Effective fuel temperature of wwer-1000

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Abstract. The main temperature characteristics of a pressurized water reactor are distinguished, supporting its safety and reliable operation. The special role of the uranium fuel effective temperature is emphasized and the accuracy of the analytical determination of the power effect is increased. The calculation of the temperature distribution along the radius of the fuel rod was carried out taking into account the temperature dependence of the thermal conductivity UO_2. The design procedure was corrected for using the Finca-Ronchi dependence for the thermal conductivity of 95% density of the theoretical one.

1 Primary temperature characteristics of reactor

These include the following integral and differential characteristics [1, 2].

The temperature effect of reactivity (TER), which is defined by the difference of reactivity, caused by the same temperature change from the cold to the hot state of all materials in reactor core (RC). The initial temperature of the cold reactor is set to 20-40 °C. Hot reactor temperature varies with a minimum controlled power level due to external heat sources. Fuel reactivity effect is responsible for forming the neutron spectrum and its leakage.

The power reactivity effect (PRE) ρ_N is determined by the mean or effective temperature of uranium fuel T_{eff} and the actual presence an isotope ^{238}U in it. Due to the Doppler effect, there are resonance levels of the uranium isotope broadening with temperature rise, which increases the neutron absorption probability, thereby reducing reactivity. The higher the fuel temperature and the lower its concentration, the greater the effect. Also, magnitude ρ_N is important for to assess the mode extension fuel campaign using the power effect reactivity.

Differential characteristics: isothermal $\alpha_T = d\rho/dT$ and power $\alpha_N = d\rho_N/dN$ reactivity coefficients determine self-regulation, self-protection, reliability and safety of the nuclear reactor. Both coefficients to provide these nuclear reactor properties must be negative: the first is near the operative point, the second is on the entire range of power transient.

The fuel effective absolute temperature is determined by the exact integral formula

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$$T_{\text{эф}} = \frac{\int_0^{r_T} T_T(r) 2\pi r dr}{\int_0^{r_T} 2\pi r dr} \tag{1}$$

Or approximate expression [1]

$$T_{\text{эф}} = T_c + 0.4(T_{max} - T_c), \tag{2}$$

where

$T_T(r)$ – temperature distributions over the fuel block;

T_{max} – temperatures on the axis of the fuel block;

T_c – temperatures on the surface of the fuel block.

2 Characteristic comparison on the coefficient of thermal conductivity

The method of direct explicit calculation $T_T(r)$ using the $\lambda_{UO_2}(T)$ is presented in handbooks [3, 4]. Research activity was based on the results [5], where the theoretical dependence of the thermal conductivity UO$_2$ was used as the equation:

$$\lambda_{UO_2} = \frac{10^2}{3.77 + 0.0258 \cdot T} + 1.1 \cdot 10^{-4} \cdot T + 1.01 \cdot 10^{-11} \cdot T^3 \cdot \exp(7.2 \cdot 10^{-4} \cdot T). \tag{3}$$

There is also the empirical formula:

$$\lambda_{UO_2} = 11.5 - 1.14 \cdot 10^{-2} \cdot T + 4.4 \cdot 10^{-6} T^2 - 5 \cdot 10^{-10} \cdot T^3, \tag{4}$$

which could be approximated by

$$\lambda_{UO_2} = \lambda_a \text{ch}\left[(T - a)/b\right], \tag{5}$$

choosing the constants: $\lambda_a = 2.3$ W/(m·K); $a = 2000\,\text{K}$; $b = 870\,\text{K}$.

More recent works are recommended to use the Fink-Ronchy formula to determine the thermal conductivity with density of 95%:

$$\lambda_{UO_2} = \frac{100}{7.5408 + 17.692 \cdot \tau + 3.6142 \cdot \tau^2} + \frac{6400}{\tau^{5/2}} \exp\left(-\frac{16.35}{\tau}\right), \tag{6}$$

where

$\tau = (T/1000)\text{K}$.

The equations (3) - (6) are presented in Fig. 1 and show the necessity of the coefficients adjustment in dependence (5).

