. 49-59, Mar. 2000.
- C. Y. Huang and Y. R. Chang, "An improved decomposition scheme for 1228 1229 assessing the reliability of embedded systems by using dynamic fault 1230 trees," Rel. Eng. Syst. Safety, vol. 92, no. 10, pp. 1403-1412, 2007.
- L. F. Rocha, C. L. T. Borges, and G. N. Taranto, "Reliability evaluation of 1231 1232 active distribution networks including islanding dynamics," IEEE Trans. Power Syst., vol. 32, no. 2, pp. 1545-1552, Mar. 2017. 1233
- 1234 [21] H. Lei and C. Singh, "Non-sequential Monte Carlo simulation for cyber-1235 induced dependent failures in composite power system reliability eval-1236 uation," IEEE Trans. Power Syst., vol. 32, no. 2, pp. 1064-1072, Mar. 1237
- [22] H. George-Williams and E. Patelli, "A hybrid load flow and event driven 1238 1239 simulation approach to multi-state system reliability evaluation," Rel. Eng. 1240 Syst. Safety, vol. 152, pp. 351-367, 2016.
- [23] H. George-Williams and E. Patelli, "Maintenance strategy optimization for 1241 1242 complex power systems susceptible to maintenance delays and operational 1243 dynamics," IEEE Trans. Rel., vol. 66, no. 4, pp. 1309–1330, Dec. 2017.
- 1244 [24] H. George-Williams, M. Lee, and E. Patelli, "A framework for power recovery probability quantification in nuclear power plant station blackout sequences," in *Proc. Probabilistic Safety Assessment Manage. Conf.*, 2016, vol. 13. [Online]. Available: 1245 1246 1247 1248 http://iapsam.org/PSAM13/program/index4.php.htm
- 1249 A. Mosleh, D. M. Rasmuson, and F. M. Marshall, "Guidelines on model-1250 ing commom-cause failures in probabilistic risk assessment," Tech. Rep. NUREG/CR-5485, U.S. Nuclear Regulatory Commission, Rockville, 1251 1252 MD. USA, 1998.
- [26] H. George-Williams and E. Patelli, "Efficient availability assessment of 1253 1254 reconfigurable multi-state systems with interdependencies," Rel. Eng. Syst. 1255 Safety, vol. 15, pp. 431–444, 2017.
- E. Patelli, "COSSAN: A multidisciplinary software suite for uncertainty 1256 1257 quantification and risk management," in Handbook of Uncertainty Quantification. New York, NY, USA: Springer, 2017, pp. 1-69. 1258
- 1259 E. Patelli, M. Broggi, M. D. Angelis, and M. Beer, "Opencossan: An 1260 efficient open tool for dealing with epistemic and aleatory uncertainties," in Proc. 2nd Int. Conf. Vulnerability Risk Anal. Manag. 6th Int. Symp. 1261 1262 Uncertainty Modeling Anal., 2014, pp. 2564-2573. [Online]. Available: http://dx.doi.org/10.1061/9780784413609.258 1263

Hindolo George-Williams received the B.Eng.(Hons.) degree in electri- 1264 cal/electronic engineering from the University of Sierra Leone, Freetown, Sierra 1265 Leone, in 2010, and the M.Sc.(Eng.) degree in energy generation from the University of Liverpool, Liverpool, U.K., in 2013. He is currently working toward 1267 the dual Ph.D. degree with the University of Liverpool and the National Tsing 1268 Hua University, Hsinchu, Taiwan. His Ph.D. research focuses on the probabilistic risk assessment of nuclear power plants. He was a Maintenance Engineer 1270 (for a period of 30 months) for the Sierra Leone affiliate of the French oil giant, 1271 TOTAL.

Mr. George-Williams received the Best Project Award from the Sierra Leone 1273 Institute of Engineers in recognition of his outstanding execution of his final 1274 B.Eng. project.

Min Lee received the Bachelor's and Master's degrees from the Department 1277 of Nuclear Engineering, NTHU, in 1977 and 1979, respectively, and the Ph.D. 1278 degree in nuclear engineering from the Massachusetts Institute of Technology, Cambridge, MA, USA, in 1985.

He is a Distinguished Professor with the Department of Engineering and 1281 System Science, National Tsing Hua University (NTHU), Hsinchu, Taiwan. 1282 He briefly worked at the Brookhaven National Laboratory after his Ph.D. and 1283 joined the Department of Engineering and System Science, NTHU, in 1989. He 1284 has held several administrative positions at NTHU, including Chairman of ESS Department, Vice President and Chief of Staff, Vice President of General Affairs, 1286 and Vice President of Student Affairs. He has also been on the board of directors 1287 of the Taiwan Power Company (a government-owned public utility) for 14 years and a member of the Nuclear Safety Committee of the same company for 12 1289 years. His research fields are probabilistic risk assessment of nuclear power 1290 plants, light water reactor severe accident phenomenology and management, 1291 source term characterization of nuclear power plants, heat transfer, and system thermal-hydraulic analyses of light water reactors.

Edoardo Patelli received the Graduate degree in nuclear engineering from the 1295 Politecnico di Milano, Milano, Italy.

He carried out his doctoral work in radiation science and technology from 1297 Politecnico di Milano in the group of Professor Marzio Marseguerra and Enrico Zio. He then moved as a Research Associate to the University of Innsbruck, 1299 Austria, in the group of Professor Schuëller. He is a Co-Principal Investigator 1300 of the Centre for Doctoral Training in Quantification and Management of Risk and Uncertainty in Complex Systems and Environments and a member of the 1302 Centre for Doctoral Training in "Next-Generation-Nuclear." He is a member of 1303 the Institute for Risk and Uncertainty, University of Liverpool, U.K., and an 1304 honorary member of the National Tsing Hua University, Hsinchu, Taiwan. He 1305 has published more than 200 contributions in international journals and pro- 1306 ceedings of international conferences. He has supervised more than 20 Ph.D. 1307 students on site and in collaboration with international partners.

Dr. Patelli is the Chair of the technical committee on simulation for safety 1309 and reliability analysis (European Safety and Reliability Association), a Guest- 1310 Editor of international journals (e.g., the International Journal of Reliability and 1311 Safety and Structural Safety), and has editorship of Springer's Encyclopaedia of 1312 Earthquake Engineering. He has also organized multidisciplinary international 1313 conferences on risk and vulnerability (e.g., ASCE-ICVRAM-ISUMA 2014, 1314 IPW2015) and a number of mini-symposia in different international confer- 1315

1316 1317