3 Updating of the design formula for the fuel block

We take the Fink-Ronchy formula (6) as the main dependence for the coefficient of the thermal conductivity UO$_2$ and approximate it by the trigonometric function (5)

$$\lambda_{\mathrm{UO}_2} = \lambda^*_a \mathrm{ch}\left[\left(T - a^*\right)/b^*\right],\tag{7}$$

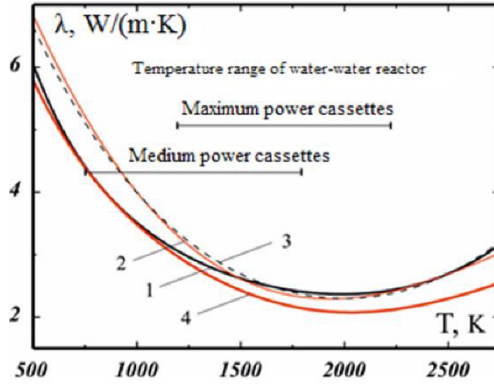which has the following constants: $\lambda^*_a = 2.1$ W/(m·K); $a^* = 2070$ K; $b^* = 980$ K.

Fig. 1. Comparison of approximations using the thermal conductivity UO_2:
1 – theoretical dependence (3) of work [5];
2 – empirical formula (4), [5];
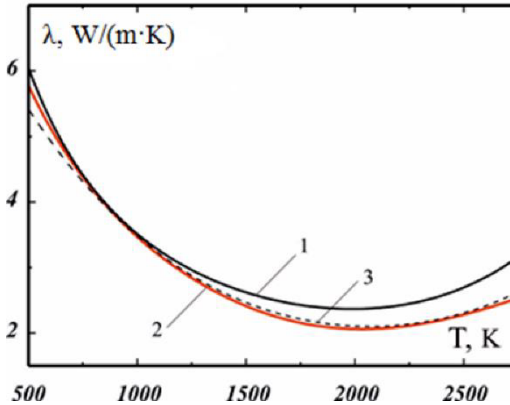3 – formula (5);
4 – Fink-Ronchi ratio (6).

Fig. 2. Updating of approximation using the thermal conductivity UO_2:
1 – theoretical dependence (3) of work [5];
2 – Fink-Ronchi ratio (6);
3 – formula (7).

4 Analysis of results

Carrying out estimations on the proposed dependencies allows us to draw the following conclusions:

• the approximation formula for determining the effective fuel temperature (2) is in error slightly more than 3%;

• the acceptable trigonometric approximation (7) of the Fink-Ronchi formula has been received; it will allow calculating the radial temperature distribution using the temperature dependence of the uranium dioxide thermal conductivity (8);

- comparison with the known results of thermal design for the reactor fuel rods (WWER– 1000 [9]) showed slightly lower values of the temperature field; this indicates, first of all, the necessity of using the effect of density changes and deviations from stoichiometry, which requires further analysis.

References

1. G.G. Bartholomew, G.A. Bat', V.D. Bajbakov, M.S. Alhutov, *Fundamentals of the Theory and Methods for Calculating Nuclear Power Reactors* (YULAND, Yekaterinburg, 2016) [in Russian]
2. V.I. Vladimirov, *Physics of Nuclear Reactors: Practical Tasks for Their Operations* (Knizhnyy dom "LIBROKOM", Moscow, 2009) [in Russian]
3. P.L. Kirillov, Yu.S. Yur'ev, V.P. Bobkov, *Handbook of Thermohydraulic Calculations (Nuclear Reactors, Heat Exchangers, Steam Generators)* (Energoatomizdat, Moscow, 1990) [in Russian]
4. P.L. Kirillov, A.V. Zhukov, N.I. Loginov, V.M. Makhin, I.L. Pioro, Yu.S. Yur'ev, Handbook of Thermohydraulic *Calculations in Nuclear Power. Volume 2. Nuclear Reactors, Heat Exchangers, Steam Generators* (IzdAt, Moscow, 2013) [in Russian]
5. V.I. Kolyadin, E.P. Il'in, A.G. Kharlamov, V.V. Yakovlev, Atomic Energy, **36**(1), 59 (1974) [in Russian]
6. Thermophysical Properties Database of Materials for Light Water Reactors and Heavy Water Reactors: Final Report of a Coordinated Research Project 1999-2005; IAEA-TECDOC-1496. – Vienna: IAEA, 2006.
7. P.L. Kirillov, M.I. Terent'eva, N.B. Deniskina, *Thermophysical properties of nuclear materials* (IzdAt, Moscow, 2007) [in Russian]
8. Yu.G. Godin, A.V. Tenishev, V.V. Novikov, *Physical Material Science. Volume 6, Part 2. Nuclear Fuel Materials* (MIFI, Moscow, 2008) [in Russian]
9. A.M. Afrov, S.A. Andrushechko, V.F. Ukraintsev, *Pressurized Water Reactor-1000: Physical Bases of Operation, Nuclear Fuel, Safety System* (Logos, Moscow, 2006) [in Russian]